

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Davis-Besse Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 4 6 1	PAGE (3) OF 0 4
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TITLE (4)  
Reactor Trip Due to Closure of Main Steam Isolation Valve

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 3	0 2	8 4	8 4	0 0 3	0 1	0 7	2 6	8 4			0 5 0 0 0
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)											

OPERATING MODE (9) 1	POWER LEVEL (10) 0 9 9	20.402(b)	20.406(e)	XX	80.73(e)(2)(iv)	73.71(b)
		20.406(a)(1)(i)	80.38(e)(1)		80.73(e)(2)(v)	73.1(a)
		20.406(a)(1)(ii)	80.38(e)(2)		80.73(e)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)
		20.406(a)(1)(iii)	80.73(e)(2)(i)		80.73(e)(2)(vii)(A)	
		20.406(a)(1)(iv)	80.73(e)(2)(ii)		80.73(e)(2)(vii)(B)	
		20.406(a)(1)(v)	80.73(e)(2)(iii)		80.73(e)(2)(viii)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Frank Swanger	TELEPHONE NUMBER AREA CODE: 4 1 9 2 5 9 - 5 0 0 0
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
B	S, B, R, V		D 2 4 3	Y	X	B, A, I, S, V		L 2 0 0	Y
X	J, E, O, B		C 5 6 0	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)	XX NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 2, 1984, Davis-Besse Nuclear Power Station was operating at 99% of rated power. At 1221 hours, the #2 Main Steam Isolation Valve went closed, isolating the steam side of Steam Generator #2. This was caused by an undetected failed relay in one safety instrumentation channel plus routine plant testing involving a second safety instrumentation channel. The closure of Main Steam Isolation Valve #2 caused feedwater and Reactor Coolant System temperature transients that led to a reactor trip on high flux. After the trip, one of the Main Steam Safety Valves did not fully close. This caused an excessive Reactor Coolant System cooldown rate and by procedure Steam Generator #2 was allowed to boil dry. After the failed Main Steam Safety Valve had been replaced, while attempting to restore level in Steam Generator #2, the auxiliary feedwater valve to Steam Generator #2 failed to open. It was opened manually to restore level in Steam Generator #2. It was discovered later that a Main Steam Safety Valve on Steam Generator #2 had failed to lift. The failed relay circuit was repaired, the safety valve that failed to close properly was replaced. The safety valve that failed to lift has been gagged and will be repaired in the future. The auxiliary feedwater valve that failed to open had its torque switch settings changed. Analyses have shown that no design parameters were exceeded on the Reactor Coolant System or the steam generator.

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