OPERATING DATA REPORT

DOCKET NO.	50-302
UNIT	FLCRP-3
DATE	January 10, 1992
COMPLETED BY	J. A. Binkowski
TELEPHONE	(904) 563-4485

OPERATING STATUS

9201270264 920110 PDR ADDCK 05000302

PDR

۹.	UNIT NAME:	CRYSTAL RIVER U(IT 3
2.	REPORTING PERIOD:	NOVEMBER 1-30 1201
3.	LICENSED THERMAL POWER (MWI):	2544
4,	NAMEPLATE RATING (GROSS MWe):	nr

8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS NUMBER 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:

NA

	A second	The second s
9.	POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWe):	N/A
	the second	1205
10	REASONS FOR RESTRICTIONS, IF ANY:	N/A
1.0.	REASONS FOR RESTRICTIONS, IF ANT.	N/A

	THIS MONTH	YR. TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	720.0	8,016.0	129,024.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	98.5	6,833.7	83,482.5
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	1,280.6
14. HRS GENERATOR ON LINE	97.2	6,785.6	81,795.7
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
16. GROSS THERMAL ENERGY GENERATED (MWH)	210,676	15,893,509	185,397,715
17. GROSS ELECTRICAL ENERGY GENERATED (MWH)	71,385	5,410,052	63,298,517
18. NET ELECTRICAL ENERGY CENERATED (MWH)	67,727	5,147,591	60,122,141
19. UNIT SERVICE FACTOR	13.5%	84.7%	63.4%
20. UNIT AVAILABLE FACTOR	13.5%	84.7%	63.4%
21. UNIT CAPACITY FACTOR (using MDC net)	11.5%	78.2%	57.8%
22. UNIT CAPACITY FACTOR (using DER net)	11.4%	77.8%	56.5%
23. UNIT FORCED OUTAGE RATE	23.3%	2.0%	19.2%
	New Work of the second statement of the database	and the second s	We can be a set of the

24. SHUTDOWNS SCHEDULED OVER NEXT SIX MONTHS (TYPE, DATE, AND DURATION OF EACH): Refuel 8 commencing on 4/30/92; duration of 56 days.

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 26. UNIT: IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):		N/A	
THIS TEM IS NOT APPLICABLE TO CR	1-3	E /PECAST	ACHIEVED
	INITIAL CRITICALITY	AK	NA
	INITIAL ELECTRICITY	NA	NA
	COMMERCIAL OPERATION	NA	NA

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-302		
UNIT	FLCRP-3		
DATE	Decembe: 03, 1991		
COMPLETED BY	J. A. Binkowski		
TELEPHONE	(904) 563-4485		

MONTH NOVEMBER

٦

DAY AVERAGE DAILY POWER LEVEL (MWe-Net)

1	0
2	0
3	0
4	0
5	0
6	0
7	0
8	0
9	0
10	0
11	0
12	0
13	0
14	0
15	0
16	0

DAY	AVERAGE	DAILY	POWER	LEVEL
	(MWe-			

17	0	
18	0	
19	0	
20	0	
21	0	
22	0	
23	0	
24	0	
25	0	
26	0	
27	386	
28	776	
29	818	
30	842	

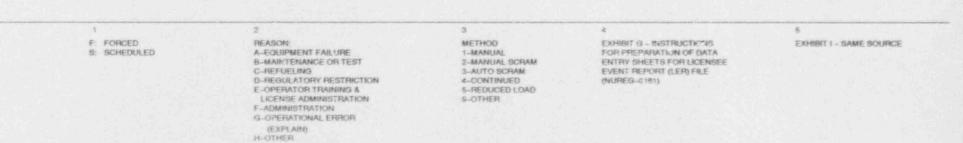
INSTRUCTIONS:

On this format, list the daily average unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

NO.	DATE	TYPE (1)	DURATION	REASON (2)	REPORT MONTH:	NOVEMBER	SYST	COMPONENT		50-302 FLCRP-3 January 10, 1902 J. A. Binkowski (904) 563-4485
			HOURS		SHUTTING DOWN REACTOR (3)	EVENT REPORT #	CODE (4)	CODE (5)	ACTION TO P	
91–40	911125	F	29.6 *	G	3	91–014	СН	INSTRU	Feedwater Pu high deaerator considered to condensate/fe startup (unit w minutes). Add level indication is being provid	n loss of both Main mps caused by r level. Root cause is be failure to stabilize edwater flow during as on ling for four itionally, deaerator in failed low. Training led to the operating vel indication was
91-41	911126	S	622.8	В	4	N/A	ZZ	777777	Completed mi outage.	d cycle maintenance

* Note : These hours are listed separately but are included in the total mid cycle duration hours.



MONTHLY OPERATIONAL SUMMARY STATEMENT

DOCKET NO. UNIT FLCRP-3 DATE COMPLETED BY TELEPHONE

50-302 January 10, 1992 J. A. Binkowski (904) 563-4485

MONTH: NOVEMBER

SUMMARY STATEMENT:

Crystal River Unit 3 completed its first planned mid-cycle maintenance outage on November 26, 1991. The outage exceeded the schedule by 10 days in order to accomodate replacement of an RCP Seal and to repair two control rod drive tube gaskets. As the plant was returning to full power, failure of the feedwater header cross tie valve and a reactor power level detector resulted in delay of the restart. An additional delay resulted when a reactor trip occurred after the plant had been on line for four minutes. The trip was attributed to high deaerator level and related instrument failure.

Some significant activities were: a) overhaul of a Raw Water Pump (RWP-3B); b) replacement of a Letdown Cooler; c) modification of a Primary Relief Valve Tailpipe (RCV-8); d) tie-in and testing of the new Turbine Building battery; e) teardown inspection of both Emergency Diesel Generators; f) repair of steam generator feedwater flange leak, and g) the leak-tightness test of the Reactor Building.