July 24, 1984

EB ackwood

Docket No. 50-289

Mr. Henry D. Hukill, Vice President and Director - TMI-1 GPU Nuclear Corporation P. O. BOx 480 Middletown, Pennsylvania 17057

Dear Mr. Hukill:

decision review schedule.

Distribution: HOrnstein Docket File Reading File R. Bosnak, Gray File RIngram JVanVliet. NRC & L PDR OELD ACRS 10 JNGrace

EJordan We have reviewed that portion of your July 16, 1984 letter that addresses pipe stresses for high energy lines. To complete our review, we need the information described in the enclosed request for additional information. As discussed with Roy Harding, we request that you respond by July 30, 1984

The reporting and/or recordkeeping requirements contained in this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

so that we may maintain our Union of Concerned Scientists 2,206 Petition

Sincerely.

John F. Stolz, Chief Operating Reactors Branch No. 4 Division of Licensing

Enclosure: As stated

cc w/enclosure See next page

ORB#4:DL JVanVliet;ef 07/12V84

QRB#4:DL

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Request For Additional Information

TMI-1

Reference 1: Letter from H. D. Hukill to J. F. Stolz, dated July 16, 1984

- 1. Items 1, 2 and 3 of attachment No. 2 of Reference 1 stated that the design code (B31.1) allowable stress limit for the line at operating conditions is not exceeded when the line is subjected to a coincident Safe Shutdown Earthquake (SSE). Quantify allowable stress limit utilized for the piping stress evaluation. In addition, provide information to clarify (1) whether you have included pipe supports in these piping system evaluations and (2) whether the results of these analyses are within the design allowable stress limits for pipe supports. (The allowable stress limits used for the pipe support evaluation also needs to be quantified.)
- 2. Item 4 of Attachment 2 of Reference 1 stated that the Auxiliary Steam Line will not experience large seismic stresses that could cause a breach of the system's pressure boundary. However, last paragraph of Reference 1 stated that the result of the Auxiliary Steam Line piping exceeds the FSAR allowable stress limit. These two statements, taken at face value, seem contradictory. Provide additional information to clarify the above statements and to justify that the Auxiliary Steam Line will remain intact during and after an SSE.
- 3. Provide additional information to clarify the statement in Item 5 of Reference 1 that "the piping downstream of COV-14A/B from the Intermediate Building to a point inside the trench of the Turbine Building may be considered to have seismic resistance based on an analysis done in response to IEB 79-14". Provide additional information, similar to that requested in question 2, to justify that the subject piping will remain intact during and after an SSE.
- 4. Provide a detailed discussion to justify the statement that the vent stacks for the main steam safety valves are judged adequate to withstand an SSE.