

NUREG-0748  
Vol. 4, No. 6

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# OPERATING REACTORS LICENSING ACTIONS SUMMARY

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UNITED STATES NUCLEAR REGULATORY COMMISSION



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**Vol. 4, No. 6**

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# **OPERATING REACTORS LICENSING ACTIONS SUMMARY**

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Prepared for: **OFFICE OF NUCLEAR REACTOR REGULATION**

Prepared by: **OFFICE OF RESOURCE MANAGEMENT**  
**U.S. NUCLEAR REGULATORY COMMISSION**  
**WASHINGTON, D.C. 20555**



## FOREWORD

THE OPERATING REACTORS LICENSING ACTIONS SUMMARY IS DESIGNED TO PROVIDE THE MANAGEMENT OF THE NUCLEAR REGULATORY COMMISSION (NRC) WITH AN OVERVIEW OF LICENSING ACTIONS DEALING WITH OPERATING POWER AND NONPOWER REACTORS. THESE REPORTS UTILIZE DATA COLLECTED FROM THE DIVISION OF LICENSING IN THE OFFICE OF NUCLEAR REACTOR REGULATION AND ARE PREPARED BY THE OFFICE OF RESOURCE MANAGEMENT.

THIS SUMMARY REPORT IS PUBLISHED PRIMARILY FOR INTERNAL NRC USE IN MANAGING THE OPERATING REACTORS LICENSING ACTIONS PROGRAM. ITS CONTENT WILL CHANGE BASED ON NRC MANAGEMENT INFORMATIONAL REQUIREMENTS.

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Definitions

I. Licensing Actions

A. Active Licensing Action - an item which has been initiated but not completed.

1. Multi-Plant Licensing Action - a licensing action topic that applies to a class of reactors.
2. Plant Specific Licensing Action - a licensing action associated with a certain facility because of the uniqueness of the technical specifications or situation, e.g., reduce the number of incore detectors.

B. Types of Licensing Actions

1. Amendment - action that results in changes to facility licenses including technical specification changes.
2. Letters to Licensees - letters sent to licensee requesting information that will resolve or aid in the resolution of technical issues. Examples of letters sent to licensees are: fuel handling inside containment, assymmetric loads or reactor vessel materials surveillance.
3. Orders/Exemptions - actions on operating plant that result in an Order for Modification of a License or an exemption to the regulation.

C. Categories of Multiplant Actions

1. A-Series: actions directly attributed to a specific regulation requirement, e.g., inservice inspection/testing requirements 10 CFR 50.55a(g). (Related to Regulations).
2. B-Series: action directly attributed to an occurrence of accumulative experience at licensed reactors, e.g., fire protection. (Related to Operating Experience).

3. C-Series: action resulting from NRC staff reviews of license applications, generic topics and operating reactor licensing actions, e.g., reporting requirements, fuel handling accidents, or filter tech spec requirements. (A result of internal review.)
4. D-Series: action required as a result of current data which shows that previously accepted requirements are no longer acceptable or are overly restrictive, e.g., ECCS input errors, asymmetric loads, equipment qualification. This new information may result from technological advancements, research or advanced analytical studies. (A result of new information.)
5. E-Series: action requested by the licensee that does not fit any of the other sources of licensing actions. Usually these actions are requested because of the licensee's desire to improve operational performance, e.g., core reloads, spent fuel pool modifications, or (N-1) loop operation. (A result of licensee requests.)
6. F-Series: multiplant functions which are the direct result of NUREG-0737 concerning TMI Lessons Learned.
7. G-Series: multiplant actions which are a direct result of NUREG-0660, a precursor to NUREG-0737.

## II. Significant Dates

- A. Initiation - This is the date an action was actually started or implementation was begun on a plant. It is the date that a letter was issued to the licensee initiating the action, the date of an amendment request, or the date a TAC was issued for an item arising out of an operating experience or new information event.
- B. PM-Target - project manager's anticipated completion date for active items.
- C. Licensing Action Complete - date on which all work, both technical and administrative was completed and sent out of the agency.



III. Definition of Priorities (Applicable to Multi-Plants or Plant Specific Actions)

A. High (1)

1. Required by Rule, Order or NRC Letter
2. Safety-related items that use definite improvements in safety or would reduce the likelihood of transients or accidents.
3. Hearing-related matters.
4. High public interest in issue.
5. Actions which, if not completed, will affect power level or scheduled restart of a facility.
6. §2.206 petitions.

B. Medium (2)

1. Items which, if not completed could result in an otherwise unnecessary IE enforcement action.
2. Spent fuel pool expansions.
3. Task should be completed within six months.

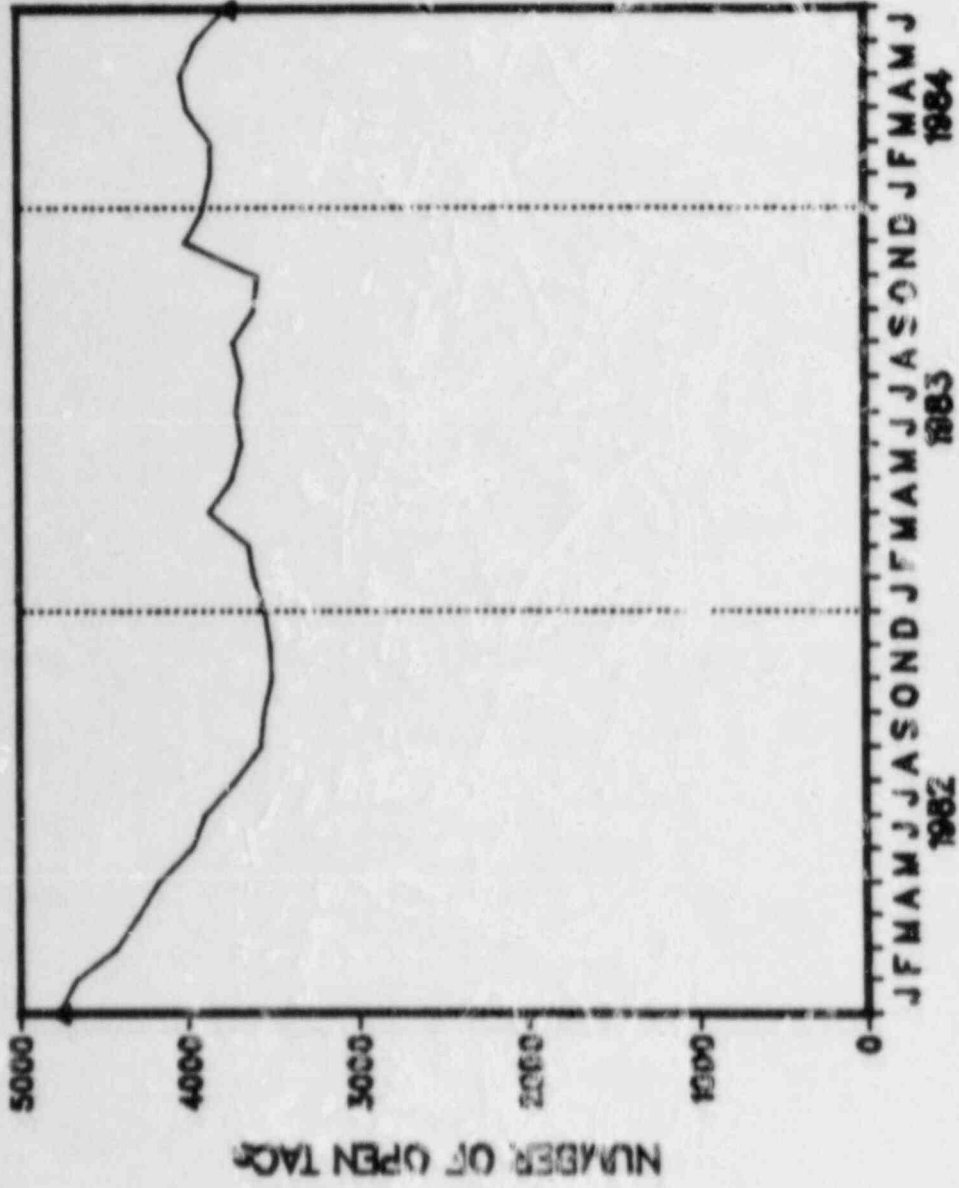
C. Low (3)

1. Licensee requests for amendments that have low safety significance.
2. Licensee requests for amendments which may have safety significance but for which NRC action is not required during the next six months (e.g., in early submittal of a core reload).

SECTION I

MANAGEMENT OVERVIEW REPORT

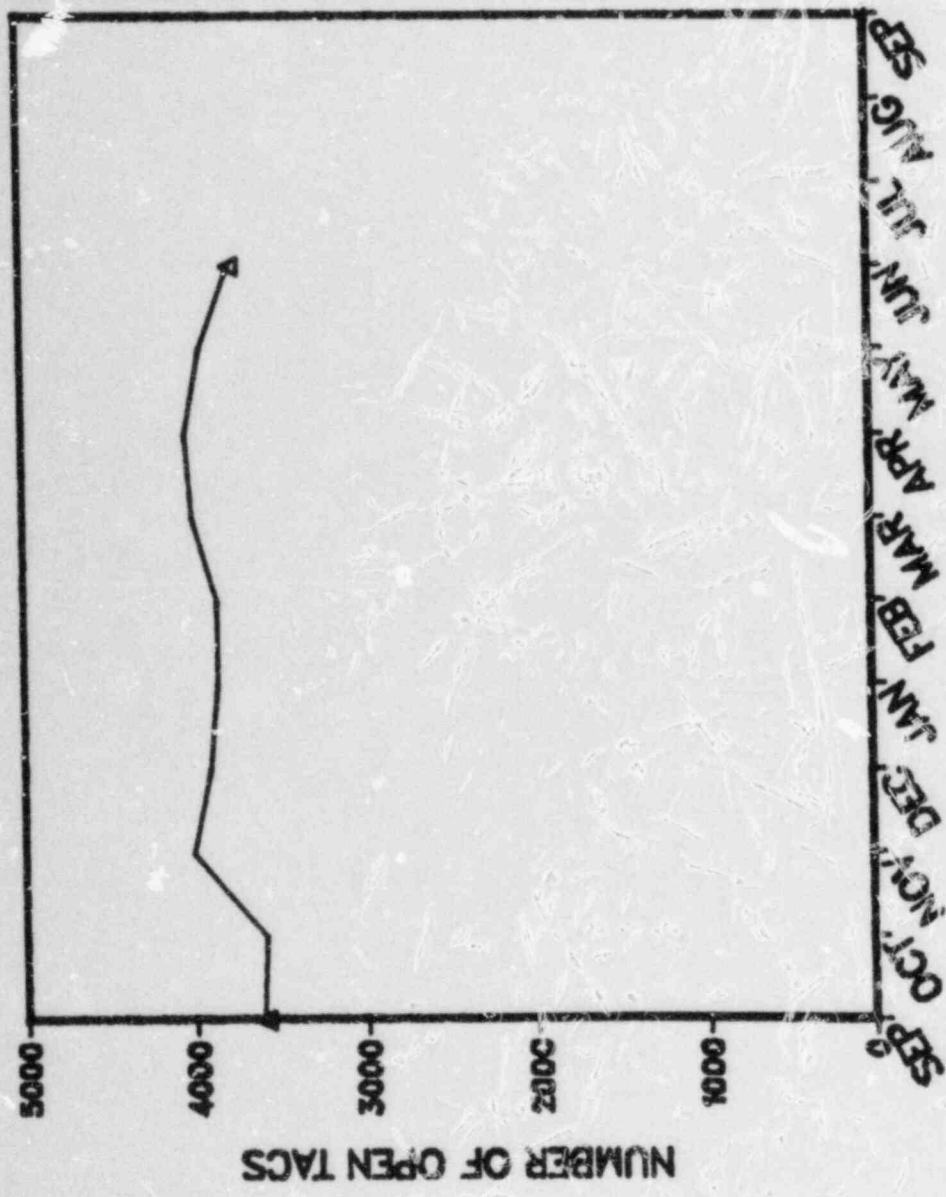
# HISTORICAL VIEW OF LICENSING ACTION INVENTORY







# TOTAL LICENSING ACTION INVENTORY DURING FY84



R-1208619-001

REPORT DATE: 07/16/84

TECHNICAL ASSIGNMENT CONTROL SYSTEM  
FY TO DATE MULTI-PLANT & PLANT SPECIFIC ACTION SUMMARY  
10/01/83 - 06/30/84

PAGE: D-1

	<u>NEW LICENSING ACTIONS</u>	<u>COMPLETED LICENSING ACTIONS</u>	<u>ACTIVE LICENSING ACTIONS</u>
<u>MULTI-PLANT ACTIONS</u>			
REGULATIONS	2	31	139
OPERATING EXPERIENCE	480	182	731
INTERNAL REVIEWS	1	51	122
NEW INFORMATION	0	82	43
LICENSE REQUIREMENTS	0	0	12
NUREG-0737 ACTIONS	19	400	912
NUREG-0660 ACTIONS	0	0	1
SUBTOTALS	502	746	1,950
<u>PLANT SPECIFIC ACTIONS</u>	1,400	1,122	1,799
TOTAL	1,902	1,868	3,759



FISCAL YEAR: 1984

## COMPLETED LICENSING ACTIONS BY FEE CLASS

REPORT DATE: 07/13/84

FEE CLASS	COMPLETED ACTIONS		MANHOURS		AVG HOURS/ACTION		AVG COST/ACTION	
	JUNE 1984	FY TO DATE	TOTAL FY 84	TOTAL TO DATE	FOR FY 84	TO DATE	FIXED BY REG	ACTUAL @ \$40/HR
NONE	229	1,305	31,311.6	94,826.6	23.9	72.6	N/A	\$2,904
1	14	171	4,655.5	7,217.1	27.2	42.2	400	\$1,688
2	9	108	2,242.5	3,942.1	20.7	36.5	1,200	\$1,460
3	23	255	12,677.3	21,103.6	49.7	82.7	4,000	\$3,308
4	5	47	2,651.0	5,434.1	56.4	115.6	12,300	\$4,624
5	0	1	8.0	35.0	8.0	35.0	25,800	\$1,400
6	0	0	0.0	0.0	0.0	0.0	45,900	\$0
TOTAL	280	1,887	53,545.9	132,558.5	28.3	70.2		\$15,384

REPORT DATE: 07/13/84

## MANPOWER SUMMARY BY FACILITY

FOR 06/01/84 - 06/30/84

FACILITY	MANHOURS EXPENDED LAST MONTH ON				MANHOURS EXPENDED TO DATE FY 1984	AVG HOURS/COMPLETED ACTION FOR		
	OR ROUTINE ACTION	HEARINGS	TMI	TOTAL		GENERIC	PLANT SPECIFIC	TOTAL
ARKANSAS 1	123.5	0.0	6.0	130.0	1,672.3	222.0	83.5	121.2
ARKANSAS 2	218.0	0.0	37.0	255.5	2,093.5	48.7	93.1	84.0
BEAVER VALLEY 1	110.1	0.0	48.0	158.1	2,305.5	26.5	35.7	33.8
BIG ROCK POINT 1	106.0	0.0	4.0	110.0	1,146.0	59.5	216.9	150.3
BROWNS FERRY 1	141.0	0.0	11.0	152.5	1,701.1	35.0	45.6	41.4
BROWNS FERRY 2	55.0	0.0	8.0	63.5	740.0	26.7	38.1	33.0
BROWNS FERRY 3	90.0	0.0	6.0	96.5	661.0	86.4	8.5	46.0
BRUNSWICK 1	104.5	0.0	81.0	186.0	1,881.0	73.1	58.5	64.4
BRUNSWICK 2	70.5	0.0	3.0	74.0	1,906.5	42.7	66.7	59.5
CALLAWAY 1	0.0	0.0	0.0	0.0	0.0	0.0	0.0	0.0
CALVERT CLIFFS 1	32.0	0.0	36.0	68.5	1,234.9	13.6	41.7	35.0
CALVERT CLIFFS 2	34.5	0.0	0.0	35.0	439.9	11.0	10.2	10.4
COOK 1	47.1	0.0	1.0	48.1	1,447.9	18.5	139.3	63.8
COOK 2	251.1	0.0	0.0	251.1	1,090.9	5.1	104.9	57.6
COOPER STATION	114.0	0.0	203.0	317.0	1,362.0	80.6	45.1	62.8
CRYSTAL RIVER 3	194.5	0.0	25.0	219.5	1,761.0	117.6	39.1	80.7
DAVIS-BESSE 1	93.0	0.0	107.0	200.5	1,663.0	114.1	35.4	61.6
DIABLO CANYON 1	556.1	289.0	30.0	875.1	8,419.3	0.0	0.0	0.0
DRESDEN 1	0.0	0.0	0.0	0.0	51.5	0.0	50.5	50.5
DRESDEN 2	123.5	0.0	15.0	139.0	2,155.0	37.8	27.6	31.8
DRESDEN 3	77.0	0.0	6.0	83.5	1,458.5	31.8	82.3	62.1
DUANE ARNOLD	124.0	0.0	41.0	165.0	1,782.0	55.3	62.5	59.4
FARLEY 1	101.1	0.0	54.0	155.1	1,770.7	18.6	56.8	47.0
FARLEY 2	93.1	0.0	33.0	126.1	728.1	9.5	29.1	23.8
FITZPATRICK	191.5	0.0	60.0	251.5	2,339.6	73.0	144.1	115.7
FORT CALHOUN 1	231.5	0.0	19.0	250.5	3,136.3	109.0	158.8	132.6
FORT ST VRAIN	16.0	0.0	20.0	36.0	1,187.0	57.7	28.0	34.5
GINNA	178.1	0.0	11.0	189.1	1,964.4	295.8	45.4	139.3
GRAND GULF 1	1,204.5	0.0	4.0	1,208.5	7,920.5	0.0	89.3	89.3
HADDAM NECK	38.0	0.0	15.0	53.0	1,772.3	99.3	42.5	72.8
HATCH 1	147.0	0.0	29.0	176.5	1,261.2	66.1	54.0	59.2
HATCH 2	170.0	0.0	7.0	177.5	1,236.3	36.2	55.6	47.5
HUMBOLDT BAY 3	0.0	0.0	0.0	0.0	140.5	0.0	68.5	68.5
INDIAN POINT 2	116.1	0.0	22.0	138.1	1,683.9	97.1	49.0	79.6
INDIAN POINT 3	76.1	0.0	0.0	76.1	2,099.9	82.4	149.3	102.1

REPORT DATE: 07/13/84

## MANPOWER SUMMARY BY FACILITY

FOR 06/01/84 - 06/30/84

FACILITY	MANHOURS EXPENDED LAST MONTH ON				MANHOURS EXPENDED TO DATE FY 1984	AVG HOURS/COMPLETED ACTION FOR		
	OR ROUTINE ACTIONS	HEARINGS	TMI	TOTAL		GENERIC	PLANT SPECIFIC	TOTAL
KEWAUNEE	58.1	0.0	26.0	84.6	853.4	16.2	61.8	39.0
LA CROSSE	144.0	0.0	6.0	150.0	2,142.8	87.9	29.0	54.6
LASALLE 1	42.5	0.0	1.0	43.5	398.5	42.0	20.4	25.2
LASALLE 2	52.5	0.0	0.0	52.5	276.5	0.0	44.8	44.8
MAINE YANKEE	118.0	0.0	17.0	135.0	2,082.4	135.0	90.9	109.8
MCGUIRE 1	116.6	0.0	109.0	225.6	1,441.3	0.0	73.2	73.2
MCGUIRE 2	51.5	0.0	4.0	55.5	705.0	0.0	39.2	39.2
MILLSTONE 1	170.0	0.0	37.0	207.0	2,038.0	65.2	33.5	41.1
MILLSTONE 2	75.5	0.0	21.0	97.0	1,307.8	42.8	131.7	95.5
MONTICELLO	57.0	0.0	16.0	73.0	2,019.5	88.1	88.2	88.2
NINE MILE POINT 1	264.0	0.0	7.0	271.0	2,214.5	135.3	120.1	127.1
NORTH ANNA 1	83.6	0.0	25.0	108.6	1,301.9	22.9	35.0	30.7
NORTH ANNA 2	54.6	0.0	1.0	55.6	1,189.9	16.0	19.3	17.9
OCONEE 1	99.0	0.0	17.0	116.0	876.5	47.6	21.1	37.7
OCONEE 2	72.0	0.0	16.0	88.0	662.0	53.4	45.5	50.6
OCONEE 3	43.0	0.0	17.0	60.0	482.5	17.4	23.7	19.7
OYSTER CREEK 1	470.0	0.0	10.0	480.0	3,558.5	20.3	134.6	80.8
PALISADES	238.5	0.0	7.0	245.5	2,226.3	66.6	98.6	82.0
PEACH BOTTOM 2	93.5	0.0	98.0	192.0	2,072.0	64.6	105.8	89.3
PEACH BOTTOM 3	118.0	0.0	1.0	119.5	796.5	20.4	27.4	24.0
PILGRIM 1	403.0	0.0	16.0	419.5	2,128.0	58.2	85.0	71.6
POINT BEACH 1	127.6	0.0	3.0	131.1	990.9	29.2	28.6	28.8
POINT BEACH 2	33.1	0.0	3.0	36.6	618.4	17.0	26.8	22.5
PRAIRIE ISLAND 1	62.1	0.0	17.0	79.1	846.9	17.6	42.7	30.1
PRAIRIE ISLAND 2	39.1	0.0	8.0	47.1	603.9	17.3	32.4	24.9
QUAD CITIES 1	116.0	0.0	9.0	125.0	696.5	100.6	24.9	73.5
QUAD CITIES 2	42.0	0.0	9.0	51.5	893.0	52.3	62.5	57.1
RANCHO SECO 1	284.0	0.0	22.0	306.0	2,786.3	88.6	211.5	147.6
ROBINSON 2	150.6	0.0	28.0	178.6	2,589.4	109.1	280.6	177.7
SALEM 1	38.1	0.0	34.0	72.1	828.9	22.4	55.2	35.5
SALEM 2	44.1	0.0	20.0	64.1	553.9	23.0	40.1	30.1
SAN ONOFRE 1	88.6	0.0	10.0	98.6	1,829.4	30.9	159.8	101.2
SAN ONOFRE 2	125.5	0.0	23.0	148.5	1,885.7	60.0	50.7	52.3
SAN ONOFRE 3	58.0	0.0	8.0	66.0	828.2	38.3	21.8	24.9
SEQUOYAH 1	30.1	0.0	20.0	50.1	938.8	18.0	32.4	28.9

REPORT DATE: 07/13/84

## MANPOWER SUMMARY BY FACILITY

FOR 06/01/84 - 06/30/84

FACILITY	MANHOURS EXPENDED LAST MONTH ON				MANHOURS EXPENDED TO DATE FY 1984	AVG HOURS/COMPLETED ACTION FOR		
	OR ROUTINE ACTIONS	HEARINGS	TMI	TOTAL		GENERIC	PLANT SPECIFIC	TOTAL
SEQUOYAH 2	6.1	0.0	9.0	15.1	771.8	12.0	32.5	29.7
ST LUCIE 1	26.0	0.0	13.0	39.5	1,338.8	80.9	139.8	110.3
ST LUCIE 2	130.5	0.0	2.0	132.5	532.0	2.0	29.5	26.4
SUMNER 1	283.0	0.0	56.0	339.0	2,452.0	8.0	83.0	69.3
SURRY 1	58.1	0.0	5.0	63.1	342.9	61.4	71.7	65.4
SURRY 2	17.1	0.0	5.0	22.1	283.9	44.7	20.4	35.0
SUSQUEHANNA 1	63.5	0.0	16.0	79.5	1,466.5	8.0	54.8	52.9
SUSQUEHANNA 2	88.0	0.0	0.0	88.0	191.0	0.0	20.7	20.7
THREE MILE ISLAND 1	829.0	195.5	85.0	1,109.5	9,014.9	91.5	404.1	306.0
TROJAI	240.1	0.0	5.0	245.1	2,250.6	60.0	103.3	88.9
TURKEY POINT 3	222.1	0.0	22.0	244.1	1,560.8	101.0	83.6	93.0
TURKEY POINT 4	87.1	0.0	14.0	101.1	935.8	59.3	52.6	56.4
VERMONT YANKEE 1	301.5	0.0	29.0	330.5	1,765.5	93.8	77.5	86.5
WASHINGTON NUCLEAR	79.0	0.0	0.0	79.0	355.0	0.0	7.0	7.0
YANKEE-ROWE 1	229.5	0.0	21.0	252.0	1,947.9	68.5	75.0	71.1
ZION 1	134.1	0.0	0.0	134.1	3,137.9	39.5	130.7	95.2
ZION 2	47.1	0.0	0.0	47.1	836.4	16.0	57.1	41.1
GRAND TOTAL	12,184.0	484.5	1,903.0	14,571.5	141,375.1	61.6	75.8	70.2



TABLE G

OPERATING POWER REACTORS HEARING

Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
Big Rock Point	R. Emch	M. Karman	Mixed Oxide Fuel	8/75	In indefinite suspension as a result of GESMO decision.
Big Rock Point	R. Emch	R. Goddard R. Bachman	Spent Fuel Pool Expansion	9/25/79	Evidentiary hearings held 6/7/82-6/12/82 and 10/25/83-11/4/83 covering all contentions - emergency preparedness, criticality, pool structural integrity, crane seismic capability, cask drop, aircraft overflights, shielding, EIA, and reliability of pool make-up line. NRC's proposed findings of fact and conclusions of law filed on Feb. 3, 1984. NRC and Licensee are being deposed by Intervenors. Deposition on High Range Stack Gas Monitor on 6/4/84. All other issues are before Board for decision.
Ginna	G. Dick	M. Young	Provisional Operating License Conversion to a Full Term Operating License	6/8/73	FTL SER issued October 1983; ACRS letter issued April 9, 1984, ASLB issued notice on 12/2/83. There was one response notifying the ASLB of a possible nontimely filing of petition for leave to intervene. Prehearing conference scheduled for June 13, 1984.



Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
Humboldt Bay	P. Erickson	M. Young	Deletion of Seismic Conditions	7/29/77	The ASLB ordered that six months after the Commission reaches a decision on safety goals and issues guidelines the licensee must report to the Board (a) readiness to take actions to permit resumed operation; or (b) commit to submit a decommissioning plan. By letter dated July 7, 1983, the licensee committed to submit a decommissioning plan to the Commission. The licensee has also requested a meeting with the staff to discuss decommissioning. A meeting was held at plant with licensee on 10/18/83. Licensee plans to submit decommissioning plan in June 1984.
Indian Point 2/3	P. Polk	J. Moore	Risk Assessment of Plant		Comments from interested parties received 2/7/84. Comments on Board findings have been received by Commission. Commission has received comments and NRR is awaiting Commission direction.

Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
LaCrosse	R. Dudley	C. Woodhead	Full Term Operating License (with seismic considerations)	6/23/78	The FTOL proceeding is contingent upon the completion of the SEP for the facility. Seismic considerations were resolved by ASLB Decision dated 4/21/83; affirmed by Appeal Board on 7/13/83 and finalized by Commission decision on September 15, 1983.
North Anna 1/2	L. Engle	D. Swanson	Turbine Missile Risk	2/27/78	On June 2, 1982, the ASLAB issued ASAB-676 wherein the Appeal Board delineated its decision resolving the North Anna Turbine Missile issue. All issues in this proceeding are terminated other than the radon emissions in the milling & mining of uranium fuel. The radon issues will be addressed following resolution in other hearings which are contested.

Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
North Anna 1/2	L. Engle	D. Swanson	Spent Fuel Pool Capacity Expansion	12/8/82	Pursuant to 10 CFR §2.751a, a notice of hearing was issued on December 8, 1982 by the ASLB and a prehearing conference was held February 16, 1983 to consider VEPCO's proposed amendments to the NA 1/2 Operating License regarding the transfer and storage of Surry spent fuel at NA and the expansion of the NA spent fuel storage capacity. On February 18, 1983 the ASLB issued two Orders. One admitted the "County" as an intervenor and the other deferred ruling on other contentions pending briefings on two issues. Interrogatories now underway. By Order dated June 10, 1983 the ASLB ruled that it had jurisdiction to consider the health and safety as well as environmental impact regarding the transshipment of spent fuel from Surry to North Anna. On July 28, 1982 VEPCO submitted a motion for directed certification to 10 CFR §2.787(a) regarding the ASLB ruling of 6/10/83 and the ASLAB was convened on 8/10/83 and the ASLAB on 9/15/83 (ALAB-741) denied VEPCO's motion for

Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
					directed certification regarding the ASLB ruling to consider health and safety and environmental impact of transshipment of fuel from Surry to North Anna. On 11/15/83, the time provided within which the Commission may act to review ASAB-741 expired and accordingly, the decision (ALAB-741) became final agency action. On 4/27/84, the County of Louisa filed a Notice of Withdrawal from the proceeding. By Order dated 5/23/84, the ASLB granted Louisa County requests for withdrawal as an intervening party and dismissed its contentions. The staff's SER and EIA regarding VEPCO's proposed amendments are presently scheduled for issuance in early June 1984.
Three Mile Island 1	J. VanVliet	R. Rawson J. Goldberg M. Wagner	TMI-1 Restart	8/79	All ASLB Partial Initial Decisions have been issued in favor of restart; ASLAB has found in favor of restart on all issues except management which is has remanded to the ASLB. The Commission has completed final agency action on emergency planning issues and is reviewing the remaining issues.
Three Mile Island 1	J. VanVliet	M. Wagner	TMI-1 Steam Generator Repair	8/83	Hearing is scheduled to begin July 16, 1984 on two issues remaining after Summery Disposition.



Facility	Project Manager	Hearing Attorney	Subject	Initiation Date	Status
Robinson 2	G. Requa	S. Treby	Radon	11/30/82	Partial Initial Decision issued on ASLB; affirmed by ASAB on 10/31/79 (ASAL-569). Pending before ASLAB on radon only.
Robinson 2	G. Requa	M. Karman	Steam Generator Repair	1/5/83	Hearing held 2/7/84. Hearing concluded 2/9/84. Board issued Decision 2/10/84. Hearing completed since interventor withdrew all contentions.
Salem 1/2	D. Fischer	J. Moore	Integrated Leak Rate	10/21/83	State of Delaware filed motion to withdraw on 1/20/84. Withdrawal granted on 1/25/84.
Turkey Pt. 3/4	D. McDonald	M. Young	Reload	11/7/83	Board established. Prehearing conference held 2/28/84. Issued Prehearing Order dated 5/17/84. Hearing will be held and discovery initiated.
Zion 1/2	J. Norris		Containment Leakage Testing	2/13/84	Notice of Withdrawal of Application signed April 18, 1984.



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NOTE: \* UNDER EST. COMP. DATE INDICATES SCHEDULE SLIPPED

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R-3	82-21	PALISADES - IST PROGRAM FOR PUMPS AND VALVES	04/23/82	07/84 *	XX/XX/XX	1163 / 11214 / WAMBACH	MEB, ORB-5
R-2	82-22	SEQUOYAH - FIRE PROTEC- TION FEATURES	03/12/82	08/83 *	XX/XX/XX	1162 / 48599 / STAHL	CHEB, LB-4
R-2	82-33	SEQUOYAH - ROD DROP PRO- TECTION FOR WESTINGHOUSE PLANTS	03/08/82	08/83 *	XX/XX/XX	1262 / 48581 / STAHL	CPB, LB-4
R-2	82-35	MCGUIRE - LOSS OF HIGH HEAD INJECTION - CONCERN THAT TECHNICAL SPECIFICA- TION ACTION STATEMENT MAY NOT BE CONSERVATIVE	04/12/82	12/83 *	XX/XX/XX	1162 / 48395 / BIRKEL	SPEB
R-3	82-43	QUAD CITIES - ELECTRIC POWER TECHNICAL SPECIFI- CATIONS	07/26/82	08/84 *	XX/XX/XX	1163 / 48667, 48668 / BEVAN	GRAB, ORB-2
R-3	82-52	FERMI 2 - CABLE TRAY SEPARATION	08/13/82	12/83 *	XX/XX/XX	---- / 141415 / LYNCH	CHEB, LB-1
IE	82-57	HATCH 1/FITZPATRICK/PIL- GRIM - TARGET ROCK SRVS	09/29/82	10/84 *	XX/XX/XX	---- / 48771 / RIVENBARK	MEB, ORB-4
I&E	82-59	BWR CRD SYSTEM HYDRAULIC LOADS	08/31/82	07/84 *	XX/XX/XX	---- / 48843 / HOUSTON	LB-2, MEB, RSB
R-2	82-72	ROBINSON/CRYSTAL RIVER - A/FW CHECK VALVE LEAKAGE	11/26/82	08/84 *	XX/XX/XX	1162 / 49064, 49065 / REQUA	MEB, ORB-1
R-5	82-75	ALLEGED DESIGN DEFICIEN- CIES IN CLASS I PIPING	11/24/82	02/84 *	XX/XX/XX	1165 / 49242 / MARTIN	MEB, LB-2
R-5	83-01	DIABLO/PALO VERDE/WNP 1-4 CABLE TRAY & CONDUIT SUPPORTS	12/16/82	03/84 *	XX/XX/XX	1165 / 49313 / BUCKLEY	SEB, MEB, LB-3, LB-4

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R-5	83-08	SAN ONOFRE S.3/ANO-2 - ESFAS DESIGN DISCREPANCY	12/29/82	03/84 *	XX/XX/XX	---- / 49485, 49486, 49320 / R. LEE	ICSB, ORAB, LB-3
R-4	83-15	FORT ST. VRAIN - INSTRU- MENTATION SETPOINTS	01/14/83	09/84 *	XX/XX/XX	---- / 49587 / COLBURN	ICSB, ORB-3
R-3	83-19 83-19A	SURVEILLANCE TESTING OF FIRE PUMPS	02/14/83	06/84 *	XX/XX/XX	1163 / 49720 / WAMBACH	CHEB, LB-3
R-1	83-20	SALEM 1 - LONGTERM ACTION PLAN	03/02/83	12/84 *	XX/XX/XX	1161 / 49795 / FISCHER	ICSB, EQB, LQB, ORB-1
R-1	83-22	CALVERT CLIFFS - SPURIOUS ACTUATION OF RPS/SFAS	02/06/83	06/84 *	XX/XX/XX	1161 / 49584 / JAFFE	ICSB, ORAB, ORB-3
NRR	83-24	GINNA - SEP ACTION ITEMS	02/22/83	06/84	XX/XX/XX	---- / 49803 / DICK, PERSINKO	SEP, ORB-5
R-3	83-26	QUAD CITIES - TECH SPEC INTERPRET ON DC POWER SYSTEMS	02/16/83	08/84 *	XX/XX/XX	1263 / 49893, 49894 / BEVAN	SSPB, PSB, ORB-2
NRR	83-28	PALISADES SEP ACTION ITEMS	03/07/83	12/84 *	XX/XX/XX	---- / 42199 / WAMBACH, MICHAELS	SEP, ORB-5
R-3	83-33	LASALLE/BYRON/BRAIDWOOD - PIPE WHIP RESTRAINTS	03/21/83	08/84 *	XX/XX/XX	---- / 141415, 141413 / STEVENS	MEB, LB-1

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R-3	83-35	POINT BEACH - AFW SYSTEM OPERABILITY	03/22/83	11/84 *	XX/XX/XX	1163 / 51042 / 51043 / COLBURN	ASB, SSPB, ORB-3
R-5	83-38	SAN ONOFRE 2&3 - NUISANCE ALARMS	03/24/83	09/84 *	XX/XX/XX	1165 / 51315 / 51317 / ROOD	HFEB, LQB, LB-3
R-3	83-40	MIDLAND 1&2 - SOILS ISSUES	03/16/83	06/84	XX/XX/XX	1163 / 51341 / HOOD	SGEB, LB-4
NRR	83-41	MILLSTONE 1 SEP ACTION ITEMS	04/06/83	12/84	XX/XX/XX	---- / 49786 / SHEA, PERSINKO	ORB-5 SEPB
R-3	83-42	BIG ROCK POINT - FIRE DAMPERS TECH SPEC	04/13/83	09/84 *	XX/XX/XX	1163 / 51394 / EMCH	CHEB, SSPB, ORB-5
R-2	83-43	SUMMER - PIPING ANALYSIS REVIEW	04/14/83	04/84 *	XX/XX/XX	1162 / 51411 / HOPKINS	LB-1, MEB
NRR	83-45	DRESDEN 2 - SEP ACTION ITEMS	04/18/83	12/84	XX/XX/XX	---- / 52436 / GILBERT, CHERY	ORB-5 SEPB
NRR	83-46	OYSTER CREEK - SEP ACTION ITEMS	03/29/83	12/84	XX/XX/XX	---- / 49809 / LOMBARDO, CHERY	SEPB, ORB-5
R-3	83-52	LASALLE - SCRAM RESPONSE TIME TESTING	04/22/83	06/84 *	12/20/83	---- / 51494 / BOURNIA	ICSB, SSPB, LB-2
R-2	83-54	ST. LUCIE 2 - SEISMIC ANALYSIS OF PIPING	05/02/83	07/84 *	XX/XX/XX	1162 / 51554 / SELLS	MEB, ORB-3

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IE	83-58	FIRE PROTECTION INSPEC- TION ASSISTANCE APPENDIX R	05/19/83	06/84 *	XX/XX/XX	VARIOUS / 51395, 51396, 51397, 51398, 51399, 52144 / WAMBACH	CHEB, ORB-5
R-3	83-57	BIG ROCK/QUAD CITIES - CILRT	05/11/83	09/84 *	XX/XX/XX	1163 / 51557, 51611, 51612 / EMCH	CSB, ORB-5
R-1	83-58	OYSTER CREEK - INSOLATION CONDENSER TECH SPECS	02/10/84	10/84 *	XX/XX/XX	1161 / 51403 / LOMBARDO  HOPKINS	SSPD,RSB, ORC-5
R-3	83-68	FIRE PROTECTION TECHNICAL SPECIFICATIONS	07/12/83	06/84 *	XX/XX/XX	1057 / 40002 / WAMBACH	CHEB, ORB-5
NRR	83-69	YANKEE ROWE - SEP ACTION ITEMS	07/14/83	12/84	XX/XX/XX	---- / 51903 / ERICKSON, BOYLE	SEP, ORB-5
R-3	83-70	QUAD CITIES - IGI TEST PROGRAM	04/22/83	07/84 *	XX/XX/XX	1163 / 52080, 52081 / SEVAN	MEB,ORB-2
R-5	83-73	PALO VERDE - RCS DAMAGE	08/04/83	05/84 *	XX/XX/XX	---- / 141435 / LICITRA	MEB, EQB



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R-1	83-74	BEAVER VALLEY 2, HOPE CREEK, LIMERICK - HEAVY WALLED PIPING	07/26/83	03/84 *	XX/XX/XX	---- / 141431 / LAZO	MEB, LB-3
R-3	83-78	MIDLAND/CLINTON - DEFICI- CIENCIES IN HVAC	08/04/83	01/84	XX/XX/XX	---- / 141433 / HOOD	MEB, MTEB, LB-4
NRR	83-79	LA CROSSE - SEP ACTION ITEMS	08/12/83	12/84	XX/XX/XX	---- / 51880 / DUDLEY, BROWN	ORB-3, SEPB
R-3	83-80	ZION - ACTIVE FIRE BARRIERS	07/14/83	09/84 *	XX/XX/XX	1163 / 52120, 52121 / NORRIS	CHEB, SEPB, ORB-1
R-3	83-81	MONTICELLO - LOSS OF OFFSITE POWER EVENT	08/11/83	08/84 *	XX/XX/XX	---- / 52089 / ROONEY	ORAB, PSB, ORB-2
R-1	83-82	SHOREHAM - D/G CRANKSHAFT FAILURE	08/24/83	09/84 *	XX/XX/XX	---- / 141431 / CARUSO	PSB, MEB, MTEB, LB-2, ORAB
R-3	83-83	DAVIS BESSE - FIRE PRO- TECTION DEFICIENCIES	08/02/84	02/84 *	XX/XX/XX	1163 / 52156 / DEGAZIO	CHEB, ORB-4
R-5 R-3	83-87	EMERGENCY LIGHTING REQUIREMENTS	07/29/83	06/84 *	XX/XX/XX	1163 / 52038 / WAMBACH	CHEB, PSB ORB-5
NRR	83-89	HADDAM NECK - SEP ACTION ITEMS	08/15/83	10/85	XX/XX/XX	---- / 52010 / LYONS, BROWN	ORB-3 SEPB
R-2	83-90	BRUNSWICK - REFUELING TECHNICAL SPECIFICATIONS	08/30/83	07/84 *	XX/XX/XX	1162 / 52365 / 52366 / GROTENHUIS	SSPB, ORB-2



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R-5	83-91	SAN ONOFRE 2,3 - SNUBBER ALLEGATION	09/13/83	06/84 *	XX/XX/XX	1165 / 52481, 52492 / ROOD	MEB, LB-3
IE	83-92	RUDEL-GOULD MODE SWITCH MALFUNCTIONS	09/29/83	07/84 *	XX/XX/XX	1562 / 40011 / PERCH	ICSB, ORAB, LB-2
R-5	83-93	WNP-2 - CONSTRUCTION DEFICIENCIES	08/19/83	06/84 *	XX/XX/XX	---- / 141435 / AULUCK	MEB, SGEB LB-2
R-2	83-94	GRAND GULF - DELAVEL D/G EVALUATION	09/23/83	06/84 *	XX/XX/XX	---- / 52411 / HOUSTON	PSB, LB-2
R-5	83-96	SONGS 2/3 - REACTOR TRIP BREAKERS	10/07/83	03/84 *	XX/XX/XX	---- / 52527 / ROOD	EQB, LB-3
R-5	83-98	SAN ONOFRE 2,3 - RTB UV COIL ARMATURE PROBLEM	11/04/83	03/84 *	XX/XX/XX	---- / 52624, 52627 / ROOD	EQB, ORAB, LB-3
R-5	83-100	DIABLO CANYON 1 - STRUCTURAL ALLEGATIONS	10/02/84	02/84 *	XX/XX/XX	1165 / 52622 / SCHIERLING	SGEB, MEB, LB-3
R-5	83-101	DIABLO CANYON - RHR ALLEGATIONS	10/26/83	02/84 *	XX/XX/XX	1165 / 52623 / SCHIERLING	ICSB, ASB, LB-3
R-4	83-105	FT. ST. VRAIN - FIRE PROTECTION DEFICIENCIES	11/02/83	03/84 *	XX/XX/XX	1164 / 52647 / COLBURN	CHEB, ASB, ORB-3
R-3	83-106	DC COOK 2 - CONTAINMENT SPRAY ADDITIVE SYSTEM	11/04/83	07/84 *	XX/XX/XX	1163 / 52673 / WIGGINTON	CSB, ORB-1
R-3	83-107	DRESDEN 3/QUAD CITIES 2 - BWR PIPE CRACKS	11/04/83	06/84 *	XX/XX/XX	---- / 52670, 52094 / BEVAN	MTEB, RAB, ORB-2

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R-1	83-108	TMI-1&2 - FIRE BRIGADE TRAINING	09/26/83	01/84	XX/XX/XX	1161 / 52489 / VANVLIET	LQB, ORB-4
R-2	83-109	ROBINSON 2 - REACTOR TRIP SYSTEM	11/07/83	07/84	XX/XX/XX	1162 / 53222 / REQUA	ICSB, ORB-1
R-2	83-111	GRAND GULF ET.AL. - DEFI- NITION OF CORE ALTERATIONS	11/21/83	08/84 *	XX/XX/XX	1162 / 53231 / HOUSTON	SSPB, LB-2
R-1	83-112	VERMONT YANKEE - LOW LEVEL RADIOACTIVE RELEASE	11/07/83	06/84 *	XX/XX/XX	1161 / 53220 / ROONEY	METB, ORB-2
R-2	83-114	ROBINSON 2 - CONTAINMENT RECIRCULATION SPRAY	11/04/83	12/84 *	XX/XX/XX	1162 / 53223 / REQUA	CSB, ORB-1
R-2	83-115	BEILLEFONTE - FIRE PUMP/ SPRINKLER SYSTEMS REQUIRE- MENTS	11/14/83	01/84	XX/XX/XX	---- / 141432 / STAHL	CMEB, LB-4
R-3	83-116	LASALLE - TECH.SPEC. IN- TERPRETATION - SNUBBERS	11/01/83	06/84 *	XX/XX/XX	1163 / 52714 / BOURNIA	SSPB,MEB, LB-2
R-3	83-117	CALLAWAY - CLOSURE VERIFI- CATION OF CATEGORY C CHECK VALVES	11/08/83	04/84	XX/XX/XX	XXXX / 141433 / HOLONICH	MEB, LB-1
R-3	83-120	LACROSSE - BWR PIPE CRACKS	12/02/83	07/84 *	XX/XX/XX	---- / 53284 / DUDLEY	METB,RAB, ORB-5
R-1	83-121	MAINE YANKEE - MULTIPLE RTB UV CO. 1 FAILURE	12/09/84	09/84 *	XX/XX/XX	1161 / 53267 / HEITNER	ORAB,EQB, ORB-3

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IE	83-122	FIRE PROTECTION INSPEC- TION ASSISTANCE - APPEN- DIX R	10/11/83	12/84	XX/XX/XX	---- / VARIOUS / WAMBACH	CHEB, ASB, ORB-5
R-5	83-123	SONGS 2/3 - POST-ACCIDENT SAMPLING	11/09/83	02/84	XX/XX/XX	1165 / 53304, 53305 / ROOD	METB, LB-3
R-3	83-129	MIDLAND - DIESEL GENERATOR BUILDING	11/03/83	02/84	XX/XX/XX	---- / 141433 / HOOD	PSB, SGEB, MEB, LB-4
R-5	84-02	PALO VERDE - FOUNDATION SOILS ALLEGATION	12/16/83	03/84	XX/XX/XX	---- / 141435 / LICITRA	SGEB, LB-3
R-5	84-03	PALO VERDE - KAMAN RADIA- TION MONITORS	12/20/83	03/84	XX/XX/XX	---- / 141435 / LICITRA	MTEB, LB-3
R-3	84-05	LASALLE/DRESDEN - MA- TERIALS OF INDETERMINATE QUALITY	12/05/83	06/84 *	XX/XX/XX	1163 / 53421, 53422 / BOURNIA	EQB, LB-2
R-3	84-07	PALISADES - STEAM GENERA- TOR TUBE DEGRADATION	01/18/84	07/84 *	XX/XX/XX	---- / 53508 / PAULSON	METB, MEB, RAB, ORB-5
R-5	84-08	SAN ONOFRE 3 - HIGH PRI- MARY COOLANT ACTIVITY	01/19/84	06/84 *	XX/XX/XX	1165 / 53463 / H. ROOD	CPB, METB, RAB, LB-3
R-2	84-09	HATCH 2 - VENT HEADER CRACK	02/15/84	12/84 *	XX/XX/XX	1162 / 54150 / RIVENBARK	CSB, MTEB, ORAB, ORB-4
R-3	84-10	DAVIS DESSE - 50.59 REVIEWS	01/10/84	06/84	XX/XX/XX	1163 / 53448 / DEAGAZIO	ICSB, MEB, SGEB, EQB, AEB, ORB-4

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R-3	84-11	QUAD CITIES/DRESDEN - FIRE PROTECTION ISSUES	12/28/83	08/84 *	XX/XX/XX	1163 / 53501, 53502, 53499, 53500 / GILBERT	MEB, MEB, RAB, ORB-5
R-3	84-12	LASALLE - EQUIPMENT EXPO- SURE TO HIGH TEMPERATURES	01/30/84	07/84	XX/XX/XX	1163 / 54153 / BOURNIA	EQB, LB-2
R-5	84-13	PALO VERDE - LPSI PUMP PROBLEMS	02/06/84	04/84	XX/XX/XX	---- / 141435 / LICITRA	EQB, RSB, LB-3
R-1	84-14	SEABROOK - TORNADO MIS- SILE PATH	02/07/84	11/84 *	XX/XX/XX	---- / 141431 / NERSES	ASB, LB-3
R-3	84-16	MONTICELLO - PIPE CRACKS	02/29/84	09/84	XX/XX/XX	1163 / 54455 / ROONEY	MTEB, RAB, ORB-2
R-3	84-18	POINT BEACH 1 - CONTROL ROD GUIDE TUBE PIN FAILURES	03/09/84	03/84	XX/XX/XX	1163 / 54356 / COLBURN	MEB, CPB, MTEB, ORB-3
R-5	84-20	SALEM - FIRE PROTECTION	02/22/84	06/84	XX/XX/XX	1161 / 54507, 54508 / FISCHER	CHEB, ASB, ORB-1
R-1	84-21	VERMONT YANKEE - FIRE PROTECTION	02/22/84	06/84	XX/XX/XX	1161 / 54496 / ROONEY	CHEB, ASB, ORB-2



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R-4	84-22	FT. ST. VRAIN - CRACKED FUEL ELEMENTS	03/07/84	06/84	XX/XX/XX	1164 / 54494 / COLBURN	CPB, ORB-3
R-1	84-23	SUSQUEHANNA 1/2 - POWER SUPPLY TECH SPECS	02/14/84	05/84	XX/XX/XX	1161 / 54490 / PERCH	PSB, LB-2
R-5	84-24	RANCHO SECO - HYDROGEN EXPLOSION, FIRE, LOSS OF NNI	04/13/84	07/84	XX/XX/XX	1163 / 54487 / MINER	ICSB, PSRB, ORB-4
R-1	84-26	NINE MILE POINT 1 - CRD STUB TUBES CRACKS	04/03/84	07/84 *	XX/XX/XX	1161 / 54628 / HERMANN	MTEB, ORB-2
R-3	84-27	DUANE ARNOLD - MATERIALS OF INDETERMINATE QUALITY	04/03/84	08/84	XX/XX/XX	1163 / 54679 / THADANI	EQB, ORB-2
R-2	84-28	ST. LUCIE - FIRE PROTEC- TION FEATURES	04/06/84	09/84 *	XX/XX/XX	1162 / 54746 / SELLS	CHEB, ORB-3
R-3	84-31	BYRON - FIRE PROTECTION EVALUATIONS	04/19/84	06/84	XX/XX/XX	---- / 141415 / OLSHAN	CHEB, ASB, LB-1
R-4	84-32	FT. ST. VRAIN - PCRV TEN- DONS	04/13/84	08/84 *	XX/XX/XX	1164 / 54639 / COLBURN	SGEB, ORB-3
R-3	84-34	QUAD CITIES 1 - PIPE CRACKS	04/25/84	08/84	XX/XX/XX	---- / 54955 / BEVAN	MTEB, RAB, ORB-2
R-3	84-35	PALISADES - WATER HAMMER DAMAGE TO AFW SYSTEM	04/18/84	07/84 *	XX/XX/XX	1163 / 54856 / PAULSON	ACB, MTEB, ORB-5, MEB
R-2	84-36	MCGUIRE - FIRE PROTECTION INSPECTION	04/19/84	08/84	XX/XX/XX	1162 / 54878, 54879 / BIRKEL	ASB, CHEB, LB-4
R-5	84-37	TROJAN - MAIN STEAM VALVES PROBLEMS	05/04/84	07/84	XX/XX/XX	1165 / 54988 / XXXXXX	MEB, RSB, ORB-3



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R-3	84-38	QUAD CITIES - DC BATTERY 8-HR RATING	05/16/84	07/84	XX/XX/XX	1163 / 55008 / BEVAN	PSB, ORB-2
R-5	84-39	RANCHO SECO - EFFLUENT REPORT	05/18/84	07/84	XX/XX/XX	1165 / 55038 / MINER	METB, RAB, ORB-4
R-3	84-40	BYRON/BRAIDWOOD - DC SYS- TEM	05/01/84	08/84	XX/XX/XX	---- / 141433 / STEVENS	CMEB, ASB, PSB, LB-1
R-4	84-41	FT. CALHOUN - S/G TUBE FAILURE	05/29/84	06/84	XX/XX/XX	1111 / 55023 / TOURIGNY	MTEB, CHEB, RAB, ORB-3
R-2	84-42	BROWNS FERRY 3 - FIRE PROTECTION DISCREPANCIES	05/30/84	10/84	XX/XX/XX	1162 / 55142, 55143, 55144 / CLARK	CMEB, ORB-2
R-2	84-43	BROWNS FERRY 3 - PIN HOLE LEAKS	05/24/84	09/84	XX/XX/XX	---- / 55140 / CLARK	MTEB, RAB, ORB-2
R-2	84-44	OCONEE 2 - ISI FLAW EVALUATIONS	05/23/84	09/84	XX/XX/XX	1162 / 55160 / NICOLARAS	MTEB, ORB-4
R-3	84-XX	DAVIS BESSE - EVALUATION OF S/G TUBE VIBRATION	05/18/84	11/84	XX/XX/XX	1163 / XXXXX / DEAGAZIO	MEB, RSB, ORB-4
R-3	84- <sup>46</sup> <del>XX</del>	FERMI 2 - FIRE PROTECTION ISSUES	05/29/84	11/84	XX/XX/XX	---- / 141433 / LYNCH	CMEB, ASB, LB-1
R-1	84-XX	NINE MILE PT. 1 - CORE SPRAY PIPE CRACKS	05/29/84	09/84	XX/XX/XX	---- / XXXXX / HERMANN	MTEB, RAB, ORB-2
R-1	84- <sup>45</sup> <del>XX</del>	SUSQUEHANNA 2 - VACUUM BREAKER VALVES	05/29/84	09/84	XX/XX/XX	1161 / XXXXX / PERCH	CSB, ICSB, LB-2
R-3	84- <sup>47</sup> <del>XX</del>	CALLAWAY - EXHAUST GAS IN CONTROL ROOM	05/22/84	10/84	XX/XX/XX	---- / 141433 / HOLONICH	AEB, LB-1 SAB

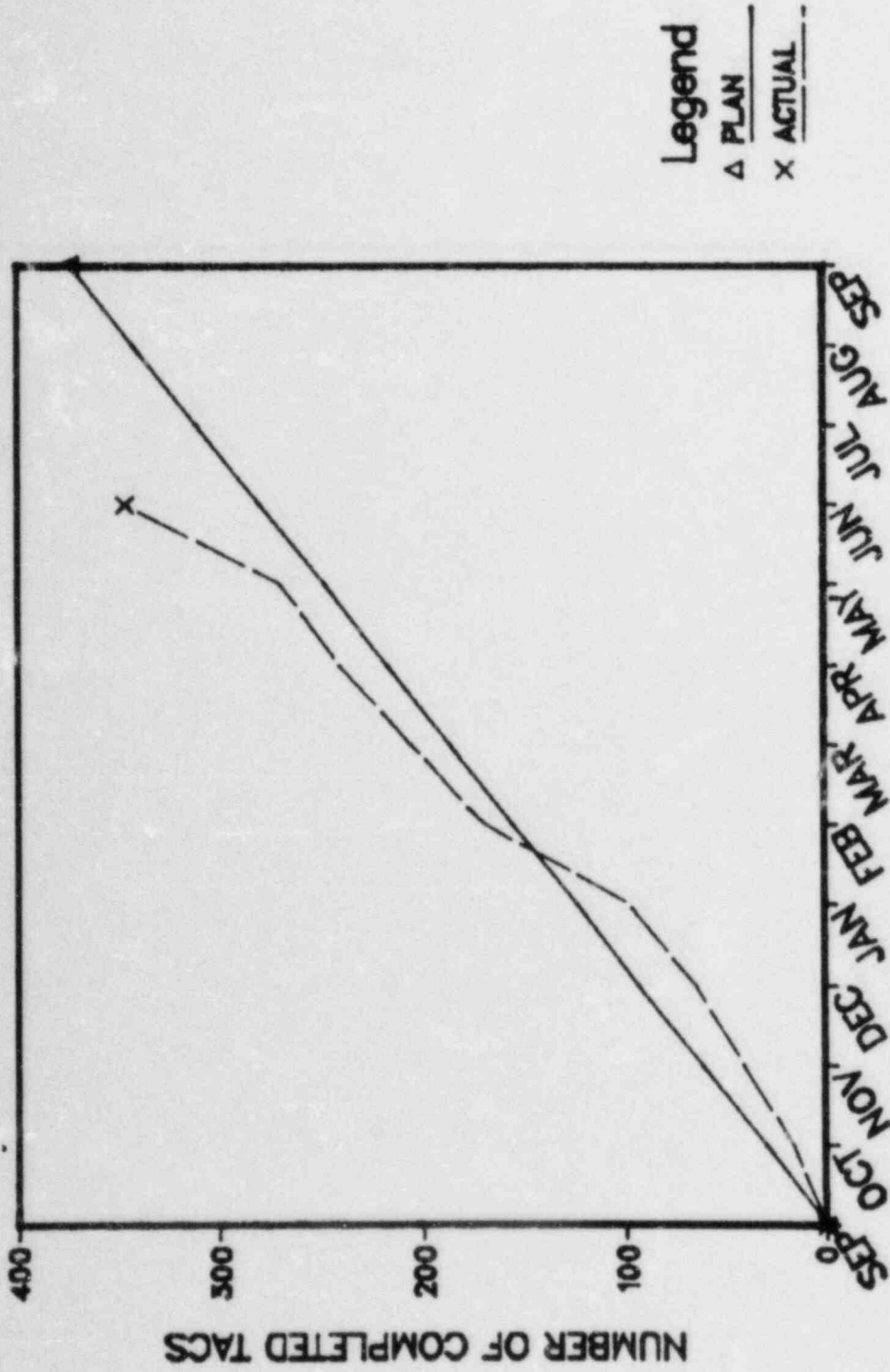
REQUESTS FOR NRR ASSISTANCE - STATUS REPORT  
 DIVISION OF LICENSING  
 06/14/84  
 OPEN ITEMS

RE- QSTG OFCE	TIA #	*****SUBJECT*****	REQUEST **DATE**	EST. COMP. DATE*	ACTUAL COMP. **DATE**	PA/ TAC # / *****	NRR BRANCHES	RS*
R-2	84-XX	BROWNS FERRY - OFFSITE MANUAL FIREFIGHTING CAPABILITIES	05/31/84	10/84	XX/XX/XX	1162 / 55145 / 55146 / 55147 / CLARK	CMEB, ORB-2	
R-5	84-XX	RANCHO SECO - EMERGENCY PREPAREDNESS - METEOROLOGY	05/31/84	06/84	XX/XX/XX	1165 / 55153 / MINER	MTEB, ORB-4	
R-1	84-XX	PILGRIM - INCONEL WELD MATERIAL CRACKING	06/08/84	XX/XX	XX/XX/XX	---- / 55218 / LEECH	MTEB, ORB-2	
R-5	84-XX	TROJAN - CONTROL ROD GUIDE TUBE SPLIT PIN	06/08/84	XX/XX	XX/XX/XX	---- / XXXXX / TRAMMELL	MTEB, MEB, CPB, ORB-3	
R-1	84-XX	OYSTER CREEK - ISO CONDENSER PIPE CRACKS	06/07/84	XX/XX	XX/XX/XX	---- / 54987 / LOMBARDO	MTEB, ORB-5	
R-1	84-XX	MILLSTONE 1 - RECIRC SYSTEM PIPE CRACKS	06/07/84	07/84	XX/XX/XX	---- / XXXXX / SHEA	MTEB, ORB-5	
R-1	84-XX	PEACH BOTTOM 2/3 - CRACKS IN JET PUMP INSTRUMENT NOZZLES	06/XX/84	XX/XX	XX/XX/XX	---- / XXXXX / GEARS	MTEB, RAB, ORB-4	

SECTION II

MULTIPLANT ACTION OVERVIEW

# MULTIPLANT ACTIONS VS PLANNED COMPLETIONS FOR FY84 does not include NUREG 0737 items





TECHNICAL ASSIGNMENT CONTROL SYSTEM  
MULTI-PLANT ISSUES SUMMARY REPORT

07/13/84

CODE	GENERIC SUBJECT	ACTIVE	FUTURE	COMPL. CURRENT FY (84)	COMPL. PRIOR FY	TOTAL
A-01	10 CFR 50.55 A(G) - ISI	0	0	2	68	70
A-02	APPENDIX I - ALARA	40	0	14	16	70
A-03	SECURITY REVIEWS-MODIFIED AMENDMENT PLANS	0	0	0	73	73
A-04	APPENDIX J - CONTAINMENT LEAK TESTING	6	0	5	43	54
A-05	GE MARK I CONTAINMENT TECH SPECS-SHORT TERM	0	0	0	20	20
A-06	RESPIRATORY PROTECTION SYSTEM	0	0	0	12	12
A-07	APPENDIX G - FRACTURE TOUGHNESS	0	0	1	8	9
A-08	ECCS EVALUATION-GENERIC PER 50.46 COMPLIANCE	0	0	0	8	8
A-09	PRESSURE VESSEL BELTLINE MATERIAL SURVEILLANCE	0	0	0	67	67
A-10	CONTINGENCY PLANNING	0	0	0	67	67
A-11	GUARD TRAINING PLANS	0	0	0	68	68
A-12	VITAL AREA ANALYSTS	0	0	0	68	68
A-13	NON POWER REACTOR SAFEGUARDS PLANS	0	0	0	0	0
A-14	10CFR 50.55 A(G) - INSERVICE TESTING	15	0	6	30	51
A-15	QUALITY ASSURANCE REQUEST REGARDING DIESEL GENERATOR FUEL OIL	0	0	1	36	37
A-16	QUALIFICATIONS OF EXAMINATION, INSPECTION, TESTING AND AUDIT PERSONNEL	0	0	0	57	57
A-17	INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)	78	0	2	0	80
B-01	DIESEL GENERATOR LOCKOUT	0	0	0	26	26
B-02	FIRE PROTECTION	0	0	0	65	65
B-03	PWR MODERATOR DILUTION	0	0	0	41	41
B-04	REACTOR VESSEL OVERPRESSURE PROTECTION	2	0	2	34	38
B-05	STRESS CORROSION CRACKING - BWR RCSPB	3	0	21	25	49
B-06	BWR RELIEF VALVE	0	0	0	23	23
B-07	STEAM GENERATOR FEEDWATER FLOW INSTABILITY	0	0	0	26	26
B-08	PWR HPSI-LPSI FLOW RESISTANCE	0	0	0	15	15
B-09	CHARGING SYSTEM PIPE VIBRATIONS	0	0	0	15	15
B-10	BURNABLE POISON ROD FAILURE - B&W	0	0	0	2	2
B-11	FLOOD OF EQUIPMENT IMPORTANT TO SAFETY	0	0	0	10	10
B-12	STEAM GENERATOR TUBE INSPECTION	0	0	0	12	12
B-13	FUEL ROD BOW	0	0	0	8	8
B-14	CEA GUIDE TUBE WEAR	0	0	0	39	39
B-15	C-E POISON ROD GROWTH	0	0	0	1	1
B-16	EMERGENCY PLANNING AND REVISIONS	2	0	0	64	66
B-17	TECH SPEC SURVEILLANCE FOR HYDRAULIC SNUBBERS	17	0	2	46	65
B-18	WORTHINGTON RHR PUMP SHAFT INTEGRITY	0	0	0	2	2
B-19	NEUTRON SHIELDING - CE REACTORS	0	0	0	2	2
B-20	CONTAINMENT LEAKAGE DUE TO SEAL DETERIORATION	0	0	0	57	57
B-21	LOSS OF 125-V DC BUS VOLTAGE WITH LOSS OF ANNUNCIATOR SYSTEM	0	0	0	66	66
B-22	TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS	15	0	5	50	70
B-23	DEGRADED GRID VOLTAGE	5	0	1	61	67
B-24	VENTING AND PURGING CONTAINMENTS WHILE AT FULL POWER AND EFFECT ON LOCA	14	0	19	101	134
B-25	BWR FEEDWATER NOZZLE CRACKING	1	0	6	16	23
B-26	INADVERTANT SAFETY INJECTION DURING COOLDOWN	0	0	0	40	40
B-27	REVIEW RESPONSES TO IE BULLETIN 78-03 (OFFGAS EXPLOSIONS)	0	0	0	23	28
B-28	BWR JET PUMP FLOW INDICATION ELIMINATION	0	0	0	5	5
B-29	BWR FEEDWATER PUMP TRIP	0	0	0	5	5
B-30	STEAM GENERATOR REPLACEMENT PROGRAM	0	0	0	8	8
B-31	LONG SHAFT LHSI & OUTSIDE RECIRC. PUMP DEGRADATION	0	0	0	3	3
B-32	BLOCKED SI SIGNAL DURING COOLDOWN	1	0	7	15	23
B-33	IODINE SPIKING	0	0	0	0	0
B-34	BWR JET PUMP INTEGRITY ASSURANCE	0	0	0	0	0

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## MULTI-PLANT ISSUES SUMMARY REPORT

CODE	GENERIC SUBJECT	ACTIVE	FUTURE	COMPL. CURRENT FY (84)	COMPL. PRIOR FY	TOTAL
B-35	ORIFICE ROD ASSEMBLY INTEGRITY - P&W	0	0	0	6	6
B-36	RESISTANCE TEMPERATURE DETECTOR (RTD) RESPONSE - CE	0	0	0	7	7
B-37	STEAM GENERATOR TUBE DENTING AND SUPPORT PLATE MODIFICATIONS - CE	0	0	0	11	11
B-38	TENDON SURVEILLANCE - BECHTEL CONTAINMENTS	0	0	0	3	3
B-39	PWR PRESSURE - TEMPERATURE LIMIT TECH SPECS	0	0	0	40	40
B-40	PIPE SUPPORT BASE PLATES	0	0	0	-1	1
B-41	FIRE PROTECTION - FINAL TECH SPECS (INCLUDES SER SUPPLEMENTS)	19	0	18	121	158
B-42	TMI FOLLOW UP - ALL PLANTS	0	0	0	87	87
B-43	PWR FEEDWATER LINE CRACKS	0	0	0	71	71
B-44	LESSONS LEARNED IMPLEMENTATION	0	0	0	809	809
B-45	WASH 1400 EVENT V, "PRIMARY COOLANT SYSTEM PRESSURE ISOLATION VALVES"	0	0	0	65	65
B-46	ANALYSIS OF TURBINE DISC CRACKS	1	0	0	69	70
B-47	ECCS; CLAD SWELLING AND RUPTURE	0	0	0	45	45
B-48	ADEQUACY OF STATION ELECTRIC DISTRIBUTION VOLTAGE	5	0	6	58	69
B-49	PWR CONTROL ROD MISALIGNMENT	0	0	0	9	9
B-50	AUXILIARY FEEDWATER SYSTEM EVALUATION	0	0	0	0	0
B-51	EVALUATION OF BULLETIN 79-06 AND 79-08	0	0	0	6	6
B-52	REVIEW OF SAFETY ASPECT OF INADVERTENT SAFETY ACTIONS DURING SUR. TEST	0	0	0	28	28
B-53	LESSONS LEARNED CATEGORY B (ITEMS)	0	0	0	5	5
B-54	LESSONS CATEGORY A TECH SPEC	0	0	0	19	19
B-55	B&O REPORT ON BWRs	0	0	0	3	3
B-56	CONTROL RODS FAILURE TO INSERT. BWR	0	0	0	5	5
B-57	DHR CAPABILITY	7	0	2	22	31
B-58	SDV CAPABILITY	0	0	3	38	41
B-59	MASONRY WALL DESIGN	45	0	8	17	70
B-60	ENVIRONMENTAL QUALIFICATION	72	0	2	6	80
B-61	LOSS OF NON CLASS IE I&C POWER	0	0	0	66	66
B-62	ESF RESET CONTROL DESIGN DEFICIENCY	0	0	1	66	67
B-63	INTERIM PROCEDURES FOR SHORT TERM BLACKOUT	0	0	0	71	71
B-64	B&C INDUCED FLUX ERRORS	0	0	0	8	8
B-65	SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN BWR SCRAM SYSTEM	17	0	0	9	26
B-66	NATURAL CIRCULATION COOLDOWN	8	0	17	24	49
B-67	THERMAL SHOCK	0	0	0	16	16
B-68	ECCS PUMP DEADHEADING	0	0	0	0	0
B-69	IEB 80-4 MAINSTEAM LINE BREAK WITH CONTINUED FEEDWATER ADDITION	2	0	1	46	49
B-70	FATIGUE TRANSIENT LIMIT TS	0	0	0	47	47
B-71	120 VAC VITAL INSTRUMENT BUSES AND INVERTER TECHNICAL SPECS.	0	0	0	0	0
B-72	NUREG 0737 TECH SPECS (GENERIC LETTER 82-16)	36	0	32	7	75
B-73	PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION	0	0	8	0	8
B-74	THERMAL SHIELD FOLLOW UP ANALYSIS	1	0	3	0	4
B-75	REVIEW OF B&W CORE BARREL BOLTS CRACKING	0	0	0	0	0
B-76	ITEM 1.1 - POST TRIP REVIEW; PROGRAM DESCRIPTION & PROCEDURES	80	0	2	0	82
B-77	ITEM 2.1 - EQUIPMENT CLASSIFICATION & VENDOR INTERFACE - RTS COMPONENTS	80	0	2	0	82
B-78	ITEMS 3.1.1 & 3.1.2 -POST MAINTENANCE TEST PROCEDURES & VENDOR RECOMM.	79	0	2	0	81
B-79	ITEM 3.1.3 - POST MAINTENANCE TESTING - CHANGES TO TECH SPECS - RTS COMP.	79	0	2	0	81
B-80	ITEM 4.1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODS	52	0	2	0	54
B-81	ITEMS 4.2.1 & 4.2.2 -PREVENTATIVE MAINT PROG FOR REACTOR TRIP BREAKERS	54	0	0	0	54
B-82	ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACH. FOR WEST & B&W	34	0	8	0	42
C-01	PWR SECONDARY WATER CHEMISTRY MONITORING REQUIREMENTS	0	0	1	37	38
C-02	BWR-RECIRC. PUMP TRIP (ATWS)	0	0	0	23	23
C-03	QUALIFICATIONS OF RADIATION PROTECTION MANAGER	0	0	0	14	14

TECHNICAL ASSIGNMENT CONTROL SYSTEM  
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CODE	GENERIC SUBJECT	ACTIVE	FUTURE	COMPL. CURRENT FY (84)	COMPL. PRIOR FY	TOTAL
C-04	FILTER TECH SPECS	0	0	3	12	15
C-05	CONVERSION TO STANDARD TECH SPECS	0	0	0	5	5
C-06	PUMP SUPPORT-LAMELLAR TEARING	0	0	0	38	38
C-07	FUEL HANDLING ACCIDENT INSIDE CONTAINMENT	2	0	2	35	39
C-08	BWR POST LOCA H2 CONTROL	0	0	0	8	8
C-09	PWR AUX FW PUMPS	0	0	0	7	7
C-10	CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL	28	0	32	20	80
C-11	RPS POWER SUPPLY	4	0	8	12	24
C-12	BORON SOLUBILITY DURING LONG TERM COOLING FOLLOWING LOCA	0	0	0	6	6
C-13	LOSS OF OFFSITE POWER	0	0	0	2	2
C-14	AUXILIARY FEEDWATER SEISMIC QUALIFICATION	8	0	4	35	47
C-15	CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA# C-10)	80	0	1	0	81
D-01	GE MARK I CONTAINMENT EVALUATION - LONG TERM	14	0	8	0	22
D-02	GE MARK I CONTAINMENT EVALUATION - LONG TERM	0	0	0	20	20
D-03	ECCS ZIRC CLAD MODEL ERROR-COMPLIANCE WITH 10 CFR-46	0	0	0	14	14
D-04	PRESSURIZER HEATUP RATE ERROR	0	0	4	14	18
D-05	PWR REACTOR VESSEL CAVITY SEAL RING MISSILE POTENTIAL	6	0	0	0	6
D-06	PLANT UPI MODEL PROBLEM	0	0	0	6	6
D-07	PEAKING MODEL CHANGE FOR CE REACTOR CORE	0	0	0	3	3
D-08	BWR POWER LEVEL FOR RWM	0	0	0	3	3
D-09	DEFICIENCY IN CHEM ADDITION TO CONTAINMENT SPRAYS	0	0	0	1	1
D-10	GE ECCS INPUT ERRORS	12	0	22	5	39
D-11	ASYMMETRIC LOCA LOADS	0	0	0	65	65
D-12	FISSTON GAS RELEASE	0	0	0	6	6
D-13	NON-JET PUMP FOR CORE SPRAY PERFORMANCE	0	0	0	7	7
D-14	B&W SMALL SYSTEM ERROR	0	0	0	11	11
D-15	REACTOR VESSEL WELD - WIRE DEFICIENCY	0	0	44	22	66
D-16	HIGH ENERGY PIPE BREAK & CONSEQUENTIAL SYSTEM FAILURE	0	0	0	18	18
D-17	REVIEW OF OPERATOR MANAGEMENT CAPABILITY	11	0	3	56	70
D-18	DEFINITION OF OPERABLE	0	0	1	7	8
E-01	NUREG 0630 OPERABLE MODELS (B&W PLANTS)	0	0	0	39	39
E-02	SPENT FUEL STORAGE EXPANSIONS	0	0	0	8	8
E-03	FUEL CASK TROP	0	0	0	37	37
E-04	CORE RELOADS REQUIRING PRIOR NRC APPROVAL	8	0	0	11	19
E-05	BWR SINGLE LOOP OPERATION	4	0	0	5	9
E-06	W N-1 LOOP OPERATION	0	0	0	7	7
E-07	CEA POSITION INDICATION FAILURES - CE	0	0	0	5	5
F-01	REACTOR PROTECTION SYSTEM LOGIC - CE	0	0	1	70	71
F-02	I.A.1.1 SHIFT TECHNICAL ADVISOR	2	0	12	55	69
F-03	I.A.1.3 SHIFT MANNING	2	0	6	63	71
F-04	I.A.2.1 UPGRADING OF RO AND SRO TRAINING	21	0	45	6	72
F-05	I.C.1.2 INADEQUATE CORE COOLING GUIDELINES AND PROCEDURES	68	0	1	4	73
F-06	I.C.1.3 ABNORMAL TRANSIENT OPERATOR GUIDELINES AND PROCEDURES	1	0	0	72	73
F-07	I.C.5 FEEDBACK OF OPERATING EXPERIENCE	1	0	0	70	71
F-08	I.C.6 CORRECT PERFORMANCE OF OPERATING ACTIVITIES	82	0	1	0	83
F-09	I.D.1 CONTROL ROOM DESIGN REVIEW	79	0	2	0	81
F-10	I.D.2 SAFETY PARAMETER DISPLAY SYSTEM	0	0	16	58	74
F-11	II.B.1 RCS HIGH POINT VENTS	4	0	15	51	70
F-12	II.B.2 PLANT SHIELDING	39	0	30	6	75
F-13	II.B.3 POST ACCIDENT SAMPLING	1	0	6	64	71
F-14	II.B.4 TRAINING FOR MITIGATING CORE DAMAGE	62	1	10	12	85
F-15	II.D.1 RV AND SV TRAINING					



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CODE	GENERIC SUBJECT	ACTIVE	FUTURE	COMPL. CURRENT FY (84)	COMPL. PRIOR FY	TOTAL
F-15	II.E.1.1 AFW SYSTEM EVALUATION	1	0	6	38	45
F-16	II.E.1.2.1 AFW SYSTEM INITIATION	0	0	2	43	45
F-17	II.E.1.2.2 AFW SYSTEM FLOW INDICATION	0	0	2	44	46
F-18	II.E.4.1 DEDICATED HYDROGEN PENETRATION	1	0	0	70	71
F-19	II.E.4.2 CONTAINMENT ISOLATION DEPENDABILITY	10	0	4	57	71
F-20	II.F.1.1 NOBLE GAS MONITOR	2	0	0	70	72
F-21	II.F.1.2 IODINE/PARTICULATE SAMPLING	1	0	0	71	72
F-22	II.F.1.3 CONTAINMENT HIGH RANGE MONITOR	0	0	1	70	71
F-23	II.F.1.4 CONTAINMENT PRESSURE INSTRUMENT	4	0	7	60	71
F-24	II.F.1.5 CONTAINMENT WATER LEVEL INSTRUMENT	4	0	7	60	71
F-25	II.F.1.6 CONTAINMENT HYDROGEN MONITOR	6	0	6	59	71
F-26	II.F.2 INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING	72	0	6	2	80
F-27	II.K.2.9 FMEA ON ICS	0	0	0	8	8
F-28	II.K.2.10 SAFETY GRADE ARTS	0	0	0	7	7
F-29	II.K.2.11 CONTINUED OPERATOR TRAINING AND DRILLING	0	0	0	7	7
F-30	II.K.2.13 THERMAL-MECHANICAL REPORT	6	0	45	3	54
F-31	II.K.2.14 LIFT FREQUENCY OF PORVS AND SVS	0	0	0	8	8
F-32	II.K.2.16 RCP SEAL DAMAGE	0	0	0	7	7
F-33	II.K.2.17 POTENTIAL FOR VOIDING IN RCS	1	0	38	9	48
F-35	II.K.2.20 SYSTEM RESPONSE TO SB LOCA	0	0	0	7	7
F-36	II.K.3.1 AUTO PORV ISOLATION	1	0	17	28	46
F-37	II.K.3.2 REPORT ON PORV FAILURES	1	0	16	29	46
F-38	II.K.3.3 REPORTING SV AND RV FAILURES AND CHALLENGES	1	0	0	70	71
F-39	II.K.3.5 AUTO TRIP OF RCPS	1	0	0	50	51
F-40	II.K.3.9 PID CONTROLLER	0	0	0	30	30
F-41	II.K.3.10 ANTICIPATORY TRIP MODIFICATIONS	0	0	0	30	30
F-42	II.K.3.12 ANTICIPATORY TRIP ON TURBINE TRIP	0	0	0	30	30
F-43	II.K.3.13 HPCI AND RCIC INITIATION LEVELS	0	0	0	25	25
F-44	II.K.3.14 ISOLATION CONDENSER ISOLATION MODIFICATION	0	0	0	8	8
F-45	II.K.3.15 ISOLATION OF HCPI AND RCIC MODIFICATIONS	0	0	0	19	19
F-46	II.K.3.16 CHALLENGES AND FAILURES OF RELIEF VALVES	21	0	1	3	25
F-47	II.K.3.17 ECCS OUTAGES	0	0	7	62	69
F-48	II.K.3.18 ADS ACTUATION	12	0	6	8	26
F-49	II.K.3.19 INTERLOCK RECIRCULATION PUMP MODIFICATION	1	0	0	24	25
F-50	II.K.3.21 RESTART OF CSS AND LPCI	0	0	0	24	24
F-51	II.K.3.22 RCIC SUCTION	4	0	4	9	17
F-52	II.K.3.24 SPACE COOLING FOR HPCI/RCIC MODIFICATION	0	0	0	19	19
F-53	II.K.3.25 POWER ON PUMP SEALS	5	0	0	56	61
F-54	II.K.3.27 COMMON REFERENCE LEVEL	0	0	0	25	25
F-55	II.K.3.28 QUALIFICATION OF ADS ACCUMULATORS	14	0	3	5	22
F-56	II.K.3.29 PERFORMANCE OF ISOLATION CONDENSERS	0	0	1	7	8
F-57	II.K.3.30 SB LOCA METHODS	44	0	26	3	73
F-58	II.K.3.31 COMPLIANCE WITH 10 CFR 50.46	50	0	22	3	75
F-59	II.K.3.44 ANTICIPATED TRANSIENTS WITH SINGLE FAILURES	0	0	2	23	25
F-60	II.K.3.45 MANUAL DEPRESSURIZATION	1	0	2	22	25
F-61	II.K.3.46 MICHELSON CONCERNS	0	0	0	0	0
F-62	II.K.3.57 MANUAL ACTUATION OF ADS	0	0	0	0	0
F-63	III.A.1.2 TECHNICAL SUPPORT CENTER	71	0	2	2	75
F-64	III.A.1.2 OPERATIONAL SUPPORT CENTER	70	0	2	2	74
F-65	III.A.1.2 EMERGENCY OPERATIONS FACILITY	67	0	3	4	74
F-66	III.A.1.2 NUCLEAR DATA LINK	0	0	0	72	72



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F-67	III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE	1	0	6	66	73
F-68	III.A.2.2 METEOROLOGICAL DATA UPGRADE	68	0	3	1	72
F-69	III.D.3.3 IMPLANT RADIATION MONITORING	0	0	0	71	71
F-70	III.D.3.4 CONTROL ROOM HABITABILITY	8	0	9	55	72
G-01	REACTOR COOLANT PUMP TRIP (D660 - II.K.3.5)	1	0	0	0	1
TOTAL GENERIC ISSUES		1,959	1	747	6,203	8,910









MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL					Remaining Inputs to be Completed	FY 1985	FY 1986	Schedule for Actions Issued by DL			FY1984 Technical Review Performed by		
					FY 1984								FY 1984	FY 1985	FY 1986	In- House Staff	Contract	
					1st	2nd	3rd	4th	Total								Under DL	Under Div.
B-13	Fuel Rod BOW		Complete															
B-14	CEA Guide Tube Wear		Complete															
B-15	C-E Posion Rod Growth		Complete															
B-16	Emergency Planning and Revisions		Complete															
B-17	Tech Spec Surveillance for Hydraulic Snubbers	(15/18)	3	REG	5	5	5	0	15	0	0	0	18	0	0		15	
B-18	Worthinton RHR Pump Shaft Integrity		Complete															
B-19	Neutron Shielding - CE Reactors		Complete															
B-20	Containment leakage due to Seal Deterioration		Complete															
B-21	Loss of 125-V DC Bus Voltage with Loss of Annunciator System		Complete															
B-22	Tech Spec Surveillance Requirements for Mechanical Snubbers	(15/18)	2	REG	5	5	5	0	15	0	0	0	18	5	0		15	
B-23	Degraded Grid Voltage (5 yr. old)	(5/7)	2	PSB	0	1	2	1	4	0	1	0	6	1	0		4	













MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL					Schedule for Actions Issued by DL			FY1984 Technical Review Performed by					
					FY 1984					Remaining Inputs to be Completed	FY 1985	FY 1986	FY 1984	FY 1985	FY 1986	In- House Staff	Contract	
					1st	2nd	3rd	4th	Total								Under DL	Under Div.
C-01	PWR Secondary Water Chemistry Monitoring Requirements (5 yr. old)	(2/4)	Complete		2	0	0	0	2	0	0	0	4		2			
C-02	BWR-Recirc. Pump Trip (ATWS)		Complete															
C-03	Qualifications of Radiation Protection Manager		Complete															
C-04	Filter Tech Specs (5 yr. old)	(0/3)	3	ETSB	0	2	0	1	3	0	1	0	3	1	0			
C-05	Conversion to Standard Tech Spec		Complete															
C-06	Pump Support Lamellar Tearing		Inactive															
C-07	Fuel Handling Accident Inside Containment	(2/4)	2	ORB#5	1	0	0	3	4	3	0	0	4	0		2		
C-08	BWR Post LOCA H2 Control (5 yr. old)	(4/4)	Complete															
C-09	PWR Aux. Feedwater Pumps		Complete															
C-10	Control of Heavy Loads Over Spent Fuel Pool (USI:A-36)	(5/5)	1	ASB	6	9	9	18	43	16	16	0	35	24		5		







MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL					Schedule for Actions Issued by DL			FY1984 Technical Review Performed by					
					FY 1984					Remaining Inputs to be Completed	FY 1985	FY 1986	FY 1984	FY 1985	FY 1986	In- House Staff	Under DL	Under Div.
					1st	2nd	3rd	4th	Total									
E-01	Spent Fuel Pool Expansions		Complete															
E-02	Fuel Cask Drop		Complete															
E-03	Core Reloads Requiring Prior NRC Approval		Complete															
E-04	BWR Single Loop Operation (5 yr. old)	(3/10)	<b>3</b>	RSB ORAB#2 ICSB CPB	0	0	0	0	0	10	10	0	0	10	0			
E-05	W N-1 Loop Operation	(3/3)	3	RSB ICSB HFEB	0	0	1	0	1	0	1	1	1	1	1	3		
E-06	CEA Position Indica- tion Failures CE		Complete															
E-07	Reactor Protection System Logic		Complete	RSB/ CPB														
F-01	I.A.1.1 Shift Tech Advisor		Complete															
	I.A.1.1.1 Trained to Category B Lessons Learned		Complete															
F-02	I.A.1.3.1 Minimum Shift Crew	(26/26)	Post	REG	0	2	3	3	8	3	3	8	3	0	11			



MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL					Remaining Inputs to be Completed	FY 1985	FY 1986	Schedule for Actions Issued by DL			FY1984 Technical Review Performed by		
					FY 1984								FY 1984	FY 1985	FY 1986	In- House Staff	Under DL	Under Contract Div.
					1st	2nd	3rd	4th	Total									
F-08	I.D.1 Control Room Design Review (Suppl., 1, NUREG-0737)																	
	I.D.1.1 Program Plans			HFEB	28	13	6	3	50	31	25	6						
	I.D.1.2 In-Progress Audits			HFEB	2	7	3	0	12	13	13	0						
	I.D.1.3 Summary Report			HFEB	0	5	8	4	17	46	28	18						
	I.D.1.4 Pre-Implementation Audit			HFEB	0	0	1	0	1	23	19	4						
	I.D.1.5 Post-Implementation Audit			HFEB	0	0	0	0	0	23	19	4						
F-09	I.D.2 Safety Parameter Display System (Suppl. 1, NUREG-0737)																	
	I.D.2.1 Safety Analysis Report			HFEB	0	0	8	14	32	59	32	27						
	I.D.2.2 Pre-Implementation Review			HFEB	0	1	1	1	3	2	2	0						
	I.D.2.3 Post-Implementation Audit			HFEB	0	0	0	2	2	44	23	21						
F-10	II.B.1 RCS High Point Vents																	
	II.B.1.1 Final Design & Procedures	(0/12)	Complete	RSB	0	0	0	0	0	0	0	0	12	0	0	0	0	
F-11	II.B.2 Plant Shielding II.B.2.2 Modifications	(13/20)	Post	REG	0	1	2	1	4	0	0	0	5				4	
F-12	II.B.3 Post Accident Sampling II.B.3.2 Upgraded System		Post	CEB	15	8	5	6	34	35	35	0	34	35	0	34	34	



TABLE K  
MULTIPLANT ACTION COMPLETION SCHEDULE

MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL					Remaining Inputs to be Completed	Schedule for Actions Issued by DL			FY1984 Technical Review Performed by				
					FY 1984						FY 1985	FY 1986	FY 1984	FY 1985	FY 1986	In- House Staff	Contract	
					1st	2nd	3rd	4th	Total								DL	Div.
F-13	II.B.4 Training for Mitigating Core Damage	(4/4)	Post	REG	0	0	0	4	4			4	0	0	4			
F-14	II.D.1 RV & SV Testing (15/17)		BWR	EQB	4	4	5	2	15	(BWR)		17					15	
	II.D.1.1 BWR Test Report		Post	EQB	4	6	7	7	24	28(PWR)	28	20	32					
	II.D.1.2.a BWR Plant Unique Analysis																	
	II.D.1.2.b PWR Test Report	(52/52)	Pre	MEB														
	II.D.1.3 PWR Plant Unique		Pre	MEB														
F-15	II.E.1.1 AFW Sys Eval		Pre	ASB	2	1	1	0	4	0	0	0	5	0	0	4		
F-16	II.E.1.2.1 AFW System Initiation																	
	II.E.1.2.1.b Safety Grade		Complete	ICSB	2						2		2					
F-17	II.E.1.2.2 AFW System Flow Indicat.																	
	II.E.1.2.2.b Safety Grade		Complete	ICSB	2						2		2					
F-18	II.E.4.1 Dedicated H2 Penetration  Anal.		Post	ICSB	0	0	0	0	0		0		1					



MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL					Schedule for Actions Issued by DL			FY1984 Technical Review Performed by					
					FY 1984					Remaining Inputs to be Completed	FY 1985	FY 1986	FY 1984	FY 1985	FY 1986	In- House Staff	Under DL	Under Div.
					1st	2nd	3rd	4th	Total									
F-28	II.K.2.10 Safety Grade Trip		Complete															
F-29	II.K.2.11 Continued Operator Training and Drilling		Complete															
F-30	II.K.2.13 Therm Mech Rpt		Post	RSB/ GIB	0	0	54	0	54		0	0	54	0	0			
F-31	II.K.2.14 Lift Freq of PORVs (II.K.3.7)		Complete															
F-32	II.K.2.16 RCP Seal Damage		Complete															
F-33	II.K.2.17 Voiding in RCS		Complete	RSB	30						30					30		
F-34	II.K.2.19 Benchmark Analysis for Sequen- tial AFW Flow		Complete															
F-35	II.K.2.20 System Response to SB LOCA	(1/30)	Complete															
F-36	II.K.3.1 PORV Isolation		Post	ICSB													1	
F-37	II.K.3.2 Rpt on PORV		Post	RRAB	1	0	0	0	1			30					1	
F-38	II.K.3.3 Rpt SV/RV Failures		Post	ORB#3	0	0	0	1	1				1	0	0			

MPA No.	Multi-plant Action Title	Projected FY83 Carryover (Inputs/LAs)	Priority	Review Branch	Schedule for Technical Inputs to DL					Schedule for Actions Issued by DL			FY1984 Technical Review Performed by					
					FY 1984					Remaining Inputs to be Completed	FY 1985	FY 1986	FY 1984	FY 1985	FY 1986	In- House Staff	Under DL	Under Div.
					1st	2nd	3rd	4th	Total									
F-39	II.K.3.5 Auto Trip of RC Criteria		Complete	RSB														
F-40	II.K.3.9 PID Controller		Complete															
F-41	II.K.3.10 Anticip Trip Mods		Complete	ICSB														
F-42	II.K.3.12 Anticip Trip on II II.K.3.12a Design		Complete															
F-43	II.K.3.13 HPCI/RCIC Init Lvl II.K.3.13a Design II.K.3.13b Modify		Complete Post	ICSB	Complete													
F-44	II.K.3.14 IC Mods		Complete															
F-45	II.K.3.15 Isol of HPCI/RCIC		Complete	RSB														
F-46	II.K.3.16 Chall/Fail of RVS Report		Post	RSB	0	0	0	22	22			22	0	0				
F-47	II.K.3.17 ECCS Outage (0/6) Rpt		Complete	RRAB					0			6						
F-48	II.K.3.18 ADS Actuation II.K.3.18a Study II.K.3.18b Design II.K.3.18c Modifications		Complete Post Pre	ICSB RSB								22						
F-49	II.K.3.19 Interlock RCPs		Post	RSB				1	1			1						







TABLE L

L1

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
A-01	10 CFR 50.55 A(g) - ISI	C							
A-02	Appendix I - ALARA	A	G. Gears	X		DL/DSI	M. Carrington	B-6891	FRC II/EG&G
A-03	Security Reviews - Modified Amendment Plans	C							
A-04	Appendix J - Containment Leak Testing	A	G. Dick	X		DL	S. Bajwa	B-6590	FRC I (completed)
A-05	GE Mark I Containment Tech Specs - Short Term	C							
A-06	Respiratory Protection System	C							
A-07	Appendix G - Fracture Toughness	C							
A-08	ECCS Evaluation - Generic Per 50.46 Compliance	C							
A-09	Pressure Vessel Beltline Material Surveillance	C							
A-10	Contingency Planning	C							
A-11	Guard Training Plans	C							
A-12	Vital Area Analysis	C							
A-13	Non Power Reactor Safeguards Plan	I							
A-14	10 CFR 50.55 A (G) - Inservice Testing	A	P. Tam		X				
A-15	Quality Assurance Request Regarding Diesel Generator Fuel Oil	C							

\* A = Active, C = Complete, I = Inactive

TABLE I.

L2

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
A-16	Qualification of Inspection, Examination, Testing and Audit Personnel	C							
A-17	Instrumentation to Follow the Course of an Accident (R.G. 1.97)	A	J. Shea						
B-01	Diesel Generator Lockout	C							
B-02	Fire Protection								
B-03	PWR Moderator Dilution	C							
B-04	Reactor Vessel Overpressure Protection	A	A. DeAgazio	X		DSI			EG&G
B-05	Stress Corrosion Cracking - BWR RCSPB	A	R. Clark	X		DL	J. Donohew	A-6429	INEL
B-06	BWR Relief Valve	C							
B-07	Steam Generator Feedwater Flow Instability	C							
B-08	PWR HPSI-LPSI Flow Resistance	C							
B-09	Charging Systems Pipe Vibrations	C							
B-10	Burnable Poison Rod Failure - B&W	C							
B-11	Flood of Equipment Important to Safety	C							
B-12	Steam Generator Tube Inspection	C							
B-13	Fuel Rod Bow	C							

\* A = Active, C = Complete, I = Inactive.



TABLE L

L3

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 07/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No	Contractor
B-14	CEA Guide Tube Wear	C							
B-15	C-E Poison Rod Growth	C							
B-16	Emergency Planning and Revisions	C							
B-17	Tech Spec Surveillance for Hydraulic Snubbers	A	L. Engle	X	X**	Region I	J. Donohew	A-0250	LLL
B-18	Worthington RHR Pump Shaft Integrity	C							
B-19	Neutron Shielding - CE Reactors	C							
B-20	Containment Leakage Due to Seal Deterioration	C							
B-21	Loss of 125-V DC Bus Voltage with Loss of Annunciator System	C							
B-22	Tech Spec Surveillance Requirements for Mechanical Snubbers	A	L. Engle	X	X**	Region I	J. Donohew	A-0250	LLL
B-23	Degraded Grid Voltage	A	R. Dudley	X		DL	J. Donohew	A-0250 A-6423	LLL & INEL
B-24	Venting and Purging Containment While at Full Power and Effect on LOCA	A	E. Reeves	X		DL	S. Bajwa	B-6590	FRC I (completed)
B-25	BWR Feedwater Nozzle Cracking	A	R. Gilbert		X				
B-26	Inadvertant Safety Injection During Cooldown	C							

\* A = Active, C = Complete, I = Inactive.

\*\*review contracted only in some Regions.

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
B-27	Review Responses to IE Bulletin 78-03 (Offgas Explosions)	C							
B-28	BWR Jet Pump Flow Indication Elimination	C							
B-29	BWR Feedwater Pump Trip	C							
B-30	Steam Generator Replacement Program**	C							
B-31	Long Shift LHSI and Outside Recirculation Degradation	C							
B-32	Blocked SI Signal During Cooldown	C							
B-33	Iodine Spiking	C							
B-34	BWR-Weld Failure of Jet Pump Retainer Bolt	C							
B-35	Orifice Rod Assembly Integrity - B&W	C							
B-36	Resistance Temperature Detector (RTD) Response - CE	C							
B-37	Steam Generator Tube Denting and Support Plate Modifications - CE	C							
B-38	Tendon Surveillance - Bechtel Containments	C							
B-39	PWR Pressure - Temperature Limit Tech Specs	C							
B-40	Pipe Support Base Plates	C							

\* A = Active, C = Complete, I = Inactive

\*\*All new and pending reviews are being handled as plant specific actions.

TABLE L

L5

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
B-41	Fire Protection - Final Tech Specs (Includes SER Supplements)	A	T. Wambach	X		DSI			BNL
B-42	TMI Follow Up-All Plants	C							
B-43	PWR Feedwater Line Cracks	C							
B-44	Lessons Learned Implementation	C							
B-45	WASH 1400 Event V, "Primary Coolant Pres. Isolation Valves"	C							
B-46	Analysis of Turbine Disc Cracks	C							
B-47	ECCS; Clad Swelling and Rupture	C							
B-48	Adequacy of Station Electric Distrib. Voltage	A	R. Dudley	X		DL	J. Donohew	A-6429 A-0250	LLI & INEL
B-49	PWR Control Rod Misalignment	C							
B-50	Auxiliary Feedwater System Evaluation	C							
B-51	Evaluation of Bulletin 79-05, 79-06, 79-08	C							
B-52	Review of Safety Aspect of Inadvertent - Safety Actions During Surveillance Testing	C							
B-53	Lessons Learned Category B (Items)	C							
B-54	Lessons Learned Category A Tech Spec	C							

TABLE L

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
B-55	B&O Report on BWRs	C							
B-56	Control Rods Fail to Insert BWR	C							
B-57	DHR Capability	A	A. DeAgazio	X		DL	J. Donohew	A-6429	INEL (completed)
B-58	SDV Capability	C							
B-59	Masonry Wall Design	A	C. Trammell	X		DL	M. Carrington	B-6891	FRC II
B-60	Environmental Qualification of Electrical Equipment	A	R. Karsch	X		DL	S. Bajwa	B-6890	FRC I
B-61	Loss of Non Class IE Instr and Control Power	C			X				
B-62	ESF Reset Design Deficiency-IE Bulletin 80-06	C					J. Donohew	A-0250	LL (completed)
B-63	Interim procedures for Station Blackout	C			X				
B-64	B&W Induced Flux Errors	C							
B-65	Safety Concerns Assoc. with Pipe Breaks in the BWR Scram System	A	V. Rooney		X				
B-66	Nat. Cir. Cooldown	A	K. Heitner						
B-67	Thermal Shock	C							
B-68	ECCS Pump Deadheading	I							
B-69	IEB 80-4 Mainsteam Line Break with Continued Feedwater Addition	A	E. Tourigny	X		DL	M. Carrington	B-6891	FRC II

\* A = Active, C = Complete, I = Inactive.



TABLE L

L7

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
B-70	Monitoring of Fatigue Transient Limits for ECCS	I							
B-71	120 VAC Vital Instrument Busses and Inverter Technical Specifications	I**	G. Rivenbark						
B-72	NUREG-0737 Tech Specs	A	C. Patel		X				
B-73	Plans for Preventing Exceeding PTS Screening Criterion	C							
B-74	Thermal Shield Support Structure Review	C							
B-75	B&W Reactor Vessel Internals Bolting	A	N. Kadambi		X				
B-76	Item 1.1 - Post-Trip Review - Program Description and Procedures	A	A. Bournia		X				
B-77	Item 2.1 - Equipment Classification and Vendor Interface - RTS Components	A	R. Auluck		X				
B-78	Items 3.1.1 and 3.1.2 - Post-Maintenance Testing Procedures and Vendor Recommendations - RTS Components	A	J. Hopkins		X				
B-79	Item 3.1.3 - Post-Maintenance Testing - Changes to Tech Specs - RTS Components	A	J. Hopkins		X				
B-80	Item 4.1 - Reactor Trip System Reliability Vendor Related Modifications	A	J. Hopkins		X				

\* A = Active, C = Complete, I = Inactive.

\*\*Action inactive pending CRGR review.

TABLE L

L8

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
B-81	Items 4.2.1 and 4.2.2 - Preventative Maintenance Program for Reactor Trip Breakers - Maintenance and Testing	A	R. Perch		X				
B-82	Item 4.3 - Automatic Actuation of Shunt Trip Attachment for W and B&W Plants	A	J. Holonich		X				
B-83	Review of the Technical Specifications for NUREG-0737 (Generic Letters 83-36 and 83-37)	A	C. Patel		X				
C-01	PWR Secondary Water Chemistry Monitoring Requirements	C							
C-02	BWR - Recirc. Pump Trip (ATWS)	C							
C-03	Qualifications of Radiation Protection Manager	C							
C-04	Filter Tech Specs	A	J. Lombardo		X				
C-05	Conversion to Standard Tech Specs	C							
C-06	Pump Support - Lamellar Tearing	C							
C-07	Fuel Handling Accident Inside Containment	A	J. Lombardo		X				
C-08	BWR Post LOCA H-2 Control	C							
C-09	PWR Aux. FW Pumps	C							
C-10	Control of Heavy Loads Over Spent Pool (Phase 1)	A	D. Neighbors	X		DL	M. Carrington	B-6891	FRC II

\* A = Active, C = Complete, I = Inactive.

TABLE L

L9

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
C-11	RPS Power Supply	A	M. Thadani	X		DL	J. Donohew	A-0250	LLL
C-12	Boron Solubility During Long Term Cooling Following LOCA	C							
C-13	Loss of Offsite Power	C							
C-14	Aux. Feedwater Seismic Qualification	A	J. Beard	X		DL	J. T. Beard	A-0429	LLL
C-15	Heavy Loads Phase 2	A	D. Neighbors	X					
D-01	GE Mark I Containment Evaluation Long Term	A	B. Siegel	X		DL	B. Barnhart	A-3713	BNL
D-02	ECCS Circ. Clad Model Error - Compliance with 10 CFR 46	C							
D-03	Pressurizer Heatup Rate Error	C							
D-04	PWR Reactor Vessel Cavity Seal Ring Missile Potential	C							
D-05	Plant LPI Model Problem	A	D. DiIanni		X	DSI			SNL
D-06	Peaking Model Change for CE Reactor Core	C							
D-07	PWR Power Level for RWM	C							
D-08	Deficiency in Chem Addition to Containment Sprays	C							
D-09	GE ECCS Input Errors	C							

\* A = Active, C = Complete, I = Inactive.

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
D-10	Asymetric LOCA Loads	A	J. Shea		X				
D-11	Fission Gas Release	C							
D-12	Non-Jet Pump BWR Core Spray Performance	C							
D-13	B&W Small Break Error	C							
D-14	Reactor Vessel Weld-Wire Deficiency	C							
D-15	High Energy Line Break and Consequential System Failure	C							
D-16	Review of Corporate Management Capability	C							
D-17	Definition of Operable	A	S. Miner	X		DL	J. Donohew	A-6429	INEL
D-18	NUREG-0630 Cladding Models - B&W Plants	C							
E-01	Spent Fuel Pool Expansions**	C							
E-02	Fuel Cask Drop	C							
E-03	Core Reloads Requiring Prior NRC Approval**	C							
E-04	BWR Single Loop Operation	A	R. Clark	X		DL	J. Donohew	A-0250	LLL
E-05	WN-1 Loop Operation	A	D. Wigginton	X		DL	J. Donohew	A-0250	LLL (completed)
E-06	CEA Position Indication Failures - CE	C							

\* A = Active, C = Complete, I = Inactive.

\*\*All new and pending reviews are being handled as plant specific actions



TABLE L

111

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
E-07	Reactor Protection System Logic - CE	C							
F-01	I.A.1.1 Shift Technical Advisor	C							
F-02	I.A.1.3 Shift Manning	A	D. Fischer		X				
F-03	I.A.2.1 Upgrading of RO and SRO Training	A	D. Wigginton	X		DL	S. Bajwa	B-8011	SAI
F-04	I.C.1.2 Inadequate Core Cooling Guidelines and Procedures	A	J. Lyons		X				
F-05	I.C.1.3 Abnormal Transient Operator Guidelines and Procedures	A	J. Lyons	X		DSI			
F-06	I.C.5 Feedback of Operating Experience	C							
F-07	I.C.6 Correct Performance of Operating Activities	A	P. Leech		X				
F-08	I.D.1 Control Room Design Review	A	G. Dick		X				
F-09	I.D.2 Safety Parameter Display System	A	G. Dick		X				
F-10	II.B.1 RCS High Point Vents	C							
F-11	II.B.2 Plant Shielding	A	E. Tourigny		X				
F-12	II.B.3 Post accident Sampling	A	J. Norris						
F-13	II.B.4 Training for mitigating Core Damage	A	D. Wigginton	X		DL	S. Bajwa	B-8011	SAI
F-14	II.D.1 RV and SV Testing	A	M. Caruso	X		DE			

\* A = Active, C = Complete, I = Inactive.

TABLE L

L12

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
F-15	II.E.1.1 AFW System Evaluation	A	R. Hernan						
F-16	II.E.1.2.1 AFW System Initiation	C							
F-17	II.E.1.2.2 AFW System Flow Indication	C							
F-18	II.E.4.1 Dedicated Hydrogen Penetrations	A	R. Hernan						
F-19	II.E.4.2 Containment Isolation Dependability	A	E. Reeves						
F-20	II.F.1.1 Noble Gas Monitor	A	H. Nicolaras						
F-21	II.F.1.2 Iodine/Particulate Sampling	A	H. Nicolaras						
F-22	II.F.1.3 Containment High Range Monitor	C							
F-23	II.F.1.4 Containment Pressure Instrument	A	H. Nicolaras						
F-24	II.F.1.5 Containment Water Level Instrument	A	H. Nicolaras						
F-25	II.F.1.6 Containment Hydrogen Monitor	A	H. Nicolaras						
F-26	II.F.2 instruments for Detection of Inadequate Core Cooling	A	J. Shea	X		DSI			ORNL
F-27	II.K.2.9 FMEA on ICS	C							
F-28	II.K.2.10 Safety Grade ARTS	C							
F-29	II.K.2.11 Continued Operator Training and Drilling	C							

\* A = Active, C = Complete, I = Inactive.

TABLE L

L13

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review		Contract Information			
				Contracted Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
F-30	II.K.2.13 Thermal-Mechanical Report	A	G. Vissing		X				
F-31	II.K.2.14 Lift Frequency of PORVs and SVs	C							
F-32	II.K.2.16 RCP Seal Damage	C							
F-33	II.K.2.17 Potential for Voiding in RCS	C							
F-35	II.K.2.20 System Response to SB LOCA	C							
F-36	II.K.3.1 Auto PORV Isolation	A	T. Colburn		X				
F-37	II.K.3.2 Report on PORV Failures	A	T. Colburn	X		DL	M. Carrington	B-6891	FRC II
F-38	II.K.3.3 Reporting SV and RV Failures and Challenges	A	T. Colburn		X				
F-39	II.K.3.5 Auto Trip of RCPs	C							
F-40	II.K.3.9 PID Controller	C							
F-41	II.K.3.10 Anticipatory Trip Modifications	C			X				
F-42	II.K.3.12 Anticipatory Trip on Turbine Trip	C			X				
F-43	II.K.3.13 HPCI and RCIC Initiation Levels	C							
F-44	II.K.3.14 Isolation Condenser Isolation Modification	C							
F-45	II.K.3.15 Isolation of HPCI and RCIC Modification	C							
F-46	II.K.3.16 Challenges and Failures of Relief Valves	A	R. Clark		X				

\* A = Active, C = Complete, I = Inactive.

TABLE L

L14

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Date as of 04/08/83

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
F-47	II.K.3.17 ECCS Outages	C							
F-48	II.K.3.18 ADS Actuation	A	R. Bevan		X				
F-49	II.K.3.19 Interlock Recirculation Pump Modification	A	J. Lombardo		X				
F-50	II.K.3.21 Restart of CSS and LPCI	C							
F-51	II.K.3.22 RCIC Suction	A	R. Hermann						
F-52	II.K.3.24 Space Cooling for HPCI/RCIC Modifications	C			X				
F-53	II.K.3.25 Power on Pump Seals	A	R. Hermann	X					
F-54	II.K.3.27 Common Reference Level	C			X				
F-55	II.K.3.28 Qualification of ADS Accumulators	A	R. Hermann		X				
F-56	II.K.3.29 Performance of Isolation Condensers	C							
F-57	II.K.3.30 SB LOCA Methods	A	D. Wigginton		X	DSI			LANL
F-58	II.K.3.31 Compliance with 10 CFR 50.46	A	B. Siegel		X				
F-59	II.K.3.44 Anticipated Transients with Single Failures	A	J. Lombardo		X				
F-60	II.K.3.45 Manual Depressurization	C							
F-61	II.K.3.46 Michelson Concerns	C							
F-62	II.K.3.47 Manual Actuation of ADS	C							

\* A = Active, C = Complete, I = Inactive.



TABLE L

## MULTIPLANT ACTIONS - LEAD ASSIGNMENTS AND CONTRACT INFORMATION

Multiplant Action Number	Title	Status*	Lead Project Manager	Review Contracted		Contract Information			
				Yes	No	Funding Division	Contract Manager	FIN No.	Contractor
F-63	III.A.1.2 Technical Support Center	A	W. Paulson		X				
F-64	III.A.1.2 Operational Support Center	A	W. Paulson		X				
F-65	III.A.1.2 Emergency Operations Facility	A	W. Paulson		X				
F-67	III.A.2.1 Emergency Plan Upgrade to Meet Rule	A	P. Erickson		X				
F-68	III.A.2.2 Meteorological Data Upgrade	A	J. Lombardo		X				
F-69	III.D.3.3 Implant Radiation Monitoring	C							
F-70	III.D.3.4 Control Room Habitability	A	R. Gilbert		X				
G-1	Reactor Coolant Pump Trip	A	O. Jaffe						

\* A = Active, C = Complete, I = Inactive.

IMPLEMENTATION OF RESOLVED USIs

M-1

Resolved USIs which result in changes on operating reactors become multiplant actions (MPAs). The progress can be readily tracked by referring to the appropriate MPA. Below is a correlation between USI numbers and MPA numbers.

<u>USI #</u>	<u>MPA #</u>	<u>Title</u>
A-2	D-10	Asymmetric LOCA Loads
A-7	D-01	GE Mark I Containment Evaluation - Long Term
A-10	B-25	BWR Feedwater Nozzle Cracking
A-24	B-60	Environmental Qualification
A-26	B-04	Reactor Vessel Overpressure Protection
A-36	C-10	Control of Heavy Loads Over Spent Fuel Pool
A-42	B-05	Stress Corrosion Cracking - BWR RCSPB

The following USIs have been resolved but not backfitted on operating reactors:

A-9	ATWS - Involved in rulemaking
A-31	Residual Heat Removal Requirements

DIVISION OF LICENSING POINTS OF CONTACT  
FOR UNRESOLVED SAFETY ISSUES

N-1

USI	CONTACT	
A-1	Water Hammer	J. Neighbors
A-3/4/5	Steam Generator Tube Integrity	R. Martin
A-8	Mark II Containment Pool Dynamic Loads	T. Bournia
A-9	Anticipated Transients Without Scram	V. Rooney
A-11	Reactor Vessel Material Toughness	A. DeAgazio
A-12	Steam Generator and Reactor Coolant Pump Supports	A. DeAgazio
A-17	Systems Interaction	T. Michaels
A-39	SRV Pool Dynamic Loads	B. Siegel
A-40	Seismic Design Criteria - Short Term Program	T. Cheng
A-43	Containment Emergency Sump Performance	D. Jaffe
A-44	Station Blackout	L. Engle
A-45	Shutdown Decay Heat Removal Requirements	D. DiIanni
A-46	Seismic Qualification of Equipment	P. Y. Chen
A-47	Safety Implications of Control Systems	J. T. Beard
A-48	Hydrogen Control Measures	V. Rooney
A-49	Pressurized Thermal Shock	G. Vissing

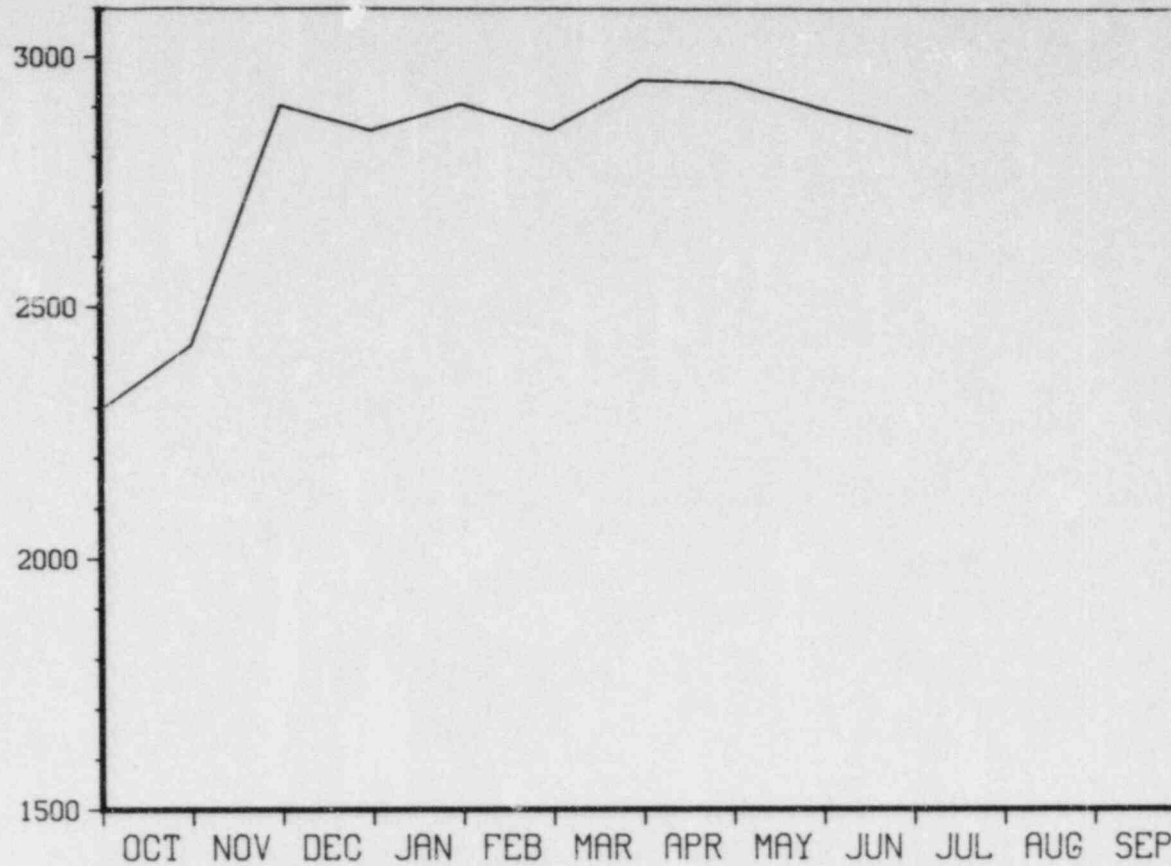
SECTION III

ROUTINE (NON-NUREG 0737)

LICENSING ACTION SUMMARY



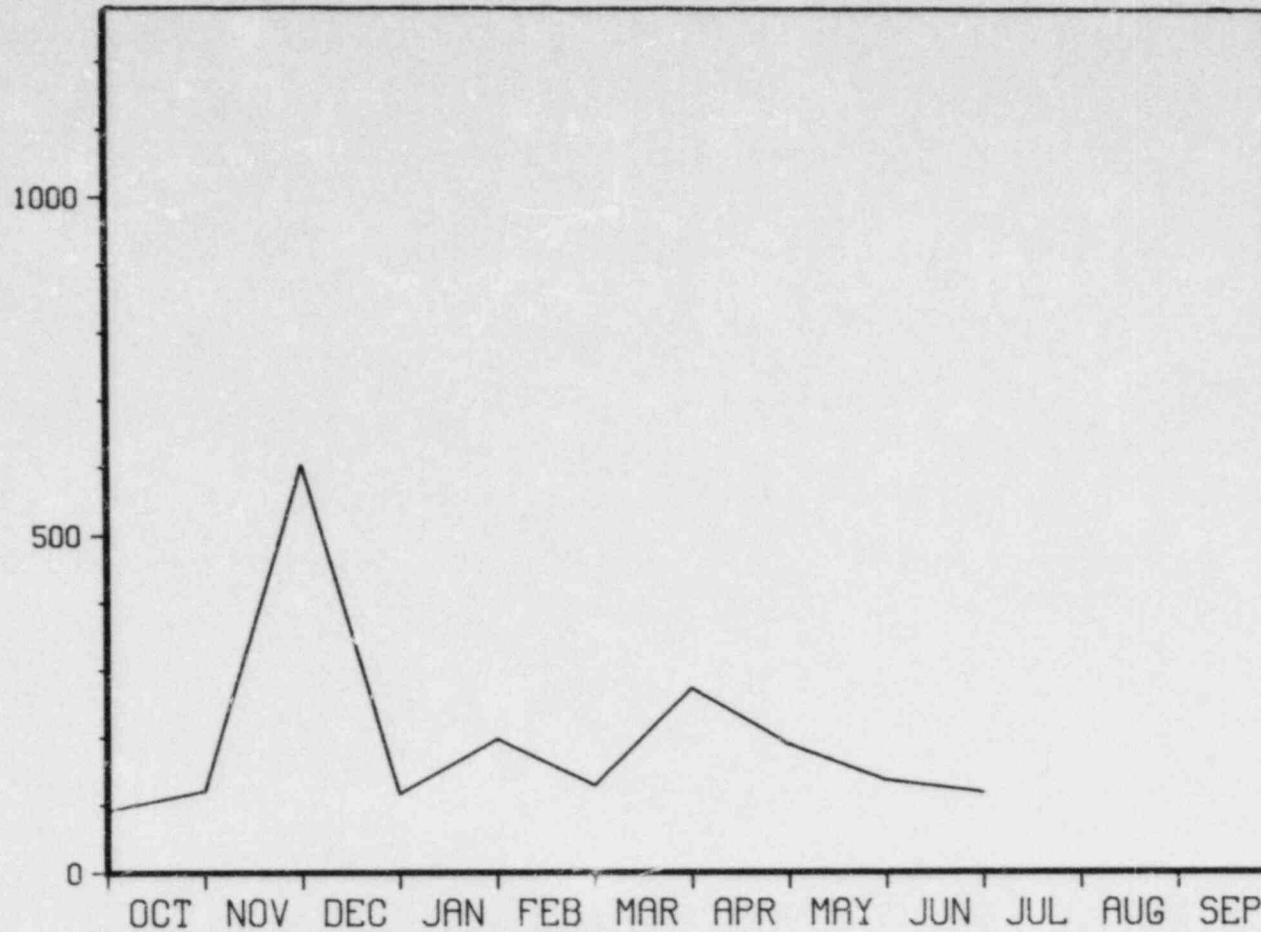
TOTAL ACTIVE ROUTINE ACTIONS  
DIVISION OF LICENSING  
(EXCLUDES NUREG-0737 ITEMS)



TOTAL	2424	2902	2852	2904	2853	2951	2945	2894	2847			
ACTIONS	2421	2899	2846	2898	2845	2942	2941	2891	2842			
HEARINGS	03	03	06	06	08	09	04	03	05			

# NEW ROUTINE ACTIONS BY MONTH

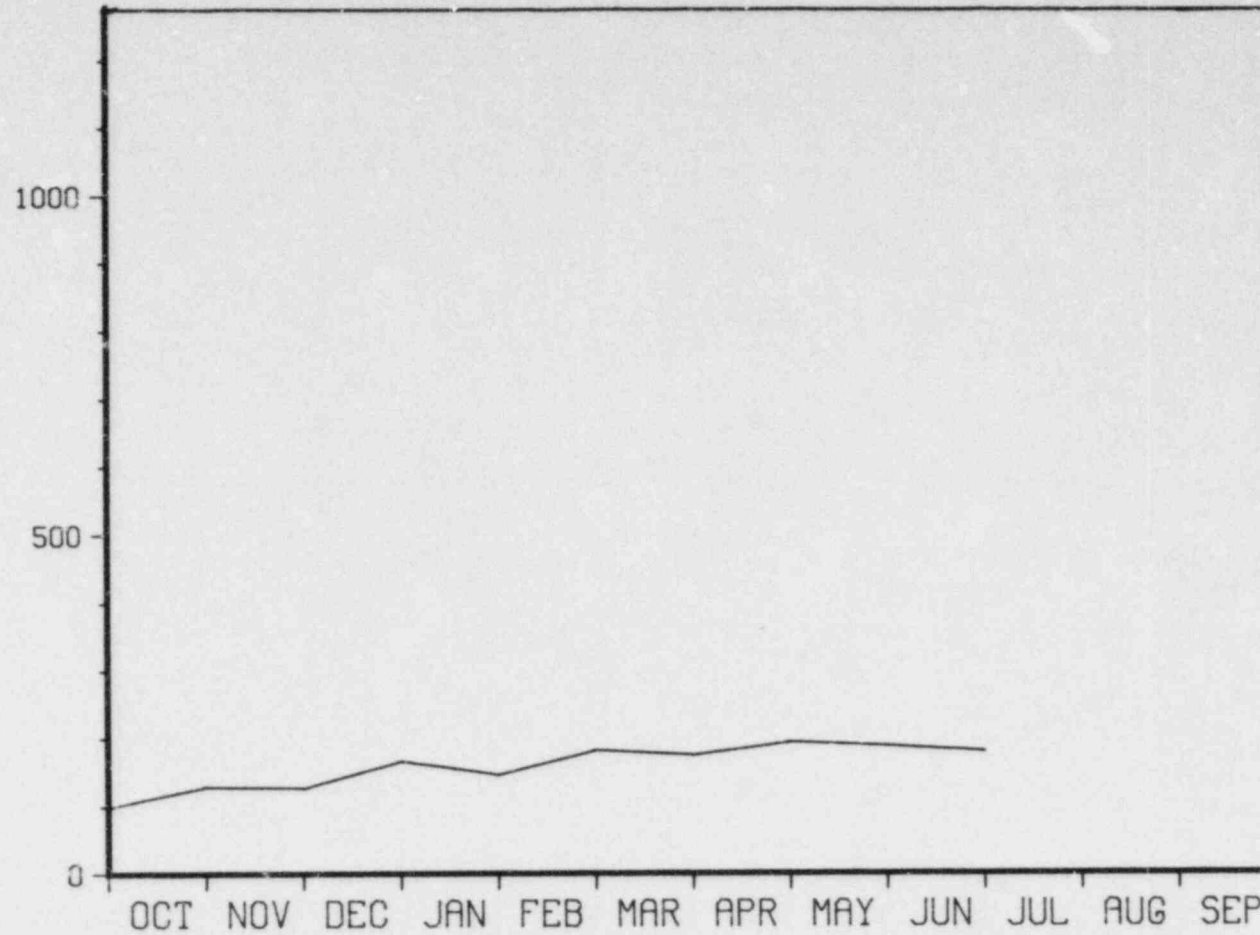
DIVISION OF LICENSING



NEW ACTIONS	121	604	117	197	129	272	189	136	118			
CUMULATIVE	121	725	842	1039	1168	1440	1629	1765	1883			

# COMPLETED ROUTINE ACTIONS BY MONTH

DIVISION OF LICENSING



COMPLETED	129	127	166	146	182	175	194	188	180			
CUMULATIVE	129	256	422	568	750	925	1119	1307	1487			

TECHNICAL ASSIGNMENT CONTROL SYSTEM  
ROUTINE LICENSING ACTIONS BY MONTH

FISCAL YEAR: 1984

REPORT DATE: 07/18/84

FACILITY NAME	OCT.	NOV.	DEC.	JAN.	FEB.	MAR.	APR.	MAY	JUNE	JULY	AUG.	SEPT.
ARKANSAS 1	30	38	37	37	37	39	38	37	38			
ARKANSAS 2	33	36	36	35	34	32	30	30	27			
BEAVER VALLEY 1	28	38	37	36	36	35	33	38	38			
BIG ROCK POINT 1	32	35	35	32	32	31	28	36	35			
BROWNS FERRY 1	54	62	56	56	57	59	57	54	45			
-----												
BROWNS FERRY 2	46	54	52	52	52	53	52	48	39			
BROWNS FERRY 3	45	52	51	65	65	67	67	65	55			
BRUNSWICK 1	38	43	32	30	31	29	29	28	33			
BRUNSWICK 2	47	48	37	34	35	33	34	29	36			
CALLAWAY 1								1	28			
-----												
CALVERT CLIFFS 1	21	24	25	24	26	24	25	24	27			
CALVERT CLIFFS 2	17	26	28	27	29	27	22	21	18			
COOK 1	22	22	22	22	21	21	20	21	20			
COOK 2	26	26	26	25	24	25	25	26	22			
COOPER STATION	17	20	21	18	18	18	18	20	19			
-----												
CRYSTAL RIVER 3	29	38	39	37	38	40	41	40	37			
DAVIS-BESSE 1	43	51	55	53	53	57	53	46	45			
DIABLO CANYON 1	26	34	34	34	34	36	36	36	36			
DRESDEN 1	1	1	1	1	1	2	2	2	2			
DRESDEN 2	37	38	36	36	33	37	37	36	35			
-----												
DRESDEN 3	31	32	30	29	29	30	30	29	28			
DUANE ARNOLD	28	32	31	30	33	30	31	28	24			
FARLEY 1	29	34	30	25	24	20	31	31	31			
FARLEY 2	36	44	41	38	39	43	43	42	42			
FITZPATRICK	31	34	32	35	33	35	33	30	26			
-----												
FORT CALHOUN 1	22	28	29	26	26	29	29	26	25			
FORT ST VRAIN	25	28	28	28	27	26	25	27	27			
GINNA	44	50	54	57	56	62	57	54	53			
GRAND GULF 1	60	61	63	68	67	78	74	75	90			
HADDAM NECK	38	45	45	46	42	47	45	46	46			
-----												
HATCH 1	27	33	34	33	33	32	30	29	29			
HATCH 2	27	34	37	35	31	30	29	28	27			
HUMBOLDT BAY 3	2	2	3	3	3	3	3	2	2			
INDIAN POINT 2	34	37	37	37	36	35	36	35	34			
INDIAN POINT 3	30	38	36	35	36	37	36	35	34			
-----												
KEWAUNEE	23	28	28	30	28	32	30	30	30			
LA CROSSE	32	37	35	37	38	39	39	39	40			
LASALLE 1	8	12	12	13	12	12	11	14	12			
LASALLE 2	2	2	2	2	5	2	1	4	4			
MAINE YANKEE	35	38	36	40	40	39	38	38	37			



FISCAL YEAR: 1984

## ROUTINE LICENSING ACTIONS BY MONTH

REPORT DATE: 07/18/84

<u>FACILITY NAME</u>	<u>OCT.</u>	<u>NOV.</u>	<u>DEC.</u>	<u>JAN.</u>	<u>FEB.</u>	<u>MAR.</u>	<u>APR.</u>	<u>MAY</u>	<u>JUNE</u>	<u>JULY</u>	<u>AUG.</u>	<u>SEPT.</u>
MCGUIRE 1	16	26	26	28	24	25	23	23	24			
MCGUIRE 2	19	29	30	29	25	24	21	21	22			
MILLSTONE 1	46	48	46	49	48	47	51	48	43			
MILLSTONE 2	31	37	32	32	32	34	31	30	28			
MONTICELLO	42	45	43	37	36	36	37	35	35			
NINE MILE POINT 1	24	33	32	36	34	35	34	31	30			
NORTH ANNA 1	22	32	32	39	35	38	36	33	31			
NORTH ANNA 2	22	31	32	36	34	38	37	35	34			
OCONEE 1	27	35	36	33	35	34	35	36	37			
OCONEE 2	28	35	37	33	35	36	37	39	38			
OCONEE 3	27	35	36	32	35	35	35	35	35			
OYSTER CREEK 1	42	47	55	51	48	48	48	48	45			
PALISADES	30	35	35	38	37	37	37	36	31			
PEACH BOTTOM 2	38	42	43	42	37	35	37	39	39			
PEACH BOTTOM 3	36	41	39	38	35	33	35	38	39			
PILGRIM 1	29	33	32	31	31	34	31	31	30			
POINT BEACH 1	21	31	30	28	28	34	31	31	31			
POINT BEACH 2	22	31	30	28	27	30	28	29	29			
PRAIRIE ISLAND 1	12	19	17	18	17	17	17	17	11			
PRAIRIE ISLAND 2	10	17	16	17	16	16	16	16	10			
QUAD CITIES 1	24	28	28	27	27	30	27	29	28			
QUAD CITIES 2	28	32	31	28	25	27	25	25	24			
RANCHO SECO 1	43	51	50	49	49	51	51	51	48			
ROBINSON 2	41	47	46	47	45	46	46	45	43			
SALEM 1	20	30	30	29	28	30	30	29	26			
SALEM 2	18	30	30	29	29	32	32	32	30			
SAN ONOFRE 1	22	30	31	32	28	31	31	34	35			
SAN ONOFRE 2	40	46	46	46	48	53	81	66	70			
SAN ONOFRE 3	32	38	38	38	40	42	71	60	65			
SEQUOYAH 1	33	41	32	40	36	33	34	27	26			
SEQUOYAH 2	29	37	25	29	29	26	26	23	23			
ST LUCIE 1	29	35	35	34	34	30	29	28	28			
ST LUCIE 2	19	26	27	27	27	28	30	29	29			
SUMMER 1	21	28	29	32	30	30	26	26	26			
SURRY 1	28	35	36	33	28	30	30	29	28			
SURRY 2	29	36	37	34	29	30	31	29	27			
SUSQUEHANNA 1	25	28	28	30	31	28	27	39	36			
SUSQUEHANNA 2				1	2	2	5	6	6			
THREE MILE ISLAND 1	40	48	55	56	60	60	53	46	41			
TROJAN	26	30	30	31	30	30	35	35	34			

TECHNICAL ASSIGNMENT CONTROL SYSTEM  
 ROUTINE LICENSING ACTIONS BY MONTH

FISCAL YEAR: 1984

REPORT DATE: 07/18/84

<u>FACILITY NAME</u>	<u>OCT.</u>	<u>NOV.</u>	<u>DEC.</u>	<u>JAN.</u>	<u>FEB.</u>	<u>MAR.</u>	<u>APR.</u>	<u>MAY</u>	<u>JUNE</u>	<u>JULY</u>	<u>AUG.</u>	<u>SEPT.</u>
TURKEY POINT 3	24	28	27	27	26	29	29	31	31			
TURKEY POINT 4	24	28	27	27	26	28	29	32	32			
VERMONT YANKEE 1	31	36	35	33	35	40	39	38	38			
WASHINGTON NUCLEAR 2	2	2	8	10	11	22	22	22	22			
YANKEE-ROWE 1	31	38	38	38	38	42	42	42	42			
-----												
ZION 1	33	39	36	61	60	60	55	55	56			
ZION 2	32	38	35	60	59	59	54	55	55			
GRAND TOTAL	2,424	2,902	2,852	2,904	2,853	2,951	2,945	2,894	2,847			
-----												

REPORT DATE: 07/19/84

## ROUTINE LICENSING ACTIONS

## POWER REACTORS

FOR 06/01/84 - 06/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
ARKANSAS 1	2	23	0	19	38	0	38
ARKANSAS 2	0	16	3	25	27	0	27
BEAVER VALLEY 1	2	35	2	26	38	0	38
BIG ROCK POINT 1	0	25	1	19	35	0	35
BROWNS FERRY 1	0	17	9	27	45	0	45
BROWNS FERRY 2	0	15	9	23	39	0	39
BROWNS FERRY 3	0	31	10	22	55	0	55
BRUNSWICK 1	6	27	1	29	33	0	33
BRUNSWICK 2	8	36	1	42	36	0	36
CALLAWAY 1	27	28	0	0	28	0	28
CALVERT CLIFFS 1	3	31	1	24	27	0	27
CALVERT CLIFFS 2	0	25	3	23	18	0	18
COOK 1	0	11	1	13	20	0	20
COOK 2	0	14	4	16	22	0	22
COOPER STATION	0	12	1	10	19	0	19
CRYSTAL RIVER 3	0	21	3	12	37	0	37
DAVIS-BESSE 1	0	21	1	19	45	0	45
DIABLO CANYON 1	0	13	0	0	35	1	36
DRESDEN 1	0	2	0	1	2	0	2
DRESDEN 2	0	13	1	16	35	0	35
DRESDEN 3	0	15	1	19	28	0	28
DUANE ARNOLD	0	17	5	23	24	0	24
FARLEY 1	1	28	1	27	31	0	31
FARLEY 2	1	23	1	19	42	0	42
FITZPATRICK	0	12	4	17	26	0	26
FORT CALHOUN 1	1	20	2	17	25	0	25
FORT ST VRAIN	3	22	3	21	27	0	27
GINNA	0	29	1	24	53	0	53
GRAND GULF 1	21	55	6	23	90	0	90
HADDAM NECK	0	20	0	11	46	0	46
HATCH 1	2	16	2	17	29	0	29
HATCH 2	0	16	1	19	27	0	27
HUMBOLDT BAY 3	0	2	0	1	2	0	2
INDIAN POINT 2	0	13	1	13	34	0	34
INDIAN POINT 3	0	12	1	8	34	0	34

REPORT DATE: 07/19/84

## ROUTINE LICENSING ACTIONS

## POWER REACTORS

FOR 06/01/84 - 06/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
KEWAUNEE	1	18	1	12	30	0	30
LA CROSSE	1	15	0	16	40	0	40
LASALLE 1	0	12	2	9	12	0	12
LASALLE 2	0	7	0	5	4	0	4
MAINE YANKEE	0	24	1	22	37	0	37
MCGUIRE 1	1	16	0	10	24	0	24
MCGUIRE 2	1	13	0	11	22	0	22
MILLSTONE 1	0	16	5	23	43	0	43
MILLSTONE 2	1	19	3	21	28	0	28
MONTICELLO	2	11	2	16	35	0	35
NINE MILE POINT 1	0	27	1	21	30	0	30
NORTH ANNA 1	0	38	2	23	31	0	31
NORTH ANNA 2	0	38	1	21	34	0	34
OCONEE 1	1	19	0	8	37	0	37
OCONEE 2	1	21	2	11	38	0	38
OCONEE 3	1	20	1	10	35	0	35
OYSTER CREEK 1	0	14	3	11	45	0	45
PALISADES	1	33	6	20	31	0	31
PEACH BOTTOM 2	2	24	2	23	39	0	39
PEACH BOTTOM 3	2	20	1	18	39	0	39
PILGRIM 1	0	16	1	12	30	0	30
POINT BEACH 1	0	29	0	20	31	0	31
POINT BEACH 2	0	24	0	18	29	0	29
PRAIRIE ISLAND 1	0	11	12	19	11	0	11
PRAIRIE ISLAND 2	0	11	13	19	10	0	10
QUAD CITIES 1	0	13	1	9	28	0	28
QUAD CITIES 2	0	8	1	12	24	0	24
RANCHO SECO 1	0	28	3	20	48	0	48
ROBINSON 2	0	28	2	19	43	0	43
SALEM 1	0	14	3	9	26	0	26
SALEM 2	0	18	2	7	30	0	30
SAN ONOFRE 1	2	26	1	14	35	0	35
SAN ONOFRE 2	4	60	0	40	70	0	70
SAN ONOFRE 3	5	59	0	28	65	0	65
SEQUOYAH 1	0	19	1	26	26	0	26



REPORT DATE: 07/19/84

## ROUTINE LICENSING ACTIONS

## POWER REACTORS

FOR 06/01/84 - 06/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
SEQUOYAH 2	0	14	0	20	23	0	23
ST LUCIE 1	0	12	0	11	28	0	28
ST LUCIE 2	1	19	1	8	29	0	29
SUMMER 1	2	22	3	20	26	0	26
SURRY 1	0	17	1	19	28	0	28
SURRY 2	0	16	2	20	27	0	27
SUSQUEHANNA 1	0	37	3	23	36	0	36
SUSQUEHANNA 2	0	10	0	4	6	0	6
THREE MILE ISLAND 1	4	39	9	41	38	3	41
TROJAN	2	25	3	14	34	0	34
TURKEY POINT 3	0	26	0	20	31	0	31
TURKEY POINT 4	0	27	0	20	32	0	32
VERMONT YANKEE 1	4	21	4	14	38	0	38
WASHINGTON NUCLEAR 2	0	22	0	2	22	0	22
YANKEE-ROWE 1	1	21	1	13	42	0	42
ZION 1	1	40	0	15	55	1	56
ZION 2	0	40	0	15	55	0	55
GRAND TOTAL	118	1,883	180	1,487	2,842	5	2,847

REPORT DATE: 07/10/84

## ACTIVE LICENSING ACTIONS

## NON - POWER REACTORS

FOR 06/01/84 - 06/30/84

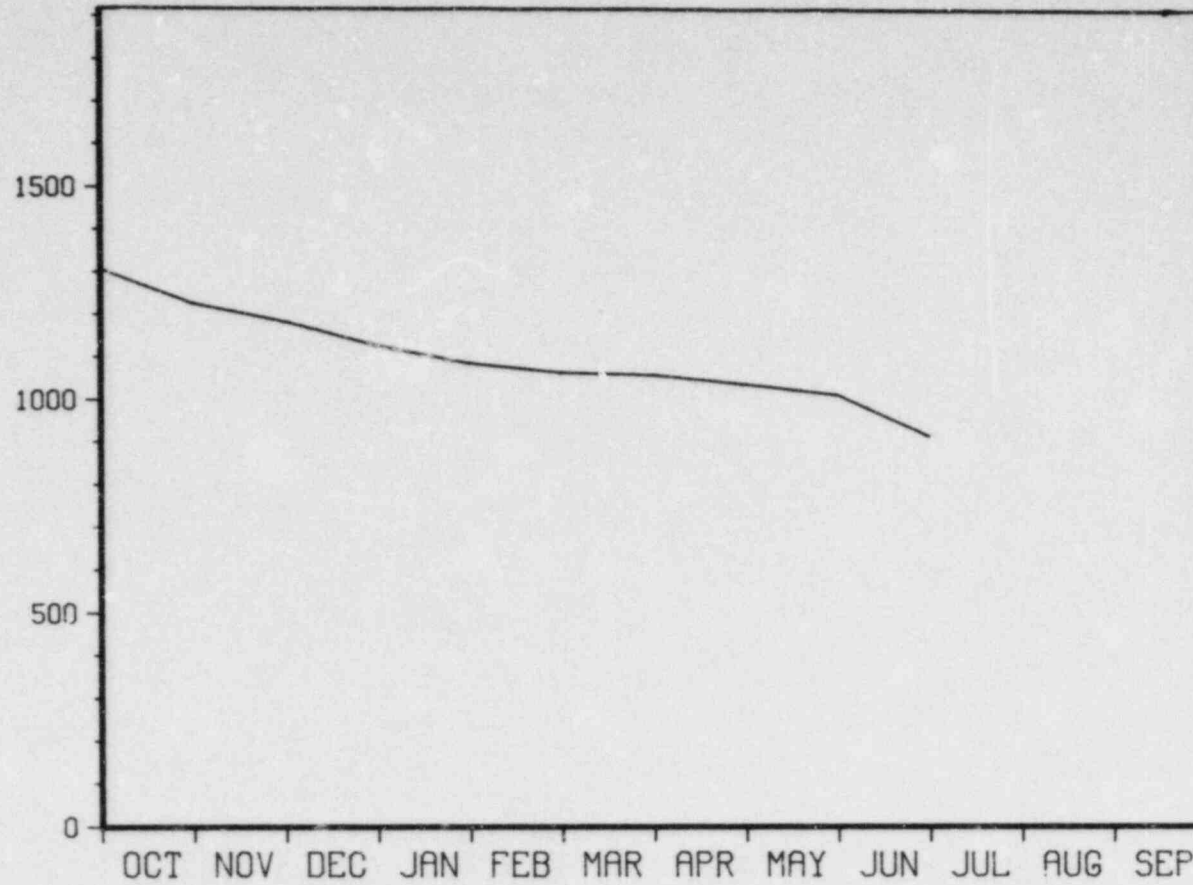
FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
MICHIGAN STATE	0	0	0	0	1	0	1
GRAND TOTAL	0	0	0	0	1	0	1

SECTION IV

NUREG-0737 LICENSING ACTIONS

# TOTAL ACTIVE NUREG-0737 ACTIONS

DIVISION OF LICENSING

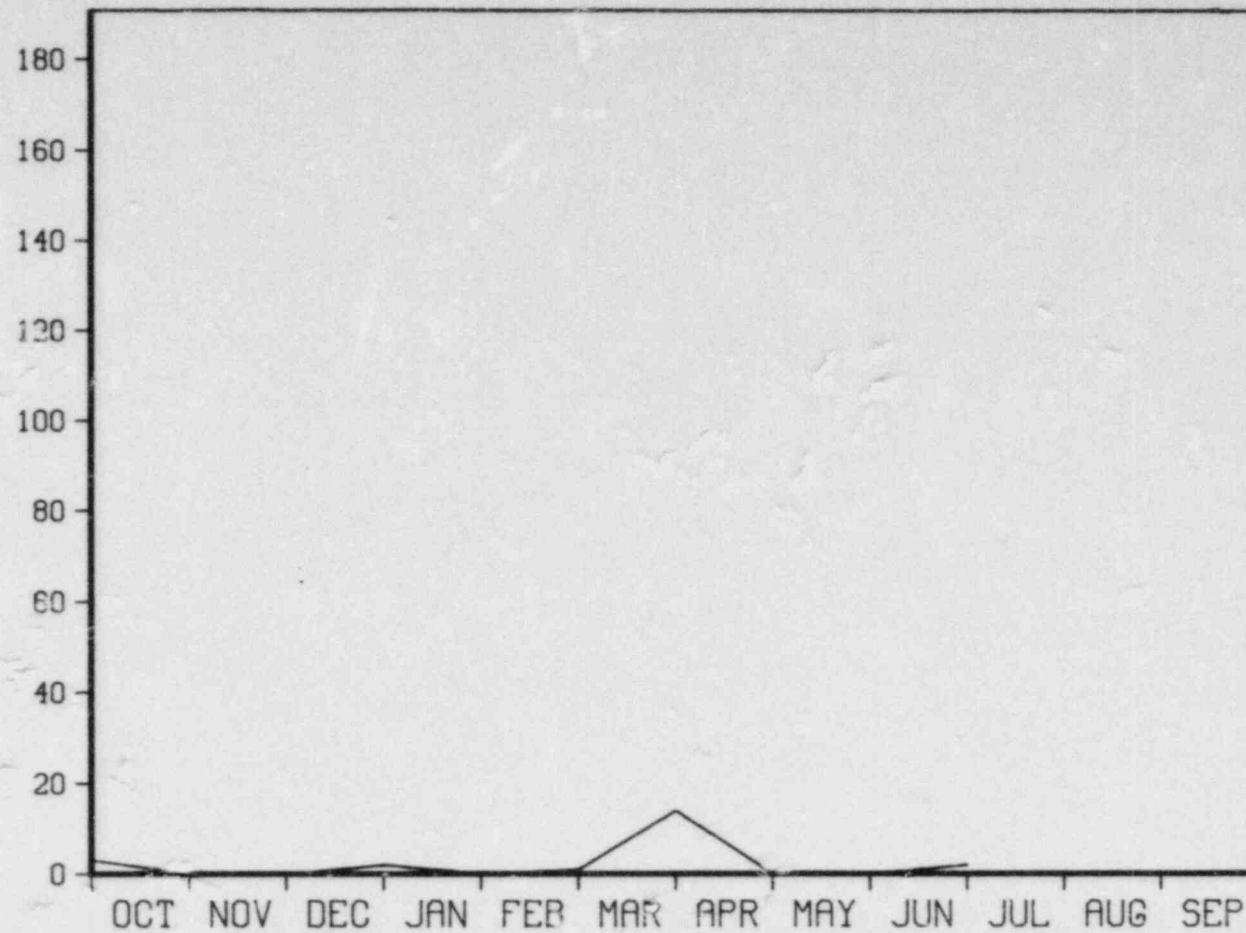


TOTAL	1227	1183	1126	1085	1063	1057	1035	1010	912			
ACTIONS	1227	1183	1126	1085	1063	1057	1035	1010	912			
HEARINGS	00	00	00	00	00	00	00	00	00			



## NEW NUREG-0737 ACTIONS BY MONTH

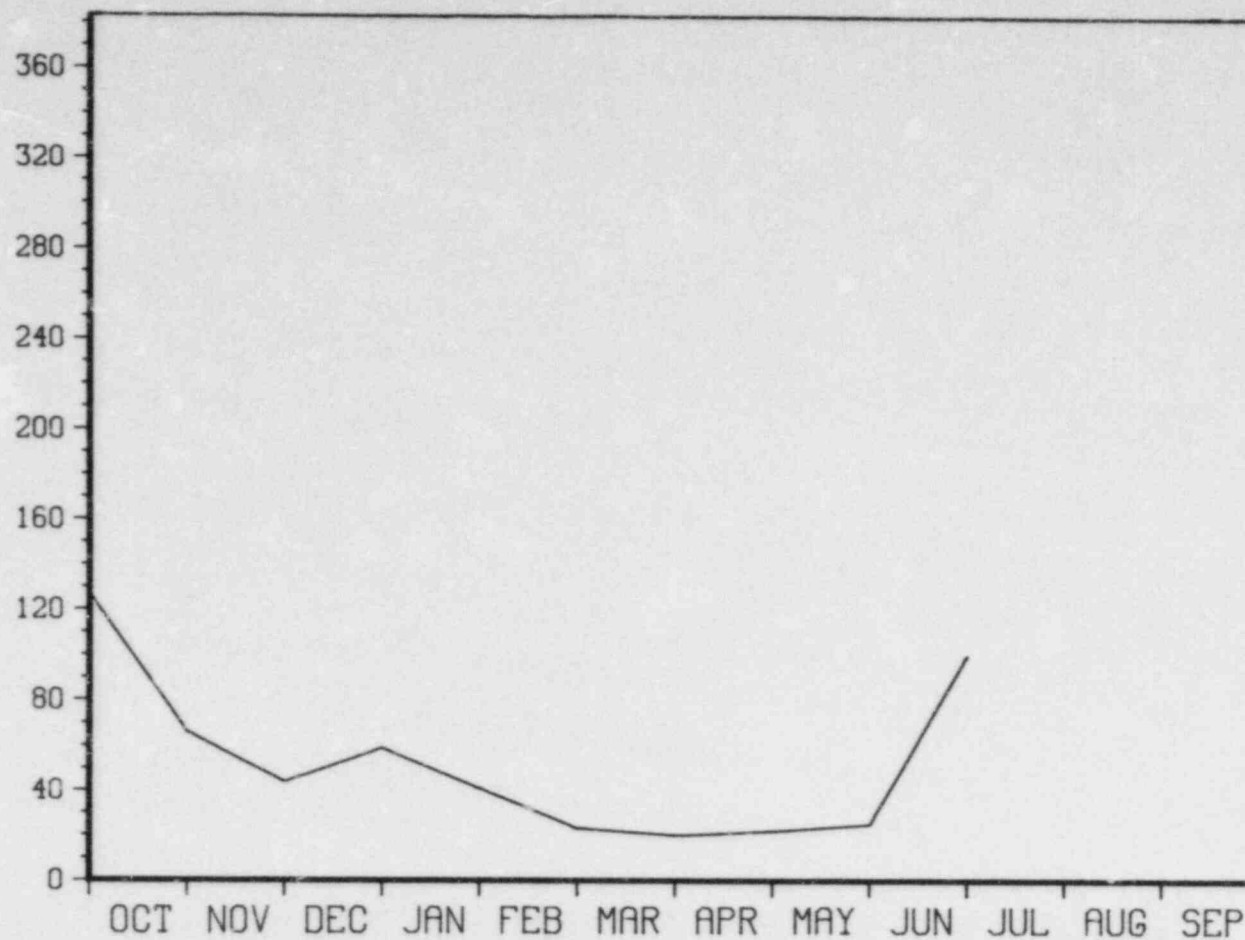
DIVISION OF LICENSING



NEW ACTIONS	00	00	02	00	01	14	00	00	02			
CUMULATIVE	0	0	2	2	3	17	17	17	19			

## COMPLETED NUREG-0737 ACTIONS BY MONTH

DIVISION OF LICENSING



COMPLETED	66	44	59	41	23	20	22	25	100			
CUMULATIVE	66	110	169	210	233	253	275	300	400			

TECHNICAL ASSIGNMENT CONTROL SYSTEM  
 NUREG - 0737 LICENSING ACTIONS BY MONTH

FACILITY NAME	OCT.	NOV.	DEC.	JAN.	FEB.	MAR.	APR.	MAY	JUNE	JULY	AUG.	SEPT.
ARKANSAS 1	14	14	14	14	14	14	14	14	12			
ARKANSAS 2	15	14	14	14	14	14	14	14	12			
BEAVER VALLEY 1	15	15	14	14	14	14	14	14	12			
BIG ROCK POINT 1	19	19	16	16	16	15	13	13	12			
BROWNS FERRY 1	18	18	16	16	15	15	15	15	14			
BROWNS FERRY 2	18	18	16	16	15	15	15	15	14			
BROWNS FERRY 3	18	18	16	16	15	15	15	15	15			
BRUNSWICK 1	18	18	17	16	16	16	16	14	13			
BRUNSWICK 2	18	18	17	16	16	16	16	13	13			
CALVERT CLIFFS 1	16	16	16	16	16	16	16	16	14			
CALVERT CLIFFS 2	16	16	16	16	16	16	16	16	14			
COOK 1	14	14	14	14	14	14	14	14	12			
COOK 2	14	14	13	13	13	13	13	13	11			
COOPER STATION	14	14	11	11	11	11	11	11	10			
CRYSTAL RIVER 3	18	17	17	17	17	17	16	15	14			
DAVIS-BESSE 1	18	18	18	18	17	17	17	15	14			
DIABLO CANYON 1	5	5	5	5	5	5	5	5	5			
DRESDEN 2	19	19	18	15	15	15	15	15	14			
DRESDEN 3	18	18	17	14	14	14	14	14	13			
DUANE ARNOLD	18	17	15	14	14	14	13	12	11			
FARLEY 1	15	14	13	12	12	12	12	12	11			
FARLEY 2	15	14	13	12	12	12	12	12	12			
FITZPATRICK	18	18	16	16	16	16	16	16	15			
FOR: CALHOUN 1	13	13	13	13	13	13	13	13	12			
FOR: ST VRAIN	11	11	11	11	11	11	11	10	10			
GINNA	19	19	18	18	18	18	17	13	11			
GRAND GULF 1	5	5	5	5	5	6	6	6	5			
HADDAM NECK	20	20	19	19	19	19	19	19	16			
HATCH 1	17	17	16	13	13	13	13	13	13			
HATCH 2	17	17	16	13	13	13	13	13	12			
INDIAN POINT 2	15	15	14	13	13	11	11	11	10			
INDIAN POINT 3	17	17	14	13	13	11	11	11	10			
KEWAUNEE	16	16	16	15	15	15	14	14	12			
LA CROSSE	17	17	17	17	15	14	14	11	10			
LASALLE 1	3	3	3	3	3	4	4	4	4			
LASALLE 2						1	1	1	1			
MAINE YANKEE	15	15	15	15	15	15	15	15	12			
MCGUIRE 1	4	4	4	4	4	4	4	4	4			
MCGUIRE 2	3	3	3	3	3	4	4	4	4			
MILLSTONE 1	17	17	15	15	15	15	15	15	15			

FISCAL YEAR: 1984

NUREG - 0737 LICENSING ACTIONS BY MONTH

REPORT DATE: 07/19/84

FACILITY NAME	OCT.	NOV.	DEC.	JAN.	FEB.	MAR.	APR.	MAY	JUNE	JULY	AUG.	SEPT.
MILLSTONE 2	16	16	16	16	15	15	15	15	11			
MONTICELLO	18	17	15	15	15	13	13	13	12			
NINE MILE POINT 1	18	18	17	15	15	15	14	13	13			
NORTH ANNA 1	17	16	16	15	14	13	13	13	12			
NORTH ANNA 2	18	17	16	15	14	13	13	13	12			
OCONEE 1	20	14	14	13	13	13	13	13	13			
OCONEE 2	20	14	14	13	13	13	13	13	12			
OCONEE 3	20	14	14	13	13	13	13	13	12			
CYSTER CREEK 1	20	19	17	17	16	16	16	16	14			
PALISADES	18	17	17	17	17	17	16	16	14			
PEACH BOTTOM 2	17	16	16	16	13	13	13	12	10			
PEACH BOTTOM 3	17	16	16	16	13	13	13	12	10			
PILGRIM 1	21	21	19	19	19	16	16	16	15			
POINT BEACH 1	16	14	14	13	13	13	13	13	11			
POINT BEACH 2	16	14	14	13	13	13	13	13	11			
PRAIRIE ISLAND 1	15	14	13	13	13	13	13	13	11			
PRAIRIE ISLAND 2	15	14	13	13	13	13	13	13	11			
QUAD CITIES 1	16	15	13	13	13	13	13	13	11			
QUAD CITIES 2	16	15	13	13	13	13	13	13	11			
RANCHO SECO 1	14	14	14	14	14	14	14	14	12			
ROBINSON 2	19	19	19	19	19	19	16	16	13			
SALEM 1	16	15	15	14	14	14	14	14	12			
SALEM 2	15	14	14	13	13	13	13	13	11			
SAN ONOFRE 1	20	20	19	19	17	17	14	14	12			
SAN ONOFRE 2	7	7	7	7	7	9	7	6	5			
SAN ONOFRE 3	6	6	6	6	6	9	6	5	5			
SEQUOYAH 1	16	16	12	12	12	11	11	9	9			
SEQUOYAH 2	17	17	15	15	15	15	15	15	15			
ST LUCIE 1	13	13	13	13	13	13	13	13	11			
ST LUCIE 2	4	4	4	4	4	4	4	4	3			
SUMMER 1	6	6	6	6	5	5	5	5	4			
SURRY 1	15	15	15	14	13	13	13	13	12			
SURRY 2	15	15	15	14	13	13	13	13	11			
SUSQUEHANNA 1	3	3	4	4	4	4	4	4	4			
SUSQUEHANNA 2						2	2	2	2			
THREE MILE ISLAND 1	20	20	20	20	20	15	12	12	11			
TROJAN	15	15	15	13	13	13	13	13	11			
TURKEY POINT 3	16	14	14	13	13	13	13	13	12			
TURKEY POINT 4	16	14	14	13	13	13	13	13	11			
VERMONT YANKEE 1	17	17	17	15	14	14	14	14	13			



FISCAL YEAR: 1984

## NUREG-0737 LICENSING ACTIONS BY MONTH

REPORT DATE: 07/19/84

<u>FACILITY NAME</u>	<u>OCT.</u>	<u>NOV.</u>	<u>DEC.</u>	<u>JAN.</u>	<u>FEB.</u>	<u>MAR.</u>	<u>APR.</u>	<u>MAY</u>	<u>JUNE</u>	<u>JULY</u>	<u>AUG.</u>	<u>SEPT.</u>
WASHINGTON NUCLEAR 2	16	16	13	13	14	1	1	1	1			
YANKEE-ROWE 1	14	14	14	13	13	14	14	13	11			
ZION 1	13	13	13	12	12	13	13	13	12			
ZION 2	13	13	13	12	12	12	12	12	11			
GRAND TOTAL	1,227	1,183	1,126	1,085	1,063	1,057	1,035	1,010	912			

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TECHNICAL ASSIGNMENT CONTROL SYSTEM  
NUREG - 0737 LICENSING ACTIONS  
POWER REACTORS  
FOR 06/01/84 - 06/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
ARKANSAS 1	0	0	2	3	12	0	12
ARKANSAS 2	0	0	2	4	12	0	12
BEAVER VALLEY 1	0	0	2	3	12	0	12
BIG ROCK POINT 1	0	0	1	7	12	0	12
BROWNS FERRY 1	0	0	1	6	14	0	14
BROWNS FERRY 2	0	0	1	6	14	0	14
BROWNS FERRY 3	0	0	0	5	15	0	15
BRUNSWICK 1	0	0	1	8	13	0	13
BRUNSWICK 2	0	0	0	8	13	0	13
CALLAWAY 1	2	2	0	0	2	0	2
CALVERT CLIFFS 1	0	0	2	5	14	0	14
CALVERT CLIFFS 2	0	0	2	5	14	0	14
COOK 1	0	1	2	3	12	0	12
COOK 2	0	0	2	3	11	0	11
COOPER STATION	0	0	1	4	10	0	10
CRYSTAL RIVER 3	0	0	1	5	14	0	14
DAVIS-BESSE 1	0	0	1	5	14	0	14
DIABLO CANYON 1	0	0	0	0	5	0	5
DRESDEN 2	0	0	1	6	14	0	14
DRESDEN 3	0	0	1	6	13	0	13
DUANE ARNOLD	0	0	1	7	11	0	11
FARLEY 1	0	0	1	4	11	0	11
FARLEY 2	0	0	0	3	12	0	12
FITZPATRICK	0	0	1	3	15	0	15
FORT CALHOUN 1	0	0	1	2	12	0	12
FORT ST VRAIN	0	0	0	2	10	0	10
GINNA	0	0	2	8	11	0	11
GRAND GULF 1	0	1	0	0	6	0	6
HADDAM NECK	0	0	3	4	16	0	16
HATCH 1	0	0	0	4	13	0	13
HATCH 2	0	0	1	5	12	0	12
INDIAN POINT 2	0	0	1	9	10	0	10
INDIAN POINT 3	0	0	1	9	10	0	10
KEWAUNEE	0	0	2	4	12	0	12
LA CROSSE	0	0	1	7	10	0	10

REPORT DATE: 07/19/84

## NUREG-0737 LICENSING ACTIONS

## POWER REACTORS

FOR 06/01/84 - 06/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
LASALLE 1	0	1	0	0	4	0	4
LASALLE 2	0	1	0	0	1	0	1
MAINE YANKEE	0	0	3	6	12	0	12
MCGUIRE 1	0	0	0	0	4	0	4
MCGUIRE 2	0	1	0	0	4	0	4
MILLSTONE 1	0	0	0	2	15	0	15
MILLSTONE 2	0	0	4	6	11	0	11
MONTICELLO	0	0	1	6	12	0	12
NINE MILE POINT 1	0	0	0	5	13	0	13
NORTH ANNA 1	0	0	1	8	12	0	12
NORTH ANNA 2	0	0	1	9	12	0	12
OCONEE 1	0	0	0	8	13	0	13
OCONEE 2	0	0	1	9	12	0	12
OCONEE 3	0	0	1	9	12	0	12
OYSTER CREEK 1	0	0	2	6	14	0	14
PALISADES	0	0	2	7	14	0	14
PEACH BOTTOM 2	0	0	2	7	10	0	10
PEACH BOTTOM 3	0	0	2	7	10	0	10
PILGRIM 1	0	0	1	6	15	0	15
POINT BEACH 1	0	0	2	5	11	0	11
POINT BEACH 2	0	0	2	5	11	0	11
PRAIRIE ISLAND 1	0	0	2	5	11	0	11
PRAIRIE ISLAND 2	0	0	2	5	11	0	11
QUAD CITIES 1	0	0	2	5	11	0	11
QUAD CITIES 2	0	0	2	5	11	0	11
RANCHO SECO 1	0	0	2	5	12	0	12
ROBINSON 2	0	0	3	6	13	0	13
SALEM 1	0	0	2	6	12	0	12
SALEM 2	0	0	1	5	12	0	12
SAN ONOFRE 1	0	0	2	8	12	0	12
SAN ONOFRE 2	0	3	1	5	5	0	5
SAN ONOFRE 3	0	3	0	4	5	0	5
SEQUOYAH 1	0	0	0	7	9	0	9
SEQUOYAH 2	0	0	0	2	15	0	15
ST LUCIE 1	0	0	2	5	11	0	11

REPORT DATE: 07/19/84

NUREG - 0737 LICENSING ACTIONS

POWER REACTORS

FOR 06/01/84 - 06/30/84

FACILITY	NEW ACTIONS FOR THIS REPORTING PERIOD	NEW ACTIONS TO DATE IN FY 84	COMPLETED ACTIONS FOR THIS REPORTING PERIOD	COMPLETED ACTIONS TO DATE IN FY 84	NUMBER OF ROUTINE ACTIONS	NUMBERS OF HEARINGS	TOTAL ACTIONS
ST LUCIE 2	0	0	1	1	3	0	3
SUMMER 1	0	0	1	2	4	0	4
SURRY 1	0	0	1	4	12	0	12
SURRY 2	0	0	2	5	11	0	11
SUSQUEHANNA 1	0	2	0	1	4	0	4
SUSQUEHANNA 2	0	2	0	0	2	0	2
THREE MILE ISLAND 1	0	0	1	10	11	0	11
TROJAN	0	0	2	4	11	0	11
TURKEY POINT 3	0	0	1	4	12	0	12
TURKEY POINT 4	0	0	2	5	11	0	11
VERMONT YANKEE 1	0	0	1	4	13	0	13
WASHINGTON NUCLEAR 2	0	1	0	0	1	0	1
YANKEE-ROWE 1	0	1	2	7	11	0	11
ZION 1	0	0	1	3	12	0	12
ZION 2	0	0	1	3	11	0	11
GRAND TOTAL	2	19	100	400	912	0	912



STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNTZN	ORDERS ISSUED	SUBMTTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
I.A.1.1.1 STA ON DUTY	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	65(65)	A	10/81	65(65)
I.A.1.1.3 STA TRAINED PER LL CAT B (F-01)	Y	Y	POST	LQB	65(65)	65(65)	65(65)	65(65)	65(65)	B	06/82	65(65)
I.A.1.1.4 STA LONG TERM PROGRAM	Y	Y	POST	LQB	65(65)	65(65)	65(65)	NA	NA	B	06/82	65(65)
I.A.1.2 SHIFT SUPERVSR DUTIES	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
I.A.1.3.1 LIMIT OVERTIME (F-01)	Y	Y	POST	IE	65(65)	65(65)	65(65)	65(65)	NA	B	12/81	65(65)
I.A.1.3.2 MIN SHIFT CREW (F-02)	Y	Y	POST	LQB	NE	65(65)	65(65)	65(65)	65(65)	D	09/84	65(65)
I.A.2.1.1 SRO EXPERIENCE	Y	Y	POST	OLB	NE	NA	65(65)	NA	NA	B	03/80	65(65)
I.A.2.1.2 SRO BE RO 1 YR	Y	Y	POST	OLB	NE	NA	65(65)	NA	NA	B	03/80	65(65)
I.A.2.1.3 3 MO TRAINING ON SHIFT	Y	Y	POST	OLB	NE	NA	65(65)	NA	NA	B	03/80	65(65)
I.A.2.1.4 MODIFY TN PROGRAM (F-03)	Y	Y	POST	OLB	65(65)	65(65)	65(65)	65(65)	NA	D	09/83	65(65)
I.A.2.1.5 FACILITY CERTI	Y	Y	POST	OLB	NE	NA	65(65)	NA	NA	C	03/80	65(65)
I.A.2.3 INSTR COMPLETE SRO TRAINING	Y	Y	POST	OLB	NE	NA	65(65)	NA	NA	B	03/80	65(65)

LICENSEE SCHEDULE CODE KEYS: A- STILL  
 B- THROUGH 6/30/81 EXCLUDING STILL  
 C- 6/30/81 TO 03/01/82  
 D- AFTER 03/01/82  
 E- SUPPLEMENT 1

OTHER KEYS: XX(XX)- NO. COMPLETE(TOTAL NO. TO BE COMPLETED)  
 Y- COMPLETE  
 NA- NOT APPLICABLE  
 NE- NO ENTRY  
 TBD- TO BE DETERMINED

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHKL REVIEW ORGNZTN	ORDERS ISSUED	SUBMTTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
I.A.3.1.1 SCOPE OF LICENSING EXAM	Y	Y	OLB	NA	NE	NA	65(65)	NA	NA	B	03/80	65(65)
I.A.3.1.2 PASSING GRADE	Y	Y	NA	OLB	NE	NA	65(65)	NA	NA	B	03/80	65(65)
I.A.3.1.3 SIMULATOR EXAM	Y	Y	NA	OLB	65(65)	65(65)	65(65)	65(65)	NA	C	10/81	65(65)
I.C.1.1 SB LOCA PROCED	Y	Y	POST	RSB	NA	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
I.C.1.2A ICC GUIDELINES (F-04)	Y	Y	PRE	PTB	65(65)	65(65)	65(65)	65(65)	NA	B	12/81	65(65)
I.C.1.2B ICC PROCEDURES (F-04)	Y	Y	PRE	PTB/FRC	0(65)	0(65)	0(65)	0(65)	NA	E	09/84	0(65)
I.C.1.3A TRANSIENT/ACC GUIDELINES (F-05)	Y	Y	PRE	RSB	65(65)	65(65)	65(65)	65(65)	NA	B	09/82	65(65)
I.C.1.3B TRANS/ACCIDENT PROCEDURES (F-05)	Y	Y	PRE	PTB/FRC	0(65)	0(65)	0(65)	0(65)	NA	E	09/84	0(65)
I.C.2 SHIFT/RELIEF TURNOVER PROCD	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
I.C.3 SHIFT SUP RESP	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
I.C.4 CONT RM ACCESS	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
I.C.5 FEEDBACK OF OPERATG EXPERC	Y	Y	POST	IE	65(65)	NA	65(65)	65(65)	NA	B	12/81	65(65)
I.C.6 PERF OF OPERTG ACTIVITIES	Y	Y	POST	IE	65(65)	NA	64(65)	64(65)	NA	C	09/83	64(65)

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNTZN	ORDERS ISSUED	SUBMTTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
I D 1 CONTROL ROOM DESIGN REVIEW (F-08)	NE	NE	NE	HFEB/LLL	0(65)	1(65)	0(65)	0(65)	NA	E	TBD	0(65)
I D 2 1 SPDS DESCRIPTN	NE	NE	NE	HFEB/LLL	0(65)	0(65)	0(65)	0(65)	NA	E	TBD	0(65)
I D 2 2 SPDS INSTALLED (F-09)	NE	NE	NE	HFEB/LLL	NE	0(65)	0(65)	0(65)	0(65)	E	TBD	0(65)
II B 1 1 RCS VENT DESGN (F-10)	Y	Y	POST	RSB/LLL	NE	65(65)	65(65)	65(65)	NA	C	09/83	65(65)
II B 1 2 RCS VENT INSTL (F-10)	Y	Y	PRE	RSB/LLL	NE	65(65)	65(65)	65(65)	23(65)	D	09/84	23(65)
II B 1 3 RCS VENT PROCD (F-10)	Y	Y	PRE	RSB/LLL	NE	65(65)	65(65)	65(65)	NA	D	09/83	65(65)
II B 2 1 SHIELDG REVIEW (F-11)	Y	Y	POST	DL/EGG	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
II B 2 2 SHIELDG MODS (F-11)	Y	Y	POST	RAB/EGG	65(65)	65(65)	62(65)	62(65)	NA	C	09/84	62(65)
II B 2 3 EQUIP QUAL (B-60)	Y	Y	POST	EQB/FRC	NE	65(65)	65(65)	NA	NA	D	09/83	65(65)
II B 3 1 ACC SAMP INTER SYSTEM	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
II B 3 2 UPGRADE ACC SAMP (F-12)	Y	Y	POST	ETB/INEL	65(65)	65(65)	26(65)	26(65)	3(65)	C	09/84	3(65)

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STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIW ORGNTZN	ORDERS ISSUED	SUBMTTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.B.4 TRG PROG CORE DAM (F-13)	Y	Y	POST	OLB	65(65)	65(65)	65(65)	65(65)	NA	C	09/83	65(65)
II.D.1.1 RV/SV TEST PROGRAM (F-14)	Y	Y	POST	DL/EGG	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
II.D.1.2A TEST PROGRAM COMP. BWR'S (F-14)	Y	Y	POST	MEB/EGG	NE	24(24)	24(24)	NA	NA	C	07/82	24(24)
II.D.1.2A* TEST PROGRAM COMP. PWR'S (F-14)	Y	Y	POST	MEB/EGG	NE	40(40)	40(40)	NA	NA	C	09/83	40(40)
II.D.1.2.B* PLANT SPEC RPTS. PWR'S (F-14)	Y	Y	PRE	MEB/EGG	40(40)	40(40)	0(40)	0(40)	NA	D	09/84	0(40)
II.D.1.2B PSPEC RPT. BWR (F-14)	Y	Y	PRE	EQB	24(24)	24(24)	16(24)	16(24)	NA	D	09/84	16(24)
II.D.1.3 BK VALVE TESTING (F-14)	Y	Y	PRE	MEB/EGG	40(40)	40(40)	0(40)	0(40)	NA	D	09/84	0(40)
II.D.3.1 VALVE POS INDIC INSTL	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	65(65)	A	09/81	65(65)
II.D.3.3 VALVE POS ENVIR QUAL (B-60)	Y	Y	POST	EQB/FRC	NE	65(65)	65(65)	NA	NA	D	09/83	65(65)
II.E.1.1 AFS EVAL (F-15)	Y	Y	PRE	ASB	NE	40(40)	38(40)	38(40)	36(40)	C	09/83	36(40)
II.E.1.2.1A AFS INIT CONT GRADE (F-16)	Y	Y	PRE	DL	40(40)	40(40)	40(40)	40(40)	40(40)	A	08/81	40(40)



STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

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	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNTZN	ORDERS ISSUED	SUBMTTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.E.1.2.1.B AFS INIT SAFE GRADE (F-16)	Y	Y	POST	ICSB	40(40)	40(40)	40(40)	40(40)	27(40)	C	09/83	27(40)
II.E.1.2.2.A AFS FLOW CONT GRADE (F-17)	Y	Y	POST	DL	40(40)	40(40)	40(40)	40(40)	40(40)	A	08/81	40(40)
II.E.1.2.2.B AFS FLOW SAFE GRADE (F-17)	Y	Y	POST	ICSB	40(40)	40(40)	40(40)	40(40)	29(40)	C	09/83	29(40)
II.E.3.1 EMERG PWR FOR HEATERS	Y	Y	POST	DL	40(40)	40(40)	40(40)	40(40)	40(40)	A	09/81	40(40)
II.E.4.1.1 DED H2 PENET DESIGN (F-18)	Y	Y	PRE	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
II.E.4.1.2 DED H2 PENET INSTALL (F-18)	Y	Y	POST	CSB	NE	65(65)	65(65)	65(65)	NA	C	12/81	65(65)
II.E.4.2.1-4 DIVERSE ISO SIG (F-19)	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	65(65)	A	09/81	65(65)
II.E.4.2.5A CONT PRESS SETPT (F-19)	Y	Y	PRE	CSB/LLL	65(65)	65(65)	65(65)	65(65)	NA	B	09/81	65(65)
II.E.4.2.5B CONT PRESS MODIFY (F-19)	Y	Y	POST	CSB/LLL	10(10)	10(10)	10(10)	10(10)	10(10)	C	09/83	10(10)

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## STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

Z-6

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNTZN	ORDERS ISSUED	SUBMTTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.E.4.2.6 CONT PURGE VALVES (F-19)	Y	Y	POST	CSB/LLL	65(65)	65(65)	60(65)	60(65)	NA	C	09/83	59(65)
II.E.4.2.7 RAD SIG ON PURGE VAL (F-19)	Y	Y	POST	CSB/LLL	65(65)	65(65)	63(65)	60(65)	14(65)	C	09/83	14(65)
II.F.1.1 NOBLE GAS MONITOR (F-20)	Y	Y	POST	ETSB/ INEL	65(65)	65(65)	65(65)	65(65)	3(65)	C	09/84	3(65)
II.F.1.2 IODINE/PART MONITOR (F-21)	Y	Y	POST	ETSB/ INEL	65(65)	65(65)	65(65)	65(65)	4(65)	C	09/84	4(65)
II.F.1.3 CONT HI-RANGE MONITOR (F-22)	Y	Y	POST	ETSB/ INEL	65(65)	65(65)	65(65)	65(65)	3(65)	C	09/83	3(65)
II.F.1.4 CONT PRESS MONITOR (F-23)	Y	Y	POST	CSB/LLL	65(65)	65(65)	60(65)	60(65)	4(65)	C	09/83	4(65)
II.F.1.5 CONT H2O LEVEL (F-24)	Y	Y	POST	CSB/LLL	65(65)	65(65)	61(65)	61(65)	2(65)	C	09/83	2(65)
II.F.1.6 CONT HYDROGEN (F-25)	Y	Y	POST	CSB/ LLL	65(65)	65(65)	59(65)	59(65)	2(65)	C	09/83	2(65)
II.F.2.1 SUB COOL METER	Y	Y	POST	DL	40(40)	40(40)	40(40)	40(40)	40(40)	A	10/81	40(40)
II.F.2.3 INSTRUMENTATN (F-26)	Y	Y	POST	CPB	07(65)	65(65)	1(65)	1(65)	1(65)	D	09/84	1(65)

STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGN TZN	ORDERS ISSUED	SUBMTTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.G.1 UPGRADE POWER PORV. LVL INST	Y	Y	POST	DL	40(40)	40(40)	40(40)	40(40)	40(40)	A	09/81	40(40)
II.K.1 IE BULLETINS	Y	Y	POST	DL	NA	65(65)	65(65)	65(65)	NA	B	06/81	65(65)
II.K.2.8 UPGRADE AFW	Y	Y	PRE	ASB	07(07)	7(07)	7(07)	7(07)	7(07)	B	08/81	7(07)
II.K.2.9 FEMA ON ICS (F-27)	Y	Y	POST	ICSB	07(07)	7(07)	7(07)	7(07)	NA	B	06/82	7(07)
II.K.2.10 SAFE GRADE TRIP (F-28)	Y	Y	POST	ICSB	07(07)	7(07)	7(07)	7(07)	7(07)	C	06/82	7(07)
II.K.2.11 OPER TNG & DRILLING (F-29)	Y	Y	POST	OLB	07(07)	7(07)	7(07)	7(07)	NA	B	11/81	7(07)
II.K.2.13A THERM MECH RPT B&W (F-30)	Y	Y	POST	RSB/LLL	07(07)	7(07)	7(07)	7(07)	7(07)	C	09/84	7(07)
II.K.2.13B THERM MECH RPT CE&W (F-30)	Y	Y	POST	RSB/LLL	NE	33(33)	33(33)	33(33)	33(33)	C	09/84	33(33)
II.K.2.14 LIFT FREQ PORV &SV (F-31)	Y	Y	POST	RRAB	07(07)	7(07)	7(07)	7(07)	NA	B	06/82	7(07)
II.K.2.15 SLUG FLOW	Y	Y	POST	RSB	07(07)	7(07)	7(07)	7(07)	NA	B	01/81	7(07)
II.K.2.16 RCP SEAL DAMAGE (F-32)	Y	Y	POST	RSB/EGG	07(07)	7(07)	7(07)	7(07)	NA	B	12/81	7(07)

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## STATUS OF NUREG-0737 REQUIREMENTS SCHEDULE

Z-8

	DEVELOPMT COMPLTD	ISSUED TO OPERATG REACTORS	PRE/POST IMPLMNTN	TECHNL REVIEW ORGNTZN	ORDERS ISSUED	SUBMTTLS RECEIVED	TECHNICL REVIEW COMPLETE	SER ISSUED	TECHNL SPECS ISSUED	LICENSEE SCHEDULE CODE	NRC COMPLTN SCHEDULE	COMPLETE
II.K.2.17 VOIDING IN RCS (F-33)	Y	Y	POST	RSB	07(07)	40(40)	40(40)	40(40)	NA	C	09/84	40(40)
II.K.2.19A SEQ AFS FLOW, B&W (F-34)	Y	Y	POST	RSB	07(07)	7(07)	7(07)	7(07)	NA	C	02/81	7(07)
II.K.2.19B SEQ AFS FLOW, CE&W (F-34)	Y	Y	POST	RSB	NE	33(33)	33(33)	33(33)	NA	C	07/81	33(33)
II.K.2.20 SYS RESP TO SB LOCA (F-35)	Y	Y	POST	RSB	07(07)	7(07)	7(07)	7(07)	NA	B	TBD	7(07)
II.K.3.1 PORV ISOL (F-36)	Y	Y	POST	ICSB	NE	40(40)	40(40)	40(40)	NA	C	09/84	40(40)
II.K.3.2 RPT PORV FAILURES (F-37)	Y	Y	POST	RRAB/FRC	40(40)	40(40)	40(40)	40(40)	NA	C	09/84	40(40)
II.K.3.3 RPT SV & RV FAILURES (F-38)	Y	Y	POST	RRAB	65(65)	65(65)	65(65)	65(65)	28(65)	C	09/83	28(65)
II.K.3.5A RCP AUTO TRIP DESIGN (F-39)	Y	Y	POST	RSB	NE	40(40)	40(40)	40(40)	NA	C	09/82	40(40)
II.K.3.5B RCP AUTO TRIP MODIFY (F-39)	Y	Y	PRE	RSB	NE	40(40)	40(40)	40(40)	NA	C	09/82	40(40)
II.K.3.9 PID CONTROLLER (F-40)	Y	Y	POST	IE/DL	25(25)	25(25)	25(25)	25(25)	NA	B	09/81	25(25)



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II.K.3.10 ANTICIP TRIP (F-41)	Y	Y	PRE	ICSB	NE	25(25)	25(25)	25(25)	NA	B	06/82	25(25)
II.K.3.11 USE OF CERTAIN PORV	Y	Y	POST	RSB	NA	NA	65(65)	NA	NA	B	09/81	65(65)
II.K.3.12 ANTIC TRIP ON TT (F-42)	Y	Y	POST	ICSB	25(25)	25(25)	25(25)	25(25)	NA	C	09/81	25(25)
II.K.3.13A HPCI/RCIC INIT LEV DESIGN (F-43)	Y	Y	POST	RSB	17(17)	17(17)	17(17)	17(17)	NA	B	10/81	17(17)
II.K.3.13B HPCI/RCIC MODS (F-43)	Y	Y	POST	ICSB	NE	17(17)	17(17)	17(17)	5(17)	C	09/83	5(17)
II.K.3.14 ISO COND MODS (F-44)	Y	Y	POST	CSB	NE	7(07)	7(07)	7(07)	NA	C	12/81	7(07)
II.K.3.15 ISO OF HPCI/RCIC (F-45)	Y	Y	POST	RSB	19(19)	19(19)	19(19)	19(19)	17(19)	C	09/83	17(19)
II.K.3.16A RV FAIL REPORT (F-46)	Y	Y	POST	RRAB/FRC	24(24)	24(24)	24(24)	24(24)	NA	B	09/83	24(24)
II.K.3.16B RV FAIL MODS (F-46)	Y	Y	PRE	RSB/FRC	NE	24(24)	0(24)	0(24)	0(24)	D	09/83	0(24)

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II.K.3.17 ECCS OUTAGE REPORT (F-47)	Y	Y	POST	RRAB/FRC	65(65)	65(65)	65(65)	65(65)	NA	C	09/83	65(65)
II.K.3.18A ADS ACT STUDY (F-48)	Y	Y	POST	RSB	24(24)	24(24)	24(24)	24(24)	NA	B	04/82	24(24)
II.K.3.18B ADS ACT DESIGN (F-48)	Y	Y	POST	ICSB/RSB	24(24)	24(24)	24(24)	24(24)	24(24)	D	09/84	24(24)
II.K.3.18C ACS ACT MODS (F-48)	Y	Y	PRE	ICSB/RSB	NE	24(24)	11(24)	11(24)	2(24)	D	09/84	2(24)
II.K.3.19 INTERLOCK RECIRC (F-49)	Y	Y	POST	RSB	3(03)	3(03)	2(03)	2(03)	2(03)	C	09/83	2(03)
II.K.3.20 LOSS SVC WATER AT BRP	Y	Y	POST	ASB	NA	1(01)	1(01)	1(01)	NA	C	06/81	1(01)
II.K.3.21A RESTART LPCI/ CSS DESIGN (F-50)	Y	Y	POST	RSB/EGG	24(24)	24(24)	24(24)	24(24)	NA	B	09/82	24(24)
II.K.3.21B RESTART LPCI/ CSS MODS (F-50)	Y	Y	NE	ICSB/EGG	NE	24(24)	24(24)	24(24)	NA	C	09/83	24(24)
II.K.3.22A RCIC SUCT PROCEDURES (F-51)	Y	Y	POST	IE	17(17)	NA	17(17)	17(17)	NA	B	08/81	17(17)
II.K.3.22B RCIC SUCTION MODS (F-51)	Y	Y	POST	ICSB	17(17)	17(17)	13(17)	13(17)	3(17)	C	09/83	3(17)
II.K.3.24 SPACE COOLING FOR RCIC/HPCI (F-52)	Y	Y	POST	RSB	15(19)	19(19)	19(19)	19(19)	NA	C	09/83	19(19)

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II.K.3.25A PUMP SEAL STUDY (BWR) (F-53)	Y	Y	POST	RSB	NE	24(24)	24(24)	24(24)	NA	C	12/81	24(24)
II.K.3.25A PUMP SEAL STUDY (CE&W) (F-53)	Y	Y	POST	RSB	NE	33(33)	33(33)	33(33)	NA	C	06/82	33(33)
II.K.3.25B PUMP SEAL MODS (BWR) (F-53)	Y	Y	PRE	IE	NE	24(24)	24(24)	24(24)	NA	D	09/83	24(24)
II.K.3.25B PUMP SEAL MODS (CE&W) (F-53)	Y	Y	PRE	IE	NE	33(33)	28(33)	28(33)	NA	D	09/83	27(33)
II.K.3.27 COMMON REF LEVEL (F-54)	Y	Y	POST	ORB2		24(24)	24(24)	24(24)	24(24)	C	01/82	24(24)
II.K.3.28 QUAL OF ADS ACC (F-55)	Y	Y	POST	EQB	NE	21(21)	5(21)	5(21)	0(21)	C	09/83	0(21)
II.K.3.29 PERF OF ISO COND (F-56)	Y	Y	POST	RSB	07(07)	7(07)	7(07)	7(07)	NA	B	10/81	7(07)
II.K.3.30A SB LOCA SCHEDULE (F-57)	Y	Y	POST	RSB	65(65)	65(65)	65(65)	NA	NA	B	04/81	65(65)
II.K.3.30B SB LOCA METHOD (F-57)	Y	Y	PRE	RSB/EGG	0(65)	65(65)	24(65)	24(65)	NA	D	09/84	24(65)

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II.K.3.31 SB LOCA PUA (F-58)	Y	Y	PRE	RSB	0(65)	22(65)	22(65)	22(65)	22(65)	D	09/84	22(65)
II.K.3.44 TRANS W/SINGLE FAILLRE (F-59)	Y	Y	POST	RSB	24(24)	24(24)	22(24)	22(24)	NA	C	09/83	22(24)
II.K.3.45 MANUAL DEPRESS (F-60)	Y	Y	POST	RSB/EGG	24(24)	24(24)	24(24)	24(24)	NA	C	09/83	24(24)
II.K.3.46 MICHELSON CONCERNS (F-61)	Y	Y	POST	RSB	NE	24(24)	24(24)	24(24)	NA	B	08/81	24(24)
II.K.3.57 MAN ACT OF ADS (F-62)	Y	Y	POST	RSB	NE	24(24)	24(24)	NA	NA	B	TBD	24(24)
III.A.1.1 EMERG PLAN SHORT TERM	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
III.A.1.2.1 INTERIM FACILITIES	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
III.A.1.2.2 DESIGN UPGRD FACIL (F-63,64)	Y	Y	POST	EPLB/PNL	NE	0(65)	0(65)	0(65)	0(65)	E	09/84	0(65)
III.A.2.1 UPGRADE EMER PLAN (F-67)	Y	Y	POST	IE	NE	65(65)	62(65)	62(65)	NA	D	TBD	62(65)
III.A.2.2 METEOROLOGICAL DATA (F-68)	Y	Y	POST	IE	0(65)	0(65)	0(65)	0(65)	0(65)	E	09/84	0(65)
III.D.1.1 PRIMARY COOLANT LEAK	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	09/81	65(65)



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III D.3.3.1 I DETECT SHORT TERM	Y	Y	POST	DL	65(65)	65(65)	65(65)	65(65)	NA	A	05/80	65(65)
III D.3.3.2 I DETECT UPGRADE (F-69)	Y	Y	POST	IE/INEL	65(65)	65(65)	65(65)	65(65)	NA	B	03/82	65(65)
III D.3.4.1 CR HABIT REVIEW (F-70)	Y	Y	POST	AEB/PNL	65(65)	65(65)	65(65)	65(65)	NA	B	09/83	65(65)
III D.3.4.2 CR HABIT MODS (F-70)	Y	Y	POST	AEB/PNL	65(65)	65(65)	58(65)	58(65)	0(65)	D	09/83	0(65)

LICENSING ACTION TRACKING SYSTEM  
NRR IMPLEMENTATION REVIEW  
COMPLETION STATUS

LICENSEE SCHEDULE CODE	NUMBER OF ACTIONS	SUBMITTALS RECEIVED		SER ISSUED		TECH SPEC ISSUED		ACTIONS COMPLETED
		# ACTIONS	# COMPLETED	# ACTIONS	# COMPLETED	# ACTIONS	# COMPLETED	
A	1,240	1,240	1,240	1,240	1,240	395	395	1,240
B	1,430	893	893	821	821	72	72	1,430
C	1,933	1,803	1,803	1,804	1,711	863	263	1,843
D	948	948	905	818	530	397	137	660
E	455	455	1	455	0	195	0	0
TOTALS	6,006	5,339	4,842	5,138	4,302	1,922	867	5,173

LICENSEE SCHEDULE CODE KEYS:  
 A- STILL  
 B- THROUGH 6/30/81 EXCLUDING STILL  
 C- 6/30/81 TO 03/01/82  
 D- AFTER 03/01/82  
 E- SUPPLEMENT 1

SECTION V  
CONDENSED MANAGEMENT REPORT  
OF  
LICENSING ACTIONS BY POWER REACTOR

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: ARKANSAS 1

PLANT LOCATION: 6 MI WNW OF RUSSELLVILLE, AR  
 DOCKET NUMBER: 050-00313  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: W. JOHNSON

LICENSED POWER: 2568 MWT  
 DESIGN POWER: 0850 MWE  
 NSSS VENDOR: B&W

PROJECT MANAGER: G. VISSING  
 BRANCH CHIEF: J. STOLZ  
 LIC. ASSISTANT: R. INGRAM

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
07862	AND 1 - PRESSURE VESSEL SUPPORT (ASYMMETRICAL LOADS)		2	<u>COMPLETE</u> 02/14/84
53165	ARKANSAS 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			05/14/84
<u>ACTIVE ACTIONS</u>				
52735	ARKANSAS 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52816	ARKANSAS 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52897	ARKANSAS 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52978	ARKANSAS 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53059	ARKANSAS 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53112	ARKANSAS 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
52206	ARKANSAS 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
11236	AND 1 - REVISED IST - TESTING		1	04/30/84
43632	AND 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	06/28/84
08105	AND 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/30/84
07971	ARKANSAS 1 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	07/30/84
11266	AND 1 - MECHANICAL SNUBBERS		2	09/30/84
11267	AND 1 - HYDRAULIC SNUBBERS		3	09/30/84
42514	AND 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/30/84
47181	ARKANSAS 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	09/30/84
51069	ARKANSAS 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
42634	ARKANSAS 1 - NUREG-0737, II.E.1.1, AFW SYSTEM EVALUATION		1	<u>COMPLETE</u> 10/06/83
44213	AND 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
43425	AND 1 - ITEM II K.2.13 THERMAL-MECH EFFECTS OF HPI ON VESSEL		1	06/07/84
<u>ACTIVE ACTION:</u>				
45112	AND 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/83
44419	AND 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
44560	ARKANSAS 1 - RV AND SV TESTING		1	06/30/84
45803	AND 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
45923	AND 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
45994	AND 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46066	AND 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46282	AND 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
51219	ARKANSAS 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
42842	ARKANSAS 1 - NUREG-0737, I.C.1.3, ABNORMAL TRANSIENT OPER GUIDELINES & PROCEDURES		1	11/30/84
48144	ARKANSAS 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85
51359	ARKANSAS 1 - I.D.1 - CONTROL ROOM DESIGN REVIEW			11/30/85



DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ARKANSAS 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
51010	ARKANSAS NUCLEAR ONE, UNIT 1, APPENDIX R SCHEDULAR EXEMPT REQUEST FOR ONE HOUR FIRE BARRIERS			10/02/83
52091	ANO-1 TS CHANGE CONCERNING ANO& RVSP CAPSULE SCHEDULE			10/08/83
51926	ANO-1 SALP REVIEW JUL 1, 1982 TO JUNE 30, 1983			10/14/83
52162	ANO-1 - EVALUATION OF BACKUP TSC HABITABILITY			10/24/83
51575	ANO-1 - ELECT 1 AND C AND TSS RELATED TO OVER PRES. PROT.			11/07/83
51736	ANO 1: STATION DIST. VOLTAGE VERIF. TEST			12/19/83
41013	ANO 1 - IREP			12/23/83
51928	ANO-1/SOURCE RANGE NEUTRON FLUX MONITORING INSTRUMENTATION		1	01/06/84
47553	ANO-1 - REVIEW 5 YR TENDON SURVEILLANCE REPORT			01/16/84
51779	ANO-1 SHUNT TRIP DEVICE DESIGN FOR CRDTB			01/24/84
49807	ANO-1 TECH SPEC ADMINISTRATIVE CHANGES			02/01/84
53235	ANO-1 INTERPRETATION AT TECHNICAL SPECIFICATIONS			02/28/84
53425	ANO-1 EXEMPTION REQUEST FOR A FULL SCALE EMERGENCY EXERCISE			03/22/84
54272	ANA0-1 PROJECT MANAGER'S BRIEFING BOOK			03/30/84
54224	ANO-1, COMMISSION PAPER CONCERNING EOF			04/24/84
54286	ANO-1 - STAFF REQUEST FOR T.S. CONCERNING ELECT. DIST.			05/09/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
51353	ARKANSAS NUCLEAR 1 UNIT 1 - APPENDIX R SCHEDULAR EXEMPTION REQUEST RELATED TO SUPPRESSION/DETECTION SYS/FIRE BARRIERS			05/30/84
54744	ANO - LICENSEE'S 1 REP RESPONSE			06/15/84
54348	ANO-1 P/T HEAT UP & COOLDOWN LIMITATIONS			06/30/84
53530	ANO-1 ADMIN. CHANGE-CORRECT ERRORS			06/30/84
53259	ANO-1 REVIEW EFW ISSUE OPEN ITEMS GL-9 & GL-2			06/30/84
52382	ANO-1 - LEAK TESTING OF SEALED RADIOACTIVE SOURCES			06/30/84
48030	ANG-1 REACTOR COOLANT PUMP SEAL DAMAGE		3	06/30/84
49722	- ARKANSAS 1 NUREG 0737 TECH SPECS (GENERIC LTR 82-16)			06/30/84
12374	ANO 1 - REVISE ENVIRONMENTAL (T.S. APPENDIX B)		3	06/30/84
43595	ANO 1 - T.S. FOR OPERABILITY OF PORV		1	06/30/84
54483	ANO-1 TECH. SPEC. FOR OVERPRESSURE PROTECTION			07/20/84
55099	ANO-1, REVIEW REQUEST RELATED TO OPERATOR LICENSING EXAMINATIONS			08/30/84
54311	ANO-1 - IST RELIEF FOR CF-1A, CF-1B, BS-4A, BS-4B			09/05/84
53310	ANO-1 - BOUNDING ANALYSIS OF NUREG 0630 MODEL WITH FLECSSET			09/30/84
54514	ARKANSAS 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54516	BEAVER VALLEY 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54380	ARKANSAS 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
49826	ARKANSAS NUCLEAR 1- SMALL BREAK LOCA PROCED & MAINTAINING PROPER SG LEVEL			09/30/84
43164	ANO 1 - WASTE GAS H/O LIMITS T.S. CHANGE		1	09/30/84
55304	ANO 1 - SALP REPORT FOR PERIOD 7-1-83 TO 6-30-84			10/30/84
51584	ANO-1, TS CHANGES RELATED TO KEY LOCK SWITCHES FOR PURGE VALVES			12/30/84
49699	ARKANSAS 1 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53567	ARKANSAS 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53650	ARKANSAS 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53733	ARKANSAS 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53816	ARKANSAS 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53899	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ARKANSAS 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				<u>INITIATION</u>
53952	ARKANSAS 1 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
53960	ARKANSAS 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54043	ARKANSAS 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: ARKANSAS 2

PLANT LOCATION: 6 MI WNW OF RUSSELLVILLE, AR  
 DOCKET NUMBER: 650-00368  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: L. CALLAN

LICENSED POWER: 2815 MWT  
 DESIGN POWER: 0912 MWE  
 NSSS VENDOR: COMB

PROJECT MANAGER: R. LEE  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47161	ARKANSAS 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 03/27/84
43633	ANO 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		3	06/26/84
<u>ACTIVE ACTIONS</u>				
51070	ARKANSAS 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		3	<u>TARGET</u> 06/30/84
49723	ARKANSAS 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			07/30/84
42496	ANO 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/30/84
08159	ANO 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	07/30/84
12069	ANO 2 - SURVEILLANCE FOR HYDRAULIC SNUBBERS (B-17)		3	08/30/84
12070	ANO 2 - T.S. SURVEILLANCE MECHANICAL SNUBBERS (B-22) (B-22)		3	08/30/84
12542	ANO 2 - MISC. CHANGES TO FIRE PROTECTION T.S.		3	08/30/84
52736	ARKANSAS 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		1	11/01/84
52817	ARKANSAS 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		1	11/01/84
52898	ARKANSAS 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		1	11/01/84
52979	ARKANSAS 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		1	11/01/84
53060	ARKANSAS 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		1	11/01/84
53113	ARKANSAS 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		1	11/01/84
12946	ARKANSAS 2 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	12/30/84
52207	ARKANSAS 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47791	ARKANSAS 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 10/18/83
44660	ANO 2 - NUREG-0737 II.E.1.1, EFW SYSTEM EVALUATION EVALUATION		1	11/18/83
46877	ANO 2 - II K2.13 THERMAL MECHANICAL REPORT		1	06/05/84
44211	ANO 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
44420	ANO 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>TARGET</u> 07/30/84
44561	ARKANSAS 2 - RV AND SV TESTING		1	09/30/84
45924	ANO 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
45995	ANO 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46067	ANO 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
51140	ARKANSAS 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/84
51220	ARKANSAS 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
45804	ANO 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/30/84
45113	ANO 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/30/85
44281	ANO 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	12/30/85
48145	ARKANSAS 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		3	12/30/85
46283	ANO 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/86

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ARKANSAS 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51009	ARKANSAS NUCLEAR ONE, APPENDIX R SCHEDULAR EXEMPT REQUEST FOR ONE HOUR FIRE BARRIERS (UNIT 2)			<u>COMPLETE</u>
51927	ANO-2 SALP REVIEW, JULY 1, 1982 TO JUNE 30, 1983			10/07/83
52161	ANO-2 - EVALUATION OF BACKUP TSC HABITABILITY			10/20/83
49447	ANO-2, APPENDIX R SCHEDULAR EXEMPTION REQUEST, FIREZONE 2100Z AND 2101AA SWITCHGEAR MOD			10/24/83
52104	ANO-2 T.S. CHANGE REQUEST - FUEL ENRICHMENT			11/01/83
51417	ANO-2 - TECH SPEC CHANGES RELATED TO INCORE DETECTORS SURVEILLANCE REQUIREMENTS & ADMINISTRATIVE CORRECTIONS			11/07/83
52356	ANO-2 - EQ DEADLINE EXTENSION			11/10/83
49502	ANO-2, CPC SOFTWARE ERROR			11/16/83
52065	ANO-2 T.S. CHANGE REQUEST - SURVEILLANCE OF EFW PUMP			11/28/83
52066	ANO-2 T.S. CHANGE REQUEST - EFW FLOW PATH VERIFICATION			12/13/83
51929	ANO-2/SOURCE RANGE NEUTRON FLUX MONITORING INSTRUMENTATION			12/13/83
53309	ANO-2 - APP. R - SCHEDULAR EXEMPTION REQUEST PERTAINING TO SOURCE RANGE MONITORING CAPABILITY			01/06/84
49808	ANO-2 TECH SPEC ADMINISTRATIVE CHANGES			01/23/84
48918	ANO 2 - FUEL SHOULDER GAP DECREASE			02/01/84
12081	ANO 2 - VERIFICATION OF TRANSIENT ANALYSIS CODE CESEC CESEC			02/24/84
53426	ANO-2 EXEMPTION REQUEST FOR A FULL SCALE EMERGENCY EXERCISE		1	03/20/84
52064	ANO-2 T.S. CHANGE REQUEST - INCORE DETECTION SYSTEM		1	03/22/84
53350	ANO-2, TECH SPEC CHANGE REQUEST - ELECTRICAL POWER SYSTEMS			03/26/84
12466	ANO 2 - T.S. CHANGE 2 SEISMIC INSTRUMENTATION			03/27/84
54225	ANO-2, COMMISSION PAPER CONCERNING EOF		2	04/13/84
54223	ANO-2, PM BRIEFING BOOK			04/25/84
51352	ARKANSAS NUCLEAR 1 UNIT 2 - APPENDIX SCHEDULAR EXEMPTION REQUEST RELATED TO SUPPRESSION/DETECTION SYS/FIRE BARRIERS		3	05/30/84
49320	ARKANSAS NUCLEAR ONE - UNIT 2 - ESFAS DESIGN DISCREPANCY			06/05/84
<u>ACTIVE ACTIONS</u>				
43165	ANO 2 - WASTE GAS H/O LIMITS T.S. CHANGE		2	<u>TARGET</u>
52383	ANO-2 - LEAK TESTING OF SEALED RADIOACTIVE SOURCES		3	06/15/84
52635	ANO-2, TECH SPEC CHANGE REQUESTS FOR REMOTE SHUTDOWN PANEL INSTRUMENTATION AND H-H ANALYZER CALIB.		2	06/30/84
52636	ANO-2, LICENSE AMENDMENT FOR IAEA SAFEGUARD INSPECTION PROGRAM			07/30/84
54761	ANO-2, TSCR PERTAINING TO DIESEL GENERATOR SURVEILLANCE REQUIREMENTS			07/30/84
12083	ANO 2 - FIRE PROTECTION-LICENSE CONDITION RESOLUTION		3	07/30/84
55100	ANO-2 REVIEW REQUEST RELATED TO OPERATOR LICENSING EXAMINATIONS			08/30/84
54381	ARKANSAS 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54515	ARKANSAS 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
51585	ANO-2, TS CHANGES RELATED TO KEY LOCK SWITCHES FOR PURGE VALVES		3	09/30/84
12062	ANO 2 - INSERVICE TESTING PROGRAM (A-14)		3	12/30/84
49676	ARKANSAS 2 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				
53651	ARKANSAS 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			<u>INITIATION</u>
53734	ARKANSAS 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53817	ARKANSAS 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53900	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53961	ARKANSAS 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84



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TECHNICAL ASSIGNMENT CONTROL SYSTEM

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ARKANSAS 2

TAC #    TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

54044    ARKANSAS 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES  
53568    ARKANSAS 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY

INITIATION

11/01/84

11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: BEAVER VALLEY 1

PLANT LOCATION: 5 MI E OF E. LIVERPOOL, OH  
 DOCKET NUMBER: 050-00334  
 ARCH/ENGINEER: S&W  
 IE INSPECTOR: W. TROSKOSKI

LICENSED POWER: 2660 MWT  
 DESIGN POWER: 0835 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: P. TAM  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49724	BEAVER VALLEY 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"			<u>COMPLETE</u> 10/21/83
12263	BEAVER VALLEY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE BEAVER VALLEY 1		3	01/31/84
49417	BEAVER VALLEY 1 - BLOCKING SI SIGNAL DURING COOLDOWN		2	02/09/84
<u>ACTIVE ACTIONS</u>				
52737	BEAVER VALLEY 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		1	<u>TARGET</u>
52818	BEAVER VALLEY 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		1	
52899	BEAVER VALLEY 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		1	
52980	BEAVER VALLEY 1 - ITEM 3.1.3 - POST-MAINT TESTING - CHNGS TO TECH SPECS - RTS COMPONENTS		1	
53061	BEAVER VALLEY 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		1	
53114	BEAVER VALLEY 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		1	
53166	BEAVER VALLEY 1 - ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	
10386	BEAVER VALLEY 1 - N-1 LOOP OPERATION		2	06/30/84
42459	BEAVER VALLEY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLT-21)		1	09/30/84
07972	BEAVER VALLEY 1 - CNTRL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	10/30/84
08597	BEAVER VALLEY 1 - REVIEW OF ASYMMETRIC LOCA LOADS		2	12/31/84
52208	BEAVER VALLEY 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			12/31/84
51071	BEAVER VALLEY 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	03/31/85
TAC #	TAC DESCRIPTION	IMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47792	BEAVER VALLEY 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 12/19/83
44212	BEAVER VALLEY 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
46878	BEAVER VALLEY 1 II K2.13 THERMAL MECHANICAL REPORT		1	06/15/84
<u>ACTIVE ACTIONS</u>				
44421	BEAVER VALLEY 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>TARGET</u> 05/31/84
45805	BEAVER VALLEY 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	07/31/84
45925	BEAVER VALLEY 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
45996	BEAVER VALLEY 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46068	BEAVER VALLEY 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44282	BEAVER VALLEY 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	09/30/84
51141	BEAVER VALLEY 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	09/30/84
51221	BEAVER VALLEY 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	09/30/84
45114	BEAVER VALLEY 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/30/84
46284	BEAVER VALLEY 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	12/31/84
44562	BEAVER VALLEY 1 - RV AND SV TESTING		1	06/30/85
48146	BEAVER VALLEY 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BEAVER VALLEY 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
49436	BEAVER VALLEY 1 - REVIEW OF REPORT ON THIMBLE DELETION FOR TS CHANGE			10/21/83
51048	BEAVER VALLEY UNIT 1 - SURVEILLANCE REQUIREMENTS FOR HEPA FILTERS AND CHARCOAL ADSORBER UNITS			10/27/83
51818	BEAVER VALLEY UNIT 1 - TECH SPEC FOR OVERPRESSURE PROTECTION SYSTEM			10/31/83
52364	BEAVER VALLEY UNIT 1 - ROD BANK WORTH MEASUREMENT WCAP-9863		2	11/04/83
52554	BEAVER VALLEY UNIT - REVISED CYCLE 4 RELOAD REPORT			11/05/83
52068	BEAVER VALLEY UNIT 1 - ADMINISTRATIVE CHANGES, CHANGE REQUEST NO. 86		3	12/13/83
52070	BEAVER VALLEY UNIT 1 - ONSITE POWER DISTRIBUTION SYSTEMS		2	12/13/83
51577	BEAVER VALLEY UNIT 1 - RHR HEAT EXCHANGER ISI RELIEF		2	12/15/83
52184	BEAVER VALLEY UNIT 1 - RCS VENT SYSTEM SCHEDULAR EXEMPTION		1	01/12/84
52051	BEAVER VALLEY UNIT 1 - DELAY IN INSTALLATION OF NEW DIESEL FUEL TANK		2	01/31/84
53542	BEAVER VALLEY UNIT 1 - PRIVACY INFORMATION IN EMERGENCY PLAN		3	02/14/84
52183	BEAVER VALLEY UNIT 1 - MAINE FEEDWATER SYSTEM SNUBBERS			02/27/84
53415	BEAVER VALLEY UNIT 1 - BRIEFING BOOK FOR EMERGENCIES		2	02/28/84
54219	BEAVER VALLEY UNIT 1 - REVISION TO APP. R SER (USE OF CETS VS. T-HOT)		2	03/22/84
54358	BEAVER VALLEY UNIT 1 - LICENSEE COMMENTS ON NUREG-1021		3	03/26/84
52555	BEAVER VALLEY UNIT - DOSE PROJECTION USING COMPUTER CODE IRDAM		1	04/09/84
53472	BEAVER VALLEY UNIT 1 - INFRARED AERIAL PHOTOS		2	04/11/84
53207	BEAVER VALLEY UNIT 1 - ELIMINATION OF APP. B. TS RADIOLOGICAL		1	04/17/84
53315	BEAVER VALLEY UNIT 1 - 1983 CONTAINMENT LEAK RATE TEST		3	05/07/84
54258	BEAVER VALLEY UNIT 1 - EXTENSION OF ISI INTERVAL		2	05/21/84
52530	BEAVER VALLEY UNIT 1 - REANALYSIS OF LOCA DOSES		3	05/31/84
52531	BEAVER VALLEY 1 - REVISION TO REACTOR VESSEL ISI		2	06/04/84
53291	BEAVER VALLEY UNIT 1 - ELIMINATION OF MECHANICAL VACUUM PUMP TECH. SPECS.		2	06/19/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54841	BEAVER VALLEY UNIT 1 - REORGANIZATION OF OFFSITE ORGANIZATION		2	
55235	BEAVER VALLEY UNIT 1 - TECHNICAL SPECIFICATION CHANGE DUE TO NEW LOCA AND FUEL HANDLING ANALYSIS			
54198	BEAVER VALLEY 1 - RIVER WATER SYSTEM CLASS 3 PIPING SUPPORT - ISI			04/30/84
52071	BEAVER VALLEY UNIT 1 - BYPASSES TO BLOCK TRIP SYSTEM INSTRUMENTS AND ESF CHANNELS		2	05/30/84
52027	BEAVER VALLEY UNIT 1 - NOZZLE THERMAL STRESSES DUE TO INADVERTENT SI ACTUATION		2	06/30/84
51513	BEAVER VALLEY UNIT 1 - REVISED INSERVICE INSPECTION PROGRAM		1	06/30/84
53225	BEAVER VALLEY UNIT 1 - SALP REPORT FOR 1983			06/30/84
53380	BEAVER VALLEY UNIT 1 - ADDITIONAL FIRE PROTECTION EXEMPTION REQUESTS		1	06/30/84
55245	BEAVER VALLEY UNIT 1 - TECHNICAL SPECIFICATION FOR STATION BATTERY SURVEILLANCE			07/31/84
55010	BEAVER VALLEY UNIT 1 - TS CHANGES TO CONFORM WITH NEW 50.72 AND 50.73		2	07/31/84
52691	BEAVER VALLEY UNIT 1 - CLASS 3 PIPE SUPPORT RELIEF REQUEST		2	07/31/84
52069	BEAVER VALLEY UNIT 1 - CONTAINMENT AIR LOCKS		2	07/31/84
49437	BEAVER VALLEY 1 - RESPONSE TO IST SER		2	07/31/84
55244	BEAVER VALLEY UNIT 1 - TECHNICAL SPECIFICATION FOR CONTAINMENT EMERGENCY AIR LOCK			08/30/84
53208	BEAVER VALLEY UNIT 1 - ELIMINATION OF APPENDIX B TS, NON-RADIOLOGICAL		2	08/31/84
55101	BEAVER VALLEY UNIT 1 - TECH SPEC CHANGE FOR RADIATION PROCESS MONITORS		2	09/15/84
55102	BEAVER VALLEY UNIT 1 - TECH SPEC CHANGE FOR ERROR CORRECTION, CLARIFICATION ETC.		2	09/15/84
55103	BEAVER VALLEY UNIT 1 - TECH SPEC CHANGE FOR REACTOR LEAKAGE DETECTION SYSTEM		2	09/15/84
55104	BEAVER VALLEY UNIT 1 - TECH SPEC CHANGE FOR VESSEL IRRADIATION SURVEILLANCE		2	09/15/84
55105	BEAVER VALLEY UNIT 1 - ASSOPTED FIRE PROTECTION TECH SPEC CHANGE		2	09/15/84
55106	BEAVER VALLEY UNIT 1 - TECH SPEC CHANGE FOR FACILITY ORGANIZATION		2	09/30/84
55024	BEAVER VALLEY UNIT 1 - TECH. SPEC. CHANGES FOR N-1 LOOP OPERATION		2	09/30/84
54382	BEAVER VALLEY 1 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		2	09/30/84
49695	BEAVER VALLEY 1 - REACTOR COOLANT PUMP TRIP			02/01/85

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BEAVER VALLEY 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
54627	BEAVER VALLEY UNIT 1 - SALP FOR 4/84 TO 10/85		3	<u>TARGET</u> 12/31/85
<u>ANTICIPATED ACTIONS</u>				
53569	BEAVER VALLEY 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53652	BEAVER VALLEY 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53735	BEAVER VALLEY 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53818	BEAVER VALLEY 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53901	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53962	BEAVER VALLEY 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES 2 TEST INTERVALS			11/01/84
54045	BEAVER VALLEY 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: BIG ROCK POINT 1

PLANT LOCATION: 4 MI NE OF CHARLEVOIX, MICH  
 DOCKET NUMBER: 050-00155  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: G. WRIGHT

LICENSED POWER: 0240 MWT  
 DESIGN POWER: 0072 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. EMCH  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08441	BIG ROCK POINT - MECHANICAL SNUBBERS		2	<u>COMPLETE</u> 01/11/84
51668	BIG ROCK PT 1 - NUREG 0737 TECH SPECS			02/10/84
46646	BIG ROCK PT 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL PLANTS (NUREG 0612)		2	05/09/84
46651	BIG ROCK POINT - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/26/84
<u>ACTIVE ACTIONS</u>				
52738	BIG ROCK POINT 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52819	BIG ROCK POINT 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52900	BIG ROCK POINT 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52981	BIG ROCK PT 1-ITEM 3.1.3-POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
42907	BIG ROCK POINT - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
42515	BIG ROCK POINT - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-C'I-I-21)		1	08/30/84
08096	BIG ROCK POINT - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	09/01/84
43044	BIG ROCK POINT - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	09/30/84
51072	BIG ROCK POINT 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	09/30/84
52009	BIG ROCK POINT 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
48147	BIG ROCK POINT -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>COMPLETE</u> 12/27/83
45806	BIG ROCK POINT - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/27/83
46427	BIG ROCK POINT - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	12/30/83
45875	BIG ROCK POINT - NUREG-0737 II.K.3.44, TRANSIENTS WITH SINGLE FAILURES		1	03/15/84
47581	BIG ROCK POINT - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		1	04/16/84
47652	BIG ROCK POINT - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	04/16/84
44213	BIG ROCK POINT - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
44283	BIG ROCK POINT - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	<u>TARGET</u> 06/30/84
44422	BIG ROCK POINT - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
47723	BIG ROCK POINT - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	06/30/84
47920	BIG ROCK POINT - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	06/30/84
45115	BIG ROCK POINT - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
45926	BIG ROCK POINT - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
45997	BIG ROCK POINT - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46069	BIG ROCK POINT - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46285	BIG ROCK POINT - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
51142	BIG ROCK POINT 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	09/30/84
51222	BIG ROCK POINT 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	09/30/84
44563	BIG ROCK POINT 1 - RV AND SV TESTING		1	09/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## BIG ROCK POINT 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
52340	BIG ROCK POINT - FOR G13 & G14 FUEL MAPLHGR			<u>COMPLETE</u> 11/14/83
49590	BIG ROCK POINT - CONTAINMENT LEAK TEST T.S.		2	12/27/83
52520	BIG ROCK POINT - EXEMPTION REQUEST - 20.54(M) (2) & (3)		1	12/30/83
48708	BIG ROCK POINT - SAFETY MANAGEMENT FUNCTION, TS CHG		2	01/04/84
49304	BIG ROCK POINT - REVISION OF ADMINISTRATIVE CONTROLS T.S.		2	01/04/84
51976	BIG ROCK POINT - REVISION TO GUARD TRAINING PLAN			02/17/84
54251	BIG ROCK POINT - II.F.1(1) HIGH RANGE STACK GAS MONITOR DEVIATION			03/07/84
52657	BIG ROCK POINT - EXEMPTION TO 10CFR, APPR, II.G.3		2	03/20/84
52658	BIG ROCK POINT - REACTOR VESSEL PRESSURE TEMPERATURE LIMITS T.S.		1	04/12/84
51495	BIG ROCK POINT - SECURITY PLAN CHANGE		2	04/18/84
54763	BIG ROCK POINT - DEVIATION TO II.F.1(1) STACK GAS MONITOR			04/20/84
53356	BIG ROCK POINT - REACTOR COOLANT IODINE T.S.			04/30/84
12979	BIG ROCK POINT - RISK ASSESSMENT		1	05/17/84
49298	BIG ROCK POINT - MCPR T.S.		1	05/18/84
54250	BIG ROCK POINT - STATE EXERCISE PARTICIPATION EXEMPTION			05/23/84
<u>ACTIVE ACTIONS</u>				
52012	BIG ROCK POINT - CONTROL ROD WITHDRAWAL RATE LIMIT T.S.			<u>TARGET</u>
55045	BIG ROCK POINT - INCORPORATION OF BYPRODUCT LICENSE			
55076	BIG ROCK POINT - VALVE RELIABILITY			
55077	BIG ROCK POINT - BOP QA			
55074	BIG ROCK POINT - INCORE DETECTORS			
52355	BIG ROCK POINT - SCRAM DISCHARGE VOLUME LEVEL INSTRUMENTATION			
52485	BIG ROCK POINT - REQUEST FOR EXEMPTION TO 10 CFR 50.44 (C)(3)(III) - HIGH POINT VENTS		1	06/30/84
51988	BIG ROCK POINT - TRANSFER CASK TRIP TEST T.S.			06/30/84
51900	BIG ROCK POINT - VENT VALVE SEISMIC QUAL. & AIR CYLINDER			06/30/84
49499	BIG ROCK POINT - DEBRIS SCREENS, VENT VALVE TESTS, VALVE PERCENT TIME OPEN		2	06/30/84
08277	BIG ROCK POINT - ATWS RECIRCULATION PUMP TRIP		1	06/30/84
55068	BIG ROCK POINT - STRUCTURAL CAPABILITY FOR SEVERE WEATHER			07/24/84
53379	BIG ROCK POINT - ISI RELIEF REQUESTS (REV. 3)			07/30/84
51974	BIG ROCK POINT - CONTAINMENT PRESSURE & WATER LEVEL LEVEL MONITOR T.S.			08/30/84
51975	BIG ROCK POINT - STACK GAS MONITORING SYSTEM T.S.			08/30/84
55073	BIG ROCK POINT - SCRAM DUMP TANK VALVES			08/31/84
49589	BIG ROCK POINT - TEN YEAR IST PUMP & VALVE PROGRAM		2	09/01/84
54517	BIG ROCK POINT 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
55069	BIG ROCK POINT - STRUCTURAL DESIGN AND SEISMIC LOADS			07/31/85
55067	BIG ROCK POINT - ALTERNATE SHUTDOWN INSTALLATION & PROCEDURES			01/31/86
55075	BIG ROCK POINT - FIRE PROTECTION MODS			01/31/86
55072	BIG ROCK POINT - STEAM STABILITY			02/28/86
55078	BIG ROCK POINT - UPDATED DESIGN DATA			06/30/86
55071	BIG ROCK POINT - RDS MODIFICATIONS			06/31/86
55070	BIG ROCK POINT - CONTAINMENT ISOLATION			03/31/89
<u>ANTICIPATED ACTIONS</u>				
53570	BIG ROCK POINT 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53653	BIG ROCK POINT 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53736	BIG ROCK PT 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53819	BIG ROCK POINT 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BIG ROCK POINT 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53963	BIG ROCK POINT 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			<u>INITIATION</u> 11/01/84
54046	BIG ROCK POINT 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: BROWNS FERRY 1

PLANT LOCATION: 10 MI NW OF DECATUR, ALA  
 DOCKET NUMBER: 050-00259  
 ARCH/ENGINEER: TVA  
 IE INSPECTOR: J. PAULK

LICENSED POWER: 3293 MW  
 DESIGN POWER: 1065 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. CLARK  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
48134	BROWNS FERRY 1 - FIRE PROTECTION APPENDIX R		3	<u>COMPLETE</u> 10/12/83
12295	BROWN'S FERRY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE		3	02/01/84
07974	BROWNS FERRY 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/06/84
08435	BROWN FERRY 1 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING		1	06/06/84
46652	BROWNS FERRY 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/15/84
52392	BROWNS FERRY 1 - APPENDIX R EXEMPTION REQUESTS			06/18/84
42873	BROWNS FERRY 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/27/84
<u>ACTIVE ACTIONS</u>				
52192	BROWNS FERRY 1 - RPS MODIFICATION			<u>TARGET</u>
52210	BROWNS FERRY 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
51073	BROWNS FERRY 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52739	BROWNS FERRY 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52820	BROWNS FERRY 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52901	BROWNS FERRY 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52982	BROWNS FERRY 1-ITEM 3.1.3-POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
08715	BROWNS FERRY 1 - CONTAINMENT LEAK TESTING - APP. J (GENERIC)		2	09/01/83
42568	BROWNS FERRY 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	09/30/83
11324	BROWNS FERRY 1 - INSERVICE TESTING		1	11/11/83
48131	BROWNS FERRY 1 - SINGLE LOOP OPERATION		3	11/21/83
51669	BROWNS FERRY 1 NUREG 0737 TECH SPECS			03/01/84
08133	BROWNS FERRY 1 - APPENDIX I T.S. IMPLEMENTATION REVIEW REVIEW		3	03/16/84
43727	BROWNS FERRY 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	03/30/84
07929	BROWNS FERRY 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	06/30/84
08929	BROWNS FERRY 1 - RPS POWER SUPPLY		1	08/10/84
42481	BROWNS FERRY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/07/84
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
44143	BROWNS FERRY 1 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	<u>COMPLETE</u> 10/12/83
44493	BROWNS FERRY 1 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	10/12/83
45807	BROWNS FERRY 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
48148	BROWNS FERRY 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/28/83
44072	BROWNS FERRY 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			02/28/84
44214	BROWNS FERRY 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
46286	BROWNS FERRY 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u>
51143	BROWNS FERRY 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51223	BROWNS FERRY 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45680	BROWNS FERRY 1 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	09/30/83
45584	BROWNS FERRY 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	11/15/83
44832	BROWNS FERRY 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	11/30/83



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
44284	BROWNS FERRY 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83
44423	BROWNS FERRY 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	12/31/83
48260	BROWNS FERRY 1 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS		1	04/30/84
45116	BROWNS FERRY 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	07/30/84
47849	BROWNS FERRY 1 - NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS		3	08/31/84
45927	BROWNS FERRY 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
45998	BROWNS FERRY 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46070	BROWNS FERRY 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
<u>TAC # TAC DESCRIPTION</u>				<u>PRIORITY</u>
<u>PLANT SPECIFIC</u>				<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
52193	BROWNS FERRY 1 - CORE RELOAD (AMEND 91)			12/01/83
52187	BROWNS FERRY 1 - TORUS MODIFICATIONS (AMEND 92)			12/12/83
52188	BROWNS FERRY 1 - MODIFICATION TO DRYWELL AIR SUPPLY			12/12/83
52189	BROWNS FERRY 1 - MODIFICATION TO PERSONNEL AIRLOCK DOORS			12/12/83
52190	BROWNS FERRY 1 - SCRAM DISCHARGE INSTRUMENT VOLUME MODIFICATION (AMEND 92)			12/12/83
52191	BROWNS FERRY 1 - ANALOG TRIP SYSTEM (AMEND 93)			12/16/83
51749	BROWNS FERRY 1 - AUGMENTED LEAK DETECTION REQUIREMENTS (AMEND 94)			12/27/83
52401	BROWNS FERRY 1 - AUDIT OF SAFEGUARDS PLAN (AMEND 95)			04/23/84
51752	BROWNS FERRY 1 - GROUP 7 ISOLATION VALVES			04/24/84
52579	BROWNS FERRY 1 - TURBINE CONTROL VALVE FAST CLOSURE (AMEND 97)			04/25/84
47455	BROWNS FERRY 1 - REVISED ISI PROGRAM (AMEND 98)		3	04/30/84
48479	BROWNS FERRY 1 - ORGANIZATIONAL CHANGES			05/23/84
53238	BROWNS FERRY 1 - GAP CONDUCTANCE CALCULATIONS			05/23/84
53353	BROWNS FERRY 1 - REPORTING REQUIREMENTS OF 10 CFR 50.73 (AMEND 66)			05/30/84
52505	BROWNS FERRY 1 - REPORTING ACTUATIONS OF SRVS (AMEND 99)			05/30/84
43698	BROWNS FERRY 1 - CAD SYSTEM VALVES		1	05/30/84
52576	BROWNS FERRY 1 - ROD BLOCK INSTRUMENTATION			06/06/84
51859	BROWNS FERRY 1 - VERIFICATION TEST OF STATION DISTRIBUTION VOLTAGES			06/19/84
52699	BROWNS FERRY 2 - CORE RELOADS USING SRMS			06/25/84
52341	BROWNS FERRY 1 - DEPTH OF WATER OVER FUEL			06/26/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52344	BROWNS FERRY 1 - SRO FOR FUEL HANDLING			
52441	BROWNS FERRY 1 - NDT PVS T CURVES THRU 6 EFPY			
52444	BROWNS FERRY 1 - LIFE OF PLANT NDT CURVES			
52455	BROWNS FERRY 1 - AUDIT OF SECURITY PLAN/QA REVIEWS			
41011	BROWNS FERRY 1 - IREP		1	
52582	BROWNS FERRY 1 - ISOLATION DAMPERS, CONTROL ROOM			
52508	BROWNS FERRY 1 - SUPPRESSION CHAMBER DRAIN VALVES			
53496	BROWNS FERRY 1 - RESPONSE ON II.K.3.22			
54167	BROWNS FERRY 1 - ANALOG TRANSMITTER FAILURES			
54629	BROWNS FERRY 1 - LOWERING MSIV SETPOINT			
54687	BROWNS FERRY 1 - REVISED PHYSICAL SECURITY PLAN			
54805	BROWNS FERRY UNIT 1, APRIL 9, 1984, TECH. SPEC. AMDT REQ. #TS-197			
55212	BROWNS FERRY 1 - POTENTIAL OVER VOLTAGE CONDITION			
49018	BROWNS FERRY 1 - UPDATE PRESSURE VS TEMPERATURE CURVE		1	08/15/83
11569	BROWNS FERRY 1 - BOTTLED UP OPERATION		2	09/12/83
49984	BROWNS FERRY 1 - BYPASSING MAIN STEAM LINE TUNNEL TEMPERATURE SWITCHES			09/20/83

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>		<u>(CONTINUATION)</u>		<u>TARGET</u>
42805	BROWNS FERRY 1 - MAIN STEAM LINE SPACE HIGH TEMP ISOLATIONS		2	10/01/83
12611	BROWNS FERRY 1 - ISOLATION OF MAIN STEAM LINES ON HIGH TEMPERATURE		2	10/21/83
49011	BROWNS FERRY 1 - CUMULATIVE OUT OF SERVICE TIME FOR RBM (TS-176)		2	10/30/83
49014	BROWNS FERRY 1 - PERTURBATION OF REACTOR WATER LEVEL			10/30/83
49227	BROWNS FERRY 1 - REVISED INSERVICE PUMP AND VALVE TESTING PROGRAM		1	11/11/83
11938	BROWNS FERRY 1,2,3- ISI AND IMPLEMENTATION DATES.		1	11/11/83
49224	BROWNS FERRY 1 - FREQUENCY OF MSIV TESTING		2	11/28/83
48476	BROWNS FERRY 1 - CRD MAINTENANCE REQUIREMENTS AND OTHER CHANGES TO T.S.			12/17/83
53352	BROWNS FERRY 1 - HYDRO TEST PRESSURE RELIEF			01/06/84
55028	BROWNS FERRY - APPENDIX R MODS			09/28/84
54518	BROWNS FERRY 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
55034	BROWNS FERRY - APPENDIX R MODS			10/31/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53571	BROWNS FERRY 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53654	BROWNS FERRY 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53737	BROWNS FERRY 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53820	BROWNS FERRY 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53964	BROWNS FERRY 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54047	BROWNS FERRY 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: BROWNS FERRY 2

PLANT LOCATION: 19 MI NW OF DECATUR, ALA  
 DOCKET NUMBER: 050-00260  
 ARCH/ENGINEER: TVA  
 IE INSPECTOR: J. PAULK

LICENSED POWER: 3293 MWT  
 DESIGN POWER: 1065 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. CLARK  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
48135	BROWNS FERRY 2 - FIRE PROTECTION APPENDIX R		1	<u>COMPLETE</u> 10/12/83
12296	BROWN'S FERRY 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE BROWN'S FERRY 2		3	02/01/84
07975	BROWNS FERRY 2 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/06/84
08436	BROWNS FERRY 2 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKLING		1	06/06/84
46653	BROWNS FERRY 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		1	06/15/84
52393	BROWNS FERRY 2 - APPENDIX R EXEMPTION REQUESTS			06/18/84
42874	BROWNS FERRY 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/27/84
<u>ACTIVE ACTIONS</u>				
51074	BROWNS FERRY 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52211	BROWNS FERRY 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52740	BROWNS FERRY 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52821	BROWNS FERRY 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52902	BROWNS FERRY 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52983	BROWNS FERRY 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING - CHNGS TO TECH SPECS - RTS COMPONENTS			
08716	BROWNS FERRY 2 - CONTAINMENT LEAK TESTING - APP. J (GENERIC)		2	09/01/83
11325	BROWNS FERRY 2 - INSERVICE TESTING		1	11/11/83
48132	BROWNS FERRY 2 - SINGLE LOOP OPERATION		3	11/23/83
42569	BROWNS FERRY 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	02/17/84
51670	BROWNS FERRY 2 NUREG 0737 TECH SPECS			03/01/84
08134	BROWNS FERRY 2 - APPENDIX I T.S. IMPLEMENTATION REVIEW REVIEW		3	03/16/84
43731	BROWNS FERRY 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	03/30/84
07930	BROWNS FERRY 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	06/30/84
08930	BROWNS FERRY 2 - RPS POWER SUPPLY		1	08/10/84
42482	BROWNS FERRY 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/07/84
<u>TAC</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44144	BROWNS FERRY 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	<u>COMPLETE</u> 10/12/83
44494	BROWNS FERRY 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	10/12/83
48149	BROWNS FERRY 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/28/83
45808	BROWNS FERRY 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
44073	BROWNS FERRY 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			02/28/84
44215	BROWNS FERRY 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
51144	BROWNS FERRY 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51224	BROWNS FERRY 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45681	BROWNS FERRY 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/30/83
45585	BROWNS FERRY 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	11/15/83
44285	BROWNS FERRY 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83
44833	BROWNS FERRY 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	11/30/83
44424	BROWNS FERRY 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	12/31/83



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>		<u>(CONTINUATION)</u>		<u>TARGET</u>
48261	BROWNS FERRY 2	- NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES	1	04/30/84
46287	BROWNS FERRY 2	- NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE	1	06/01/84
45117	BROWNS FERRY 2	- NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION	1	07/30/84
47850	BROWNS FERRY 2	- NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS	1	08/31/84
45928	BROWNS FERRY 2	- NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER	1	09/30/84
45999	BROWNS FERRY 2	- NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER	1	09/30/84
46071	BROWNS FERRY 2	- NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY	1	09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
49321	BROWNS FERRY 2	- RECIRCULATION SYSTEM PIPE CRACKS (AMEND 87)	1	12/27/83
51750	BROWNS FERRY 2	- AUGMENTED LEAK DETECTION REQUIREMENTS (AMEND 87)		12/27/83
52348	BROWNS FERRY 2	- INCREASED CORE FLOW (AMEND 88)		03/06/84
51503	BROWNS FERRY 2	- INTEGRATED LEAK RATE TEST FAILURE		04/20/84
52402	BROWNS FERRY 2	- AUDIT SAFEGUARDS PLAN (AMEND 89)		04/23/84
51753	BROWNS FERRY 2	- GROUP 7 ISOLATION VALVES (AMEND 90)		04/24/84
52580	BROWNS FERRY 2	-TURBINE CNTRL VALVE FAST CLOSURE (AMEND 91)		04/25/84
47456	BROWNS FERRY 2	- REVISED ISI PROGRAM (AMEND 92)	3	04/30/84
48480	BROWNS FERRY 2	- ORGANIZATIONAL CHANGES		05/23/84
53239	BROWNS FERRY 2	- GAP CONDUCTANCE CALCULATIONS		05/23/84
53354	BROWNS FERRY 2	- REPORTING REQUIREMENTS OF 10 CFR 50.73 (AMEND 93)		05/30/84
52506	BROWNS FERRY 2	- REPORTING ACTUATIONS OF SRVS (AMEND 93)		05/30/84
52577	BROWNS FERRY 2	- ROD BLOCK INSTRUMENTATION		06/06/84
51860	BROWNS FERRY 2	- VERIFICATION TEST OF STATION DISTRIBUTION VOLTAGES		06/19/84
52700	BROWNS FERRY 2	- CORE RELOADS USING SRMS		06/25/84
52342	BROWNS FERRY 2	- DEPTH OF WATER OVER FUEL		06/26/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52345	BROWNS FERRY 2	- SRO FOR FUEL HANDLING		
52583	BROWNS FERRY 2	- ISOLATION DAMPERS, CONTROL ROOM		
52509	BROWNS FERRY 2	- SUPPRESSION CHAMBER DRAIN VALVES		
52442	BROWNS FERRY 2	- NDT PVS T CURVES THRU 6 EFPY		
52445	BROWNS FERRY 2	- LIFE OF PLANT NDT CURVES		
52456	BROWNS FERRY 2	- AUDIT OF SECURITY PLANT/QA REVIEWS		
53497	BROWNS FERRY 2	- RESPONSE ON II.K.3.22		
54791	BROWNS FERRY 2	- APPENDIX J TEST EXEMPTION		
54630	BROWNS FERRY 2	- LOWERING MSIV SETPOINT		
54688	BROWNS FERRY 2	- REVISED PHYSICAL SECURITY PLAN		
54690	BROWNS FERRY 2	- ALLOWABLE RESPONSE TIME ON SDIV		
54806	BROWNS FERRY UNIT 2,	APRIL 9, 1984, TECH. SPEC. AMDT REQ. #TS-197		
55213	BROWNS FERRY 2	- POTENTIAL OVER VOLTAGE CONDITION		
11570	BROWNS FERRY 2	- BOTTLED UP OPERATION	2	09/12/83
42806	BROWNS FERRY 2	- MAIN STEAM LINE SPACE HIGH TEMP ISOLATIONS	2	10/01/83
12612	BROWNS FERRY 2	- ISOLATION OF MAIN STEAM LINES ON HIGH TEMPERATURE	2	10/21/83
46998	BROWNS FERRY 2	- FLOW BIASED PEAKING FACTOR	3	10/28/83
49012	BROWNS FERRY 2	- CUMULATIVE OUT OF SERVICE TIME FOR RBM (TS-176)	2	10/30/83
49015	BROWNS FERRY 2	- PERTURBATION OF REACTOR WATER LEVEL	2	10/30/83
49228	BROWNS FERRY 2	- REVISED INSERVICE PUMP AND VALVE TESTING PROGRAM	1	11/11/83
49985	BROWNS FERRY 2	- BYPASSING MAIN STEAM LINE TUNNEL TEMPERATURE SWITCHES		11/17/83



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
48477	BROWNS FERRY 2 - CRD MAINTENANCE REQUIREMENTS AND OTHER CHANGES TO T. S.			12/17/83
54519	BROWNS FERRY 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53572	BROWNS FERRY 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53655	BROWNS FERRY 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53738	BROWNS FERRY 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53821	BROWNS FERRY 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53965	BROWNS FERRY 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54048	BROWNS FERRY 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: BROWNS FERRY 3

PLANT LOCATION: 10 MI NW OF DECATUR, ALA  
 DOCKET NUMBER: 050-00296  
 ARCH/ENGINEER: TVA  
 IE INSPECTOR: J. PAULK

LICENSED POWER: 3293 MWT  
 DESIGN POWER: 1065 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. CLARK  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
48136	BROWNS FERRY 3 - FIRE PROTECTION APPENDIX R		3	<u>COMPLETE</u> 10/12/83
42232	BROWNS FERRY 3 - BWR SCRAM DISCHARGE VOLUME CAPABILITY		1	11/15/83
12297	BROWN'S FERRY 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE BROWN'S FERRY 3		3	02/01/84
08437	BROWNS FERRY 3 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING		1	06/06/84
08438	BROWNS FERRY 3-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/06/84
46654	BROWNS FERRY 3 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/15/84
52394	BROWNS FERRY 3 - APPENDIX R EXEMPTION REQUESTS			06/18/84
42875	BROWNS FERRY 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/27/84
<u>ACTIVE ACTIONS</u>				
52741	BROWNS FERRY 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52822	BROWNS FERRY 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52903	BROWNS FERRY 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52984	BROWNS FERRY 3-ITEM 3.1.3-POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
51075	BROWNS FERRY 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52212	BROWNS FERRY 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
08717	BROWNS FERRY 3 - CONTAINMENT LEAK TESTING - APP. J (GENERIC)		2	09/01/83
42570	BROWNS FERRY 3 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	09/30/83
11326	BROWNS FERRY 3 - INSERVICE TESTING		1	11/11/83
51671	BROWNS FERRY 3 NUREG 0737 TECH SPECS			03/01/84
08135	BROWNS FERRY 3 - APPENDIX I T.S. IMPLEMENTATION REVIEW REVIEW		3	03/16/84
43736	BROWNS FERRY 3 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	03/30/84
07931	BROWNS FERRY 3 - MARK I CONTAINMENT LONG TERM PROG. IMPLEMENT.		1	06/30/84
08931	BROWNS FERRY 3 - RPS POWER SUPPLY		1	08/10/84
42483	BROWNS FERRY 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/07/84
48133	BROWNS FERRY 3 - SINGLE LOOP OPERATION		3	11/21/84
<u>TAC # TAC DESCRIPTION IMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
44495	BROWNS FERRY 3 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	<u>COMPLETE</u> 10/12/83
44145	BROWNS FERRY 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	10/12/83
45809	BROWNS FERRY 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
48150	BROWNS FERRY 3 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46			12/28/83
44074	BROWNS FERRY 3 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			02/28/84
<u>ACTIVE ACTIONS</u>				
51145	BROWNS FERRY 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51225	BROWNS FERRY 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45682	BROWNS FERRY 3 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	09/30/83
45586	BROWNS FERRY 3 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	11/15/83
44834	BROWNS FERRY 3 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	11/30/83
44216	BROWNS FERRY 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/83
44286	BROWNS FERRY 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>		(CONTINUATION)		<u>TARGET</u>
44425	BROWNS FERRY 3	- NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS	1	12/31/83
48262	BROWNS FERRY 3	- NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES	1	04/30/84
46288	BROWNS FERRY 3	- NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE	1	06/01/84
45118	BROWNS FERRY 3	- NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION	1	07/30/84
47851	BROWNS FERRY 3	- NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS	1	08/31/84
45929	BROWNS FERRY 3	- NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER	1	09/30/84
46000	BROWNS FERRY 3	- NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER	1	09/30/84
46072	BROWNS FERRY 3	- NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY	1	09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
51751	BROWNS FERRY 3	- AUGMENTED LEAK DETECTION REQUIREMENTS (AMEND 60)		12/27/83
52349	BROWNS FERRY 3	- INCREASED CORE FLOW (AMEND 61)		03/06/84
52403	BROWNS FERRY 3	- AUDIT OF SAFEGUARDS PLAN (AMEND 62)		04/23/84
51754	BROWNS FERRY 3	- GROUP 7 ISOLATION VALVES		04/24/84
52581	BROWNS FERRY 3	-TURBINE CNTRL VALVE FAST CLOSURE (AMEND 64)		04/25/84
53240	BROWNS FERRY 3	- GAP CONDUCTANCE CALCULATIONS		05/23/84
48481	BROWNS FERRY 3	- ORGANIZATIONAL CHANGES		05/23/84
53355	BROWNS FERRY 3	- REPORTING REQUIREMENTS OF 10 CFR 50.73		05/30/84
52507	BROWNS FERRY 3	- REPORTING ACTUATIONS OF SRVS (AMEND 66)		05/30/84
54807	BROWNS FERRY UNIT 2,	APRIL 9, 1984, TECH. SPEC. AMDT REQ. #TS-197		06/05/84
52578	BROWNS FERRY 3	- ROD BLOCK INSTRUMENTATION		05/06/84
54137	BROWNS FERRY 3	- ANALOG TRIP SYSTEM MODIFICATIONS		06/06/84
52701	BROWNS FERRY 3	- CORE RELOADS USING SRMS		06/25/84
52343	BROWNS FERRY 3	- DEPTH OF WATER OVER FUEL		06/26/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52346	BROWNS FERRY 3	- SRO FOR FUEL HANDLING		
52584	BROWNS FERRY 3	- ISOLATION DAMPERS, CONTROL ROOM		
53498	BROWNS FERRY 3	- RSPONSE ON II.K.3.22		
53506	BROWNS FERRY 3	- CORE RELOAD		
54131	BROWNS FERRY 3	- THERMAL POWER MONITOR		
54132	BROWNS FERRY 3	- SDIV MODIFICATIONS		
54133	BROWNS FERRY 3	- MARK I TOROS MODIFICATIONS		
54134	BROWNS FERRY 3	- LPCI SUPPRESSION POOL COOLING		
54135	BROWNS FERRY 3	- DRYWELL AND SUPPRESSION CHAMBER COATINGS		
54136	BROWNS FERRY 3	- RPS POWER SUPPLY MODIFICATIONS		
52510	BROWNS FERRY 3	- SUPPRESSION CHAMBER DRAIN VALVES		
52443	BROWNS FERRY 3	- NDT PVS T CURVES THRU 6 EFPY		
52446	BROWNS FERRY 3	- LIFE OF PLANT NDT CURVES		
52457	BROWNS FERRY 3	- AUDIT OF SECURITY PLAN/QA REVIEWS		
54138	BROWNS FERRY 3	- BOTTLED-UP OPERATION SCRAMS		
54139	BROWNS FERRY 3	- ITEM II.F.1 MODS AND T.S.		
54140	BROWNS FERRY 3	- HYDROGEN ANALYZER VALVES		
55141	BROWNS FERRY 3	- SCHEDULAR EXEMPTION TO 10 CFR 50.49		
55214	BROWNS FERRY 3	- POTENTIAL OVER VOLTAGE CONDITION		
55215	BROWNS FERRY 3	- HEAD SPRAY LINE		
54141	BROWNS FERRY 3	- TESTABLE PENETRATIONS		
54142	BROWNS FERRY 3	- DRYWELL AIR SUPPLY		

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BROWNS FERRY 3

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE TARGET
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
54143	BROWNS FERRY 3 - TORUS MAKEUP WATER SUPPLY			
54631	BROWNS FERRY 3 - LOWERING MSIV SETPOINT			
54689	BROWNS FERRY 3 - REVISED PHYSICAL SECURITY PLAN			
54853	BROWNS FERRY 3 - EXEMPTION TO APPENDIX J TEST INTERNAL			
49226	BROWNS FERRY 3 - FREQUENCY OF MSIV TESTING		2	07/30/83
11571	BROWNS FERRY 3 - BOTTLED UP OPERATION		2	09/12/83
49986	BROWNS FERRY 3 - BYPASSING MAIN STEAM LINE TUNNEL TEMPERATURE SWITCHES			09/20/83
42807	BROWNS FERRY 3 - MAIN STEAM LINE SPACE HIGH TEMP ISOLATIONS		2	10/01/83
12613	BROWNS FERRY 3 - ISOLATION OF MAIN STEAM LINES ON HIGH TEMPERATURE		2	10/21/83
46999	BROWNS FERRY 3 - FLOW BIASED PEAKING FACTOR		3	10/28/83
49013	BROWNS FERRY 3 - CUMULATIVE OUT OF SERVICE TIME FOR RBM		2	10/30/83
49016	BROWNS FERRY 3 - PERTURBATION OF REACTOR WATER LEVEL		2	10/30/83
49229	BROWNS FERRY 3 - REVISED INSERVICE PUMP AND VALVE TESTING PROGRAM		1	11/30/83
48478	BROWNS FERRY 3 - CRD MAINTENANCE REQUIREMENTS AND OTHER CHANGE TO T.S.			12/17/83
55140	BROWNS FERRY 3 - TIA, LEAKS IN INSTRUMENT NOZZLE			07/13/84
47869	BROWNS FERRY - MSIV LEAKAGE			09/28/84
54520	BROWNS FERRY 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53573	BROWNS FERRY 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53656	BROWNS FERRY 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53739	BROWNS FERRY 3 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53822	BROWNS FERRY 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING --CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53966	BROWNS FERRY 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54049	BROWNS FERRY 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: BRUNSWICK 1

PLANT LOCATION: 3 MI N OF SOUTHPORT, NC  
 DOCKET NUMBER: 050-00325  
 ARCH/ENGINEER: UEC  
 IE INSPECTOR: D. JOHNSON

LICENSED POWER: 2436 MWT  
 DESIGN POWER: 0821 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: M. GROTENHUIS  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
42571	BRUNSWICK 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	<u>COMPLETE</u> 12/01/83
08106	BRUNSWICK 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		2	12/27/83
12186	BRUNSWICK 1 - HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAILURE		3	01/31/84
07932	BRUNSWICK 1 - MARK CONTAIN. LONG TERM PROGRAM IMPLEMENTATION		1	03/19/84
51673	BRUNSWICK 1 NUREG 0737 TECH SPECS		2	03/28/84
46655	BRUNSWICK 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		1	03/31/84
07976	BRUNSWICK 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		3	05/18/84

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
52742	BRUNSWICK 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52823	BRUNSWICK 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52904	BRUNSWICK 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52985	BRUNSWICK 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
42484	BRUNSWICK 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	05/22/84
42876	BRUNSWICK 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
11264	BRUNSWICK 1 - 10 CFR 50.55A(G) 1ST PUMPS/VALVES		1	06/30/84
51076	BRUNSWICK 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/30/84
43418	BRUNSWICK 1 - NEW APPENDIX R TO 10 CFR 50 REGARDING FIRE PROTECTION FEATURES		3	07/01/84
12829	BRUNSWICK 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES		2	10/31/84
43737	BRUNSWICK 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	10/31/84
48489	BRUNSWICK 1 - SINGLE LOOP OPERATION			11/01/84
52213	BRUNSWICK 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	09/30/85

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
45614	BRUNSWICK 1 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES		1	<u>COMPLETE</u> 10/17/83
46431	BRUNSWICK 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	10/18/83
44567	BRUNSWICK 1 - II.D.1 RV AND SV TESTING		1	10/31/83
48151	BRUNSWICK 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		2	12/15/83
45810	BRUNSWICK 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/04/84
44426	BRUNSWICK 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	05/08/84
45683	BRUNSWICK 1 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	05/18/84
44217	BRUNSWICK 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
44287	BRUNSWICK 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	<u>TARGET</u>
45930	BRUNSWICK 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		3	
46001	BRUNSWICK 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		3	
46073	BRUNSWICK 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		3	
46289	BRUNSWICK 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	
51226	BRUNSWICK 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		3	
47585	BRUNSWICK 1 - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		2	06/05/84
47656	BRUNSWICK 1 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		2	06/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
47727	BRUNSWICK 1 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		2	<u>TARGET</u> 06/30/84
45119	BRUNSWICK 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		2	07/17/84
45587	BRUNSWICK 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		2	07/17/84
48263	BRUNSWICK 1 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		2	10/31/84
51146	BRUNSWICK 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		3	10/31/84
<u>PLANT SPECIFIC</u>				
<u>TAC #</u>	<u>TAC DESCRIPTION</u>		<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49273	BRUNSWICK 1 - ISOLATION ACTUATION INSTRUMENTATION			<u>COMPLETE</u> 11/28/83
51498	BRUNSWICK 1 - SALP REVIEW			12/01/83
49489	BRUNSWICK 1 - FIVE MICELLANEOUS TECH. SPECS.			12/02/83
52628	BRUNSWICK 1 - INTEGRATED LEAK RATE TEST - SHORT DURATION		2	12/09/83
49585	BRUNSWICK 1 - SECOND DIGITAL ANALOG TS CHANGE			12/12/83
43796	BRUNSWICK 1 - SPENT FUEL POOL EXPANSION			12/15/83
52610	BRUNSWICK 1 - REVISED RETS		2	12/27/83
48754	BRUNSWICK 1 - RFS MONITORING SYSTEM TS		1	12/27/83
48635	BRUNSWICK 1 - SDV INSTRUMENT MODIFICATIONS		1	12/28/83
48637	BRUNSWICK 1 - CRD RETURN LINE SNUBBERS		2	12/28/83
52486	BRUNSWICK 1 - INTEGRATED LEAK RATE TEST		1	01/11/84
52536	BRUNSWICK 1 - FEEDWATER & TURBINE BYPASS TS		3	01/18/84
47038	BRUNSWICK 1 - RADIATION AREA INTERPRETATION REQUEST		2	01/27/84
48812	BRUNSWICK 1 - DG LCO EXTENSION REQUEST		3	02/14/84
48822	BRUNSWICK 1 - MISCELLANEOUS TECHNICAL SPECIFICATIONS		2	02/27/84
53504	BRUNSWICK 1 OPERATOR REQUAL PROGRAM REV 8			02/28/84
49331	BRUNSWICK 1 - REACTIVITY ANOMALIES		2	04/17/84
54254	BRUNSWICK 1 - EXEMPTION - ANNUAL EMERGENCY EXERCISE			05/17/84
54964	BRUNSWICK 1 - REQUEST FOR SCHEDULAR EXEMPTION FROM H CONTROL			05/18/84
49775	BRUNSWICK 1: ADMINISTRATIVE CONTROLS TSC2		2	05/25/84
46768	BRUNSWICK 1 - ADMINISTRATIVE CONTROLS TECHNICAL SPECIFICATION CHANGE		2	05/25/84
54170	BRUNSWICK UNIT 1 - FUEL ENRICHMENT			06/05/84
<u>ACTIVE ACTIONS</u>				
52537	BRUNSWICK 1 - RESPONSE TO 7/22/83 CONF. ORDER			<u>TARGET</u> 03/31/84
54270	BRUNSWICK 1 - 1983 INSERVICE INSPECTION REPORT			06/15/84
51816	BRUNSWICK 1 THERMAL - HYDRAULIC CODE		3	06/20/84
53233	BRUNSWICK 1 - DISPOSAL OF WASTE OIL			06/21/84
55228	BRUNSWICK 1 - RECOMBINER CAPABILITY EXEMPTION			06/30/84
54961	BRUNSWICK 1 - ROYCHAN CABLE TEST			07/30/84
54999	BRUNSWICK 1 - DELETE SNUBBER TABLE			07/31/84
55031	BRUNSWICK 1 - CORRECTIONS TO TS - ADMIN			07/31/84
55036	BRUNSWICK 1 - SALP REVIEW			07/31/84
54190	BRUNSWICK 1 - SEALING FACTORS - RADWATER			08/30/84
55254	BRUNSWICK 1 - CONTROL ROD ASSEMBLY			08/31/84
55256	BRUNSWICK 1 - ADMINISTRATION CONTROLS			09/30/84
55258	BRUNSWICK 1 - SPENT FUEL POOL WATER LEVEL			09/30/84
54354	BRUNSWICK 1 PURGE AND VENT OPERABILITY			09/30/84
54521	BRUNSWICK 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54840	BRUNSWICK 1 - CONTAINMENT INTEGRITY - MSIV CLOSURE			09/30/84
55230	BRUNSWICK 1 - CHANNEL TRIP			10/01/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
55232	BRUNSWICK 1 - CLARIFICATION OF OPERATING CONDITION			<u>TARGET</u>
54193	BRUNSWICK 1 - DEGRADED VOLTAGE RITAG T/S			10/01/84
49276	BRUNSWICK 1 - ELECTRICAL DISTRIBUTION VOLTAGE		3	10/31/84
<u>ANTICIPATED ACTIONS</u>				
55252	BRUNSWICK 1 - MARK I CONTAINMENT TECHNICAL SPECIFICATION			<u>INITIATION</u>
53574	BRUNSWICK 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			07/11/84
53657	BRUNSWICK 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53740	BRUNSWICK 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINI TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53823	BRUNSWICK 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53967	BRUNSWICK 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54050	BRUNSWICK 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: BRUNSWICK 2

PLANT LOCATION: 3 MI N OF SOUTHPORT, NC  
 DOCKET NUMBER: 050-00324  
 ARCH/ENGINEER: UEC  
 IE INSPECTOR: D. JOHNSON

LICENSED POWER: 2436 MWT  
 DESIGN POWER: 0821 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: M. GROTENHUIS  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
42572	BRUNSWICK 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	<u>COMPLETE</u> 12/01/83
10906	BRUNSWICK 2 - APPENDIX I TECH SPEC IMPLEMENTATION		2	12/27/83
12187	BRUNSWICK 2 - HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAILURE		3	01/31/84
46656	BRUNSWICK 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		1	03/13/84
07933	BRUNSWICK 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMEN.		1	03/19/84
51672	BRUNSWICK 1 NUREG 0737 TECH SPECS		2	03/28/84
07977	BRUNSWICK 2 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		3	05/18/84
<u>ACTIVE ACTIONS</u>				
52743	BRUNSWICK 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52824	BRUNSWICK 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52905	BRUNSWICK 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52986	BRUNSWICK 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
42485	BRUNSWICK 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	05/22/84
12830	BRUNSWICK 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES		2	05/31/84
42877	BRUNSWICK 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/03/84
11265	BRUNSWICK 2 - INSERVICE TESTING AND INSPECTION (IST) PUMPS/VALVES		1	06/30/84
51077	BRUNSWICK 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/30/84
43417	BRUNSWICK 2 - NEW APPENDIX R TO 10 CFR 50 REGARDING FIRE PROTECTION FEATURES		3	07/01/84
43735	BRUNSWICK 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	10/31/84
46779	BRUNSWICK 2 - SINGLE LOOP OPERATION		2	11/01/84
52214	BRUNSWICK 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	09/30/85
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
45615	BRUNSWICK 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES		1	<u>COMPLETE</u> 10/17/83
46432	BRUNSWICK 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	10/18/83
44568	BRUNSWICK 2 - RV AND SV TESTING		1	10/31/83
48152	BRUNSWICK 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		2	12/15/83
45811	BRUNSWICK 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/04/84
44427	BRUNSWICK 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	05/08/84
45684	BRUNSWICK 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	05/13/84
45588	BRUNSWICK 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		2	05/18/84
<u>ACTIVE ACTIONS</u>				
45931	BRUNSWICK 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		3	
46002	BRUNSWICK 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		3	
46074	BRUNSWICK 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		3	
46290	BRUNSWICK 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	
44218	BRUNSWICK 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	
44288	BRUNSWICK 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	
51227	BRUNSWICK 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		3	03/15/84
47586	BRUNSWICK 2 - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		2	06/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
47657	BRUNSWICK 2 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		2	TARGET 06/30/84
47728	BRUNSWICK 2 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		2	06/30/84
45120	BRUNSWICK 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		2	07/17/84
48264	BRUNSWICK 2 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		2	10/31/84
51147	BRUNSWICK 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		3	10/31/84
<u>COMPLETED ACTIONS</u>				
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
52587	BRUNSWICK 2 - ORDER MODIFICATION REQUEST PIPE CRACK			COMPLETE 10/28/83
43695	BRUNSWICK 2 - INSERVICE SURVEILLANCE REQUIREMENTS FOR SNUBBERS		1	11/08/83
52601	BRUNSWICK 2 - PIPE CREEK ORDER MODIFICATION REQUEST 2 - SHORT RANGE			11/22/83
52602	BRUNSWICK 2 - PIPE CRACK ORDER MODIFICATIONS REQUEST 3 - LONG RANGE			11/22/83
52078	BRUNSWICK 2 - REVISED MCPR			11/28/83
49274	BRUNSWICK 2 - ISOLATION ACTUATION INSTRUMENTATION			11/28/83
51313	BRUNSWICK 2 - TMI ORDER AMENDMENT		1	12/01/83
49490	BRUNSWICK 2 - FIVE MICELLANEOUS TECH. SPECS.		2	12/02/83
52629	BRUNSWICK 2 INTEGRATED LEAK RATE TEST SHORT DURATION		2	12/09/83
52470	BRUNSWICK 2 - AUGMENTED OFF-GAS SCHEDULE		1	12/12/83
52538	BRUNSWICK 2 - REPSONSE TO AUG. 16, 1983 ORDER		1	12/13/83
43797	BRUNSWICK 2 - SPENT FUEL POOL EXPANSION		1	12/15/83
52611	BRUNSWICK 2 - REVISED RETS		2	12/27/83
48636	BRUNSWICK 2 - SDV INSTRUMENT MODIFICATIONS		1	12/28/83
48638	BRUNSWICK 2 - CRD RETURN LINE SNUBBERS		2	12/28/83
48755	BRUNSWICK 2 -RPS MONITORING SYSTEM TS		1	12/28/83
53292	BRUNSWICK 2 - BATTERY SURVEILLANCE REQUIREMENTS			12/31/83
52487	BRUNSWICK 2 - INTEGRATED LEAK RATE TEST		1	01/11/84
52535	BRUNSWICK 2 - FEEDWATER & TURBINE BYPASS TS		3	01/18/84
53293	BRUNSWICK 2 - RELIEF FROM HYDRO TEST			01/27/84
47039	BRUNSWICK 2 - RADIATION AREA INTERPRETATION REQUEST		2	01/27/84
48813	BRUNSWICK 2 - DG LCO EXTENSION REQUEST		3	02/14/84
48823	BRUNSWICK 2 - MISCELLANEOUS TECH SPEC REVISIONS		2	02/27/84
53505	BRUNSWICK 2 OPERATOR REQUAL PROGRAM REV 8			02/28/84
49332	BRUNSWICK 2 - REACTIVITY ANOMALIES		2	04/17/84
53470	BRUNSWICK 2 CORE SPRAY SYSTEM SURVEILLANCE			05/08/84
54255	BRUNSWICK 2 - EXEMPTION - ANNUAL EMERGENCY EXERCISE			05/17/84
54965	BRUNSWICK 2 - REQUEST FOR SCHEDULAR EXEMPTION FROM H CONTROL			05/18/84
49922	BRUNSWICK 2 - FUEL BURNUP CODE		2	05/18/84
49923	BRUNSWICK 2 - CORE SIMULATOR CODE		2	05/18/84
49924	BRUNSWICK 2 - ANALYSIS METHODS		2	05/18/84
49586	BRUNSWICK 2 - SECOND DIGITAL ANALOG TS CHANGE		1	05/25/84
49774	BRUNSWICK 2 - ADMINISTRATIVE CONTROLS TSC2		2	05/25/84
46767	BRUNSWICK 2 - ADMINISTRATIVE CONTROLS TECHNICAL SPECIFICATION CHANGE		2	05/25/84
06171	BRUNSWICK 2 - SINGLE LOOP OPERATION		2	06/24/84
<u>ACTIVE ACTIONS</u>				
54962	BRUNSWICK 2 - ROYCHAN CABLE TEST			TARGET 05/30/84
51817	BRUNSWICK 2 THERMAL - HYDRAULIC CODE		2	06/20/84
53234	BRUNSWICK 2 - DISPOSAL OF WASTE OIL			06/21/84
55229	BRUNSWICK 2 - RECOMBINER CAPABILITY EXEMPTION			06/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

BRUNSWICK 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54966	BRUNSWICK 2 - EQ EXTENSION REQUEST			06/30/84
54147	BRUNSWICK 2 - BATTERY CAPACITY			06/31/84
55001	BRUNSWICK 1 - CHECK VALVE SURVEILLANCE EXTENSION			07/02/84
55227	BRUNSWICK UNIT 2 - FEEDWATER SPRAYER REPLACEMENT			07/15/84
55032	BRUNSWICK 2 - CORRECTIONS TO TS - ADMIN			07/31/84
55037	BRUNSWICK 2 - SALP REVIEW			07/31/84
55000	BRUNSWICK 2 - DELET SNUBBER TABLE			07/31/84
54191	BRUNSWICK 2 - SEALING FACTORS - RADWASTE			08/30/84
55251	BRUNSWICK 2 - RELCAD 5 TECHNICAL SPECIFICATIONS			08/31/84
55255	BRUNSWICK 2 - CONTROL ROD ASSEMBLY			08/31/84
55257	BRUNSWICK 2 - ADMINISTRATION CONTROLS			09/30/84
55259	BRUNSWICK 2 - SPENT FUEL POOL WATER LEVEL			09/30/84
54355	BRUNSWICK 2 PURGE AND VENT OPERABILITY			09/30/84
54522	BRUNSWICK 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
55231	BRUNSWICK 2 - CHANNEL TRIP			10/01/84
55233	BRUNSWICK 2 - CLARIFICATION OF OPERATING CONDITION			10/01/84
54839	BRUNSWICK 2 - CONTAINMENT INTEGRITY - MSIV CLOSURE			10/30/84
54194	BRUNSWICK 2 - DEGRADED VOLTAGE RITAG T/S			10/31/84
49277	BRUNSWICK 2 - ELECTRICAL DISTRIBUTION VOLTAGE		3	10/31/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
55253	BRUNSWICK 2 - MARK I CONTAINMENT TECHNICAL SPECIFICATIONS			07/11/84
53575	BRUNSWICK 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53658	BRUNSWICK 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53741	BRUNSWICK 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53824	BRUNSWICK 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53968	BRUNSWICK 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54051	BRUNSWICK 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: CALLAWAY 1

PLANT LOCATION: 10 MI SE OF FULTON, MO  
 DOCKET NUMBER: 050-00483  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR:

LICENSED POWER: 000 MWT  
 DESIGN POWER: 1188 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: J. HOLONICH  
 BRANCH CHIEF: B. YOUNGBLOOD  
 LIC. ASSISTANT: M. RUSHBROOK

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
55193	CALLAWAY 1 - ITEM 3.1.3 - POST MAINTENANCE TESTING CHANGES TO TECH SPECS RTS COMPONENTS			<u>TARGET</u>
55194	CALLAWAY 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES AND VENDOR RECOMMENDATIONS			
55195	CALLAWAY 1 - ITEM 1.1 POST TRIP REVIEW - PROGRAM DESCRIPTION AND PROCEDURES			
55196	CALLAWAY 1 - ITEM 2.1 EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE			
55197	CALLAWAY 1 - ITEM 4.3 AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
55198	CALLAWAY 1 - ITEM 4.1 - REACTOR SYSTEM RELIABILITY VENDOR RELATED MODIFICATIONS			
55199	CALLAWAY 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
55185	CALLAWAY 1 - DETAILED CONTROL ROOM DESIGN REVIEW - I.D.1			<u>TARGET</u>
55186	CALLAWAY 1 - POST ACCIDENT SAMPLING SYSTEM II.B.3			07/20/84 07/20/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
55201	CALLAWAY 1 - ITEM 1.2 - POST TRIP REVIEW - DATA AND INFORMATION CAPABILITY			<u>TARGET</u>
55202	CALLAWAY 1 - ITEM 3.2.1 & 3.2.2 POST MAINTENANCE TESTING PROCEDURES AND VENDOR RECOMMENDATIONS			
55203	CALLAWAY 1 - ITEM 3.2.3 POST MAINTENANCE TESTING -CHANGES TO TECH SPECS			
55204	CALLAWAY 1 - ITEMS 4.2.3 & 4.2.4 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS			
55205	CALLAWAY 1 - ITEM 4.5.1 REACTOR TRIP SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			
55206	CALLAWAY 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING			
55209	CALLAWAY 1 - LOCA REANALYSIS			
55200	CALLAWAY 1 - ITEM 2.2 EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			
55207	CALLAWAY 1 - STEAM GENERATOR TUBE RUPTURE			06/13/84
55208	CALLAWAY 1 - LOW TEMPERATURE OVERPRESSURE PROTECTION MODIFICATIONS			06/13/84
55181	CALLAWAY 1 - PREOP TEST FSAR REVISIONS			06/15/84
55182	CALLAWAY 1 - PUMP ANV VALVE OPERABILITY			07/20/84
55190	CALLAWAY 1 - FIRE PROTECTION ITEMS			07/20/84
55183	CALLAWAY 1 - SEISMIC AND DYNAMIC QUALIFICATION			07/20/84
55184	CALLAWAY 1 - QUALIFICATION AND SEAL WATER INJECTION FILTER			07/20/84
55187	CALLAWAY 1 - ICC ISOLATION DEVICES			07/20/84
55188	CALLAWAY 1 - OPERATOR EXPERIENCE			07/20/84
55189	CALLAWAY 1 - PSI EXEMPTION REQUEST			07/20/84
55192	CALLAWAY 1 - ENVIRONMENTAL ERRORS FOR SECONDARY TRIPS			07/20/84
55180	CALLAWAY PLANT 1 - REG GUIDE 1.97, REV. 2			11/09/84
55191	CALLAWAY 1 - INSERVICE INSPECTION			12/31/85



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## CONDENSED MANAGEMENT REPORT

FACILITY: CALVERT CLIFFS 1

PLANT LOCATION: 40 MI S OF ANNAPOLIS, MD  
 DOCKET NUMBER: 050-00317  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: T. FOLEY

LICENSED POWER: 2700 MWT  
 DESIGN POWER: 0845 MWE  
 NSSS VENDOR: COMB

PROJECT MANAGER: D. JAFFE  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12314	CALVERT CLIFFS 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE CALVERT CLIFFS 1		3	<u>COMPLETE</u> 01/31/84
43634	CALVERT CLIFFS 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	03/21/84
<u>ACTIVE ACTIONS</u>				
51078	CALVERT CLIFFS 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52215	CALVERT CLIFFS 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52744	CALVERT CLIFFS 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52825	CALVERT CLIFFS 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52906	CALVERT CLIFFS 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52987	CALVERT CLIFFS 1-ITEM 3.1.3-POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53062	CALVERT CLIFFS 1 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53115	CALVERT CLIFFS 1 - ITEMS 4.2.1 & 4.2.2- PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
42494	CALVERT CLIFFS 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
42890	CALVERT CLIFFS 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
08136	CALVERT CLIFFS 1 - APPENDIX I TECH. SPEC IMPLEMENTATION REVIEW		3	12/01/84
08951	CALVERT CLIFFS 1 - REVIEW OF ASYMMETRIC LOCA LOADS		2	12/30/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
45260	CALVERT CLIFFS 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	<u>COMPLETE</u> 10/06/83
45306	CALVERT CLIFFS 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/06/83
47793	CALVERT CLIFFS 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	10/24/83
46879	CALVERT CLIFFS 1 - II K2.13 THERMAL MECHANICAL REPORT		1	06/05/84
44219	CALVERT CLIFFS 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
51228	CALVERT CLIFFS 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	06/15/84
44569	CALVERT CLIFFS 1 - RV AND SV TESTING		1	06/30/84
45121	CALVERT CLIFFS 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
44428	CALVERT CLIFFS 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
45812	CALVERT CLIFFS 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
47926	CALVERT CLIFFS 1-PLANT SHIELDING MODIFICATIONS (II.B.2.2) REGIONAL REVIEW		1	06/30/84
48153	CALVERT CLIFFS 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/30/84
45932	CALVERT CLIFFS 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46003	CALVERT CLIFFS 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46075	CALVERT CLIFFS 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46291	CALVERT CLIFFS 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
44289	CALVERT CLIFFS 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51148	CALVERT CLIFFS 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			12/01/84
48443	CALVERT CLIFFS 1 - NUREG 0737, II.K.3.25 POWER ON PUMP SEALS			12/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CALVERT CLIFFS :

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
52157	CALVERT CLIFFS	1 - CYCLE 7 RELOAD		<u>COMPLETE</u>
52422	CALVERT CLIFFS	UNIT 1 - CHANGE TO T.S. FOR CSAS A-3 AND B-3	1	11/17/83
52431	CALVERT CLIFFS	UNIT 1 - TS FOR CONTAINMENT VENT (MOV-6900 & 6901)	1	11/17/83
52432	CALVERT CLIFFS	UNIT 1 - CHANGE TO TS FOR REMOTE SHUTDOWN INSTR.		11/17/83
52433	CALVERT CLIFFS	UNIT 1 - CHANGE TO TS FOR POST ACCIDENT MONITORING		11/17/83
52434	CALVERT CLIFFS	UNIT 1 - FINAL AFW MODIFICATIONS		11/17/83
52568	CALVERT CLIFFS	UNIT 1 - CONTROL ROOM HVAC COOLER TS	1	11/17/83
52570	CALVERT CLIFFS	UNIT 1 - CORRECT CONTAINMENT CAM TS	1	12/30/83
52429	CALVERT CLIFFS	UNIT 1 - CHANGE TS SNUBBER NUMBER		12/30/83
52430	CALVERT CLIFFS	UNIT 1 - DELETION OF SNUBBER FROM TS	1	12/30/83
54179	CALVERT CLIFFS	UNIT 1 - MODIFICATION OF ORDER		03/02/84
53275	CALVERT CLIFFS	UNIT 1 - APPENDIX R EXEMPTION - (OIL)	1	03/15/84
53276	CALVERT CLIFFS	UNIT 1 - APPENDIX R EXEMPTION - (DOORS)	1	03/15/84
53277	CALVERT CLIFFS	UNIT 1 - APPENDIX R EXEMPTION (SPRINKLER)	1	03/15/84
52424	CALVERT CLIFFS	UNIT 1 - REED SWITCH TS	1	03/16/84
54622	CALVERT CLIFFS	UNIT 1 - EMERGENCY TS CHANGE		04/16/84
53369	CALVERT CLIFFS	UNIT 1 - CHANGE TO TS FOR 125VDC BATTERY		04/19/84
53370	CALVERT CLIFFS	UNIT 1 - CHANGE TO TS FOR RCS VENT		04/19/84
53366	CALVERT CLIFFS	UNIT 1 - TS FOR FUEL OIL TANK INSPECTION		04/19/84
53368	CALVERT CLIFFS	UNIT 1 - CHANGE TO TS FOR 69KV SMECO LINE		04/19/84
52426	CALVERT CLIFFS	UNIT 1 - TS FOR CONTAINMENT COOLING	1	05/31/84
53367	CALVERT CLIFFS	UNIT 1 - REPORTING REQUIREMENTS PER GL 83-43		06/06/84
<u>ACTIVE ACTIONS</u>				
54625	CALVERT CLIFFS	UNIT 1 - CHANGE TO ISI INTERVAL		<u>TARGET</u>
54378	CALVERT CLIFFS	UNIT 1 - EMERGENCY EXERCISE FREQUENCY		06/01/84
54774	CALVERT CLIFFS	UNIT 1 - ORGANIZATION CHANGE		06/03/84
54776	CALVERT CLIFFS	UNIT 1 - CHANGE TO ETS - REPORTING		07/01/84
54779	CALVERT CLIFFS	UNIT 1 - HIGH RAD ACCESS TS		07/01/84
54977	CALVERT CLIFFS	UNIT 1 - FIRE PUMP TESTING		07/01/84
54778	CALVERT CLIFFS	UNIT 1 - FIRE DETECTION INSTRUMENTS		07/01/84
55290	CALVERT CLIFFS	UNIT 1 - STA QUAL.		07/01/84
54810	CALVERT CLIFFS	UNIT 1 - IREP		08/01/84
55249	CALVERT CLIFFS	UNIT 1 - REVIEW OF AFW PUMP TEST		09/01/84
54383	CALVERT CLIFFS	1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		09/30/84
54523	CALVERT CLIFFS	1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		09/30/84
55289	CALVERT CLIFFS	UNIT 1 - LICENSE DATE CHANGE		10/01/84
53365	CALVERT CLIFFS	UNIT 1 - TS FOR HYDROGEN PURGE SYSTEM		10/01/84
49674	CALVERT CLIFFS	1 - REACTOR COOLANT PUMP TRIP	1	02/01/85
<u>ANTICIPATED ACTIONS</u>				
53576	CALVERT CLIFFS	1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		<u>INITIATION</u>
53659	CALVERT CLIFFS	1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		11/01/84
53742	CALVERT CLIFFS	1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		11/01/84
53825	CALVERT CLIFFS	1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		11/01/84
53902	CALVERT CLIFFS	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT		11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CALVERT CLIFFS 1

TAC #    TAC DESCRIPTIONPLANT SPECIFICPRIORITYCRITICAL DATEANTICIPATED ACTIONS (CONTINUATION)

53969	CALVERT CLIFFS 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		
54052	CALVERT CLIFFS 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES		

INITIATION

11/01/84

11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: CALVERT CLIFFS 2

PLANT LOCATION: 40 MI S OF ANNAPOLIS, MD  
 DOCKET NUMBER: 050-00318  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: T. FOLEY

LICENSED POWER: 2700 MWT  
 DESIGN POWER: 0845 MWE  
 NSSS VENDOR: COMB

PROJECT MANAGER: D. JAFFE  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12315	CALVERT CLIFFS 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE CALVERT CLIFFS 2		3	<u>COMPLETE</u> 01/31/84
43635	CALVERT CLIFFS 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	03/21/84
<u>ACTIVE ACTIONS</u>				
51079	CALVERT CLIFFS 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52216	CALVERT CLIFFS 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52745	CALVERT CLIFFS 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52826	CALVERT CLIFFS 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52907	CALVERT CLIFFS - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52988	CALVERT CLIFFS 2-ITEM 3.1.3-POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53063	CALVERT CLIFFS 2 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53116	CALVERT CLIFFS 2 - ITEMS 4.2.1 & 4.2.2- PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
42495	CALVERT CLIFFS 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
42891	CALVERT CLIFFS 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/31/84
08137	CALVERT CLIFFS 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	12/01/84
08950	CALVERT CLIFFS 2 - REVIEW OF ASYMMETRIC LOCA LOADS		2	12/30/84
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
45261	CALVERT CLIFFS 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	<u>COMPLETE</u> 10/06/83
45307	CALVERT CLIFFS 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/06/83
47794	CALVERT CLIFFS 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	10/24/83
46880	CALVERT CLIFFS 2 - II K2.13 THERMAL MECHANICAL REPORT		1	06/05/84
44220	CALVERT CLIFFS 2 - NUREG-0737 I.C.1.2.4, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
51229	CALVERT CLIFFS 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			<u>TARGET</u>
47927	CALVERT CLIFFS 2 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	12/30/83
44570	CALVERT CLIFFS 2 - RV AND SV TESTING		1	06/15/84
45122	CALVERT CLIFFS 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
45813	CALVERT CLIFFS 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
48154	CALVERT CLIFFS 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/30/84
44429	CALVERT CLIFFS 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
45933	CALVERT CLIFFS 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46004	CALVERT CLIFFS 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46076	CALVERT CLIFFS 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46292	CALVERT CLIFFS 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
44290	CALVERT CLIFFS 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51149	CALVERT CLIFFS 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			12/01/84
48444	CALVERT CLIFFS 2 - NUREG 0737, II.K.3.25 POWER ON PUMP SEALS			12/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

CALVERT CLIFFS 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
48610	CALVERT CLIFFS UNIT 2 - HYDROGEN ANALYZER OPERABILITY EXTENSION			12/29/83
52423	CALVERT CLIFFS UNIT 2 - CHANGE TO TS FOR CSAS A-3 AND B-3			12/30/83
52428	CALVERT CLIFFS UNIT 2 - ADD SNUBBER TO TS			12/30/83
52569	CALVERT CLIFFS UNIT 2 - CONTROL ROOM HVAC COOLER TS		1	12/30/83
52571	CALVERT CLIFFS UNIT 2 - CORRECT CONTAINMENT CAM TS			12/30/83
54180	CALVERT CLIFFS UNIT 2 - MODIFICATION OF ORDER			03/02/84
53279	CALVERT CLIFFS UNIT 2 - APPENDIX R EXEMPTION (DOORS)		1	03/15/84
53280	CALVERT CLIFFS UNIT 2 - APPENDIX R EXEMPTION (SPRIKLER)		1	03/15/84
53278	CALVERT CLIFFS UNIT 2 - APPENDIX R EXEMPTION (OIL)		1	03/15/84
52425	CALVERT CLIFFS UNIT 2 - REED SWITCH TS			03/16/84
53555	CALVERT CLIFFS UNIT 2 - GENERIC STEAM LINE BREAK			03/27/84
53373	CALVERT CLIFFS UNIT 2 - CHANGE TO TS FOR 69KV SMECO LINE			04/19/84
53374	CALVERT CLIFFS UNIT 2 - CHANGE TO TS FOR 125VDC BATTERY			04/19/84
53375	CALVERT CLIFFS UNIT 2 - CHANGE TO TS FOR RCS VENT			04/19/84
53376	CALVERT CLIFFS UNIT 2 - TS CHANGE FOR REPLACEMENT OF SNUBBERS WITH SWAY STRUTS			04/19/84
53377	CALVERT CLIFFS UNIT 2 - TS CHANGE FOR INSTALLATION OF DEDICATED SNUBBER RESERVOIRS			04/19/84
53371	CALVERT CLIFFS UNIT 2 - TS FOR FUEL OIL TANK INSPECTION			04/19/84
52427	CALVERT CLIFFS UNIT 2 - TS FOR CONTAINMENT COOLING			05/31/84
53372	CALVERT CLIFFS UNIT 2 - REPORTING REQUIREMENTS PER GL 83-43			06/06/84
54623	CALVERT CLIFFS UNIT 2 - WIDE RANGE NEUTRON FLUX			06/06/84
54624	CALVERT CLIFFS UNIT 2 - TS FOR CONTAINMENT PURGE VALVES			06/06/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54379	CALVERT CLIFFS UNIT 2 - EMERGENCY EXERCISE FREQUENCY			06/03/84
54775	CALVERT CLIFFS UNIT 2 - AUX. FDWTR TS			07/01/84
48946	CALVERT CLIFFS UNIT 2 - HIGH BURNUP REPORT			08/31/84
54384	CALVERT CLIFFS 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54524	CALVERT CLIFFS 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49675	CALVERT CLIFFS 2 - REACTOR COOLANT PUMP TRIP		1	02/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53577	CALVERT CLIFFS 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53660	CALVERT CLIFFS 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53743	CALVERT CLIFFS 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53826	CALVERT CLIFFS 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53903	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53970	CALVERT CLIFFS 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54053	CALVERT CLIFFS 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53556	CALVERT CLIFFS UNIT 2 - TS FOR HYDROGEN PURGE SYSTEM			12/22/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: COOK 1

PLANT LOCATION: 11 MI S OF BENTON HARBOR, MI  
 DOCKET NUMBER: 050-00315  
 ARCH/ENGINEER: AEP  
 IE INSPECTOR: E. SWANSON

LICENSED POWER: 3250 MWT  
 DESIGN POWER: 1030 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: D. WIGGINTON  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47164	COOK 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 11/17/83
49727	- COOK 1 NUREG 0737 TECH SPECS (GENERIC LTR 82-16)		2	11/23/83
42093	COOK 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS CAPABILITY TECH SPECS		2	11/23/83
42859	COOK 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	11/23/83
12264	COOK 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE COOK 1		3	01/31/84
08477	COOK 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
53167	COOK 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			06/28/84
<u>ACTIVE ACTIONS</u>				
52746	COOK 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		2	<u>TARGET</u>
52827	COOK 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		2	
52908	COOK 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECCMMENDATIONS - RTS COMPONENTS		2	
52989	COOK 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		2	
53064	COOK 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		2	
53117	COOK 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		2	
42460	COOK 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	03/22/84
11331	COOK 1 - INSERVICE TESTING - 1ST OF PUMPS & VALVES - 10 CFR 50.55A(G)		1	11/15/84
52217	COOK 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	09/30/85
51080	COOK 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	12/31/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47795	D. C. COOK 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 12/16/83
44224	COOK 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
46881	COOK 1 - II K2.13 THERMAL MECHANICAL REPORT		1	06/18/84
<u>ACTIVE ACTIONS</u>				
51150	COOK 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	<u>TARGET</u>
51230	COOK 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	
44433	COOK 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/83
45126	COOK 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/83
46296	COOK 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	11/01/83
45818	COOK 1 - NUREG-0737 II.K.3.30, SE LOCA OUTLINE		1	06/01/84
45937	COOK 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46008	COOK 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46080	COOK 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
48158	D.C. COOK 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	10/01/84
44573	COOK 1 - RV AND SV TESTING		1	10/30/84
44294	COOK 1 - NUREG-0737 I.C.1.3A, PROCEDURES GENERATION PACKAGES (SUPP 1, 0737)		1	11/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

COOK 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52439	COOK 1 - CHARGING PUMP FLOW		1	<u>COMPLETE</u> 10/07/83
48005	COOK-1 MISC T/S CHANGES 1/22/82		3	11/22/83
51321	COOK 1 - FIRE PROTECTION SAFE SHUTDOWN CAPABILITY		2	11/22/83
04958	D.C. COOK-LOTIC-3 ANALYSIS		1	11/28/83
48699	COOK 1 FIRE PROTECTION CHANGES RE STANDARD T.S.		1	05/16/84
49542	COOK 1 - LICENSE/TECH SPEC NRC UPDATE		3	04/27/84
<u>ACTIVE ACTIONS</u>				
48291	COOK 1 - MISCELLANEOUS T.S. CHANGES DATED 3.29/82 ACP659		3	<u>TARGET</u> 03/30/84
51325	COOK 1 - ISI (NRC) REQUESTED SUPPLEMENT		2	04/15/84
49520	COOK 1 - HYDROGEN CONTROL FINAL RESOLUTION		2	05/15/84
43072	COOK 1 - STRUCTURAL ANALYSIS OF THE CONTAINMENTS ULTIMATE STRENGTH		1	05/15/84
43523	COOK 1 - EQUIPMENT SURVIVABILITY: HYDROGEN BURN		1	05/15/84
53514	COOK 1 RCS VENTS TECH SPEC			05/30/84
54385	COOK 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		2	09/30/84
54525	COOK 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		2	09/30/84
51602	COOK 1 - SURVEILLANCE REFERENCE TO 4.0.5		2	11/15/84
49701	COOK 1 - REACTOR COOLANT PUMP TRIP		2	02/01/85
<u>ANTICIPATED ACTIONS</u>				
53578	COOK 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		2	<u>INITIATION</u> 11/01/84
53661	COOK 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		2	11/01/84
53744	COOK 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		2	11/01/84
53827	COOK 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		2	11/01/84
53904	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT		2	11/01/84
53971	COOK 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		2	11/01/84
54054	COOK 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: COOK 2

PLANT LOCATION: 11 MI S OF BENTON HARBOR, MI  
 DOCKET NUMBER: 050-00316  
 ARCH/ENGINEER: AEP  
 IE INSPECTOR: E. SWANSON

LICENSED POWER: 3411 MWT  
 DESIGN POWER: 1100 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: D. WIGGINTON  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47165	COOK 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 11/17/83
42094	COOK 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS CAPABILITY TECH SPECS		2	11/23/83
42860	COOK 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	11/23/83
49728	COOK 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		2	11/23/83
12265	COOK 2 - MELB AND CONSEQUENTIAL SYSTEM FAILURE DC COOK 2		3	01/31/84
08480	COOK 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
<u>ACTIVE ACTIONS</u>				
52747	COOK 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		2	<u>TARGET</u>
52828	COOK 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		2	
52909	COOK 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		2	
52990	COOK 2-ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		2	
53065	COOK 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		2	
53118	COOK 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		2	
53168	COOK 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
11334	COOK 2 - INSERVICE TESTING OF PUMP AND VALVES- 10 CFR 50.55 A(G)		1	02/15/82
42461	COOK 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-2)		1	03/22/84
52218	COOK 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	09/30/85
51081	COOK 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	12/31/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47796	D. C. COOK 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 12/16/83
44225	COOK 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
46882	COOK 2 - II K2.13 THERMAL MECHANICAL REPORT		1	06/18/84
<u>ACTIVE ACTIONS</u>				
51151	COOK 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	<u>TARGET</u>
51231	COOK 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	
46297	COOK 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
44434	COOK 2 - NUREG-0737 II.B.3.2. POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/83
45127	COOK 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/83
45819	COOK 2 - NUREG-0737 II.K.3.3a, SB LOCA OUTLINE		1	06/30/83
48159	D.C. COOK 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/83
45938	COOK 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46009	COOK 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46081	COOK 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44295	PROCEDURE GENERATION PACKAGES (SUPP 1, 0737) OPERATOR GUIDELINES		1	11/30/84
<u>ANTICIPATED ACTIONS</u>				
44574	COOK 2 - RV AND SV TESTING		1	<u>INITIATION</u> 12/15/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

COOK 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12463	COOK 2 - CHECK VALVE TESTING - LIC COND 3(C)		1	<u>COMPLETE</u> 11/05/83
48004	COCK 2 - MISCEL. T/S CHANGES 1/22/82		2	11/22/83
51322	COOK 2 - FIRE PROTECTION SAFE SHUTDOWN CAPABILITY			11/22/83
48492	COOK 2 - STUDY FOR NOT CHANGING TECH SPEC LIMITS		2	01/10/84
48700	COOK 2 FIRE PROTECTION CHANGES RE STANDARD T.S.		2	03/16/84
49543	COOK 2 - LICENSE/TECH SPEC NRC UPDATE		3	04/27/84
52497	COOK 2 - CYCLE 4 CONDITIONS ON RELOAD		1	06/18/84
52498	COOK 2 - CYCLE 5 RELOAD		1	06/18/84
55061	COOK 2 - D.C. BATTERY OPERATION - EMER. T.S.		1	06/22/84
53515	COOK 2 - RCS VENTS TECH SPEC			06/28/84
<u>ACTIVE ACTIONS</u>				
43524	COOK 2 - EQUIPMENT SURVIVABILITY: HYDROGEN BURN		1	<u>TARGET</u> 02/15/84
12619	COOK 2 - PANEL & ELEC SWITCHGEAR QUAL-LIC. COND. 3(R) COND. 3(R)		1	03/22/84
48292	COOK 2 - MISCELLANEOUS T.S. CHANGES DATED 3/29/82 AEP659		2	03/30/84
51324	COOK 2 - ISI (NRC) REQUESTED SUPPLEMENT			04/15/84
54809	COOK 2 - SOURCE RANGE MONITOR			04/26/84
49519	COOK 2 - HYDROGEN CONTROL FINAL RESOLUTION		2	05/15/84
43071	COOK 2 - STRUCTURAL ANALYSIS OF THE CONTAINMENTS ULTIMATE STRENGTH		1	05/15/84
54386	COOK 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		2	09/30/84
54526	COOK 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
51603	COOK 2 - SURVEILLANCE REFERENCE TO 4.0.5			11/15/84
49702	COOK 2 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				
04997	COOK 2 - ELEC SYS (BARTONS) QUAL FOR STEAM BREAK/LOCA LOCA		1	<u>INITIATION</u> 09/22/84
53579	COOK 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		2	11/01/84
53662	COOK 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53745	COOK 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		2	11/01/84
53828	COOK 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		2	11/01/84
53905	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT		2	11/01/84
53972	COOK 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		2	11/01/84
54055	COOK 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES		2	11/01/84



DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: COOPER STATION

PLANT LOCATION: 23 MI S OF NEBRASKA CITY, NEB  
 DOCKET NUMBER: 050-00298  
 ARCH/ENGINEER: B&R  
 IE INSPECTOR: D. DUBOIS

LICENSED POWER: 2381 MWT  
 DESIGN POWER: 0778 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: E. SYSVESTER  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
07982	COOPER STATION-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	<u>COMPLETE</u> 10/31/83
07934	COOPER - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	01/20/84
12300	COOPER - HELB AND CONSEQUENTIAL SYSTEM FAILURE COOPER		3	02/02/84
<u>ACTIVE ACTIONS</u>				
51082	COOPER STATION - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52219	COOPER STATION - CONTROL / HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52748	COOPER STATION 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52829	COOPER STATION - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52910	COOPER STATION - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52991	COOPER STATION-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		3	05/30/84
08140	COOPER - APPENDIX I T. 1 SPEC IMPLEMENTATION REVIEW		1	06/30/84
11251	COOPER - INSERVICE TESTING (IST)		1	06/30/84
42418	COOPER - SINGLE LOOP OPERATION		1	06/30/84
42486	COOPER - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
42573	COOPER - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	06/30/84
46657	COOPER - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/30/84
51674	COOPER NUREG 0737 TECH SPECS			06/30/84
43729	COOPER - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	06/31/84
TAC #	TAC DESCRIPTION	IMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44430	COOPER - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>COMPLETE</u> 12/07/83
48155	COOPER -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/28/83
45814	COOPER - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
44221	COOPER - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
51152	COOPER STATION - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51232	COOPER STATION - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45589	COOPER - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	12/30/83
48265	COOPER - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	03/30/84
46293	COOPER - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/84
44291	COOPER - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	06/30/84
45123	COOPER - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
45934	COOPER - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46005	COOPER - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46077	COOPER - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
52009	COOPER - REQUEST FOR SCHEDULAR EXTENSION OF FINAL RULE ON ENVIRONMENTAL QUALIFICATION OF ELECTRICAL EQUIPMENT			<u>COMPLETE</u> 10/03/83

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

COOPER STATION

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS (CONTINUATION)</u>				
52609	COOPER - LICENSED OPERATOR STAFFING, 50.54(M)(2)			<u>COMPLETE</u> 11/28/83
52608	COOPER - LICENSED OPERATOR STAFFING, 50.54(M)(2)(I)			01/20/84
52013	COOPER - PROPOSED CHANNGE NO 5 TO TECHNICAL SPECIFICATIONS INSERVICE INSPECTION			01/20/84
52642	PROPOSED CHANGE NO 8 TO TECHNICAL SPECIFICATIONS ORGANIZATIONAL CHANGES			03/20/84
53322	COOPER - RESPONSE TO 10 CFR 50 APPENDIX R, FIRE PROTECTION OF SAFE SHUTDOWN CAPABILITY			04/16/84
54247	COOPER - PROPOSED CHANGE #12 - REVISED LICENSEE EVENT REPORT			06/01/84
<u>ACTIVE ACTIONS</u>				
54682	COOPER - 1ST PROGRAM - SECOND TEN YEAR INTERVAL			<u>TARGET</u>
55080	COOPER - SCHEDULAR EXEMPTION REQUEST TO APPENDIX R TO 10 CFR PART 50			
53554	COOPER - PROPOSED CHANGE NO 10 TO TECHNICAL SPECIFICATIONS MIS CHANGES			05/30/84
54430	COOPER NUCLEAR STATION - NUREG-0737 TS IN RESPONSE TO GENERIC LETTER 83-36 - CNS CHANGE NO. 11			06/29/84
55005	COOPER - PROPOSED CHANGE NO. 13 - MARK/CONTAINMENT MODIFICATION			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53580	COOPER STATION - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53663	COOPER STATION - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53746	COOPER STATION- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53829	COOPER STATION - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53973	COOPER STATION - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54056	COOPER STATION - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: CRYSTAL RIVER 3

PLANT LOCATION: 7 MI NW OF CRYSTAL RIVER, FLA  
 DOCKET NUMBER: 050-00302  
 ARCH/ENGINEER: GIL  
 IE INSPECTOR: T. STETKA

LICENSED POWER: 2544 MWT  
 DESIGN POWER: 0825 MWE  
 NSSS VENDOR: B&W

PROJECT MANAGER:  
 BRANCH CHIEF: J. STOLZ  
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
03843	CRYSTAL RIVER 3 - REVIEW OF ASYMMCTRIC LOCA LOADS SUBMITTAL		3	<u>COMPLETE</u> 01/11/84
12321	CRYSTAL RIVER 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE CRYSTAL RIVER 3		3	01/31/84
12743	CRYSTAL RIVER 3 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGE		2	04/19/84
08985	CRYSTAL RIVER 3 - APPENDIX I REVIEW		1	06/27/84
<u>ACTIVE ACTIONS</u>				
42512	CRYSTAL RIVER 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	<u>TARGET</u> 07/01/84
42121	CRYSTAL RIVER 3 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		2	07/30/84
04549	CRYSTAL RIVER 3 - REVIEW REQUEST FOR RELIEF FROM ASME SEC. XI PUMP & VALVE TESTING REQUIREMENTS		2	08/01/84
49729	-CRYSTAL RIVER 3 NUREG 0737 TECH SPECS (GENERIC LTR 82-16)		1	09/30/84
07983	CRYSTAL RIVER 3-CNTRL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	10/01/84
42902	CRYSTAL RIVER 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	10/01/84
52749	CRYSTAL RIVER 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		1	11/05/84
52830	CRYSTAL RIVER 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		1	11/05/84
52911	CRYSTAL RIVER 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		1	11/05/84
52992	CRYSTAL RIVER 3-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		1	11/05/84
53066	CRYSTAL RIVER 3-ITEM 4.1 RX TRIP SYSTEM RELIABILITY-VENDOR RELATED MODIFICATIONS		1	11/05/84
53119	CRYSTAL RIVER 3 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		1	11/05/84
53169	CRYSTAL RIVER 3 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			11/05/84
52220	CRYSTAL RIVER 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			12/01/84
47166	CRYSTAL RIVER 3 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	05/01/85
51083	CRYSTAL RIVER 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			02/28/86
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
45619	CRYSTAL RIVER 3 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES		1	<u>COMPLETE</u> 10/06/83
47929	CRYSTAL RIVER 3 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	11/01/83
44668	CRYSTAL RIVER 3 - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION		1	04/30/84
45268	CRYSTAL RIVER 3 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	05/28/84
45197	CRYSTAL RIVER 3 - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT		1	06/05/84
<u>ACTIVE ACTIONS</u>				
44431	CRYSTAL RIVER 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>TARGET</u> 07/01/84
44292	CRYSTAL RIVER 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	08/01/84
44571	CRYSTAL RIVER 3 - RV AND SV TESTING		1	08/30/84
45333	CRYSTAL RIVER 3 - NUREG-0737 II.K.3.3, REPORT ON RV/SV FAILURES		1	08/30/84
45935	CRYSTAL RIVER 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	08/30/84
46006	CRYSTAL RIVER 3 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	08/30/84
46078	CRYSTAL RIVER 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	08/30/84
46294	CRYSTAL RIVER 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## CRYSTAL RIVER 3

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
45815	CRYSTAL RIVER 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	<u>TARGET</u> 12/31/84
44222	CRYSTAL RIVER 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	01/27/85
51153	CRYSTAL RIVER 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			04/03/85
48156	CRYSTAL RIVER 3 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/01/85
45124	CRYSTAL RIVER 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/85
51233	CRYSTAL RIVER 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			07/01/85
<u>COMPLETED ACTIONS</u>				
52655	CRYSTAL RIVER, UNIT 3 - EVAL OFF-SITE FIRE PROTECTION ASSISTANCE			<u>COMPLETE</u> 12/01/83
49637	CRYSTAL RIVER 3 - EMERG. F. W. FLOW INSTRUMENT ULTRASONIC INDICATION TECH. SPEC. CHANGE		1	01/17/84
11946	CRYSTAL RIVER 3 - SYSTEM CYCLs		3	03/29/84
49498	CRYSTAL RIVER 3 - STATION BATTERY ELECTROLYTE SPECIFIC GRAVITY - LCO CHANGE IN T.S.		3	04/13/84
54372	CRYSTAL RIVER, UNIT 3/REVIEW FLORIDA POWER PROPOSED ALTERNATE OFF-SITE POWER SUPPLY			05/01/84
54175	CRYSTAL RIVER UNIT 3 - REVISE STEAM GEN OPERATING LEVEL LIMITS			05/25/84
49823	CRYSTAL RIVER 3 - SMALL BREAK LOCA PROCEDURES & MAINTAINING PROPER SG LEVEL		1	06/15/84
53248	CRYSTAL RIVER, UNIT 3 REVIEW SUPPLEMENTS 1 AND 2 TO TS CHANGE REQUEST 36 (RETS)			06/27/84
<u>ACTIVE ACTIONS</u>				
54798	CRYSTAL RIVER, UNIT 3 - CHANGE PUMP INSERVICE TESTING TO QUARTERLY			<u>TARGET</u> 06/15/84
54445	CRYSTAL RIVER, UNIT 3 - REVIEW TECH SEC AMENDMENT FOR DECAY HEAT REMOVAL SYSTEM			07/30/84
54892	CRYSTAL RIVER, UNIT 3 - TSCR 110 - ADMIN CONTROL OF CONT VALVES			08/01/84
48450	CRYSTAL RIVER 3 - PHYSICAL SECURITY PLAN REVISIONS		1	08/01/84
49440	CRYSTAL RIVER 3 - GENERIC LTR 82-17 EMERG. PLAN, AND 82-23 SECURITY PLAN - 12 MONTH AUDIT INTO T. S.		2	08/01/84
48472	CRYSTAL RIVER 3 - INTAKE CANAL ACCEPTANCE CRITERIA		3	09/01/84
48298	CRYSTAL RIVER 3 - REACTOR COOLANT PUMP SEAL DAMAGE		3	09/01/84
49988	CRYSTAL RIVER 3 - CHANGE TO HIGH RADIATION AREA TECH SPEC 6.12		1	09/01/84
55004	CRYSTAL RIVER, UNIT 3/TSCR #114 DELETE LER ADMIN REQMTS			09/01/84
54527	CRYSTAL RIVER 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54387	CRYSTAL RIVER 3 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
55176	CRYSTAL RIVER, UNIT 3 - TSCR NO. 120 - CHANGE TO SECTION 3.0.4 APPLICABILITY			10/01/84
48921	CRYSTAL RIVER 3 - IREQ REVIEW		2	10/01/84
12526	CRYSTAL RIVER 3 - CALIBRATION OF NUCLEAR INSTRUMENTATION CHANGE 53		3	11/21/84
53330	CRYSTAL RIVER, UNIT 3/TECHNICAL SPECIFICATION CHANGE - FUEL POOL ENRICHMENT LIMIT			12/01/84
53405	CRYSTAL RIVER, UNIT 3/REVIEW RADWASTE PROCESS CONTROL PROGRAM		1	12/01/84
52669	CRYSTAL RIVER, UNIT 3 - REVIEW FPC PROPOSAL FOR ON-LINE ES LOGIC TESTING		2	12/01/84
49668	CRYSTAL RIVER 3 - REACTOR COOLANT PUMP TRIP		3	02/01/85
42027	CRYSTAL RIVER 3 - STEAM GEN. RUPTURE MATRIX DISCONNECT FROM AFW VALVES		3	04/01/85
49059	CRYSTAL RIVER 3 - UPGRADE OF POWER AND INSTRUMENTATION FOR A FW SYSTEM BEYOND NUREG 0737		3	06/01/85
52039	CRYSTAL RIVER, UNIT 3 - DEGRADED GRID VOLTAGE PROT TS & DESIGN			08/01/85
<u>ANTICIPATED ACTIONS</u>				
53581	CRYSTAL RIVER 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53664	CRYSTAL RIVER 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53747	CRYSTAL RIVER- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53830	CRYSTAL RIVER 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84



TECHNICAL ASSIGNMENT CONTROL SYSTEM  
CONDENSED MANAGEMENT REPORT

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(CONTINUATION)

CRYSTAL RIVER 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53906	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			<u>INITIATION</u> 11/01/84
53953	CRYSTAL RIVER 3 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
53974	CRYSTAL RIVER 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54057	CRYSTAL RIVER 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: DAVIS-BESSE 1

PLANT LOCATION: 21 MI E OF TOLEDO, OH  
 DOCKET NUMBER: 050-00346  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: W. ROGERS

LICENSED POWER: 2772 MW  
 DESIGN POWER: 0906 MWE  
 NSSS VENDOR: B&W

PROJECT MANAGER: A. DEAGAZIO  
 BRANCH CHIEF: J. STOLZ  
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12322	DAVIS BESSE 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE DAVIS BESSE 1		3	<u>COMPLETE</u> 02/02/84
10986	DAVIS BESSE 1 - RPV SUPPORTS (ASYMETRIC LOCA LOAD)		3	04/09/84
11316	DAVIS BESSE 1 - INSERVICE TESTING PROGRAM		1	05/15/84
<u>ACTIVE ACTIONS</u>				
52750	DAVIS BESSE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52831	DAVIS BESSE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52912	DAVIS BESSE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52993	DAVIS BESSE 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53067	DAVIS BESSE 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53120	DAVIS BESSE 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53170	DAVIS BESSE 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
43038	DAVIS BESSE 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	05/15/84
49730	DAVIS BESSE 1 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"		2	05/30/84
42508	DAVIS BESSE 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
08141	DAVIS BESSE 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/30/84
11311	DAVIS BESSE 1 - MECHANICAL SHUBBERS		2	09/30/84
11312	DAVIS BESSE 1 - HYDRAULIC SHUBBERS		3	09/30/84
10993	DAVIS-BESSE 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	12/31/84
47182	DAVIS BESSE 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	12/31/84
52221	DAVIS BESSE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
42903	DAVIS BESSE 1 - MASONRY WAL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	09/30/85
51084	DAVIS BESSE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	12/31/85
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
44364	DAVIS BESSE - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 10/05/83
44669	DAVIS BESSE - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION		1	02/21/84
44751	DAVIS BESSE - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION		1	05/30/84
42964	DAVIS-BESSE - NUREG-0737, II.E.1.2.1, AFW SYSTEM SAFETY GRADE INITIATION		1	05/30/84
45198	DAVIS BESSE - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT		1	06/05/84
<u>ACTIVE ACTIONS</u>				
45315	DAVIS BESSE - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	<u>TARGET</u> 09/30/84
45936	DAVIS BE SE - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46007	DAVIS BESSE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46079	DAVIS BESSE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46295	DAVIS BESSE - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
44432	DAVIS BESSE - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	09/30/84
44572	DAVIS BESSE 1 - RV AND SV TESTING		1	09/30/84
45817	DAVIS BESSE - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/31/84
51234	DAVIS BESSE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	05/30/85

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DAVIS-BESSE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
45125	DAVIS BESSE - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	TARGET 06/30/85
44223	DAVIS BESSE - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	09/30/85
44293	DAVIS BESSE - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	09/30/85
51154	DAVIS BESSE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	09/30/85
48157	DAVIS-BESSE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85
<u>COMPLETED ACTIONS</u>				
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
46676	DAVIS-BESSE 1 - DELETION OF AFW PUMP TURBINE SPEED SWITCHES		3	COMPLETE 10/26/83
48831	DAVIS-BESSE DNB MARGIN ANALYSIS		3	11/03/83
51387	DAVIS BESSE 1 - FIRE PROTECTION TECH SPECS		2	01/03/84
53427	DB-1, TECH SPEC CHANGE ON CONTAINMENT RECIRC FANS			01/20/84
51964	DAVIS DEBBS 1 - AFW FLOW PATH VERIFICATION			02/21/84
53383	DAVIS BESSE 1 - REQUEST FOR SCHEDULE EXTENSION OF II.K.1.1 & II.F.1.2			02/27/84
47043	DAVIS-BESSE 1 - DOSE EQUIVALENT I-131		3	04/19/84
43513	DAVIS BESSE 1 - DIVERSE POWERED AFW PUMP		1	04/23/84
54353	DAVIS BESSE 1 - TECH SPECS FOR CONT SUMP LEVEL, CONT PRESSURE, HIGH PT VENTS			04/24/84
54681	DAVIS BESSE 1 - APPEAL RELATIVE TO II.E.1.1.			05/10/84
54969	DAVIS BESSE - PERFORMANCE IMPROVEMENTS PREM REVIEW			05/16/84
51738	DAVIS BESSE: STATION DIST. VOLTAGE VERIF. TEST			05/21/84
51388	DAVIS BESSE 1 - BAAT HEAT TRACE		3	05/22/84
52484	DAVIS BESSE 1 - ON SITE DISPOSAL, 20.302(A) REQUEST			05/23/84
51791	DAVIS BESSE 1 - AFW LOGIC & MANUAL INITIATION CIRCUIT SURVEILLANCE			05/30/84
54497	DAVIS BESSE 1 - REQUEST FOR REVISED SCHEDULE: EMERGENCY RESPONSE CAPABILITY			06/06/84
<u>ACTIVE ACTIONS</u>				
51792	DAVIS BESSE 1 - MSIV CYCLING REQUIREMENTS			TARGET 04/30/84
49481	DAVIS BESSE 1 - INCONSISTENCY BETWEEN TSS AND 50.54(T)		3	05/30/84
49482	DAVIS BESSE 1 - INCONSISTENCY BETWEEN TSS AND PART 73.40 (D)		3	05/30/84
53298	DAVIS BESSE 1 - NON-SAFETY GRADE AFW FLOW INDICATOR			05/30/84
53300	DAVIS BESSE 1 - EXEMPTION REQUEST PART 50.54 (M)(LLL)			05/30/84
46677	DAVIS-BESSE 1 - EMERGENCY DIESEL GENERATOR SURVEILLANCE REQUIREMENTS CHANGE		3	05/30/84
54261	DAVIS BESSE 1 - CYCLE 4 COASTDOWN			05/30/84
52462	DAVIS BESSE 1 - EXEMPTION REQUEST - FIRE PROTECTION			05/30/84
54260	DAVIS BESSE 1 - DELETE SFAS TO CONTAINMENT SAMPLE VALVES			06/30/84
47040	DAVIS BESSE 1 - STEAM GENERATOR CONTAINMENT ISOLATION VALVES		3	06/30/84
43253	DAVIS BESSE 1 - REVIEW OF SFAS LEVEL 5 SINGLE FAILURE CRITERIA		3	06/30/84
53296	DAVIS BESSE 1 - ACTION FOR MISSED SURVEILLANCE TESTS			06/30/84
53297	DAVIS BESSE 1 - DEENERGIZE DHR SYSTEM VALVES			06/30/84
52415	DAVIS BESSE 1 - INCORE DETECTOR OPERABILITY			06/30/84
52416	DAVIS BESSE 1 - DHR OPERABILITY			06/30/84
52418	DAVIS BESSE 1 - ENVIRONMENTAL PROTECTION PLAN			06/30/84
52419	DAVIS BESSE 1 - CONTAINMENT ISOLATION VALVE			06/30/84
52420	DAVIS BESSE 1 - REVIEW ENVIRONMENTAL SURVEILLANCE RESULTS			06/30/84
43333	DAVIS BESSE 1 - ORGANIZATIONAL CHANGE NO. 2		2	07/30/84
52417	DAVIS BESSE 1 - CONT. PURGE/EXHAUST VALVE LEAK RATE			09/30/84
54259	DAVIS BESSE 1 - FIRE PROTECTION T.S.S.			09/30/84
54279	DAVIS BESSE 1 - DELETE POST ACCIDENT MONITORING REQUIREMENT FROM CRS			09/30/84
54388	DAVIS BESSE 1 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DAVIS-BESSE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
54528	DAVIS BESSE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			<u>TARGET</u> 09/30/84
49821	DAVIS BESSE 1 - SMALL BREAK LOCA PROCEDURES AND MAINTAINING PROPER SG LEVEL		2	12/31/84
49943	DAVIS BESSE 1 - MULTI-YEAR INTEGRATED SCHEDULE		1	12/31/84
49662	DAVIS BESSE 1 - REACTOR COOLANT PUMP TRIP		2	02/01/85
<u>ANTICIPATED ACTIONS</u>				
53582	DAVIS BESSE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53665	DAVIS BESSE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53748	DAVIS BESSE 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53831	DAVIS BESSE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53907	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53954	DAVIS BESSE 1 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
53975	DAVIS BESSE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54058	DAVIS BESSE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: DIABLO CANYON 1

PLANT LOCATION: 12 MI WSW OF SAN LUIS OBISPO  
 DOCKET NUMBER: 050-00275  
 ARCH/ENGINEER: PG&E  
 IE INSPECTOR: T. YOUNG

LICENSED POWER: 000 MWT  
 DESIGN POWER: 1084 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: H. SCHIERLING  
 BRANCH CHIEF: G. KNIGHTON  
 LIC. ASSISTANT: J. LEE

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
12131	DIABLO CANYON 1, EMERGENCY PLAN REVIEW		1	<u>TARGET</u>
52222	DIABLO CANYON 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52375	DIABLO CANYON - CONTROL OF HEAVY LOADS (PHASE I)			
52751	DIABLO CANYON 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52832	DIABLO CANYON 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52913	DIABLO CANYON 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52994	DIABLO CANYON 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53068	DIABLO CANYON 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53121	DIABLO CANYON 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53171	DIABLO CANYON 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
49424	DIABLO CANYON 1 - BLOCKING SI SIGNAL DURING COOLDOWN			03/31/83
51994	DIABLO CANYON NATURAL CIRCULATION COOLDOWN			07/30/83
<u>ACTIVE ACTIONS</u>				
52154	DIABLO CANYON UNIT 1 - CONTROL ROOM DESIGN REVIEW (API.D.1)			
52155	DIABLO CANYON UNIT 1 - SAFETY PARAMETER DISPLAY SYSTEM (API.D.2)			
51590	DIABLO CANYON RV AND SV TESTING (II.D.1)			
49111	DIABLO CANYON 1 - NUREG-0737 II.K.3.5, AUTO TRIP OF RCPS			
54986	DIABLO CANYON 1, II.K.2.13, THERMAL MECHANICAL REPORT			06/30/84
<u>ACTIVE ACTIONS</u>				
51633	DIABLO CANYON UNIT 1 PROPOSED MODS TO SECTIONS 3.7.9.1 & 4.7.9.4 OF TECH SPECS DEALING WITH FIRE PROTECTION			
51634	DIABLO CANYON UNIT 1 PROPOSED MODS TO TABLE 3.3-11 OF TECH SPECS			
52310	DIABLO CANYON UNIT 1 - TECH SPEC CHG RE INSTALLING NEW FERSUATION PUMP & THE NEED TO REPLACE HALON INIT EVERY 5 YRS			
48045	DIABLO CANYON 1 - DESIGN VERIFICATION PROGRAM			
51632	DIABLO CANYON UNIT 1 EMERGENCY SUPPORT FACILITY IMPLEMENTATION DATE GL-82-33			
51635	DIABLO CANYON UNIT 1 INSERVICE INSPECTION & TESTING PROGRAM			
51636	DIABLO CANYON UNIT 1 REPLACEMENT OF INVERTER TECH SPEC CHANGE			
51637	DIABLO CANYON UNIT 1 EMERGENCY PREPAREDNESS CHANGE FROM 24 MOS TO 12 MOS B) SAFEGUARD AUDTI 24MOS TO 12 MOS			
52309	DIABLO CANYON UNIT 1 - CONTAINMENT SPRAY TIMING			
53210	DIABLO CANYON UNIT 1 ALLEGATIONS			
48467	DIABLO CANYON UNIT 1 - PROPOSED ORGANIZATIONAL CHANGES TO SECTIONS 6.2 & 6.3 OF TECH. SPECS.			06/18/82
51008	DIABLO CANYON UNITS 1 - REQUESTED VARIANCE FROM STANDARD W COLD LICENSE PROGRAM			04/15/83
49935	DIABLO CANYON UNIT 1 - SER ON COMPONENT COOLING WATER SYSTEM			05/15/83
51583	DIABLO CANYON UNIT 1 - PROPOSED MODS TO TABLE 3.6-1 OF TECH. SPECS. DEALING WITH CONTAINMENT ISOLATION VALVES			05/31/83

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DIABLO CANYON 1

<u>TAC #</u>	<u>IAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>		<u>(CONTINUATION)</u>		<u>TARGET</u>
51633	DIABLO CANYON UNIT 1 NATURAL CIRCULATION BORAN MIXING TEST			09/30/83
52060	DIABLO CANYON UNIT 1 - APPENDIX R REVIEW & PREPARATION OF SSER			09/30/83
52622	DIABLO CANYON 1 - STRUCTURAL ALLEGATIONS			11/30/83
52623	DIABLO CANYON 1 - RHR ALEGATIONS			11/30/83
53246	DIABLO CANYON UNIT 1 - SYSTEMS INTERACTION REPORT			12/09/83
54492	DIABLO CANYON 1 - POST FUEL - LOADING INITIAL TEST PROGRAM			03/28/84
54435	DIABLO CANYON - TECH SPEC UPDATE			03/30/84
54885	DIABLO CANYON 1 - PIPING AND SUPPORTS EVALUATION			06/30/84
54434	DIABLO CANYON - RESOLUTION OF ALLEGATIONS INCLUDING SSER PREPARATION			06/30/84
49654	DIABLO CANYON 1 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53583	DIABLO CANYON 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53666	DIABLO CANYON 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53749	DIABLO CANYON 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53832	DIABLO CANYON 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53908	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53976	DIABLO CANYON 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54059	DIABLO CANYON 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: DRESDEN 1

PLANT LOCATION: 9 MI E OF MORRIS, ILL  
 DOCKET NUMBER: 050-00010  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: T. TONGUE

LICENSED POWER: 0700 MWT  
 DESIGN POWER: 0200 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. GILBERT  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

TAC # TAC DESCRIPTIONMULTI-PLANTACTIVE ACTIONS

07985 DRESDEN 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)

PRIORITYCRITICAL DATE

2

TARGET

06/11/79

TAC # TAC DESCRIPTIONPLANT SPECIFICCOMPLETED ACTIONS

54301 DRESDEN 1 - CONTROL OF RAD. CONT. MATERIALS - DE MINIMUS LEVELS

PRIORITYCRITICAL DATECOMPLETE

04/13/84

ACTIVE ACTIONS

54768 DRESDEN 1 - REVIEW OF MOBILE VOLUME REDUCTION SYSTEM

TARGET

04/30/85

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## CONDENSED MANAGEMENT REPORT

FACILITY: DRESDEN 2

PLANT LOCATION: 9 MI E OF MORRIS, ILL  
 DOCKET NUMBER: 050-00237  
 ARCH/ENGINEER: S&L  
 IE INSPECTOR: T. TONGUE

LICENSED POWER: 2527 MW  
 DESIGN POWER: 0794 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. GILBERT  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08552	DRESDEN 2 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING		1	<u>COMPLETE</u> 11/01/83
51675	DRESDEN 2 NUREG 0737 TECH SPECS			03/19/84
46658	DRESDEN 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/26/84
<u>ACTIVE ACTIONS</u>				
51085	DRESDEN 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52752	DRESDEN 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52833	DRESDEN 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52914	DRESDEN 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52995	DRESDEN 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH SPECS - RTS COMPONENTS			
42519	DRESDEN 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	04/30/84
02110	DRESDEN 2 - APPENDIX I		3	06/30/84
42603	DRESDEN 2 - LONG TERM REV CONTAINMENT PURGE AND VENT (B-24)		1	07/30/84
43719	DRESDEN 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	07/31/84
42909	DRESDEN 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/31/84
07935	DRESDEN 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	12/31/84
52223	DRESDEN 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44368	DRESDEN 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 10/03/83
45624	DRESDEN 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES		1	12/14/83
45821	DRESDEN 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/23/84
45907	DRESDEN 2 - NUREG-0737 II.K.3.45, MANUAL DEPRESSURIZATION		1	01/23/84
48160	DRESDEN 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	01/23/84
44227	DRESDEN 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
51155	DRESDEN 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51235	DRESDEN 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44297	DRESDEN 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	04/30/83
48266	DRESDEN 2 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	09/30/83
44156	DRESDEN 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	02/28/84
44436	DRESDEN 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/28/84
44506	DRESDEN 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	02/28/84
45129	DRESDEN 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	04/30/84
45591	DRESDEN 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	04/30/84
45687	DRESDEN 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/30/84
45940	DRESDEN 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46011	DRESDEN 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46083	DRESDEN 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46299	DRESDEN 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DRESDEN 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
51803	DRESDEN 2 - TECH SPEC CHANGES FOR MECHANICAL HYDRAULIC SNUBBERS			10/21/83
49364	DRESDEN UNIT 2 - HIGH ENERGY PIPE BREAK			10/27/83
51518	DRESDEN 2 - LIMIT REQUIREMENT TO VERIFY NUCLEAR RESPONSE TO CRD MOTION			11/03/83
07644	DRESDEN 2 - CONTROL ROD COUPLING VERIFICATION		3	11/03/83
52678	DRESDEN 2 - MID CYCLE PIPE CRACK INSPECTION FOR IGSCC CHANGES			11/29/83
51523	DRESDEN 2 - NUREG 0737 - ITEM II.K.3.15 - ISOLATION OF HPCI AND RCIC			12/12/83
51545	DRESDEN 2 - CONTROL ROD DRIVE SCRAM TIME TESTING TECHNICAL SPECIFICATIONS			12/12/83
52360	DRESDEN 2 - PROPOSED T.S. RE RPS POWER MONITORING SYSTEM			02/14/84
52677	DRESDEN 2 - ASSESSMENT OF CECO PROGRAM TO CORRECT DEFICIENCIES WHICH LED TO APPENDIX J EXEMPTIONS			02/24/84
52316	DRESDEN 2 - T.S. AMENDMENT RE S/R VALVE POSITION INDICATION			02/24/84
54302	DRESDEN 2 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
47033	DRESDEN 2 - ISI PROGRAM, SECOND 10 YEARS		1	04/26/84
48858	DRESDEN 2 - REVIEW CECO POSITION RE 10CFR50.44			05/22/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52326	DRESDEN 2 - REVIEW OF REVISED LICENSED OPERATOR REQUALIFICATION TOPICAL REPORT			10/28/83
49361	DRESDEN UNIT 2 - DESIGN CODES, LOAD COMBINATION			02/28/84
49363	DRESDEN UNIT 2 - CAPABILITY WITHSTAND WIND AND TORNAOES (INCLUDING MISSILES)			02/28/84
48258	DRESDEN 2 - PROPOSED TS CHANGE RE SAFETY/RELIEF VALVE POSITION INDICATION		3	02/28/84
49367	DRESDEN UNIT 2 - LEAK DETECTION			02/28/84
49370	DRESDEN UNIT 2 - REACTOR COOLANT IODOINE ACTIVITY LIMITS			02/28/84
51516	DRESDEN 2 - ECONOMIC GENERATION CONTROL			03/15/84
49366	DRESDEN UNIT 2 - THERMAL OVERLOADS			03/15/84
51066	DRESDEN UNITS 2 - MULTIPLE CHANGES TO ADMINISTRATIVE CONTROLS			03/31/84
52141	DRESDEN 2 - POL -> FTOL CONVERSION - ENVIRONMENTAL EVALUATION			03/31/84
49368	DRESDEN UNIT 2 - SHORT CIRCUIT CALCUALTIONS & BREAKER COORDINATION			04/30/84
49369	DRESDEN UNIT 2 - ELECTRICAL ISOLATION OF RPS			04/30/84
52436	DRESDEN 2 - INTEGRATED ASSESSMENT SUPPLEMENT			05/31/84
54469	DRESDEN 2 - CHANGES TO TABLE 3.7.1, CONTAINMENT ISOLATION VALVES			05/31/84
49365	DRESDEN UNIT 2 - SEISMIC QUALIFICATION			06/15/84
54662	DRESDEN 2 - UPDATE OF SNUBBER TABLES			06/29/84
54664	DRESDEN 2 - REVISION OF ENTIRE TECHNICAL SPECIFICATIONS			06/29/84
48299	DRESDEN 2 - PROPOSED TS CHANGES RE APPENDICES H AND G, METAL SURVEILLANCE & MIN. TEMP. REQ		3	06/30/84
49083	DRESDEN 2 - IST PROGRAM, SECOND 10 YRS			06/30/84
54529	DRESDEN 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
11222	DRESDEN 2 - CONVERSION POL TO FTOL		3	11/30/84
55171	DRESDEN 2: FIRE PROTECTION ISSUES			12/31/84
54769	DRESDEN 2 - REVIEW OF MOBILE VOLUME REDUCTION SYSTEM			04/30/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53584	DRESDEN 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53667	DRESDEN 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53750	DRESDEN 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53833	DRESDEN 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53977	DRESDEN 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54060	DRESDEN 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: DRESDEN 3

PLANT LOCATION: 9 MI E OF MORRIS, ILL  
 DOCKET NUMBER: 050-00249  
 ARCH/ENGINEER: S&L  
 IE INSPECTOR: J. TONGUE

LICENSED POWER: 2527 MWT  
 DESIGN POWER: 0794 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. GILBERT  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08483	DRESDEN 3 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING		1	COMPLETE 11/01/83
12301	DRESDEN 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE DRESDEN 3		3	01/31/84
51676	DRESDEN 3 NUREG 0737 TECH SPECS			03/19/84
46659	DRESDEN 3 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/26/84
<u>ACTIVE ACTIONS</u>				
51086	DRESDEN 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			TARGET
52753	DRESDEN 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52834	DRESDEN 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52915	DRESDEN 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52996	DRESDEN 3-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
42487	DRESDEN 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	01/30/84
08111	DRESDEN 3 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/30/84
42574	DRESDEN 3 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	07/30/84
43722	DRESDEN 3 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	07/31/84
07936	DRESDEN 3 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	07/31/84
42879	DRESDEN 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/31/84
52224	DRESDEN 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44369	DRESDEN 3 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	COMPLETE 10/03/83
45625	DRESDEN 3 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES		1	12/14/83
45822	DRESDEN 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/23/84
45908	DRESDEN 3 - NUREG-0737 II.K.3.45, MANUAL DEPRESSURIZATION		1	01/23/84
48161	DRESDEN 3 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	01/23/84
44228	DRESDEN 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
51156	DRESDEN 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			TARGET
51236	DRESDEN 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44298	DRESDEN 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	04/30/83
48267	DRESDEN 3 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	09/30/83
44437	DRESDEN 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/28/84
44157	DRESDEN 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	02/28/84
45130	DRESDEN 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	04/30/84
45592	DRESDEN 3 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	04/30/84
45688	DRESDEN 3 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/30/84
45941	DRESDEN 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46012	DRESDEN 3 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46084	DRESDEN 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46300	DRESDEN 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DRESDEN 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
51781	DRESDEN 3 - TECH SPEC CHANGES FOR MECHANICAL AND HYDRAULIC SNUBBERS			10/21/83
51519	DRESDEN 3 - LIMIT REQUIREMENT TO VERIFY NUCLEAR RESPONSE TO CRD MOTION			11/03/83
03536	DRESDEN 3 - CONTROL ROD COUPLING VERIFICATION		1	11/03/83
51524	DRESDEN 3 - NUREG 0737 - ITEM III.K.3.15 - ISOLATION OF HPCI AND RCIC			12/12/83
51546	DRESDEN 3 - CONTROL ROD DRIVE SCRAM TESTING TECHNICAL SPECIFICATIONS			12/12/83
52177	DRESDEN 3 - TECHNICAL SPECIFICATION CHANGE CONCERNING SAFETY/RELIEF VALVE SETPOINTS			01/04/84
52361	DRESDEN 3 - PROPOSED T.S. RE RPS POWER MONITORING SYSTEM			02/14/84
52317	DRESDEN 3 - T.S. AMENDMENT RE S/R VALVE POSITION INDICATION			02/24/84
52034	DRESDEN 3 - TS AMENDMENT TO ALLOW USE OF ASEA-ATOM CRD BLADES FOR CYCLE 9 AND BEYOND			03/09/84
52176	DRESDEN 3 - CYCLE 9 RELOAD REVIEW			03/09/84
48321	DRESDEN 3 - ANALYSIS FOR REDUCED FLOW OPERATION		2	03/09/84
52679	DRESDEN 3 - CYCLE 9 RELOAD IGSCC PIPE CRACK INSPECTION			03/15/84
54303	DRESDEN 3 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
47095	DRESDEN 3- ISI PROGRAM, SECOND 10 YRS		1	04/26/84
48057	DRESDEN 3 - REVIEW CECO POSITION RE 10CFR50.44			05/22/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54493	DRESDEN 3 - T/S CHANGE RE EXTENSION OF CERTAIN MAPLHGR CURVES			
51517	DRESDEN 3 - ECONOMIC GENERATION CONTROL			
48259	DRESDEN 3 - PROPOSED TS CHANGE RE SAFETY/RELIEF VALVE POSITION INDICATION		3	02/28/84
54151	DRESDEN 3 - REVIEW OF REVISED LICENSED OPERATOR REQUALIFICATION TOPICAL REPORT			03/30/84
51067	DRESDEN UNIT 3- MULTIPLE CHANGES TO ADMINISTRATIVE CONTROLS			03/31/84
54770	DRESDEN 3 - REVIEW OF MOBILE VOLUME REDUCTION SYSTEM			04/30/84
54470	DRESDEN 3 - CHANGES TO TABLE 3.7.1, CONTAINMENT ISOLATION VALVES			05/31/84
54239	DRESDEN 3 - T.S. CHANGE FOR SEP TOPIC VI-7.C.1-125/250 V BATTERIES			05/31/84
54240	DRESDEN 3 - T.S. CHANGE FOR SEP TOPIC XV-16-PRIMARY COOLANT ACTIVITY			05/31/84
54661	DRESDEN 3 - INSTALLATION OF BYPASS VALVE IN RWCU SYSTEM			06/29/84
54663	DRESDEN 3 - UPDATE OF SNUBBER TABLES			06/29/84
54665	DRESDEN 3 - REVISION OF ENTIRE TECHNICAL SPECIFICATIONS			06/29/84
48300	DRESDEN 3 - PROPOSED TS CHANGES RE APPENDICES H AND G, METAL SURVEILLANCE AND MIN TEMP REQ		3	06/30/84
47096	DRESDEN 3 - IST PROGRAM, SECOND 10 YRS		1	06/30/84
54530	DRESDEN 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
55172	DRESDEN 3: FIRE PROTECTION ISSUES			12/31/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53585	DRESDEN 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53668	DRESDEN 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53751	DRESDEN 3- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53834	DRESDEN 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53978	DRESDEN 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54061	DRESDEN 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: DUANE ARNOLD

PLANT LOCATION: 8 MI NW OF CEDAR RAPIDS, IA  
 DOCKET NUMBER: 050-00331  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: G. WRIGHT

LICENSED POWER: 1658 MWT  
 DESIGN POWER: 0538 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: M. THADANI  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
11035	DUANE ARNOLD - LEAK RATE TESTING		2	<u>COMPLETE</u> 01/17/84
12302	DUANE ARNOLD - HELB AND CONSEQUENTIAL SYSTEM FAILURE DUANE ARNOLD		3	02/02/84
51677	DUANE ARNOLD NUREG 0737 TECH SPECS			02/27/84
42575	DUANE ARNOLD - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	03/29/84
07988	DUANE ARNOLD CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/12/84
46660	DUANE ARNOLD - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/15/84
<u>ACTIVE ACTIONS</u>				
52225	DUANE ARNOLD - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52754	DUANE ARNOLD - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52835	DUANE ARNOLD - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52916	DUANE ARNOLD - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52997	DUANE ARNOLD-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
51037	DUANE ARNOLD - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			03/01/84
07937	DUANE ARNOLD - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	05/31/84
42680	DUANE ARNOLD - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/08/84
42488	DUANE ARNOLD - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	08/10/84
43730	DUANE ARNOLD - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		1	08/31/84
08142	DUANE ARNOLD - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	09/15/84
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
44578	DUANE ARNOLD - RV AND SV TESTING		1	<u>COMPLETE</u> 11/04/83
48162	DUANE ARNOLD -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/29/83
45823	DUANE ARNOLD - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/29/83
47855	DUANE ARNOLD - NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS		1	01/13/84
48268	DUANE ARNOLD - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	04/11/84
51237	DUANE ARNOLD - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			05/08/84
45689	DUANE ARNOLD - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/06/84
<u>ACTIVE ACTIONS</u>				
51157	DUANE ARNOLD - I.D.1 CONTROL ROOM DESIGN REVIEW			
44299	DUANE ARNOLD - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83
44847	DUANE ARNOLD - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	03/31/84
45593	DUANE ARNOLD - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	03/31/84
45131	DUANE ARNOLD - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	05/14/84
44229	DUANE ARNOLD - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	05/14/84
44438	DUANE ARNOLD - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	05/31/84
46301	DUANE ARNOLD - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	08/01/84
45942	DUANE ARNOLD - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46013	DUANE ARNOLD - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46085	DUANE ARNOLD - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

DUANE ARNOLD

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
41062	DUANE ARNOLD - DC POWER SUPPLY FAILURE EFFECT ECCS		1	<u>COMPLETE</u> 10/24/83
51863	DAEC - ORDER ON LONG TERM SDV MODS			12/05/83
49452	DAEC - ADDITIONAL APP R EXEMPTION REQUEST			12/19/83
51764	DAEC-MOD OF TURBINE CONTROL VLV OPENING (RTS-151)			01/23/84
54172	DAEC - TECH. SPEC. AMENDMENTS FOR NUREG-0737 RTS-157			02/09/84
47427	DAEC - ADMINISTRATIVE T.S. CHANGES (RTS-130)		3	02/14/84
53254	DAEC - REVIEW OF 10/28/83 REACTOR SCRAM			02/28/84
06395	DUANE ARNOLD - VERIFICATION OF SENSOR RESPONSE TIME		1	03/08/84
49334	DAEC - ADMIN CHANGES CONCERNING SAFETY COMM, SECURITY PLAN, ETC., RTS - 145		3	03/13/84
52082	DAEC - NUREG 0737 II.K.3.3 SRV CHALLENGE & FAILURE OR MALFUNCTION REPORTS (RTS-153)			03/20/84
52083	DAEC - NUREG-0737-II.K.3.13B & II.K.3.22 RCIC RESET & RCIC SUCTION (RTS-154)			04/13/84
54312	DAEC - RTS-163 TO INCORPORATE ADDITIONAL MAPLHGR LIMITS			05/14/84
54232	DAEC - TECH. SPEC. CHANGES - MINIMUM STAFFING REQUIREMENTS			05/22/84
48521	DAEC - REVIEW & EVAL OF PROPOSED CHGS CONCERNING CONTAINMENT VENT & PURGE SYS		1	05/22/84
54171	DAEC - SDV VALVE TEST/RTS-158			06/01/84
52205	DAEC SURVEILLANCE REQUIREMENTS FOR THE LOW-LOW SET LOGIC FUNCTION (RTS-155)			06/25/84
53474	DAEC REVIEW OF TS AMENDMENT RTS 143			06/29/84
<u>ACTIVE ACTIONS</u>				
52084	DAEC - ANALYSIS OF REDUCED RHR SERVICE WTR FLOW (RTS-152)			<u>TARGET</u>
51861	DAEC - CONTAINMENT PURGE & VENT VALVE OPERABILITY			
47837	DUANE ARNOLD ENERGY CENTER - APP FOR 50% POWER OPERATION WITH SINGLE RECIRC LOOP		3	
54173	DAEC - REVISION TO ENVIRONMENTAL TECH. SPECS. - ETS-30			
54227	DAEC - RTS-157 FOR II.B.3			
54228	DAEC RTS-157 CHANGES FOR II.F.1-1 AND II.F.1-2			
54229	DAEC RTS-157 CHANGES FOR II.F.1.3			
54230	DAEC RTS-157 CHANGES FOR II.F.1-4 THROUGH 6			
54231	DAEC RTS-157 CHANGES FOR III.D.3.4			
54679	DUANE ARNOLD - MATERIALS OF INDETERMINATE QUALITY - TIA			
54855	DAEC - RTS 160 REPORTING REQUIREMENTS			
54429	DAEC (DUANE ARNOLD) - INSERVICE INSPECTION/TESTING			08/12/84
48230	DUANE ARNOLD - SCHEDULAR EXEMPTION RE H2 RECOMBINERS		1	11/05/84
<u>ANTICIPATED ACTIONS</u>				
53586	DUANE ARNOLD - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53669	DUANE ARNOLD - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53752	DUANE ARNOLD- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53835	DUANE ARNOLD - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53970	DUANE ARNOLD - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54062	DUANE ARNOLD - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: FARLEY 1

PLANT LOCATION: 28 MI SE OF DOTHAN, ALA  
 DOCKET NUMBER: 050-00348  
 ARCH/ENGINEER: SSI  
 IE INSPECTOR: W. BRADFORD

LICENSED POWER: 2652 MWT  
 DESIGN POWER: 0829 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: E. REEVES  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47172	FARLEY 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 11/15/83
49418	FARLEY 1 - BLOCKING SI SIGNAL DURING COOLDOWN		2	01/09/84
12266	FARLEY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE JOSEPH M.		3	01/31/84
49731	FARLEY 1 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		2	05/07/84
<u>ACTIVE ACTIONS</u>				
52755	FARLEY 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52836	FARLEY 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52917	FARLEY 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
52998	FARLEY 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53069	FARLEY 1 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53122	FARLEY 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53172	FARLEY 1 - ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
48569	FARLEY 1 - REQUEST FOR 5 EXEMPTIONS FOR IMPLEMENTATION DATES FOR FIRE PROTECTION MODIFICATIONS		2	06/30/84
42861	FARLEY 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
51088	FARLEY 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	07/30/84
42462	FARLEY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	08/30/84
07989	FARLEY 1 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	09/30/84
52226	FARLEY 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		3	09/30/85
<u>TMI ACTIONS</u>				
TAC #	TAC DESCRIPTION		PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44371	FARLEY 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 11/07/83
47936	FARLEY 1 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	12/15/83
47797	FARLEY 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/10/84
46886	FARLEY 1 - II K2.13 THERMAL MECHANICAL REPORT		1	06/18/84
<u>ACTIVE ACTIONS</u>				
46302	FARLEY 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u>
45824	FARLEY 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
44439	FARLEY 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
44579	FARLEY 1 - RV AND SV TESTING		1	07/30/84
51158	FARLEY 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	08/30/84
51238	FARLEY 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		2	08/30/84
48163	FARLEY 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
45132	FARLEY 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
45943	FARLEY 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46014	FARLEY 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46086	FARLEY 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FARLEY 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49184	FARLEY 1 - CORPORATE ORGANIZATIONAL CHANGE AND OTHER ADMIN CHANGES		2	COMPLETE 10/14/83
52499	FARLEY 1 - SUMP MONITOR INOPERABLE		1	10/14/83
51999	FARLEY 1 - ADMINISTRATIVE TECHNICAL SPECIFICATION CHANGES		2	10/14/83
52105	FARLEY 1 - REQUEST FOR EXTENSION OF EQ FOR RG1.97 ITEMS		1	10/21/83
51437	FARLEY 1 - ADD BACKUP MET TOWER TO TECH SPEC TABLES 3.3-8 AND 4.3.5		2	10/21/83
51891	FARLEY 1 - ONE TIME EXTENSION OF VISUAL INSPECTION OF TWO HYDRAULIC SNUBBERS		1	10/31/83
52562	FARLEY 1 - ONE-TIME ACCEPTANCE OF CORE FLUXMAP OF 10/13/83 (TS 3.3.3.2A)		1	11/01/83
51436	FARLEY 1 - COMPLETION OF GENERIC LETTER 83-07 (WASTE POLICY ACT 1982) ACTION		2	12/15/83
52565	FARLEY 1 - TECH SPEC CHANGES FOR POS MTC OPERATION		1	12/30/83
52042	FARLEY 1 - T.S. CHANGE FOR SHIFT MANNING (GEN LTR 82-02 AND 82-12)		2	12/30/83
53268	FARLEY 1 - FIRE PROTECTION EXEMPTION FOR SEC III.G.2 IN CONTAINMENT		1	12/30/83
49068	FARLEY 1 - CHANGE TS 6.5.2.8.E PER GENERIC LETTER 82-17		2	01/06/84
52015	FARLEY 1 - 1ST SER CHANGES PLUS ONE RELIEF REEVALUATION		2	01/26/84
52449	FARLEY 1 - REDESIGNATE PRIMARY PRESSURE ISOLATION VALVES IN 1ST PROGRAM		2	01/26/84
52539	FARLEY 1 - ADD TURBINE OVERSPEED PROTECTIONS TS 3/4.3.4		1	01/27/84
52181	FARLEY 1 - FIRE SYSTEM TECH SPEC CHANGE (3 ADMINS)		2	02/09/84
49547	FARLEY 1 - REQUEST FOR ADDITIONAL RELIEF FROM ISI REQUIREMENTS		2	02/10/84
52618	FARLEY 1 - CHANGE ACTION STATEMENTS FOR (LCO 3.1.3.1) CONTROL RODS		2	02/17/84
52298	FARLEY 1 - CORRECT MECHANICAL AND HYDRAULIC SNUBBER LISTS		2	02/28/84
51537	FARLEY 1 - REACTOR TRIP BREAKERS (DS-416)		2	03/19/84
54471	FARLEY 1 - RELIEF FROM ASME CODE HYDRO ON SG REDUCER WELDS		1	03/30/84
54233	FARLEY 1 - EXTENSION REQUEST POST ACCIDENT EQUIPMENT PER 50.49		1	04/16/84
49374	FARLEY 1 - REVISE DG OVERLOAD TEST AND DELETE RIVER WATER SYSTEM TECH SPECS		2	06/07/84
<u>ACTIVE ACTIONS</u>				
49953	FARLEY 1 - CHANGES TO CHARCOAL FILTER TECH SPECS (3/4.7.7, 3/4.7.8 AND 3/4.9.14)		2	TARGET 06/09/84
54237	FARLEY 1 - CHANGE INOPERABLE CONTROL ROD TIME TO 36 HOURS		2	06/25/84
52020	FARLEY 1 - UV CONFIRMATORY LIFE TEST PROGRAM (DS-416 BREAKER)		2	06/30/84
51629	FARLEY 1 - REVISE TECH SPECS FOR BATTERIES PER IEEE 450-1980		2	07/02/84
54450	FARLEY 1 - REVISE MET TOWER TECH SPECS		2	07/08/84
54235	FARLEY 1 - REPORTING REQUIREMENTS TS PER GEN LTR 83-43		2	08/06/84
54452	FARLEY 1 - ADMIN TECH SPEC CHANGES		2	09/01/84
53468	FARLEY 1 - EXEMPTION FOR USE OF RADIOIODINE CANISTER		2	09/09/84
54350	FARLEY 1-TECH SPECS FOR PURGE/VENT SYSTEM		2	09/15/84
54389	FARLEY 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 075 ITEM II.B.1			09/30/84
54531	FARLEY 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		2	09/30/84
54672	FARLEY 1 - TECH. SPEC. CHANGES FOR INCREASED FUEL ENRICHMENT			10/01/84
48714	FARLEY 1 - PRESSURE ISOLATION VALVE TS CHANGE IN CRITERIA		2	10/06/84
54449	FARLEY 1 - REACTOR VESSEL CAPSULE REPORT 3:02 EFPY		2	10/14/84
54894	FARLEY 1 - TECH. SPECS. FOR 7.0 EFPY			10/22/84
55029	FARLEY 1 - ADMINISTRATIVE TECH SPECS FOR MANAGEMENT CHANGE			11/12/84
55169	FARLEY 1: 1ST RELIEF FOR ACCUM AND RWST CHECK VALVES			12/03/84
49688	FARLEY 1 - REACTOR COOLANT PUMP TRIP		2	02/01/85
<u>ANTICIPATED ACTIONS</u>				
53587	FARLEY 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53670	FARLEY 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53753	FARLEY 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53836	FARLEY 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FARLEY 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53909	-ITEMS 4.2.3 & 4.2.4-REPLACEMENT	PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT		<u>INITIATION</u> 11/01/84
53980	FARLEY 1 - ITEMS 4.5.2 & 4.5.3 - INTERVALS	REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		11/01/84
54063	FARLEY 1 - ITEM 4.5.1	REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES		11/01/84



DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: FARLEY 2

PLANT LOCATION: 28 MI SE OF DOTHAN, ALA  
 DOCKET NUMBER: 050-00364  
 ARCH/ENGINEER: SSI  
 IE INSPECTOR: W. BRADFORD

LICENSED POWER: 2652 MWT  
 DESIGN POWER: 0829 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: E. REEVES  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PAARISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47173	FARLEY 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 11/14/83
49419	FARLEY 2 - BLOCKING SI SIGNAL DURING COOLDOWN		2	01/09/84
49732	FARLEY 2 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		2	05/07/84
<u>ACTIVE ACTIONS</u>				
52756	FARLEY 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		2	<u>TARGET</u>
52837	FARLEY 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		2	
52918	FARLEY 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		2	
52999	FARLEY 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		2	
48016	FARLEY 2 - REQUEST TO DELETE LIC CONDITION 2 C (19) (D) TURBINE DISC INSPECT OR REPLACEMNT AT 1ST REFUELING OUTAGE		2	
53070	FARLEY 2 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS		2	
53123	FARLEY 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		2	
53173	FARLEY 2 - ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		2	
48570	FARLEY 2 - REQUEST FOR 5 EXEMPTIONS FOR IMPLEMENTATION DATES FOR FIRE PROTECTION MODIFICATIONS		2	06/30/84
51039	FARLEY 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	07/30/84
46590	FARLEY 2 - EQUIPMENT ENVIRONMENTAL QUALIFICATION (NUREG 0588)		2	08/30/84
43965	FARLEY 2 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		3	09/30/84
52227	FARLEY 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44372	FARLEY 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 11/07/83
47937	FARLEY 2 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	12/15/83
47798	FARLEY 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/10/84
<u>ACTIVE ACTIONS</u>				
46303	FARLEY 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u>
45825	FARLEY 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
44440	FARLEY 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
44580	FARLEY 2 - RV AND SV TESTING		1	07/30/84
51159	FARLEY 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	08/30/84
51239	FARLEY 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		2	08/30/84
45133	FARLEY 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
45944	FARLEY 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46015	FARLEY 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46087	FARLEY 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46887	FARLEY 2 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
48164	FARLEY 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FARLEY 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
52000	FARLEY 2 - ADMINISTRATIVE TECHNICAL SPECIFICATION CHANGES		2	COMPLETE 10/14/83
49185	FARLEY 2 - CORPORATE ORGANIZATIONAL CHANGE AND OTHER ADMIN CHANGES		2	10/14/83
51438	FARLEY 2 - ADD BACKUP MET TOWER TO TECH SPEC TABLES 3.3-8 AND 4.3.5		2	10/21/83
52106	FARLEY 2 - REQUEST FOR EXTENSION OF EQ FOR RG1.97 ITEMS		1	10/21/83
53308	FARLEY 2 - COMPLETION OF GENERIC LETTER 83-07 (WASTE POLICY ACT 1982) ACTION		2	12/15/83
53269	FARLEY 2 - FIRE PROTECTION EXEMPTION FOR SEC III.G.2 IN CONTAINMENT		1	12/30/83
52043	FARLEY 2 - T.S. CHANGE FOR SHIFT MANNING (GEN LTR 82-02 AND 82-12)		2	12/30/83
52566	FARLEY 2 - TECH SPEC CHANGES FOR POS MTC OPERATION		2	12/30/83
49069	FARLEY 2 - CHANGE TS 6.5.2.8.E PER GENERIC LETTER 82-17		2	01/06/84
49287	FARLEY 2 - DELETE EIGHT NON-SAFETY RELATED HYD SHUBBERS TS TABLES 3.7-4A		2	01/10/84
52450	FARLEY 2 - REDESIGNATE PRIMARY PRESSURE ISOLATION VALVES IN 1ST PROGRAM		2	01/26/84
52132	FARLEY 2 - FIRE SYSTEM TECH SPEC CHANGES (3 ADMINS)		2	02/09/84
52619	FARLEY 2 - CHANGE ACTION STATEMENTS FOR (LCO 3.1.3.1) CONTROL RODS		2	02/17/84
51536	FARLEY 2 - REACTOR TRIP BREAKERS (DS-416)		2	03/19/84
49182	FARLEY 2 - UPDATE TS FOR FIRST OUTAGE DESIGN CHANGES		2	05/17/84
54238	FARLEY 2 - CHANGE INOPERABLE CONTROL ROD TIME TO 36 HOURS		2	06/06/84
<u>ACTIVE ACTIONS</u>				
49021	FARLEY 2 - EVALUATE SAFETY GRADE BACKUP MEANS OF RCS DEPRESSURIZATION (LICENSE CONDITION 2.C.(12)(B))		3	TARGET
49067	FARLEY 2 - PAD 3.3 AS APPLICABLE TO SUCCESSIVE FUEL CYCLES (LIC COND 2.D.(19)(A))		3	
49212	FARLEY 2 - DELETE LICENSE COND. 2.C.(18) ENVIRONMENTAL QUALIFICATION (NUREG-0588)		2	
49213	FARLEY 2 - DELETE LICENSE COND. 2.C.(20) REGULATORY GUIDE 1.97		2	
49214	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) EMERGENCY OPERATING PROCEDURES		2	
49215	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) (B) REACTOR COOLANT SYS. VENTS		2	
49216	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) (G) RV LEVEL INSTR		2	
49217	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) (H) (1) ANALYSIS OF THERMAL MECH COND		2	
49218	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) (H) (2) VOIDING IN RCS		2	
49220	FARLEY 2 - DELETE LICENSE COND. 2.C.(21) (I) (4) SMALL BREAK LOCA MODEL		2	
53266	FARLEY 2 - REACTOR VESSEL CAPSULE REPORT 1.1. EFPY		3	01/10/84
47479	FARLEY 2 - MODIFIED CONTAINMENT VENT AND PURGE SYSTEM (LIC COND 2.C.(17))		3	06/28/84
49954	FARLEY 2 - CHANGES TO CHARCOAL FILTER TECH SPECS (3/4.7.7, 3/4.7.8 AND 3/4.9.14)		2	06/29/84
52021	FARLEY 2 - UV CONFIRMATORY LIFE TEST PROGRAM (DS-416 BREAKERS)		2	06/30/84
49375	FARLEY 2 - REVISE DG OVERLOAD TEST AND DELETE RIVER WATER SYSTEM TECH SPECS		2	06/30/84
51631	FARLEY 2 - REVISE TECH SPECS FOR BATTERIES PER IEEE 450-1980		2	07/02/84
53402	FARLEY 2 - SAFETY RELATED SHUBBER TS TABLE 3.7-4A		2	07/06/84
54451	FARLEY 2 - REVISE MET TOWER TECH SPECS		2	07/08/84
54236	FARLEY 2 - REPORTING REQUIREMENTS TS PER GEN LTR 83-43		2	08/06/84
54453	FARLEY 2 - ADMIN TECH SPEC CHANGES		2	09/01/84
53469	FARLEY 2 - EXEMPTION FOR USE OF RADIOIODINE CANISTER		2	09/09/84
54532	FARLEY 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		2	09/30/84
54390	FARLEY 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		2	09/30/84
54673	FARLEY 2 - TECH. SPEC. CHANGES FOR INCREASED FUEL ENRICHMENT		2	10/01/84
48715	FARLEY 2 - PRESSURE ISOLATION VALVE TS CHANGE IN CRITERIA		2	10/06/84
55030	FARLEY 2 - ADMINISTRATIVE TECH SPECS FOR MANAGEMENT CHANGE			11/12/84
55170	FARLEY 2: IST RELIEF FOR ACCUM AND RWST CHECK VALVES			12/03/84
54234	FARLEY 2 - REVISE TECH SPECS HEATUP/COOLDOWN CURVES		3	01/10/85
49689	FARLEY 2 - REACTOR COOLANT PUMP TRIP		2	02/01/85

ANTICIPATED ACTIONSINITIATION

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FARLEY 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53538	FARLEY 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		2	<u>INITIATION</u> 11/01/84
53671	FARLEY 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		2	11/01/84
53754	FARLEY 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		2	11/01/84
53837	FARLEY 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		2	11/01/84
53910	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT		2	11/01/84
53981	FARLEY 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		2	11/01/84
54064	FARLEY 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES		2	11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: FITZPATRICK

PLANT LOCATION: 8 MI NE OF OREGO, NY  
 DOCKET NUMBER: 050-00333  
 ARCH/ENGINEER: S&W  
 IE INSPECTOR: L. DOERFLEIN

LICENSED POWER: 2436 MW  
 DESIGN POWER: 0821 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: I. ABELSON  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
00952	FITZPATRICK - RPS POWER SUPPLY		1	<u>COMPLETE</u> 11/07/83
10780	FITZPATRICK IST (A-14) REVIEW		3	12/05/83
12303	FITZPATRICK - HELB AND CONSEQUENTIAL SYSTEM FAILURE FITZPATRICK		3	01/31/84
08315	FITZPATRICK - 10 CFR 50.55A(G) - INSERVICE INSPECTION - GENERIC		1	01/31/84
46661	FITZPATRICK - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/20/84
<u>ACTIVE ACTIONS</u>				
51090	FITZPATRICK - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52228	FITZPATRICK - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52757	FITZPATRICK - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52838	FITZPATRICK - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52919	FITZPATRICK 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53000	FITZPATRICK-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		2	06/15/84
43019	FITZPATRICK - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	07/15/84
42881	FITZPATRICK - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		1	09/15/84
42576	FITZPATRICK - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)			09/15/84
51678	FITZPATRICK - NUREG 0737 TECH SPECS		2	03/15/84
49371	FITZPATRICK - SINGLE LOOP OPERATION		1	10/15/84
42489	FITZPATRICK - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	10/15/84
10756	FITZPATRICK - MECHANICAL SNUBBERS		1	10/15/84
08598	FITZPATRICK - HYDRAULIC SNUBBERS		1	11/15/84
07933	FITZPATRICK - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	11/15/84
07990	FITZPATRICK -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		1	11/15/84
08045	FITZPATRICK - APPENDIX I TECH SPECS (EVALUATE RESPONSE TO NRC LETTER OF 7/11/78.)		1	11/15/84
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
48165	FITZPATRICK -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>COMPLETE</u> 12/29/83
45826	FITZPATRICK - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/29/83
44232	FITZPATRICK - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES			06/07/84
<u>ACTIVE ACTIONS</u>				
51160	FITZPATRICK - I.D.1 CONTROL ROOM DESIGN REVIEW			
51240	FITZPATRICK - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44779	FITZPATRICK - NUREG-0737 II.E.4.1.2, DEDICATED HYDROGEN PENETRATIONS		1	07/15/84
45594	FITZPATRICK - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/15/84
46304	FITZPATRICK - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/15/84
47599	FITZPATRICK - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		1	09/15/84
47670	FITZPATRICK - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	09/15/84
47741	FITZPATRICK - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	09/15/84
45945	FITZPATRICK - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46016	FITZPATRICK - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46088	FITZPATRICK - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FITZPATRICK

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
45134	FITZPATRICK - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	TARGET 09/30/84
44302	FITZPATRICK - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	09/30/84
44441	FITZPATRICK - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	10/15/84
48269	FITZPATRICK - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	11/15/84
<u>TAC # TAC DESCRIPTION PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
51390	FITZPATRICK - REVISE TS MCPR TABLE ODYN CODE INPUT ERROR			COMPLETE 12/13/83
48350	FITZPATRICK - APPENDIX R EXEMPTIONS		1	02/01/84
46709	FITZPATRICK - SHUBBER SURVEILLANCE TECH. SPECS.		3	02/28/84
51725	FITZPATRICK - TS CHANGE RE PROCESS PIPING PENETRATING PRIMARY CONTAINMENT			03/20/84
54374	FITZPATRICK - EVALUATION OF CRACK IN RECIRC. SUCTION LINE			04/13/84
49232	FITZPATRICK - PROPOSED TS CHANGES RE MANAGEMENT ORGANIZATION STRUCTURE			04/24/84
52379	FITZPATRICK - EVALUATION OF PIPE SUPPORT ALLEGATIONS IN VCS 2.206 PETITION			05/08/84
53439	FITZPATRICK - TS CHANGE RE TURBINE CONTROL VALVE MODS			05/08/84
49103	FITZPATRICK - PROPOSED TS CHANGE TO FIRE PROTECTION SYSTEM SURVEILLANCE REQUIREMENTS			05/22/84
53486	FITZPATRICK - SRO STAFFING EXEMPTION REQUEST		1	06/20/84
53560	FITZPATRICK - TS CHANGE CONTAINMENT ATM MONITOR SYSTEM ISOLATION		2	06/27/84
54294	FITZPATRICK - DEDICATED HYDROGEN PENETRATIONS		1	06/27/84
<u>ACTIVE ACTIONS</u>				
49915	FITZPATRICK - PROPOSED TS CHANGE RE SURVEILLANCE, AUDIT FREQUENCY, AND ADMINISTRATIVE ITEMS			TARGET 07/15/84
47525	FITZPATRICK - CONTROL ROD DRIVE SURVEILLANCE FREQUENCY		3	07/15/84
47394	FITZPATRICK - CONTAINMENT AIRLOCK TESTING		1	08/15/84
47018	FITZPATRICK - NUREG-0313 TECHNICAL SPECIFICATIONS		3	09/15/84
52147	FITZPATRICK - DISPOSAL OF CONTAMINATED FUEL OIL PER 10 CFR 20.302		3	09/15/84
53438	FITZPATRICK - TS CHANGE RE RBCLWS ISOLATION VALVES			09/15/84
54533	FITZPATRICK - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
06864	FITZPATRICK - MSIV LEAK DETECTION SYSTEM		2	11/15/84
53473	FITZPATRICK - TS CHANGES RE CONTAINMENT PURGE/VENT			12/15/84
<u>ANTICIPATED ACTIONS</u>				
53589	FITZPATRICK - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53672	FITZPATRICK - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53755	FITZPATRICK- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53838	FITZPATRICK - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53982	FITZPATRICK- ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54065	FITZPATRICK - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: FORT CALHOUN 1

PLANT LOCATION: 19 MI N OF OMAHA, NEB  
 DOCKET NUMBER: 050-00285  
 ARCH/ENGINEER: GHDR  
 IE INSPECTOR: L. YANDELL

LICENSED POWER: 1500 MWT  
 DESIGN POWER: 0478 MWE  
 NSSS VENDOR: COMB

PROJECT MANAGER: E. TOURIGNY  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
42586	FT. CALHOUN 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	<u>COMPLETE</u> 10/19/83
47180	FORT CALHOUN 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	10/28/83
49614	FORT CALHOUN - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION			11/23/83
12316	FT. CALHOUN 1 - HELD AND CONSEQUENTIAL SYSTEM FAILURE FT. CALHOUN		3	01/31/84
03922	FT. CALHOUN 1 - REVIEW OF ASYMETRIC LOCA LOADS SUBMITTAL		1	02/01/84
51755	FORT CALHOUN - THERMAL SHIELD FOLLOWUP ANALYSIS			05/01/84
43319	FORT CALHOUN 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		1	05/22/84
12738	FT. CALHOUN - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES		1	06/17/84
<u>ACTIVE ACTIONS</u>				
42491	FT. CALHOUN 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	<u>TARGET</u> 09/30/84
42892	FT. CALHOUN - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		1	09/30/84
08143	FT. CALHOUN 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		1	09/30/84
49733	FORT CALHOUN -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			09/30/84
51091	FORT CALHOUN 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			09/30/84
52758	FORT CALHOUN 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			12/30/84
52839	FORT CALHOUN 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			12/30/84
52920	FORT CALHOUN 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			12/30/84
53001	FT CALHOUN 1-ITEM 3.1.3-POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			12/30/84
53071	FORT CALHOUN 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			12/30/84
53124	FORT CALHOUN 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			12/30/84
52229	FORT CALHOUN 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TM1 ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
47799	FORT CALHOUN - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 10/17/83
46883	FT. CALHOUN - II K2.13 THERMAL MECHANICAL REPORT		1	06/05/84
<u>ACTIVE ACTIONS</u>				
44233	FT. CALHOUN 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	<u>TARGET</u> 06/30/84
44303	FT. CALHOUN 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	06/30/84
45827	FT. CALHOUN 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
44582	FORT CALHOUN 1 - RV AND SV TESTING		1	09/30/84
45135	FT. CALHOUN 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
45946	FT. CALHOUN 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46017	FT. CALHOUN 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46089	FT. CALHOUN 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46305	FT. CALHOUN 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
51161	FORT CALHOUN 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/84
51241	FORT CALHOUN 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
48166	FORT CALHOUN -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FORT CALHOUN 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
52194	FT. CALHOUN STATION - SALP REVIEW - SEPTEMBER 1, 1982 - AUGUST 31, 1983			<u>COMPLETE</u> 12/20/83
49491	FT. CALHOUN STATION - EVALUATION OF LICENSE RESPONSE TO GENERIC LETTERS 82-17/23			01/26/84
52512	FT. CALHOUN STATION - ADMINISTRATIVE TS CHANGES			01/26/84
52030	FT. CALHOUN STATION - RELIEF REQUESTS FROM ASME-BXPV CODE, SECTION XI			04/06/84
54181	FT. CALHOUN STATION - TS'S FOR CYCLE 9 OPERATION			04/26/84
53337	FT. CALHOUN STATION - REVIEW OF S.C.U. METHODOLOGY FOR RELOADS			05/03/84
52552	FT. CALHOUN STATION - REVIEW OF RELOAD METHODOLOGY REPORTS			05/11/84
54792	FT. CALHOUN STATION - EQ DEADLINE TIME EXTENSION			05/18/84
54857	FT. CALHOUN STATION - EVALUATION OF LICENSEE'S POSITION REGARDING ADEQUACY OF CURRENT AFW'S TECH SPECS			06/15/84
<u>ACTIVE ACTIONS</u>				
55023	FT. CALHOUN STATION - STEAM GENERATOR B MAJOR LEAKAGE EVENT			<u>TARGET</u>
55043	FT. CALHOUN, NEUTRON SOURCE DATA FOR FLUX REDUCTION VERIFICATION			
55098	FT. CALHOUN STATION - OPPD REORGANIZATION			
55084	THREE MILE ISLAND 1 - INDEPENDENT VERIFICATION OF VALVE POSITION			06/30/84
53219	FT. CALHOUN STATION - ADMINISTRATIVE TS CHANGES AS A RESULT OF RULE CHANGES			07/15/84
52513	FT. CALHOUN STATION - NUMBER AND SURVEILLANCE CAPSULES TS CHANGES			08/30/84
52353	FT. CALHOUN STATION - APPENDIX R EXEMPTION REQUESTS			09/30/84
54391	FORT CALHOUN 1 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54534	FORT CALHOUN 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54804	FT. CALHOUN STATION - PUMPS AND VALVES RELIEF REQUEST FROM ASME BXPV CODE			09/30/84
48534	FT. CALHOUN STATION - AMENDED TS'S INOPERABILITY OF RPS/ESFAS CHANNELS			09/30/84
48807	FT. CALHOUN STATION - AMEND TS'S REGARDING APPENDIX J			09/30/84
49696	FORT CALHOUN 1 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				
53590	FORT CALHOUN 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53673	FORT CALHOUN 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53756	FORT CALHOUN 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53839	FORT CALHOUN 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53911	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53983	FORT CALHOUN 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54066	FORT CALHOUN 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: FORT ST VRAIN

PLANT LOCATION: 35 MI N OF DENVER, COL  
 DOCKET NUMBER: 050-00267  
 ARCH/ENGINEER: S&L  
 IE INSPECTOR: G. PLUMLEE

LICENSED POWER: 0842 MWT  
 DESIGN POWER: 0330 MWE  
 NSSS VENDOR: GAC

PROJECT MANAGER: P. WAGNER  
 BRANCH CHIEF: E. JOHNSON  
 LIC. ASSISTANT: E. HAYCRAFT

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
48341	FORT ST. VRAIN - RETS		3	<u>COMPLETE</u> 11/23/83
47417	FORT ST VRAIN - SNUBBERS		2	01/25/84
11025	FORT ST. VRAIN-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	03/06/84
<u>ACTIVE ACTIONS</u>				
42925	FORT ST VRAIN - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	<u>TARGET</u> 07/15/84
54535	FORT ST. VRAIN - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
42527	FORT ST VRAIN - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/30/84
51092	FORT ST. VRAIN - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			12/30/84
52759	FORT ST. VRAIN - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			12/30/84
52840	FORT ST. VRAIN - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			12/30/84
52921	FORT ST. VRAIN - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			12/30/84
53002	FT ST VRAIN-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		3	12/30/84
52230	FORT ST. VRAIN - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44091	FORT ST VRAIN - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS		1	<u>COMPLETE</u> 10/13/83
45065	FORT ST VRAIN - NUREG-0737 II.F.1.3, CONTAINMENT HIGH RANGE MONITOR		1	05/14/84
<u>ACTIVE ACTIONS</u>				
44443	FORT ST VRAIN - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>TARGET</u> 06/30/84
45947	FORT ST VRAIN - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	10/30/84
46018	FORT ST VRAIN - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	10/30/84
46090	FORT ST VRAIN - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	10/30/84
46306	FORT ST VRAIN - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	10/30/84
44234	FORT ST VRAIN - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44304	FORT ST VRAIN - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
44923	FORT ST VRAIN - NUREG-0737 II.F.1.1, NOBLE GAS MONITOR		1	12/30/84
51162	FORT ST. VRAIN - I.D.1 CONTROL ROOM DESIGN REVIEW		2	12/30/84
51242	FORT ST. VRAIN - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			12/30/84
TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49023	FORT ST. VRAIN - STAFFING TECH SPECS		2	<u>COMPLETE</u> 10/13/83
47412	FORT ST. VRAIN - CONTROL ROD DRIVE HOT SPOTS		3	10/13/83
47409	FORT ST. VRAIN - PLATEOUT PROBE DATA EVALUATION		3	11/10/83
47413	FORT ST. VRAIN - PCRV HOT SPOTS		3	11/18/83
52647	FORT ST. VRAIN - RESPOND TO 2 QUESTIONS FROM R4 RE: APPLICABILITY OF APP.R TO FSV		2	12/19/83
52459	FORT ST. VRAIN - PLATEOUT PROBE REMOVAL SCHEDULE CHANGE		1	01/03/84
49623	FORT ST. VRAIN - STEAM GENERATOR TUBE LEAK INVESTIGATION		1	02/01/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

FORT ST VRAIN

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS (CONTINUATION)</u>				
53461	FORT ST. VRAIN - DISPOSAL REQUEST FOR SULPHUR 35		2	<u>COMPLETE</u> 02/14/84
53226	FORT ST. VRAIN - MOISTURE MONITOR DETECTOR RELOCATION		2	02/24/84
47422	FORT ST. VRAIN - GRAPHITE CORROSION & SURVEILLANCE		2	03/01/84
47423	FORT ST. VRAIN - CORE SUPPORT BLOCK STRESSES		3	03/01/84
49624	FORT ST. VRAIN - CYCLE 4, SEGMENT 10 RELOAD (H-451 GRAPHITE)		1	03/02/84
52458	FORT ST. VRAIN - CALIBRATION SOURCES LICENSE CONDITION		2	03/08/84
54496	FORT ST. VRAIN - NUREG 0737 TECH SPECS (G.L. 83-36 AND 37)		2	04/23/84
48863	FORT ST. VRAIN - DETECTOR DECALIBRATION		2	05/11/84
51535	FORT ST. VRAIN - TS CHANGES TO 4.4		2	06/05/84
52144	FORT ST. VRAIN - FIRE PROTECTION APPRAISAL APPENDIX R		1	06/08/84
54183	FORT ST. VRAIN - STEAM GENERATOR TUBE ISI		1	06/22/84
<u>ACTIVE ACTIONS</u>				
54639	FORT ST. VRAIN - PCRV TENDON CORROSION PROBLEM			<u>TARGET</u> 06/06/84
55222	GILINSKY COMMENTS ON THE PERFORMANCE OF REGION IV AND FORT ST. VRAIN/GREEN TICKET EDO-14474			06/14/84
51955	FORT ST. VRAIN - ELECTRICAL MODIFICATIONS INCLUSION INTO TECH SPECS		2	07/10/84
52634	FORT ST. VRAIN - LCO 4.1.9 NON-CONSERVATIVE ANALYSIS ASSUMPTIONS		1	07/15/84
54598	FORT ST VRAIN-REG IV TIA, STEAM GENERATOR TUBE LEAK			07/30/84
55053	FORT ST. VRAIN - SEVENTY PERCENT MOISTURE MONITOR RESPONSE TEST			08/22/84
54265	FORT ST. VRAIN - SECURITY PLAN REVISION 14		1	09/01/84
54373	FORT ST. VRAIN - APPENDIX R EXEMPTION REQUESTS		1	09/09/84
55286	FORT ST. VRAIN - FAILURE TO SCRAM ON 6-23-84			09/30/84
53417	FORT ST. VRAIN - ISI/IST TECH SPECS		2	09/30/84
51954	FORT ST. VRAIN - EXEMPTION TO 10CFR 50.54(W)		1	10/15/84
55287	FORT ST. VRAIN - BUILDING 10 EVALUATION			10/18/84
55051	FORT ST. VRAIN - CHANGES TO SECONDARY COOLANT ACTIVITY SR			11/15/84
55052	FORT ST. VRAIN - CHANGES TO CIRCULATION OVERSPEED TRIP SETTING			11/25/84
52354	FORT ST. VRAIN - STARTUP TEST PROGRAM CHANGES		2	11/30/84
49055	FORT ST. VRAIN - FUEL BLOCK CRACK MODELING		3	12/04/84
52035	FORT ST. VRAIN - CORE SEGMENT 2 REPORT REVIEW (CRACKED FUEL ASSEMBLIES)		3	12/30/84
47416	FORT ST. VRAIN - INSTRUMENT SETPOINTS		3	06/30/85
<u>ANTICIPATED ACTIONS</u>				
53591	FORT ST. VRAIN - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		2	<u>INITIATION</u> 11/01/84
53674	FORT ST. VRAIN - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		2	11/01/84
53757	FT ST VRAIN- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		3	11/01/84
53840	FORT ST. VRAIN - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		2	11/01/84
53984	FORT ST. VRAIN - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		2	11/01/84
54067	FORT ST. VRAIN - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES		2	11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: GINNA

PLANT LOCATION: 15 MI NE OF ROCHESTER, NY  
 DGCKET NUMBER: 050-00244  
 ARCH/ENGINEER: GIL  
 IE INSPECTOR: W. COOK

LICENSED POWER: 1520 MWT  
 DESIGN POWER: 0470 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: G. DICK  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
07992	GINNA - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	<u>COMPLETE</u> 01/18/84
49546	GINNA BLOCKING SI SIGNAL DURING COOLDOWN			01/30/84
06137	GINNA - REACTOR VESSEL SUPPORT (NRR GENERIC A-2)		2	02/09/84
42609	GINNA - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	06/21/84
<u>ACTIVE ACTIONS</u>				
08414	GINNA - TS SURVEILLANCE - HYDRAULIC SNUBBERS		3	<u>TARGET</u>
08416	GINNA - TS SURVEILLANCE FOR MECHANICAL SNUBBERS TEST RESULTS		2	
52760	GINNA - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52841	GINNA 1 - ITEM 4.2.1 - EQUIPMENT CLASSIFICATION AND VENDOR VENDOR INTERFACE - RTS COMPONENTS			
52922	GINNA - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53003	GINNA-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53072	GINNA - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53125	GINNA - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53174	GINNA - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		2	05/30/84
43047	GINNA - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		1	06/30/84
42520	GINNA - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)			07/15/84
49734	GINNA - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		3	09/30/84
07739	GINNA - ECCS EVAL THAT ALLOWS FOR UPPER PLENUM INJECTION(UPI)		2	10/30/84
42910	GINNA - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11			12/31/85
52231	GINNA - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			06/30/86
51093	GINNA - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47800	GINNA - NUREG 0737 ITEM II.K.2.17 POTENTIAL FOR VOIDING IN RCS			<u>COMPLETE</u> 12/27/83
44444	GINNA - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	04/24/84
47941	GINNA - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	05/08/84
47602	GINNA - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		1	05/10/84
47673	GINNA - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	05/10/84
47744	GINNA - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	05/10/84
44235	GINNA - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
46884	GINNA - II K2.13 THERMAL MECHANICAL REPORT		1	06/13/84
<u>ACTIVE ACTIONS</u>				
45829	GINNA - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	<u>TARGET</u>
44584	GINNA - RV AND SV TESTING		1	09/01/84
44305	GINNA - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
45137	GINNA - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/31/84
48168	GINNA -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/31/84
51163	GINNA - I.D.1 CONTROL ROOM DESIGN REVIEW			02/28/86
51243	GINNA - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			06/30/86

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

GINNA

TAC #	TAC DESCRIPTION	IMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
45948	GINNA - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	TARGET 06/30/86
46019	GINNA - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	06/30/86
46091	GINNA - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	06/30/86
46307	GINNA - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/30/86
TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49344	GINNA - SGTR - THERMOHYDRAULIC ANALYSIS		2	COMPLETE 10/05/83
49345	GINNA - RCP TRIP & RESTART CRITERIA		2	10/14/83
49349	GINNA - FIBER OPTIC INSPECTION OF S/G TUBES		2	10/25/83
49351	GINNA - STAGNANT FLOW/PTS		1	10/31/83
48914	GINNA - REORGANIZATION OF PLANT STAFF		2	11/02/83
49348	GINNA - LETDOWN SYSTEM & PER PRESSURE INST MDDS		2	11/09/83
49347	GINNA - SIMULATOR RESPONSE TIME - SGTR		2	12/14/83
51053	GINNA INSERTION INTO THE SPENT FUEL RACKS A FUEL ASSEMBLY OF PREVIOUSLY UNANALYZED DESIGN & ENRICHMENT			02/08/84
54174	GINNA - EXEMPTION FROM SHIFT STAFFING REQUIREMENTS			02/22/84
48884	GINNA - EFFECTS OF GROUNDWATER LEVEL ON STRUCTURES SEP III-3.A			04/18/84
48885	GINNA - INSERVICE INSPECTION OF WATER CONTROL STRUCTURES SEP III-3.C			04/18/84
48886	GINNA - STEAM GENERATOR BLOWDOWN VALVE RESTRAINTS SEP III-4.C			04/18/84
48891	GINNA - RHR TUBE FAILURES SEP V-10.A			04/18/84
48892	GINNA - RHR PROCEDURES SEP V-10.B			04/18/84
48893	GINNA - CONTAINMENT ISOLATION SEP TOPIC VI-4			04/18/84
48894	GINNA - ESF SWITCHOVER PROCEDURES SEP VI-7.B			04/18/84
48895	GINNA - BATTERY MONITORING SYSTEM SEP VIII-3.B			04/18/84
53430	GINNA - WESTINGHOUSE OPTIMIZED FUEL ASSEMBLY			05/01/84
54632	GINNA - EXEMPTION FROM SHIFT STAFFING REQUIREMENTS			05/04/84
54503	GINNA-STEAM GENERATOR TUBESHEET SLEEVES			05/09/84
<u>ACTIVE ACTIONS</u>				
54659	GINNA - TECH. SPEC. CHANGE - REPORTING REQUIREMENTS - CONF. WITH 50.72, 50.73, & 50.49			TARGET
55300	GINNA - EXEMPTION INSPECTION OF CONTAINMENT TENDONS FOR BROKEN WIRES			
12461	GINNA - TENDON SURVEILLANCE PROGRAM		1	
54291	GINNA - EXEMPTION REQUEST - EMERGENCY EXERCISE			06/15/84
52099	GINNA - CONSOLIDATION OF MANAGEMENT RESPONSIBILITIES			06/30/84
49346	GINNA - PROCEDURES - SGTR FOLLOW-UP		2	06/30/84
54357	GINNA - T/S CHANGE - MANAGEMENT PERSONNEL AND CORRECTION IN ORGANIZATION CHART			07/15/84
49178	GINNA - TECH SPEC CHANGE FOR RWST OPERABILITY AND OTHER CLARIFICATIONS			07/28/84
48474	GINNA - REVIEW OF ADDITIONAL SUBMITTALS ON SGTR			07/30/84
49177	GINNA - TECH SPEC CHANGE TO ALLOW LOW POWER PHYSICS TESTING PRIOR TO OR DURING CERTAIN VALVE TESTING			07/30/84
53432	GINNA - PROCESS CONTROL PROGRAM			07/30/84
52404	GINNA - TS CHANGE - REVISE LIST OF HYDRAULIC SNUBBERS			07/30/84
54509	GINNA - DISPOSAL OF LOW LEVEL RADIOACTIVE WASTE			07/30/84
54371	GINNA - SEISMIC STRUCTURAL EVAL. MAIN CONTRL BOARD SEP III-6			07/30/84
54849	GINNA - LICENSE CONVERSION SER SUPPLEMENT			07/30/84
54369	GINNA - T/S CHANGE - DELETE SPACING RESTRICTIONS OF RECENTLY DISCHARGED FUEL IN SPENT FUEL RACKS			08/15/84
53433	GINNA - TS CHANGE ADJUSTMENT OF ISI PGM 10 YR INTERVAL START DATES			08/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

GINNA

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
52399	GINNA - SEP TS CHANGES			08/30/84
49135	GINNA - PUMF & VALVE INSERVICE TESTING			08/30/84
49175	GINNA - TECH SPEC CHANGE VALVE EXERCISING AND SURVEILLANCE REQUIREMENTS		2	08/30/84
49292	GINNA - HEATUP & COOLDOWN - REACTOR SURVEILLANCE CAPSULE T		2	08/30/84
54370	GINNA - T/S CHANGE - REDUCE REQUIREMENT FOR LOW PZR WATER LEVEL AND P-10 PERMISSIVE			09/01/84
54363	GINNA - T/S CHANGE - DELETE RESTRICTION ON MOVEMENT OF SHIPPING CASKS WITH AUX BLDG CRANE			09/01/84
54392	GINNA - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54536	GINNA - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
52396	GINNA - T.S. CHANGE - DELETE RESPONSE TIME TESTING OF AUX FD SYS AND CONT. 150 VALVE CIRCUITS			09/30/84
53311	GINNA - TECHNICAL SPECIFICATION CHANGE - USE OF TSC BATTERY IN PLACE OF INOPERABLE 1E BATTERY BANK			09/30/84
54762	GINNA - INCREASE CAPACITY OF SPENT FUEL POOL STORAGE RACKS			10/01/84
53431	GINNA - ALTERNATIVE SHUTDOWN SYSTEM - APP R			10/30/84
52395	GINNA - COMPUTER ANALYSIS OF JAN. 1982 STEAM GEN TUBE RUPTURE			10/30/84
49399	GINNA - TECH SPEC TO INCREASE THE MAXIMUM ALLOWABLE ACTIVITY IN THE REACTOR COOLANT			12/30/84
11220	GINNA - CONVERSION PDL TO FTOL		1	12/30/84
49341	GINNA - RADIOLOGICAL CONSEQUENCES OF SGTR		2	12/30/84
54364	GINNA - SEPSTRUCTUAL UPGRADE PROGRAM FOLLOWUP			12/30/84
43307	GINNA - NON-TECH SPEC TMI RELATED ITEMS		1	12/31/84
48880	GINNA - TECH SPEC CHANGES FOR SEP TOPICS II-A AND III-7.A			01/02/85
49677	GINNA - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53592	GINNA - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53675	GINNA - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53758	GINNA- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53841	GINNA - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53912	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53985	GINNA - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54068	GINNA - ITEM 4 5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: GRAND GULF 1

PLANT LOCATION: 25 MI S OF VICKSBURG, MISS  
 DOCKET NUMBER: 050-00416  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: A. WAGNER

LICENSED POWER: 3833 MW  
 DESIGN POWER: 1250 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: L. KINTNER  
 BRANCH CHIEF: E. ADENSAM  
 LIC. ASSISTANT: M. DUNCAN

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52842	GRAND GULF - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			<u>TARGET</u>
52923	GRAND GULF - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53004	GRAND GULF-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
51094	GRAND GULF - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52232	GRAND GULF - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52761	GRAND GULF - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
49994	GRAND GULF UNIT 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL PJOL			06/30/84
51679	GRAND GULF 1 NUREG 0737 TECH SPECS			08/01/84
49996	GRAND GULF UNIT 1 - MASONRY WALL DESIGN IE BULLETIN 80-11 IE BULLETIN 80-11			06/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
49459	GRAND GULF 1 - INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING (II.F.2)			
51164	GRAND GULF - I.D.1 CONTROL ROOM DESIGN REVIEW			
51244	GRAND GULF - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
54587	GRAND GULF 1 - II.D.1 - RV & SV TESTING			
49992	GRAND GULF UNIT 1 - QUALIFICATION OF ADS ACCUMULATORS NUREG-0737, II.K.3-28			07/30/84
49993	GRAND GULF UNIT 1 - POST ACCIDENT SAMPLING NUREG-0737, II.B.3			09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51337	GRAND GULF UNIT 1 - FIRE PROTECTION PLAN			11/10/83
51479	GRAND GULF UNIT 1 - FIRE PROTECTION TS CHANGES			11/10/83
51862	GRAND GULF 1 - CHLORINE ACCIDENT ANALYSIS			11/30/83
53434	GRAND GULF UNIT 1 - STARTUP TESTS			02/03/84
51957	GRAND GULF UNIT 1 - TECH SPEC PACKAGE #8			02/21/84
51771	GRAND GULF UNIT 1 - TECH SPEC PKG #6			02/21/84
52367	GRAND GULF UNIT 1 - SRM TECH. SPEC. CHANGE			02/21/84
51478	GRAND GULF UNIT 1 - TECH SPEC CHANGE PACKAGE #3			02/21/84
52139	GRAND GULF UNIT 1 - TECH SPEC CHANGE PKG. 11			04/17/84
52523	GRAND GULF UNIT 1 - TECH SPEC CHANGE PKG. 14			04/18/84
52524	GRAND GULF UNIT 1 - TECH SPEC CHANGE PKG. 15			04/18/84
54602	GRAND GULF 1-T.S. CHANGE, DTD 3/29/84, ITEM 2 MINIMUM OPERABLE CHANNELS-RADIATION MONITOR			04/18/84
54603	GRAND GULF 1-T.S. CHANGE, DTD 3/29/84, ITEM 1 SNUBBER ADDITION			04/18/84
54604	GRAND GULF 1-T.S. CHANGE, DTD 3/29/84, ITEM 3 CONTAINMENT SPRAY INSTRUMENTATION			04/18/84
54605	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 03/29/84, ITEM 4, ADS INSTRUMENTATION			04/18/84
54606	GRAND GULF 1-T.S. CHANGE, DTD 3/29/84, ITEM 5 CHLORINE DETECTION INSTRU. SURVEILLANCE			04/18/84
54318	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 6 - ECCS RESPONSE TIMES			04/18/84
54185	GRAND GULF 1 - TECH SPEC AUDIT			06/01/84
54493	GRAND GULF UNIT 1 - HIGH GROUNDWATER LEVEL			06/01/84
53419	GRAND GULF 1 - AGASTAT RELAYS			06/01/84
54321	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 9 - SERVICE WATER FOR ESF			06/04/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

GRAND GULF 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS - (CONTINUATION)</u>				
54323	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 11 - A. C. POWER OPERABILITY			<u>COMPLETE</u>
55108	GRAND GULF UNIT 1 - SWS REVIEW			06/04/84 06/22/84
<u>ACTIVE ACTIONS</u>				
54458	GRAND GULF - 1, TECH SPEC CHANGE PROBLEM SHEETS (001-382)			<u>TARGET</u>
54478	GRAND GULF - 1, T/S CHANGE, DTD 03/20/84, ITEM 1, ADS VALUE OPERABILITY REQUIREMENTS (PROBLEM SHEET 001)			
54479	GRAND GULF - 1, T/S CHANGE, DTD 03/20/84, ITEM 2, REACTOR WATER CLEANUP SYSTEM ISOLATION (PROBLEM SHEET 005)			
54457	GRAND GULF - 1, TECH. SPEC. CHANGE DRAFTS FOR 32 PROBLEM SHUTS			
55218	GRAND GULF 1 - EQ - L.C.2.C(12)			
52412	GRAND GULF UNIT 1 - REVIEW OF REVISED TECH SPECS			12/15/83
55054	GRAND GULF UNIT 1 - TECHNICAL SPECIFICATION PROBLEM SHEET (TSPS) REVIEW			05/31/84
55109	GRAND GULF UNIT 1 - SWS REVIEW			06/15/84
55110	GRAND GULF UNIT 1 - SWS REVIEW			06/15/84
55150	GRAND GULF 1 - GDC17 EXEMPTION REQUEST			06/24/84
52049	GRAND GULF 1 - EXTENSION OF DATES IN LICENSE CONDITIONS			07/15/84
53491	GRAND GULF 1 - TS CHANGES, PLANT ORG. STAFF			07/15/84
52631	GRAND GULF UNIT 1 - 50.55 INSPECTION RELIEF			07/30/84
51866	GRAND GULF 1: STRUCTURAL INTEGRITY TEST			07/30/84
52368	GRAND GULF UNIT 1 - TECH. SPEC. CHANGE PKG. 13			07/30/84
54683	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 4/10/84, ITEM NO. 1 ROOM AIR TEMPERATURE TRIPS			07/30/84
54684	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 4/10/84, ITEM 2 CONTAINMENT ISOLATION VALVES			07/30/84
54685	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 4/10/84, ITEM 3 ACCIDENT MONITORING INSTRUMENTS			07/30/84
54686	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 04/11/84, ITEM NO. 1 ECCS SURVEILLANCE			07/30/84
54315	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 3 - RE EMBANKMENT SLOPE STABILITY AND BASES			07/30/84
54316	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 4 - REACTOR VESSEL TEMPERATURE			07/30/84
54317	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 5 - RADIATION INSTRUMENTS			07/30/84
54319	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 7 - ECCS ACTUATION INSTRUMENTS			07/30/84
54320	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 8 - RCS ISOLATION VALVE LEAKAGE			07/30/84
54808	GRAND GULF - 1, TECH. SPEC. CHANGE PROBLEM SHEETS			07/30/84
54324	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 12 - CONTAINMENT H2 CONTROL			07/30/84
54325	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 13 - CONTROL ROD BLOCK INTERLOCK			07/30/84
54326	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 14 - INITIAL CONTAINMENT PRESSURE			07/30/84
54327	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 15 - SUPPRESSION POOL LEVEL			07/30/84
54328	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 16 - VENTILATION RADIATION MONITORS			07/30/84
54329	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 17 - REPORTING OF SRV CHALLENGES			07/30/84
54330	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 18 - RPS & ESFAS INSTRUMENTATION			07/30/84
54331	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 19 - VENTILATION FLOW MONITORS			07/30/84
54341	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 30 - RECORDS REQUIREMENT			07/30/84
54342	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 32 - CONTROL ROD INSERTION LINES			07/30/84
54343	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 33 - CONTROL ROOM TEMPERATURE			07/30/84
54344	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 1/18/84, ITEM 1			07/30/84
54345	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 2/27/84, ITEM 1 - APRM SCRAM SETPOINT			07/30/84
54322	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 10 - ISOLATION VALVE RESPONSE TIME			07/30/84
54222	GRAND GULF UNIT 1 - ONSITE/OFFSITE POWER RELIABILITY			07/30/84
54313	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 1 - SNUBBER IDENTIFICATION			07/30/84
54314	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 2 OFFGAS RADIATION MONITOR			07/30/84
54332	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 21 - ECCS ACTUATION			07/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

GRAND GULF 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54333	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 22 - DRYWALL PRESSURE			07/30/84
54334	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 23 - FIRE HOSE STATIONS			07/30/84
54335	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 24 - LPCI PUMP ACTUATION			07/30/84
54336	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 25 - CONTAINMENT VALVE LIS V			07/30/84
54337	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 26 - A.C. POWER SURVEILLANCE			07/30/84
54338	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 27 - RPS INSTRUMENTATION			07/30/84
54339	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 28 - ECCS INSTRUMENTS			07/30/84
54340	GRAND GULF - 1, TECH. SPEC. CHANGE, DTD 9/9/83, ITEM 29 - DEFINITION OF UNRESTRICTED AREA			07/30/84
55242	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 18, 1984, ITEM A.02			07/31/84
55243	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 19, 1984			07/31/84
55260	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 20, 1984, ATTACHMENT 1			07/31/84
55261	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 20, 1984, ATTACHMENT 2			07/31/84
55262	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 21, 1984, ATTACHMENT 1			07/31/84
55263	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGES, DATED JUNE 21, 1984, ATTACHMENT 2			07/31/84
55268	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 22, 1984, ATTACHMENT 1			07/31/84
55269	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 22, 1984, ATTACHMENT 2			07/31/84
55270	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 22, 1984, ATTACHMENT 3			07/31/84
55241	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 18, 1984, ITEMS A.01, B, C, D			07/31/84
55238	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 17, 1984, ATTACHMENT 1			07/31/84
55239	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 17, 1984, ATTACHMENT 2			07/31/84
55240	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 17, 1984, ATTACHMENT 3			07/31/84
55271	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 22, 1984, ATTACHMENT 4			07/31/84
55272	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 22, 1984, ATTACHMENT 5			07/31/84
55273	GRAND GULF 1 - TECHNICAL SPECIFICATION CHANGE, DATED JUNE 22, 1984, ATTACHMENT 6			07/31/84
52603	GRAND GULF UNIT 1 - STAFFING REVIEW			08/01/84
51641	GRAND GULF UNIT 1 - EMERGENCY RESPONSE CAPABILITY (82-33)			08/01/84
53395	GRAND GULF UNIT 1 - ISOLATION INSTR. TECH SPECS			08/01/84
53418	GRAND GULF 1 - REMOTE PANEL TRANSFER SWITCHES			08/01/84
53490	GRAND GULF 1 - CONTROL ROOM LEAKAGE			08/01/84
52370	GRAND GULF UNIT 1 - ISOLATION INSTRUMENTATION			08/30/84
52175	GRAND GULF UNIT 1 - BWR PIPE CRACK			08/30/84
49997	GRAND GULF UNIT 1 - INSERVICE INSPECTION			08/30/84
53548	GRAND GULF 1 - CONTAINMENT/DRYWELL PURGE			09/01/84
51477	GRAND GULF UNIT 1 - INSERVICE TESTING AND INSPECTION (IST) PUMPS/VALVES			09/30/84
54537	GRAND GULF - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
52411	GRAND GULF UNIT 1 - DELAVAL DIESEL GENERATOR			10/01/84
49995	GRAND GULF UNIT 1 - SOIL STRUCTURE INTERACTION			06/30/85
48736	GRAND GULF UNIT 1 - HYDROGEN CONTROL MEASURES			06/30/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53593	GRAND GULF - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53676	GRAND GULF - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53759	GRAND GULF- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53842	GRAND GULF - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53986	GRAND GULF - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54069	GRAND GULF - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: HADDAM NECK

PLANT LOCATION: 13 MI E OF MERIDEN, CONN  
 DOCKET NUMBER: 050-00213  
 ARCH/ENGINEER: S&W  
 IE INSPECTOR: T. REBELOWSKI

LICENSED POWER: 1825 MW  
 DESIGN POWER: 0582 MW  
 NSSS VENDOR: WEST

PROJECT MANAGER: J. LYONS  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
11234	HADDAM NECK - INSERVICE TESTING (IST)		1	<u>COMPLETE</u> 12/29/83
49735	HADDAM NECK - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			01/30/84
11976	HADDAM NECK - REACTOR CAVITY SEAL RING GENERIC ISSUE		2	02/01/84
03463	HADDAM NECK - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
51095	HADDAM NECK - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52233	HADDAM NECK - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52762	HADDAM NECK - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52843	HADDAM NECK - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52924	HADDAM NECK - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53005	HADDAM NECK - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53073	HADDAM NECK - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53126	HADDAM NECK - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53175	HADDAM NECK - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
43625	HADDAM NECK - APPENDIX R-EXEMPTION REQUEST		1	06/30/84
07993	HADDAM NECK - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL PLANTS (NUREG 0612)		2	07/01/84
42521	HADDAM NECK - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/30/84
42911	HADDAM NECK - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	07/31/84
08098	HADDAM NECK - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	11/30/84
08465	HADDAM NECK - TECH SPEC SURVEILLANCE FOR HYD. SNUBBERS		3	12/01/84
08469	HADDAM NECK - TECH SPEC SURVEILLANCE FOR MECH. SNUBBERS		2	12/01/84

TAC #	TAC DESCRIPTION	TML ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47801	HADDAM NECK - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 12/22/83
41066	HADDAM NECK - NUREG-0737 - II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/05/84
44236	HADDAM NECK - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
46885	HADDAM NECK - II K2.13 THERMAL MECHANICAL REPORT		1	06/13/84

TAC #	TAC DESCRIPTION	TML ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
51165	HADDAM NECK - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51245	HADDAM NECK - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
47674	HADDAM NECK - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	05/14/84
47745	HADDAM NECK - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	05/14/84
47603	HADDAM NECK - NUREG 0737 ITEM II F1.4 - CONTAINMENT PRESSURE INSTRUMENT		1	05/15/84
44585	HADDAM NECK - RV AND SV TESTING		1	06/01/84
46308	HADDAM NECK - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/84
45138	HADDAM NECK - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
48420	HADDAM NECK - NUREG 0737, II.K.3.25 POWER ON PUMP SEALS		1	07/31/84
45830	HADDAM NECK - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	08/30/84
45949	HADDAM NECK - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## HADDAM NECK

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>		(CONTINUATION)		<u>TARGET</u>
46020	HADDAM NECK	- NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER	1	09/30/84
46092	HADDAM NECK	- NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY	1	09/30/84
46450	HADDAM NECK	- NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY	1	09/30/84
44306	HADDAM NECK	- NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES	1	11/30/84
48165	HADDAM NECK	-NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46	1	12/31/84
TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
51931	HADDAM NECK	- VENTILATION SYSTEMS		11/01/83
51475	HADDAM NECK	- DELETION OF TS ON TRANSMISSION LINE RIGHTS-OF-WAY MGMT		01/31/84
51995	HADDAM NECK	- CONTAINMENT AND RHR LEAK ROM AND CAR TS		02/21/84
54249	HADDAM NECK	- ULTIMATE HEAT SINK		02/28/84
54797	HADDAM NECK	- EQUIPMENT QUALIFICATION EXTENSION		04/05/84
53247	HADDAM NECK	- INTEGRATED SAFETY ASSESSMENT		04/05/84
54615	HADDAM NECK	- TECHNICAL SPECIFICATIONS FOR COASTDOWN		05/18/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54295	HADDAM NECK	- STA TRAINING		
54299	HADDAM NECK	- TECH. SPEC. TO LIMIT OVERTIME		
54300	HADDAM NECK	- REPORTING SAFETY AND RELIEF VALVE FAILURES		
54444	HADDAM NECK	- RCP ISI EXEMPTION		
54837	HADDAM NECK	- DC POWER SYSTEM BUS VOLTAGE MONITORING		
55113	HADDAM NECK	- ECCS PUMPS DISCHARGE PRESSURE REQUIREMENTS		
51599	HADDAM NECK	- MISCELLANEOUS ADMINISTRATIVE TS		04/01/84
49852	HADDAM NECK	- COMBUSTIBLE GAS CONTROL IN CONTAINMENT		05/30/84
48825	HADDAM NECK	- TECH SPEC DEFINITION OF CONT INTEGRITY REMOVAL PLAN FOR REACTOR VESSEL SURVEILLANCE CAPSULES		05/31/84
53301	HADDAM NECK	- SHIFT STAFFING TECH SPEC CHANGE		05/31/84
51932	HADDAM NECK	- SERVICE WATER SYSTEM		06/30/84
51933	HADDAM NECK	- RPS ISOLATION		06/30/84
51934	HADDAM NECK	- OHSITE STANDBY AC POWER SYSTEM		06/30/84
51935	HADDAM NECK	- CONTAINMENT ISOLATION SYSTEM		06/30/84
51938	HADDAM NECK	- MISSILES		06/30/84
51939	HADDAM NECK	- INTEGRATED STRUCTURAL ANALYSIS		06/30/84
51639	HADDAM NECK	- ADM CONTROLS TS CHANGE		06/30/84
54900	HADDAM NECK	- SHUTDOWN MARGIN DURING THREE-LOOP OPS.		06/30/84
49314	HADDAM NECK	- FIRE PROTECTION ALTERNATE SAFE SHUTDOWN EXEMPTION REQUEST		06/30/84
47367	HADDAM NECK	- SURVEILLANCE AND OPERABILITY REQUIREMENTS FOR DIESEL GENERATORS	3	06/31/84
54538	HADDAM NECK	- TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		09/30/84
54393	HADDAM NECK	- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		09/30/84
52074	HADDAM NECK	- DELETION OF TS FOR EQ OF ELECT EQUIP.		09/30/84
51936	HADDAM NECK	- PIPE BREAKS		10/15/84
51937	HADDAM NECK	- SEISMIC		10/15/84
51930	HADDAM NECK	- FLOODING EVALUATION		10/15/84
52010	HADDAM NECK	INTEGRATED ASSESSMENT SUPPLEMENT		10/30/84
48019	HADDAM NECK	- STANDARD TECH SPEC REVIEW	3	12/15/84
51704	HADDAM NECK	- DEGRADED GRID VOLTAGE PROCEDURES		12/31/84
49667	HADDAM NECK	- REACTOR COOLANT PUMP TRIP		02/01/85

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HADDAM NECK

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS</u>				
53594	HADDAM NECK - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53677	HADDAM NECK - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53760	HADDAM NECK- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53843	HADDAM NECK - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53913	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53987	HADDAM NECK - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54070	HADDAM NECK - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: HATCH 1

PLANT LOCATION: 11 MI N OF BAXLEY, GA  
 DOCKET NUMBER: 050-00321  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: R. ROGERS

LICENSED POWER: 2436 MWT  
 DESIGN POWER: 0777 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: G. RIVENBARK  
 BRANCH CHIEF: J. STOLZ  
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
42219	HATCH 1 - BWR SCRAM DISCHARGE VOLUME CAPABILITY		1	01/04/84
42603	HATCH 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	01/16/84
07939	HATCH 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	01/25/84
47007	HATCH 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL		2	04/19/84
46662	HATCH 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	05/29/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52234	HATCH 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52763	HATCH 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52844	HATCH 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52925	HATCH 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53006	HATCH 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH SPECS - RTS COMPONENTS		1	05/30/84
11260	HATCH 1 - 1ST		1	06/15/84
13277	HATCH 1 - DEFN OF OPERABLE		1	06/15/84
42505	HATCH 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	06/15/84
10710	HATCH 1 - CONTAINMENT LEAK TESTING APP J(GENERIC)		2	06/30/84
08310	HATCH 1 - SURV. OF MECHANICAL SNUBBERS			07/30/84
51680	HATCH 1 NUREG 0737 TECH SPECS		2	08/30/84
08043	HATCH 1 - APPENDIX I TECH SPECS		1	12/30/84
51096	HATCH 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	06/30/85
43733	HATCH 1 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM			
TAC #	TAC DESCRIPTION	IMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
44586	HATCH 1 - II.D.1 RV AND SV TESTING		1	12/12/83
45831	HATCH 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/04/84
48170	HATCH 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	01/04/84
44855	HATCH 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	01/16/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
51166	HATCH 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	
51246	HATCH 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44094	HATCH 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS		1	06/30/84
45595	HATCH 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/30/84
44445	HATCH 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	08/30/84
44307	HATCH 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	09/30/84
45691	HATCH 1 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	09/30/84
45139	HATCH 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
45950	HATCH 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46021	HATCH 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46093	HATCH 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46309	HATCH 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
44237	HATCH 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	09/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HATCH 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
43684	HATCH 1 - ADDS 3 VALVES IN SCRAM DISCH. VOL. SYST. TO T.S LIST OF ISOLATION VALVES		2	10/03/83
48294	HATCH 1 - HPCI AND RCIC SYSTEM OPERABILITY PRESSURE LCO AT 150 PSIG		2	10/03/83
52320	HATCH 1 - REVISE CONFIRMATORY ORDER ON NUREG 0737 ITEM II.B.3			10/14/83
52448	HATCH 1 - MAPHGRT MCPR CHANGES FOR REPLACEMENT OF LEAKING FUEL ASSEMBLIES			12/06/83
53306	HATCH UNIT 1 - REVISE TS TO OPERATE WITHOUT RECIRCULATION FLOW			02/06/84
51392	HATCH UNIT 1 - AUDIT FREQUENCY CHANGE FOR EMERGENCY AND SECURITY PLANS		3	02/22/84
49071	HATCH 1 - MINIMUM SHIFT CREW INCREASE NUREG-0737 RELATED T.S. CHANGE		1	02/22/84
51998	HATCH UNIT #1 TECH SPEC DELETION OF DRYWELL TO TORUS D/P SYSTEM			03/06/84
53478	HATCH 1 - EXTEND COMPLETION SCHEDULE FOR NUREG-0737-II.B.3			03/08/84
48748	HATCH 1 - FIRE PROTECTION PROGRAM - APP.R EXEMPTION REQUESTS & SAFE SHUTDOWN REPORT		1	04/18/84
48520	HATCH 1 - PROVIDES TECH SPEC LCO FOR SECONDARY CONTAINMENT		2	06/20/84
48120	HATCH 1 - ADMIN. TECH. SPEC. CHANGE RELATED TO TYPE B AND TYPE C TEST REPORTING		2	06/20/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
55236	HATCH 1 - PIPING INSPECTION PROGRAM FOR 1984 REFUELING OUTAGE			06/15/84
54168	HATCH 1 - ANALOG TRANSMITTER TRIP SYSTEM TS			06/15/84
54432	HATCH 1 - APRM/RBM TECH SPEC CHANGES			06/15/84
51984	HATCH #1 PURGE & VENT VALVE SYST DESIGN AND TS CHANGES			06/15/84
55087	HATCH UNIT 1 - CONTAINMENT SPRAY TECH. SPEC. SURVEILLANCE INTERVAL EXTENSION			06/19/84
54459	HATCH #1 - EMERGENCY EXERCISE EXEMPTION			06/30/84
53327	HATCH #1 - CLOSURE TIME FOR SDV VENT AND DRAIN VALVES			06/30/84
54461	HATCH #1 - TECH SPECS FOR TMI ACTION PLAN ITEM II.F.1.1, .2, .3			07/15/84
49076	HATCH 1 - TITLE CHANGES OF PLANT REVIEW BOARD MEMBERS IN APP.A & B T.S.			07/30/84
53454	HATCH UNIT 1 - H2 AND O2 MONITORING TS			07/31/84
54495	HATCH #1 - HPCI AND RCIC LCO CHANGE TO 150 PSI			08/15/84
53476	HATCH - 1 TSIODINE LIMITS			08/30/84
54802	HATCH #1 APPENDIX R SCHEDULAR EXEMPTIONS			08/30/84
49090	HATCH 1 - PROPOSED T.S. CHANGE FOR FLOW MONITORING OF GASEOUS RELEASES FROM VENT		2	08/30/84
48771	HATCH 1 - TWO STAGE TARGET ROCK SAFETY/RELIEF VALVE PROBLEMS (TIA)		1	09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53595	HATCH 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53678	HATCH 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53761	HATCH 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53844	HATCH 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53988	HATCH 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54071	HATCH 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84



DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: HATCH 2

PLANT LOCATION: 11 MI N OF BAXLEY, GA  
 SOCKET NUMBER: 050-00366  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: R. ROGERS

LICENSED POWER: 2436 MWT  
 DESIGN POWER: 0784 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: G. RIVENBARK  
 BRANCH CHIEF: J. STOLZ  
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
42218	HATCH 2 - BWR SCRAM DISCHARGE VOLUME CAPABILITY		1	<u>COMPLETE</u> 01/04/84
42604	HATCH 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	01/16/84
07940	HATCH 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	01/25/84
47008	HATCH 2 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL		2	04/19/84
46663	HATCH 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	05/29/84
<u>ACTIVE ACTIONS</u>				
52235	HATCH 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52764	HATCH 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52845	HATCH 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52926	HATCH 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53007	HATCH 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
11261	HATCH 2 - IST		1	05/30/84
13278	HATCH 2 - DEFN OF OPERABLE		2	06/30/84
42506	HATCH 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
08298	HATCH 2 - SURV. OF MECHANICAL SNUBBERS		2	06/30/84
51681	HATCH 2 NUREG 0737 TECH SPECS			07/30/84
08095	HATCH 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		2	08/30/84
51097	HATCH 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	12/30/84
43738	HATCH 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	06/30/85
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44587	HATCH 2 - RV AND SV TESTING		1	<u>COMPLETE</u> 12/12/83
45832	HATCH 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/04/84
48171	HATCH 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	01/04/84
44856	HATCH 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	01/16/84
45692	HATCH 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/05/84
<u>ACTIVE ACTIONS</u>				
51167	HATCH 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	
51247	HATCH 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	
44095	HATCH 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS		1	06/30/84
44446	HATCH 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	08/30/84
45596	HATCH 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/30/84
45951	HATCH 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46022	HATCH 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46094	HATCH 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46310	HATCH 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
45140	HATCH 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
44238	HATCH 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	09/30/84
44308	HATCH 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	09/30/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

HATCH 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
43685	HATCH 2 - ADDS 3 VALVES IN SCRAM DISCH. VOL. SYST. TO T.S. LIST OF ISOLATION VALVES		1	10/03/83
52115	HATCH 2 - SPECIAL IGSCC INSPECITON PROGRAM			10/06/83
52321	HATCH 2 - REVISE CONFIRMATORY ORDER ON NUREG 0737 ITEM II.B.3			10/14/83
52103	HATCH 2 - TURBINE BYPASS AND LEVEL 8 TRIP SYSTEM TECH SPEC			10/28/83
53307	HATCH UNIT 2 - REVISE TS TO OPERATE WITHOUT RECIRCULATION FLOW			02/06/84
51393	HATCH UNIT 2 - AUDIT FREQUENCY CHANGE FOR EMERGENCY AND SECURITY PLANS		3	02/22/84
48296	HATCH 2 - ROD PATTERN ADJUSTMENTS - EXTENDS LCO TO SIX HOURS FROM TWO HOURS		2	02/22/84
49072	HATCH 2 - MINIMUM SHIFT CREW INCREASE NUREG-0737 RELATED T.S. CHANGE			02/22/84
49263	HATCH 2 - CORRECT ERROR IN T.S. RELATED TO SECTIONAL REFERFNCS		2	02/22/84
52483	HATCH 2 - PIPE REPLACEMENT PLAN - RECIRC SYSTEM			02/28/84
53479	HATCH 2 - EXTEND COMPLETION SCHEDULE FOR NUREG 0737 ITEM II.B.3			03/08/84
48297	HATCH 2 - INOPERABLE LPC1 INVERTER - EXTENDS LCO TO SEVEN DAYS FROM 44 HOURS		2	03/27/84
48749	HATCH 2 - FIRE PROTECTION PROGRAM - APP.R EXEMPTION REQUESTS & SAFE SHUTDOWN REPORT		1	04/18/84
48121	HATCH 2 - ADMIN. TECH SPEC CHNG RELATED TO REPORTING RESULTS OF TYPE B&C LEAK TESTS		2	06/20/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
51985	HATCH #2 PURGE & VENT VALVE - SYST-DESIGN & TS CHANGES			06/15/84
54607	HATCH 2 - AMENDMENT - (CYCLE 5) RELOAD 4			06/15/84
54169	HATCH 2 - ANALOGUE TRANSMITTER TRIP SYSTEM - TS			06/15/84
54433	HATCH 2 - APRM/RBM TECH SPEC CHANES			06/15/84
54460	HATCH #2 EMERGENCY EXERCISE EXEMPTION			06/30/84
52640	HATCH UNIT 2 - ADD TECH SPECS FOR ITEM II.K.3.22 RCIC SUCTION AND TRANSFER			06/30/84
53328	HATCH #2 - CLOSURE TIME FOR SDV VENT AND DRAIN VALVES			06/30/84
54462	HATCH #2 TECH SPECS FOR TMI ACTION ITEM IIF.1.1, 2 AND 3			07/15/84
11158	HATCH 2 - SETTLEMENT OF CLASS I STRUCTURES		1	07/30/84
49077	HATCH 2 - TITLE CHANGES OF PLANT REVIEW BOARD MEMBERS IN APP.A & B T.S.		2	07/30/84
53398	HATCH 2 - RELAXATION OF TIP LCO			07/31/84
53455	HATCH #2 H2 AND O2 MONITORING TS			07/31/84
53477	HATCH 2 - IODINE LIMITS			08/30/84
49091	HATCH 2 - PROPOSED T.S. CHANGE FOR FLOW MONITORING OF GASEOUS RELEASES FROM VENT		2	08/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53596	HATCH 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53679	HATCH 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53762	HATCH 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53845	HATCH 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53989	HATCH 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54072	HATCH 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: HUMBOLDT BAY 3

PLANT LOCATION: 4 MI SW OF EUREKA, CA  
 DOCKET NUMBER: 050-00133  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: P. MORRILL

LICENSED POWER: 0220 MW  
 DESIGN POWER: 0065 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: P. ERICKSON  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

TAC #    TAC DESCRIPTIONPLANT SPECIFICPRIORITYCRITICAL DATECOMPLETED ACTIONS

52637    HUMBOLDT BAY UNIT 2 -- DISPOSAL OF VERY LOW-LEVEL RADIOACTIVE WASTE

COMPLETE  
 05/07/84

ACTIVE ACTIONS

01863    HUMBOLDT BAY - SEISMIC CONCERNS  
 53378    HUMBOLDT BAY UNIT 3 - MICROSEISMIC MONITORING NETWORK

1

TARGET  
 05/01/84  
 07/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: INDIAN POINT 2

PLANT LOCATION: 25 MI N OF NEW YORK CITY, NY  
 DOCKET NUMBER: 050-00247  
 ARCH/ENGINEER: UEC  
 IE INSPECTOR: T. REBELOWSKI

LICENSED POWER: 2758 MWT  
 DESIGN POWER: 0873 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: P. POLK  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
42862	INDIAN POINT 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	COMPLETE 10/19/83
03296	INDIAN POINT 2 - 10 CFR 50.55A(G) - INSERVICE - 1ST		1	1/28/83
08700	INDIAN POINT 2 - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT		2	02/21/84
06901	INDIAN POINT 2 - ASYMETRIC LOCA LOADS		2	03/23/84
06603	INDIAN POINT 2 - REACTOR VESSEL OVERPRESSURE PROTECTION SYSTEM		2	04/24/84

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
51098	INDIAN POINT 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			TARGET
52236	INDIAN POINT 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52765	INDIAN POINT 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52846	INDIAN POINT 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52927	INDIAN POINT 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53008	INDIAN PT 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53074	INDIAN POINT 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53127	INDIAN POINT 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53176	INDIAN POINT 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		2	03/30/84
07996	INDIAN PT. 2 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	03/30/84
42096	INDIAN POINT 2 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		2	03/31/84
43006	INDIAN POINT 2 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	03/31/84
08113	INDIAN POINT 2 - APPENDIX I TECH SPEC IMPLEMENTATION		3	03/31/84
43963	INDIAN POINT 2 - SAFE SHUTDOWN CAPABILITY		1	04/30/84
08763	INDIAN POINT 2 - HYDRAULIC SNUBBERS		3	06/30/84
49736	INDIAN POINT 2 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"			06/30/84
46839	INDIAN POINT 2 - PWR MSLB WITH CONTINUED FEEDWATER ADDITION		2	06/30/84
42464	INDIAN POINT 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-20)		1	12/30/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44239	INDIAN POINT 2 - NUREG-0737 I.C.1.2.A INADEQUATE CORE COOLING GUIDELINES			COMPLETE 10/03/83
46239	INDIAN POINT 2 - NUREG-0737 III.A.1.2, EMERGENCY PLAN UPGRADE TO MEET RULE			10/03/83
45636	INDIAN POINT 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES			10/19/83
47945	INDIAN POINT 2 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	10/26/83
45833	INDIAN POINT 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE			12/15/83
47802	INDIAN POINT 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/03/84
46311	INDIAN POINT 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE			03/15/84
45141	INDIAN POINT 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION			03/20/84
46888	INDIAN PT. 2 - II K2.13 THERMAL MECHANICAL REPORT		1	06/15/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
51168	INDIAN POINT 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			TARGET
51248	INDIAN POINT 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
48172	INDIAN POINT 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	03/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## INDIAN POINT 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
44447	INDIAN POINT 2	- NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		<u>TARGET</u> 03/30/84
44588	INDIAN POINT 2	- RV AND SV TESTING		03/31/84
51912	INDIAN POINT 2	- NUREG 0737 TECH. SPEC. CHANGE PER 10/7/81 AND 11/19/81 II.F.1.1 & II.F.1.2		03/31/84
45952	INDIAN POINT 2	- NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		09/30/84
46023	INDIAN POINT 2	- NUREG-0737 OPERATIONAL SUPPORT CENTER		09/30/84
46095	INDIAN POINT 2	- NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		09/30/84
44309	INDIAN POINT 2	- NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		09/30/84
<u>PLANT SPECIFIC</u>				
<u>TAC #</u>	<u>TAC DESCRIPTION</u>		<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52521	INDIAN POINT 2	- APPENDIX H SCHEDULAR EXEMPTION		<u>COMPLETE</u> 11/07/83
51923	INDIAN POINT 2	- OPERABILITY TECHNICAL SPECIFICATION CHANGE		11/15/83
08776	INDIAN POINT 2	- PWR PUMP & S/G SUPPORTS - LAMELLAR TEARING & FRACTURE TOUGHNESS	2	11/23/83
51919	INDIAN POINT 2	- WATER ABOVE REACTOR TECHNICAL SPECIFICATION		02/28/84
51917	INDIAN POINT 2	- REACTOR VESSEL SURVEILLANCE CAPSULE PROGRAM TECH. SPEC.		03/07/84
51916	INDIAN POINT 2	- POST ACCIDENT CONTAINMENT VENTING TECH. SPEC.		03/14/84
54963	INDIAN POINT 2	- REVIEW AND APPROVE STEAM GENERATOR EXAM PROGRAM		05/30/84
53429	INDIAN POINT 2	- CONTAINMENT RESPONSE WITH SPRAY ISOLATED		06/28/84
<u>ACTIVE ACTIONS</u>				
54221	INDIAN POINT 2	- ORGANIZATION STRUCTURE TS CHANGE		<u>TARGET</u>
54680	INDIAN POINT 2	- EMERGENCY PREPAREDNESS EXERCISE EXTENSION		
48223	INDIAN POINT 2	- BORON INJECTION TANK OPERATION AND SURVEILLANCE REQUIREMENTS	3	03/30/84
49799	INDIAN POINT 2	- CONTAINMENT SEISMIC CAPABILITY		03/30/84
12842	INDIAN POINT 2	- GAS TURBINE OPERATION	1	03/31/84
51913	INDIAN POINT 2	- INSERVICE INSPECTION PROGRAM TECH. SPEC. CHANGE		03/31/84
51921	INDIAN POINT 2	- EXTERNAL FLOODING CONDITIONS TECH. SPEC.		03/31/84
51922	INDIAN POINT 2	- CONTROL OF HEAVY LOADS TECHNICAL SPECIFICATION		03/31/84
51918	INDIAN POINT 2	- BORIC ACID ADDITION TECH. SPEC. CHANGE		05/31/84
51925	INDIAN POINT 2	- STS UPGRADE		05/31/84
51914	INDIAN POINT 2	- ALTERNATE SHUTDOWN CAPABILITY TECH. SPEC. CHANGE		06/30/84
48055	INDIAN POINT 2	- PRA	3	06/30/84
54394	INDIAN POINT 2	- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		09/30/84
54539	INDIAN POINT 2	- TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		09/30/84
49672	INDIAN POINT 2	- REACTOR COOLANT PUMP TRIP		02/01/85
54226	INDIAN POINT 2	- IST PROGRAM II		02/28/85
<u>ANTICIPATED ACTIONS</u>				
53597	INDIAN POINT 2	- ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		<u>INITIATION</u> 11/01/84
53680	INDIAN POINT 2	- ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		11/01/84
53763	INDIAN PT 2	- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		11/01/84
53846	INDIAN POINT 2	- ITEM 3.2.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		11/01/84
53914	INDIAN POINT 2	- ITEMS 4.2.3 & 4.2.4 - PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS - LIFE TESTING & REPLACEMENT		11/01/84
53990	INDIAN POINT 2	- ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		11/01/84

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TECHNICAL ASSIGNMENT CONTROL SYSTEM

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

INDIAN POINT 2

IAC #    IAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

INITIATION

54073    INDIAN POINT 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: INDIAN POINT 3

PLANT LOCATION: 25 MI N OF NEW YORK CITY, NY  
 DOCKET NUMBER: 050-00286  
 ARCH/ENGINEER: UEC  
 IE INSPECTOR: T. KENNY

LICENSED POWER: 3025 MWT  
 DESIGN POWER: 0965 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: P. POLK  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08773	INDIAN POINT 3 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	<u>COMPLETE</u> 03/28/84
49421	INDIAN POINT 3 - BLOCKING SI SIGNAL DURING COOLDOWN			04/26/84
08337	INDIAN POINT 3 - REACTOR VESSEL OVERPRESSURE PROTECTION		2	06/28/84
<u>ACTIVE ACTIONS</u>				
51099	INDIAN POINT 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52237	INDIAN POINT 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52766	INDIAN POINT 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52847	INDIAN POINT 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52928	INDIAN POINT 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53009	INDIAN PT 3-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53075	INDIAN POINT 3 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53128	INDIAN POINT 3 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53177	INDIAN POINT 3 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
10029	INDIAN POINT 3 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE		2	04/30/83
12993	INDIAN POINT 3 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES		2	04/31/83
08114	INDIAN POINT 3 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/30/83
08765	INDIAN POINT 3 - HYDRAULIC SNUBBERS		3	07/01/83
42097	INDIAN POINT 3 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		2	07/01/83
42558	INDIAN POINT 3 - LONG TERM CONTAINMENT PURGE AND VENT (B-24)		1	07/01/83
07997	INDIAN PT 3 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POGL (NUREG 0612)		2	08/31/83
49737	INDIAN POINT 3 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"			09/30/83
42474	INDIAN POINT 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/31/83
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44240	INDIAN POINT 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	<u>COMPLETE</u> 10/03/83
46240	INDIAN POINT 3 - III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE		1	10/03/83
45237	INDIAN POINT 3 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	12/15/83
45283	INDIAN POINT 3 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	12/15/83
45834	INDIAN POINT 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/15/83
47803	INDIAN POINT 3 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/18/84
47946	INDIAN POINT 3 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	02/28/84
46312	INDIAN POINT 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	03/15/84
46889	INDIAN PT. 3 - II K2.13 THERMAL MECHANICAL REPORT		1	06/15/84
<u>ACTIVE ACTIONS</u>				
51169	INDIAN POINT 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51249	INDIAN POINT 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
48173	INDIAN POINT 3 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/30/83
45142	INDIAN POINT 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/83
48422	INDIAN POINT 3 - NUREG 0737, II.K.3.25 POWER ON PUMP SEALS			07/01/83

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## INDIAN POINT 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
44448	INDIAN POINT 3 - NUREG-0737	II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS	1	TARGET 06/30/84
45953	INDIAN POINT 3 - NUREG-0737	III.A.1.2, TECHNICAL SUPPORT CENTER	1	09/30/84
46024	INDIAN POINT 3 - NUREG-0737	III.A.1.2, OPERATIONAL SUPPORT CENTER	1	09/30/84
46096	INDIAN POINT 3 - NUREG-0737	III.A.1.2, EMERGENCY OPERATIONS FACILITY	1	09/30/84
44310	INDIAN POINT 3 - NUREG-0737	I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES	1	11/30/84
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
08777	INDIAN POINT 3 - PWR PUMP & S/G SUPPORTS -	LAMELLAR TEARING & FRACTURE TOUGHNESS	2	COMPLETE 12/02/83
53227	INDIAN POINT 3 -	SHIFT MANNING - PLANT SPECIFIC		12/31/83
49591	INDIAN POINT 3 -	SHUTDOWN REACTIVITY		01/31/84
54650	INDIAN POINT 3 -	CHARCOAL DISPOSAL		04/30/84
51947	INDIAN POINT 3 -	BOP CHEMISTRY TECH SPEC CHANGE		05/30/84
<u>ACTIVE ACTIONS</u>				
49873	INDIAN POINT 3 -	REVIEW OF CONTAINMENT COOLING TECHNICAL SPECIFICATIONS		TARGET 06/30/83
51944	INDIAN POINT 3 -	CONTAINMENT ISOLATION VALVES TECH SPEC CHANGE		12/31/83
51949	INDIAN POINT 3 -	SNUBBER SURVEILLANCE TECH SPEC CHANGE		12/31/83
51950	INDIAN POINT 3 -	MANAGEMENT REORGANIZATION TECH SPEC CHANGE (6/82)		12/31/83
51951	INDIAN POINT 3 -	MANAGEMENT REORGANIZATION TECH SPEC CHANGE (3/83)		12/31/83
51946	INDIAN POINT 3 -	RETS TECH SPEC CHANGE		12/31/83
51952	INDIAN POINT 3 -	FIRE PROTECTION MODIFICATIONS TECH SPEC CHANGE		12/31/83
12817	INDIAN POINT 3 -	STANDARD TECH SPECS	1	12/31/83
51943	INDIAN POINT 3 -	SITE MANAGEMENT REORGANIZATION TECH SPEC CHANGE		01/30/84
51945	INDIAN POINT 3 -	CONTAINMENT ISOLATION VALVE TECH SPEC CHANGE		02/20/84
54395	INDIAN POINT 3 -	RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		09/30/84
54540	INDIAN POINT 3 -	TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		09/30/84
48056	INDIAN POINT 3 -	PRA	3	12/30/84
54850	INDIAN POINT 3 -	LER ADMINISTRATIVE CONTROLS TECHNICAL SPECIFICATION		12/31/84
49687	INDIAN POINT 3 -	REACTOR COOLANT PUMP TRIP		02/01/85
43373	INDIAN POINT 3 -	SI REVIEW AND EVALUATION	1	09/30/85
<u>ANTICIPATED ACTIONS</u>				
53764	INDIAN PT 3 -	ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		INITIATION 11/01/84
53847	INDIAN POINT 3 -	ITEM 3.2.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		11/01/84
53915	INDIAN POINT 3 -	ITEMS 4.2.3 & 4.2.4 - PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS - LIFE TESTING & REPLACEMENT		11/01/84
53991	INDIAN POINT 3 -	ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		11/01/84
54074	INDIAN POINT 3 -	ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES		11/01/84
53598	INDIAN POINT 3 -	ITEM 1.2 - POST TRIP REVIEW - DATA AND INFORMATION CAPABILITY		11/01/84
53681	INDIAN POINT 3 -	ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: KEWAUNEE

PLANT LOCATION: 27 MI E OF GREEN BAY, WI.  
 DOCKET NUMBER: 050-00305  
 ARCH/ENGINEER: PSE  
 IE INSPECTOR: R. NELSON

LICENSED POWER: 1650 MWT  
 DESIGN POWER: 0535 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: D. NEIGHBORS  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08239	KEWAUNEE - PWR SECONDARY CHEMISTRY MONITORING REQUIREMENTS - GENERIC		2	<u>COMPLETE</u> 10/06/83
12269	KEWAUNEE - HELB AND CONSEQUENTIAL SYSTEM FAILURE KEWAUNEE		3	02/02/84
49738	- KEWAUNEE NUREG 0737 TECH SPECS (GENERIC LTR 82-16)			02/10/84
46747	KEWAUNEE -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL NUREG-0612, MPA MC-10.		2	03/16/84
<u>ACTIVE ACTIONS</u>				
42465	KEWAUNEE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	<u>TARGET</u>
51100	KEWAUNEE - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52767	KEWAUNEE - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION DESCRIPTION AND PROCEDURES			
52848	KEWAUNEE - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52929	KEWAUNEE - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53010	KEWAUNEE-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53076	KEWAUNEE - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53129	KEWAUNEE - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53178	KEWAUNEE - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42864	KEWAUNEE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	12/30/83
08525	KEWAUNEE - TECH SPEC SURVEILLANCE OF HYDRAULIC SNUBBERS		3	12/30/83
10754	KEWAUNEE - ASYMMETRIC LOCA LOADS		2	12/31/83
43008	KEWAUNEE - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	12/31/83
08145	KEWAUNEE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	12/31/83
06886	KEWAUNEE - REACTOR VESSEL OVERPRESSURE PROTECTION		2	01/31/84
08350	KEWAUNEE - WESTINGHOUSE UPPER PLENUM ECCS INJECTION - LONG RANGE		3	06/30/84
52238	KEWAUNEE - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47804	KEWAUNEE - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 01/09/84
44098	KEWAUNEE - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			04/18/84
46890	KEWAUNEE - II K2.13 THERMAL MECHANICAL REPORT		1	06/18/84
44241	KEWAUNEE - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/28/84
<u>ACTIVE ACTIONS</u>				
46313	KEWAUNEE - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u>
44449	KEWAUNEE - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	
44590	KEWAUNEE - RV AND SV TESTING		1	
45835	KEWAUNEE - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	
48174	KEWAUNEE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	
51170	KEWAUNEE - I.D.1 CONTROL ROOM DESIGN REVIEW			
51250	KEWAUNEE - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45143	KEWAUNEE - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/31/83
45954	KEWAUNEE - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46025	KEWAUNEE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

KEWAUNEE

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMJ ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
46097	KEWAUNEE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	<u>TARGET</u> 09/30/84
44311	KEWAUNEE - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49860	KEWAUNEE - OPERABILITY OF PRATT PURGE VALVES			<u>COMPLETE</u> 11/21/83
52148	KEWAUNEE - PURGE & VENT			11/21/83
51500	KEWAUNEE - PROP. AMEND 54 - MISCEL 4/29/83			12/23/83
49360	KEWAUNEE			02/09/84
52389	KEWAUNEE - APPENDIX R SCHEDULE EXEMPTION 2			02/28/84
54476	KEWAUNEE - EXEMPTION FROM APPENDIX J			04/24/84
54209	KEWAUNEE - UNDERVOLTAGE TRIP SETPOINTS			04/30/84
53358	KEWAUNEE - TOTAL PEAKING FACTOR AND EXPOSURE RANGE			06/19/84
<u>ACTIVE ACTIONS</u>				
49609	KEWAUNEE RESPONSE TO GL 82-17			<u>TARGET</u> 12/31/83
49859	KEWAUNEE - ANALOG ROD POSITION INDICATION		1	12/31/83
49131	KEWAUNEE - APPENDIX J TECH. SPECS.		3	02/01/84
46817	KEWAUNEE: PRESSURE ISOLATION VALVES			02/24/84
53488	KEWAUNEE - EVALUATE REMOVAL OF RHR AUTOCLOSURE FEATURE			03/15/84
54195	KEWAUNEE - RELIEF REQUESTS (HYDRO TESTS)			06/30/84
53493	KEWAUNEE - ISI TECHNICAL SPECIFICATIONS			09/30/84
54396	KEWAUNEE- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54541	KEWAUNEE - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			10/01/84
55221	KEWAUNEE - ADMINISTRATIVE TECHNICAL SPECIFICATION CHANGES			02/01/85
49671	KEWAUNEE - REACTOR COOLANT PUMP TRIP			04/30/85
54739	KEWAUNEE IST - 2ND TEN YEAR PROGRAM			04/30/85
54740	KEWAUNEE - ISI 2ND YEAR PROGRAM			
<u>ANTICIPATED ACTIONS</u>				
53599	KEWAUNEE - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53682	KEWAUNEE - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53765	KEWAUNEE- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53848	KEWAUNEE - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53916	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53992	KEWAUNEE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54075	KEWAUNEE - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: LA CROSSE

PLANT LOCATION: 19 MI S OF LACROSSE, WISC  
 DOCKET NUMBER: 050-00409  
 ARCH/ENGINEER: SSL  
 IE INSPECTOR: J. WEIBE

LICENSED POWER: 0165 MWT  
 DESIGN POWER: 0050 MWE  
 NSSS VENDOR: AC

PROJECT MANAGER: R. DUDLEY  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
46664	LACROSSE - IMPLEMENTATION OF NUREG-0313, REV. 1		2	<u>COMPLETE</u> 10/14/83
43980	LACROSSE - APPENDIX R REVIEW		1	04/04/84
42611	LACROSSE - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	04/30/84
<u>ACTIVE ACTIONS</u>				
52849	LACROSSE - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR VENDOR INTERFACE - RTS COMPONENTS			<u>TARGET</u>
52930	LACROSSE - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53011	LACROSSE-ITEM 3.1.3 --POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
52768	LACROSSE - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION DESCRIPTION AND PROCEDURES			
08099	LACROSSE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW			
51682	LA CROSSE NUREG 0737 TECH SPECS		3	06/15/84
52239	LACROSSE - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			08/15/84
42522	LACROSSE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	09/30/84
42912	LACROSSE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	10/15/84
51101	LACROSSE - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			01/01/86
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
48175	LACROSSE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>COMPLETE</u> 02/02/84
45836	LACROSSE - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	02/02/84
45889	LACROSSE - NUREG-0737 II.K.3.44, TRANSIENTS WITH SINGLE FAILURES		1	03/26/84
47609	LACROSSE - NUREG 0737 ITEM II F1.4 CONTAINMENT PRESSURE INSTRUMENT		1	05/14/84
47680	LACROSSE - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	05/14/84
47751	LACROSSE - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	05/14/84
44242	LACROSSE - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
45955	LACROSSE - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	<u>TARGET</u> 09/30/84
46026	LACROSSE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46098	LACROSSE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46314	LACROSSE - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
51171	LACROSSE - I.D.1 CONTROL ROOM DESIGN REVIEW		1	09/30/84
44450	LACROSSE - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/30/84
44591	LA CROSSE - RV AND SV TESTING		1	11/30/84
44312	LACROSSE - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	07/15/85
45144	LACROSSE - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/01/85
51251	LACROSSE - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	09/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51876	LACROSSE - SEP TOPIC VI-7.C.1, APPENDIX K - ELECTRICAL REVIEWS			<u>COMPLETE</u> 10/07/83
49529	LA CROSSE - DEGRADED GRID VOLTAGE TECH SPECS		3	10/14/83



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

LA CROSSE

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
47536	LACROSSE - REVIEW FIRE PROTECTION TECH SPECS		2	10/14/83
51039	LACROSSE - REVIEW ES/SS SURVEILLANCE REQUIREMENTS		3	10/14/83
47533	LACROSSE TECH SPEC CHANGES (MISC)		3	10/14/83
49971	LACROSSE - EQUIPMENT QUALIFICATION SCHEDULE EXEMPTION REQUEST		2	10/19/83
51877	LACROSSE - SEP TOPIC VII-1.A, ISOLATION OF RPS			10/27/83
51872	LACROSSE - SEP TOPIC III-5.B, PIPE BREAK OUTSIDE CONTAINMENT			10/27/83
53224	LA CROSSE - TEMPORARY CHANGE TO OPERATION REQUALIF PROGRAM			12/02/83
49040	LA CROSSE: VENTILATION VALVE SEAL TESTING		2	12/12/83
52306	LA CROSSE - OPERATOR STAFFING RULE EXEMPTION			12/30/83
51875	LACROSSE - SEP TOPIC VI-4, CONTAINMENT ISOLATION			04/26/84
51879	LACROSSE - SEP TOPIC IX-5, VENTILATION SYSTEMS			05/14/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
53264	LA CROSSE - BYPASS LINWELD FAILURE			04/15/84
52171	LA CROSSE - RT-NDT SHIF? TECH SPEC			04/30/84
52172	LA CROSSE - NUCLEAR INST. TECH SPEC			04/30/84
52169	LA CROSSE - ECCS TECH SPEC REVISION			05/01/84
52170	LA CROSSE - SAFETY VALVES			05/01/84
53413	LACROSSE - SEP TECH SPECS			06/01/84
53414	LACROSSE - WATER QUALITY & FLOODING TECH SPECS		2	06/01/84
49088	LA CROSSE - SECURITY PLAN AMENDMENT			06/15/84
51871	LACROSSE - SEP TOPIC III-5.A, PIPE BREAK INSIDE CONTAINMENT			07/01/84
51868	LACROSSE - SEP TOPIC III-3.A, EFFECTS ON HIGH WATER LEVEL ON STRUCTURES			07/01/84
51870	LACROSSE - SEP TOPIC III-4.B, TURBINE MISSILES			07/01/84
54197	LA CROSSE - POST ACCIDENT TECH SPECS			07/01/84
51318	LACROSSE - VENT VALVE OPERABILITY TEST		1	07/15/84
51880	LACROSSE INTEGRATED ASSESSMENT SUPPLEMENT			07/31/84
51878	LACROSSE - SEP TOPIC VIII-3.B, DC BUS VOLTAGE MONITORING			08/01/84
54621	LA CROSSE - AIRBORNE DOSE CALCULATIONS			08/30/84
51867	LACROSSE - SEP TOPIC III-2, WIND & TORNADO LOADINGS			08/30/84
51869	LACROSSE - SEP TOPIC III-4.1, TORNADO MISSILES		1	09/30/84
49136	LACROSSE - IN-SERVICE TESTING			09/30/84
51874	LACROSSE - SEP TOPIC III-7.B, DESIGN CODES, DESIGN CRITERIA, LOAD COMBINATIONS			09/30/84
54660	LA CROSSE - SEP TOPIC II-3.C: LOSS OF ULTIMATE HEAT SINK			09/30/84
54666	LA CROSSE - LER RULE REPORTING REQUIREMENTS			09/30/84
54895	LA CROSSE - APPENDIX J: AILOCK EXEMPTION			09/30/84
54542	LACROSSE - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		1	09/30/84
11233	LACROSSE - FTL CONVERSION			10/30/84
51873	LACROSSE - SEP TOPIC III-6, SEISMIC DESIGN CONSIDERATIONS			10/31/84
55250	LA CROSSE - CONTAINMENT PRESSURE MONITOR		2	11/01/84
48681	LA CROSSE - CONTAINMENT LEAK TESTING		2	11/01/84
49895	LA CROSSE - LOCAL LEAK DETECTION CALCULATIONS			01/31/85
52514	LA CROSSE - CONFIRMATION OF SEP ACTION ITEMS			
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53600	LACROSSE - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53683	LACROSSE - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53766	LACROSSE- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53849	LACROSSE - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84



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(CONTINUATION)

IAC # IAC DESCRIPTION

ANTICIPATED ACTIONS (CONTINUATION)

53993 LACROSSE - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS

54076 LACROSSE - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

TECHNICAL ASSIGNMENT CONTROL SYSTEM  
CONDENSED MANAGEMENT REPORT

LA CROSSE

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

INITIATION

11/01/84

11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: LASALLE 1

PLANT LOCATION: 11 MI SE OF OTTAWA, ILL  
 DOCKET NUMBER: 050-00373  
 ARCH/ENGINEER: S&L  
 IE INSPECTOR: R. HEISHMAN

LICENSED POWER: 3323 MWT  
 DESIGN POWER: 1078 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: A. BOHRNIA  
 BRANCH CHIEF: A. SCHWENCER  
 LIC. ASSISTANT: E. HYLTON

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51683	LA SALLE 1 NUREG 0737 TECH SPECS			<u>COMPLETE</u> 02/09/84
48482	LA SALLE 1 - IMPLEMENTATION OF NUREG-0313, REV. 1			06/11/84
<u>ACTIVE ACTIONS</u>				
52240	LASALLE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52769	LASALLE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52850	LASALLE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52931	LASALLE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53012	LASALLE 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			12/21/84
51102	LASALLE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	03/31/85
42536	LASALLE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
51172	LASALLE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51252	LASALLE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
49460	LA SALLE 1 - INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING (II.F.2)			
54582	LASALLE 1 - II.D.1 - RV & SV TESTING			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52073	LA SALLE UNIT 1 - REVIEW OF SEISMIC & LOCA LOADS ON FUELS			<u>COMPLETE</u> 11/01/83
52716	LASALLE 1 - REACTOR FEEDWATER INBOARD CHECK VALVE TYPE C TEST			12/06/83
53382	LA SALLE UNIT 1 - TECH SPEC CHANGES FOR SPECIFICATION 30.4 NOT BEING APPLICABLE WITH CONTROL ROD NOT OPERABLE			01/17/84
53466	LA SALLE UNIT 1 - 2.206 REQUEST FROM GOGAL ON INTEGRATED LEAK RATE TESTING OF CONTAINMENT			03/16/84
53303	LA SALLE UNIT 1 - EXIGENT TECH SPEC CHANGE REGARDING DIESEL GENERATORS FAST STARTS			03/19/84
54308	LASALLE 1 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
54971	LA SALLE UNIT 1 - MEETING TO DISCUSS FINE MOTION CONTROL ROD DRIVE DEMONSTRATION			06/08/84
<u>ACTIVE ACTIONS</u>				
51494	LA SALLE 1 - TECH SPEC REQUIREMENTS RESPONSE TIME TESTING OF TURBINE STOP VALVE CLOSURE SCRAM & RECIRC PUMP TRIP			<u>TARGET</u> 06/29/84
55081	LASALLE UNIT 1 - EXIGENT T/S CHANGE TO TABLE 3.3.2.2 TO MODIFY STEAMLINE TUNNEL TRIP SETPOINT AND ALLOWABLE VALVE			06/30/84
53480	LASALLE UNIT 1 UPGRADE THE TECH SPEC TO MAKE THEM CONSISTENT WITH THE UNIT 2 TECH SPECS			07/27/84
52297	LA SALLE UNIT 1- REVIEW OF EXTENDED BLOWDOWN TEST EVALUATION OF SUPPRESSION POOL TEMPERATURE MEASUREMENTS			09/28/84
54995	LA SALLE UNIT 1 - REVIEW INSERVICE TESTING OF PUMPS AND VALVES			09/28/84
<u>ANTICIPATED ACTIONS</u>				
53601	LASALLE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

LASALLE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				<u>INITIATION</u>
53684	LASALLE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53767	LASALLE 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53850	LASALLE 1 - ITEM 3.2 - POST-MAINTENANCE TESTING - CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53994	LASALLE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54077	LASALLE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
54970	LASALLE 1 - ITEM 1.1 - POST TRIP REVIEW (PROGRAM DESCRIPTION AND PROCEDURES)			04/01/85



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## CONDENSED MANAGEMENT REPORT

FACILITY: LASALLE 2

PLANT LOCATION: 11 MI SE OF OTTAWA, ILL  
 DOCKET NUMBER: 050-00374  
 ARCH/ENGINEER: S&L  
 IE INSPECTOR: R. HEISHMAN

LICENSED POWER: 3323 MWT  
 DESIGN POWER: 1078 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: A. BOURNIA  
 BRANCH CHIEF: A. SCHWENCER  
 LIC. ASSISTANT: E. HYLTON

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
42537	LASALLE 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	<u>TARGET</u> 03/31/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
54583	LASALLE 2 - II.D.1 - RV & SV TESTING			<u>TARGET</u>
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
53467	LASALLE UNIT 2 2.206 REQUEST FROM GOGOL ON INTEGRATED LEAK RATE TESTING OF CONTAINMENT			<u>COMPLETE</u> 03/16/84
54178	LA SALLE UNIT 2 - RESPOND TO APPEAL LETTER ON FIRE PROTECTION			03/22/84
54248	LA SALLE 2 - REVIEW OF FIRE PROTECTION SUBMITTALS			03/22/84
54184	LA SALLE 2 - TECH SPEC AUDIT			03/27/84
54309	LASALLE 2 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
<u>ACTIVE ACTIONS</u>				
55062	LA SALLE UNIT2 - EXIGENT TECH SPEC CHANGE TO TABLE 3.3.2-2 TO MODIFY STEAMLINE TUNNEL TRIP SETPOINT & ALLOWABLE VALVE			<u>TARGET</u> 06/08/84
55055	LA SALLE UNIT 2: TECH SPEC CHANGE TO INCORPORATE REACTOR SCRAM ON LOW CONTROL ROD DRIVE PUMP DISCHARGE PRESSURE			06/29/84
54996	LA SALLE UNIT 2 - REVIEW INSERVICE TESTING OF PUMPS AND VALVES			09/28/84
<u>ANTICIPATED ACTIONS</u>				
53995	LASALLE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			<u>INITIATION</u> 11/01/84
54078	LASALLE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53602	LASALLE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53685	LASALLE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53768	LASALLE 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53851	LASALLE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: MAINE YANKEE

PLANT LOCATION: 10 MI N OF BATH, ME  
 DOCKET NUMBER: 050-00309  
 ARCH/ENGINEER: S&W  
 IE INSPECTOR: C. HOLDEN

LICENSED POWER: 2630 MWT  
 DESIGN POWER: 0825 MWE  
 NSSS VENDOR: COMB

PROJECT MANAGER: K. HEITNER  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49613	MAINE YANKEE - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION			<u>COMPLETE</u>
43328	MAINE YANKEE - CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANE		2	12/02/83
12317	MAINE YANKEE - HELB AND CONSEQUENTIAL SYSTEM FAILURE MAINE YANKEE		3	12/30/83
08909	MAINE YANKEE - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT		2	01/31/84
10032	MAINE YANKEE - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE		2	03/14/84
51756	MAINE YANKEE - THERMAL SHIELD FOLLOW UP ANALYSIS		2	03/27/84
				05/01/84
<u>ACTIVE ACTIONS</u>				
51103	MAINE YANKEE - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52241	MAINE YANKEE - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52770	MAINE YANKEE ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52851	MAINE YANKEE - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52932	MAINE YANKEE - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53013	MAINE YANKEE-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
53077	MAINE YANKEE - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53130	MAINE YANKEE - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
08898	MAINE YANKEE - REVIEW OF ASYMMCTRIC LOCA LOADS SUBMITTAL		2	06/01/84
08904	MAINE YANKEE - HYDRAULIC SNUBBERS		3	06/01/84
42490	MAINE YANKEE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/01/84
42893	MAINE YANKEE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
43653	MAINE YANKEE - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	08/30/84
07752	MAINE YANKEE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	09/30/84
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
45263	MAINE YANKEE - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	<u>COMPLETE</u>
45309	MAINE YANKEE - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/04/83
47805	MAINE YANKEE - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	10/11/83
44243	MAINE YANKEE - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/01/84
44664	MAINE YANKEE - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION		1	06/04/84
46891	MAINE YANKEE - II K2.13 THERMAL MECHANICAL REPORT		1	06/06/84
<u>ACTIVE ACTIONS</u>				
51173	MAINE YANKEE - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51253	MAINE YANKEE - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45837	MAINE YANKEE - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
44451	MAINE YANKEE - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
45956	MAINE YANKEE - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46027	MAINE YANKEE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46099	MAINE YANKEE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
43176	MAINE YANKEE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
44592	MAINE YANKEE - RV AND SV TESTING		1	10/31/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MAINE YANKEE

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
44313	MAINE YANKEE	- NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES	1	TARGET 11/30/84
46315	MAINE YANKEE	- NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE	1	12/01/84
45145	MAINE YANKEE	- NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION	1	12/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52095	MAINE YANKEE	- FINANCIAL QUALS - SAFETY REVIEW		COMPLETE 10/04/83
49285	MAINE YANKEE	- FIRE EXEMPTION REQUESTS FOR SAFE SHUTDOWN		11/01/83
52174	MAINE YANKEE	- MODIFICATIONS ON OPERABILITY SPECS		11/14/83
52386	MAINE YANKEE	- MFW TRIP DESIGN DEFICIENCY		11/14/83
52301	MAINE YANKEE	- ISI TECH SPECS		12/21/83
52488	MAINE YANKEE	- SECOND EXEMPTION SHIFT MANNING RULE		01/12/84
53333	MAINE YANKEE	- APPEAL ON LIMIT OVERTIME		01/26/84
48674	MAINE YANKEE	- REVIEW OF FEEDWATER TRIP SYSTEM DESIGN		02/14/84
48962	MAINE YANKEE	- TECH SPEC FOR VENT & PURGE		02/14/84
49832	MAINE YANKEE	- YAEC-1 TECH SPECS		03/21/84
52300	MAINE YANKEE	- BOARD NOTIFICATION ON SECY PAPER		03/22/84
48612	MAINE YANKEE	- SPENT FUEL POOL HEARING AND LICENSE AMENDMENT		04/02/84
54960	MAINE YANKEE	- PROPOSED CHANGE 107		05/10/84
53460	MAINE YANKEE	- STEAM GENERATOR WATER LEVEL		05/31/84
53547	MAINE YANKEE	- CYCLE 8 RELOAD		05/31/84
51406	MAINE YANKEE	- SCHEDULE FOR NUREG-0737 SUPPLEMENT 1		06/14/84
<u>ACTIVE ACTIONS</u>				
51703	MAINE YANKEE	0 DEGRADED GRID VOLTAGE PROCEDURES		TARGET 12/31/83
52011	MAINE YANKEE	- SECOND ISI INTERVAL		02/28/84
48999	MAINE YANKEE	- ESFA3 & RP5 SENSOR TECH SPEC		02/28/84
49711	MAINE YANKEE	- SECOND IST INTERVAL		03/01/84
52173	MAINE YANKEE	- SEISMIC AND OTHER QUALS ON 8IN. VENT/PURGE VALVE		03/30/84
48632	MAINE YANKEE	- APP J TECH SPEC CHANGE		06/30/84
53546	MAINE YANKEE	- TECH SPEC FOR AFW INITIATION		06/30/84
53475	MAINE YANKEE	- APS VOLTAGE STUDY		06/30/84
53407	MAINE YANKEE	- REVIEW OF 10CFR 61		06/30/84
53449	MAINE YANKEE	- REVISED DEFINITION OF CONTAINMENT INTEGRITY		06/30/84
53450	MAINE YANKEE	- CONTAINMENT LEAK TESTING		06/30/84
54431	MAINE YANKEE	- EMERGENCY EXERCISE EXEMPTION REQUEST		06/30/84
49475	MAINE YANKEE	- BRIEFINGS ON SEISMIC REVIEW PLAN		06/31/84
51389	MAINE YANKEE	- ADMINISTRATIVE TECH. SPEC. REVIEW		07/30/84
48582	MAINE YANKEE	- FIRE PROTECTION EXEMPTION FOR		09/01/84
54543	MAINE YANKEE	- TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		09/30/84
54958	MAINE YANKEE	- THERMAL SHIELD INSPECTION AND REPAIR		09/30/84
54959	MAINE YANKEE	- LTOP TECHNICAL SPECIFICATION CHANGES		09/30/84
54397	MAINE YANKEE	- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		09/30/84
53513	MAINE YANKEE	- TECH SPECS FOR SAFE SHUTDOWN		09/30/84
55092	MAINE YANKEE	- CASKIN SPENT FUEL POOL		12/30/84
53408	MAINE YANKEE	- FIRE PROTECTION - EXTRA EXEMPTION REQUESTS		01/01/85
49673	MAINE YANKEE	- REACTOR COOLANT PUMP TRIP		02/01/85

ANTICIPATED ACTIONSINITIATION

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MAINE YANKEE

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
55295	MAINE YANKEE - AFW SYSTEM RESOLUTION			<u>INITIATION</u> 07/02/84
55296	MAINE YANKEE - PROPOSED CHANGE 105			07/02/84
53996	MAINE YANKEE - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54079	MAINE YANKEE - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53603	MAINE YANKEE - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53686	MAINE YANKEE - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53769	MAINE YANKEE- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53852	MAINE YANKEE - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53917	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: MCGUIRE 1

PLANT LOCATION: 17 MI N OF CHARLOTTE, NC  
 DOCKET NUMBER: 050-00369  
 ARCH/ENGINEER: DUKE  
 IE INSPECTOR: W. ORDERS

LICENSED POWER: 3411 MW  
 DESIGN POWER: 1180 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: R. BIRKEL  
 BRANCH CHIEF: E. ADENSAM  
 LIC. ASSISTANT: N. DUNCAN

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52771	MCGUIRE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52852	MCGUIRE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52933	MCGUIRE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53014	MCGUIRE 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53078	MCGUIRE 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53131	MCGUIRE 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53179	MCGUIRE 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
51104	MCGUIRE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52242	MCGUIRE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
47127	MCGUIRE 1 - CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANTS (NUREG 0612)		2	01/02/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
51174	MCGUIRE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51254	MCGUIRE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			03/01/84
45146	MCGUIRE - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	07/01/84
46316	MCGUIRE - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	07/01/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49804	MCGUIRE UNIT #1 - EVALUATE REQUEST (2/18/83) TO TRANSFER SPENT FUEL BETWEEN MCGUIRE UNITS 1 AND 2			<u>COMPLETE</u> 10/19/83
52111	MCGUIRE UNIT 1 - EVAL TECH SPEC CHANGES RELATING TO D/G & TURBINE VALVE SURVEILLANCE TEST'G (REF: DUKE LTR 8/1/83)			10/26/83
52390	MCGUIRE UNIT #1: EVALUTE REQUEST TO INCREASE CONTAINMENT LOWER CMPTMNT TMP TO 125 DEG F(T.S.3.6.1.5) REF: DUKE LTR)			12/02/83
53214	MCGUIRE UNIT #1: EVAL PROPOSED T.S. CHANGE RELATING TO MINIMUM RCS FLOW REQ'T AT 100% 3 REF: DUKE LTR NOV 18, 1983			02/04/84
52328	MCGUIRE UNIT 1 - EVAL T.S. CHANGE RELATING TO ADMIN & TYPO ERROR & DELETION OF 1 SNUBBER (TBL3.7-4B) REF: DUKE 8/2/83			02/28/84
52198	MCGUIRE UNIT 1: EVAL CHANGE IN STEAM LINE RESPONSE TIME (TS TBL 3.3.5) AND INOP S/G LEVEL INSTRU (TS TBL 3.3-1)			02/28/84
52329	MCGUIRE UNIT 1 - EVAL T.S. CHANGE RELATING TO POSITION OF INACCESSIBLE SPRINKLER SYS VALV (4.7.10.2) REF:DUKE 8/2/83			02/29/84
52164	MCGUIRE UNIT 1: EVAL PROPOSAL TO ALLOW CONTROL VENTILATION SYS TO BE INOP IN MODES 5/6;TS 3.7.6(REF: DUKE 8/2/83 LTR)			03/19/84
53257	MCGUIRE UNIT #1: EVAL REVISION TO EMERGENCY LIGHTING FOR FIRE PROTECTION; REF: DUKE LTR 11/18/83			04/16/84
53319	MCGUIRE UNIT 1 - EVALUATE MCGUIRE 1/CYCLE 2 OPTIMIZED FUEL ASSEMBLY RELOADS REF: DUKE LETTER 12/12/83			04/20/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MCGUIRE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52062	MCGUIRE UNIT #1 10CFR PART 21 INTERPRETATION REQUEST BY DUKE POWER CO (REF: LTR 7/20/83)			
53503	MCGUIRE UNIT 1 - EVALUATE REVISED IMPLEMENTATION DATE FOR LICENSE CONDITION 2.C.(11)F.(3)-ITEM II.F.2; REF DUKE LTR			03/25/84
54882	MCGUIRE UNIT #1 - RELIEF REQUEST ASME CODE SECTION XI, SNUBBER TESTING; REF: DUKE LETTER 4/12/84			05/15/84
55177	MCGUIRE UNIT #1 - EVALUATE MODIF TO POWER DIST LIMITS FOR BASE LOAD OPERATIONS REF = DUKE LTR-6/14/84			06/22/84
53531	MCGUIRE UNIT #1 EVALUATE MCGUIRE STATION SPENT FUEL POOLS RERACKING PROGRAM			06/30/84
53255	MCGUIRE UNIT #1: EVAL PROPOSED T.S. SURVEIL CHG RELATING TO DIESEL GENERATOR FUEL; T.S.4.B.1.1.2; REF: DUKE LTR 11/83			07/01/84
52160	MCGUIRE UNIT 1: EVAL PROPOSAL ALLOWING UNIT OP LESS THAN OR EQ TO 46% RATED THERMAL POWER WITH UHI INOP TS 3.5.1.2			07/01/84
51610	MCGUIRE UNIT #1: EVALUATE LICENSEE JUSTIFICATION FOR LONGTERM OPERATION WITH RCS THERMAL SLEEVES REMOVED			07/01/84
52178	MCGUIRE UNIT 1 - EVALUATE PROPOSED TECH. SPEC. SURV CHANGE RELATING TO I.C. INLET DOORS (T.S. 4.6.5.3.1)			07/01/84
52327	MCGUIRE UNIT 1 - EVAL T.S. CHANGES RELATED TO CONTAINMENT INTEGRITY (4.6.1.1.A) BYPASS LEAKAGE (TLB.3.6-1) & DISTRIB			07/01/84
55306	MCGUIRE UNIT 1 - EVALUATE APPLICATION FOR APPROVAL TO DISPOSE OF VERY LOW LEVEL RA WASTE; REF: DUKE LTR 6-18-84			08/15/84
54398	MCGUIRE 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54544	MCGUIRE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49653	MCGUIRE 1 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53604	MCGUIRE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53687	MCGUIRE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53770	MCGUIRE 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53853	MCGUIRE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53918	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53997	MCGUIRE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54080	MCGUIRE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: MCGUIRE 2

PLANT LOCATION: 17 MI N OF CHARLOTTE, NC  
 DOCKET NUMBER: 050-00370  
 ARCH/ENGINEER: DUKE  
 IE INSPECTOR: W. COTTLE

LICENSED POWER: 3411 MW  
 DESIGN POWER: 1180 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: R. BIRKEL  
 BRANCH CHIEF: E. ADENSAM  
 LIC. ASSISTANT: M. DUNCAN

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52243	MCGUIRE 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52772	MCGUIRE 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52853	MCGUIRE 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52934	MCGUIRE 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53015	MCGUIRE 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53079	MCGUIRE 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53180	MCGUIRE 2 - ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
51105	MCGUIRE 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			01/01/84
53132	MCGUIRE 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			09/08/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
54601	MCGUIRE 2 - II.D.1 - RV & SV TESTING			<u>TARGET</u>
51175	MCGUIRE 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			03/01/84
54985	MCGUIRE 2, II.K.2.13, THERMAL MECHANICAL REPORT			06/30/84
51255	MCGUIRE 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			07/01/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52112	MCGUIRE UNIT 2 - EVAL TECH SPEC CHANGES RELATING TO D/G & TURBINE VALVE SURVEILLANCE TEST'G (REF: DUKE LTR 8/1/83)			<u>COMPLETE</u> 10/26/83
52391	MCGUIRE UNIT 2: EVAL REQ TO INCREASE CONTAINMENT LOWER COMPARTMENT TMP TO 125 DEG F(T.S.3.6.1.5) REF:DUKE LTR			12/02/83
53289	MCGUIRE UNIT #2 - EXEMPTION TO MULTIPLE CRISIS MANAGEMENT CENTER STAFF DRILLS (REF: TAC# 51035)			01/06/84
53215	MCGUIRE UNIT #2: EVAL PROPOSED T.S. CHANGE RELATING TO MINIMUM RCS FLOW REQTS AT 100% REF: DUKE LTR 11/18/83			02/04/84
52331	MCGUIRE UNIT 2 - EVAL T.S. CHANGE RELATING TO ADMIN & TYPO ERROR & DELETION OF 1 SNUBBER (TBL3.7-4B) REF: DUKE 8/2/83			02/28/84
52199	MCGUIRE UNIT 2: EVAL CHANGE IN STEAM LINE RESPONSE TIME (TS TBL 3.3.5) AND INOP S/G LEVEL INSTRU (TS TBL 3.3-1)			02/18/84
52332	MCGUIRE UNIT 2 - EVAL T.S. CHANGE RELATING TO POSITION OF INACCESSIBLE SPRINKLER SYS. VALVES(4.7.10.2) REF: DUKE LTR			02/29/84
52163	MCGUIRE UNIT 2: EVAL PROPOSAL TO ALLOW CONTROL VENTILATION SYS TO BE INOP IN MCODES 5/6;TS 3.7.6(REF: DUKE 8/2/83 LTR)			03/19/84
49981	MCGUIRE UNIT #2: EVALUATE PROPOSED T.S. CHANGE 3/4.5.2 CONCERNING MAXIMUM CENTRIFUGAL CHARGING PUMP FLOW RATE			04/13/84
53258	MCGUIRE UNIT #2: EVALUATE REVISION TO EMERGENCY LIGHTING FOR FIRE PROTECTION; REF: DUKE LTR - NOV 18/1983			04/16/84
53320	MCGUIRE UNIT #2 - EVALUATE MCGUIRE 1/CYCLE 2 OPTIMIZED FUEL ASSEMBLY RELOAD; DUKE LTR. 12/12/83			04/20/84

ACTIVE ACTIONSTARGET

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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MCGUIRE 2

PLANT SPECIFIC

PRIORITY CRITICAL DATE

IAC # IAC DESCRIPTION

(CONTINUATION)

ACTIVE ACTIONS  
 52063 MCGUIRE UNIT #2: 10CFR21 INTERPRETATION REQUEST BY DUKE POWER CO. (REF: LTR 7/20/83)  
 54833 MCGUIRE UNIT #1 - RELIEF REQUEST ASME CODE SECTION XI, SNUBBER TESTING; REF: DUKE LETTER 4/12/84  
 55178 MCGUIRE UNIT #2 - EVALUATE MODIF TO POWER DIST LIMITS FOR BASE LOAD OPERATIONS REF = DUKE LTR-6/14/84  
 53532 MCGUIRE UNIT #2: EVALUATE MCGUIRE STATION SPENT FUEL POOLS RERACKING PROGRAM  
 53290 MCGUIRE UNIT #2 - REACTOR COOLANT PUMP TRIP (REF: TAC# 49653)  
 53256 MCGUIRE UNIT #2: EVAL PROPOSED T.S. SURVEIL CHG RELATING TO DIESEL GENERATOR FUEL: T.S. 4.B.1.1.2: REF: 11/18/83  
 53287 MCGUIRE UNIT #2 - NUREG-0737, III.A.2.2, METEOROLOGICAL DATA UPGRADE (REF: IAC# 46316)  
 53288 MCGUIRE UNIT #2 - NUREG-0737, II.F.2.3. INADEQUATE CORE COOLING INSTRUMENTATION (REF: TAC# 45146)  
 52330 MCGUIRE UNIT 2 - EVAL T.S. CHANGES RELATED TO CONTAINMENT INTEGRITY (4.6.1.1.A); BYPASS LEAKAGE(TBL 3.6-1) & DISTRIB  
 51609 MCGUIRE UNIT #2: EVALUATE LICENSEE JUSTIFICATION FOR LONGTERM OPERATION WITH RCS THERMAL SLEEVES REMOVED  
 52179 MCGUIRE UNIT 2 - EVALUATE PROPOSED TECH. SPEC. SURV CHANGES RELATING TO I.C INLET DOORS (T.S.4.6.5.3.1)  
 52159 MCGUIRE UNIT 2: EVAL PROPOSAL ALLOWING UNIT OP LESS THAN OR EQ TO 46% RATED THERMAL POWER WITH UHI INOP TS 5.3.5.2  
 55307 MCGUIRE UNIT 2 - EVALUATE APPLICATION FOR APPROVAL TO DISPOSE OF VERY LOW LEVEL RA WASTE; REF: DUKE LTR 6-19-84

ANTICIPATED ACTIONS

53605 MCGUIRE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY  
 53688 MCGUIRE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS  
 53771 MCGUIRE 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS  
 53854 MCGUIRE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS  
 53919 -ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT  
 53998 MCGUIRE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS  
 54081 MCGUIRE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

INITIATION

11/01/84  
 11/01/84  
 11/01/84  
 11/01/84  
 11/01/84  
 11/01/84  
 08/15/84

TARGET

05/15/84  
 06/22/84  
 06/30/84  
 07/01/84  
 07/01/84  
 07/01/84  
 07/01/84  
 07/01/84  
 08/15/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: MILLSTONE 1

PLANT LOCATION: 5 MI SW OF NEW LONDON, CONN  
 DOCKET NUMBER: 050-00245  
 ARCH/ENGINEER: EBASCO  
 IE INSPECTOR: T. SHEDLOSKY

LICENSED POWER: 2011 MWT  
 DESIGN POWER: 0660 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: J. SHEA  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51684	MILLSTONE 1 NUREG 0737 TECH SPECS			<u>COMPLETE</u> 03/19/84
43050	MILLSTONE 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	05/23/84
10910	MILLSTONE 1 - RPS POWER SUPPLY		1	06/14/84
46665	MILLSTONE 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/20/84
<u>ACTIVE ACTIONS</u>				
52244	MILLSTONE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52773	MILLSTONE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52854	MILLSTONE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52935	MILLSTONE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53016	MILLSTONE 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
08060	MILLSTONE 1 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	09/30/82
07941	MILLSTONE 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	07/12/84
42523	MILLSTONE 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/30/84
51106	MILLSTONE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			12/12/84
42612	MILLSTONE 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	12/15/84
10137	MILLSTONE 1 - CONTAINMENT LEAK TESTING - APPENDIX J (GENERIC)		2	12/30/84
42913	MILLSTONE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	12/31/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
45839	MILLSTONE 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	<u>COMPLETE</u> 12/19/83
48178	MILLSTONE 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/19/83
<u>ACTIVE ACTIONS</u>				
51176	MILLSTONE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51256	MILLSTONE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44863	MILLSTONE 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	10/30/82
46533	MILLSTONE 1 - NUREG-0737, II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	01/28/83
46317	MILLSTONE 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
45147	MILLSTONE 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/83
48280	MILLSTONE 1 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	09/30/83
44315	MILLSTONE 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83
47754	MILLSTONE 1 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	05/14/84
46541	MILLSTONE 1 - NUREG-0737, II.K.3.45, MANUAL DEPRESSURIZATION		1	06/30/84
44453	MILLSTONE 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
45958	MILLSTONE 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46029	MILLSTONE 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46101	MILLSTONE 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44245	MILLSTONE 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## MILLSTONE 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
49376	MILLSTONE UNIT 1 - INTEGRATED STRUCTURAL ANALYSIS			10/03/83
49377	MILLSTONE UNIT 1 - MISSILES			10/03/83
49379	MILLSTONE UNIT 1 - SEISMIC DESIGN			10/03/83
49380	MILLSTONE UNIT 1 - MOV INTERLOCKS			10/03/83
49382	MILLSTONE UNIT 1 - WATER CHEMISTRY			10/03/83
52044	MILLSTONE - 1 CLARIFY T.S. RELATED TO FIRE PROTECTION DETECTION			11/02/83
51586	MILLSTONE 1 - REVIEW OF MODIFIED AMENDED SECURITY PLANS PER EISENHUT MEMORANDUM MAY 16, 1983			11/10/83
51987	MILLSTONE 1 - REQUIRED ATWS ACTIONS G.L.83-28			11/25/83
51978	MILLSTONE 1 - INTEGRATED LEAK RATE TEST IN LESS THAN 24 HOURS			12/19/83
49247	MILLSTONE 1 - SALP-1983			12/19/83
52529	MILLSTONE 1 - REV. TO ADM. TECH. SPEC. SECT. 6 LIC. OPERATOR STAFFING			02/16/84
53299	MILLSTONE 1 - PREPARE RESPONSE TO NNECO 11/18/83 STA TRAINING			02/28/84
52022	MILLSTONE 1 - T.S. CHANGES - ADMINISTRATIVE PROVIDE SER			03/19/84
49457	MILLSTONE 1 - AUDITS OF EMERGENCY & SECURITY PLANS RE: GEN LETTERS 82-17 AND 82-23			03/19/84
53446	MILLSTONE 1 - RECIRC LINE PIPE CRACK - CLAMPS			05/11/84
49381	MILLSTONE UNIT 1 - LEAK DETECTION			05/23/84
54656	MILLSTONE-1 - PREPARE SER FOR FUEL VUNDLE "BP8DRB 300"			06/14/84
54658	MILLSTONE-1 - RELOAD 9 (PREPARE SER)			06/14/84
49388	MILLSTONE UNIT 1 - BATTERY INSTRUMENTATION AND TS LIMITS			06/21/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
49786	MILLSTONE 1 IPSAR SUPPLEMENT			12/30/83
51986	MILLSTONE 1 - REVIEW & PREPARE SER OF NNECO QUAL ASS. REPORT SUBMITTED BY NNECO LTR 6/9/83			12/30/83
51961	MILLSTONE 1 - ALTERNATIVE APPENDIX I TECH. SPEC. REVIEW			03/06/84
53349	MILLSTONE 1 - SALP 1984			04/13/84
53294	MILLSTONE 1 - INTEGRATED SAFETY ASSESSMENT PROGRAM			04/21/84
49384	MILLSTONE 1 - FAULT TRANSFERS			04/27/84
49385	MILLSTONE UNIT 1 - ELECTRICAL ISOLATION			04/30/84
49780	MILLSTONE UNIT 1 - FLOODING (NON-STRUCTURAL)			05/01/84
54189	MILLSTONE-1 - NEW STEAM TUNNEL VENTILATION SYSTEM			06/12/84
49488	MILLSTONE 1 - REVIEW FINAL ENVIRONMENTAL STATEMENT UPDATE			06/30/84
54657	MILLSTONE-1 PROVIDE COMMENTS ON ATTACHED NNECO SUBMITTAL DATED 3/27/84 (ISO CONDENSER)			08/01/84
11210	MILLSTONE 1 - INSERVICE TESTING(IST)		1	08/10/84
53445	MILLSTONE 1 - RESPONSE TO GEN LTR 81-34 RE MPA 8-65		3	08/10/84
48229	MILLSTONE 1 - SUBCOOLED POST DBLOCA RADIOLYSIS			09/29/84
51698	MILLSTONE - 1 T.S. CHANGES TO MEET FINAL RULE ON ENVIR QUAL 10 CFR 50.49			10/17/84
54491	MILLSTONE - 1: REVIEW RESPONSE TO GE SIL-402 DRYWELL VENT HEADER DISTRIBUTER IN TORUS CRACKS			10/30/84
49290	MILLSTONE 1 - SAFETY FACTOR FOR PENETRATION X-10A		2	11/30/84
07426	MILLSTONE 1 - CONTAINMENT LEAK RATE CONTINUOUS MONITORING			11/30/84
54693	MILLSTONE 1 - RECIRCULATION LOOP DECONTAMINATION			12/08/84
48664	MILLSTONE - 1 EXCEPTION REQUESTS - FIRE PROT APP.R RULE			12/12/84
54545	MILLSTONE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			12/12/84
51702	MILLSTONE 1 - DEGRADED GRID VOLTAGE PROCEDURES			12/31/84
49386	MILLSTONE UNIT 1 - GRID SEPARATION PROCEDURES			
51305	MILLSTONE 1 - PREPARE SER CONSIDERING LICNESEE COMMENTS & IREP-REPORT. NUREG/CR 3085			
51531	MILLSTONE-1. IREP - REPORT EVALUATION - NNECO FEEDBACK BY 7-1-83			
51795	MILLSTONE - 1 RESPONSE TO NRC 6/8/83 LETTER OPTION ON II K.3.18			
53435	MILLSTONE 1 - OCCUPATIONAL DOSE REDUCTION BNL			
48293	MILLSTONE 1 - SUBCOOLED POST DBLOCA RADIOLYSIS		3	
49387	MILLSTONE 1 - EMERGENCY POWER			

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## MILLSTONE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
51559	MILLSTONE 1 - REVIEW NEED FOR SCRAM DISCHARGE VOLUME DIVERSE LEVEL INSTRUMENTATION			<u>TARGET</u> 12/31/84
11221	MILLSTONE 1 - CONVERSION POL TO FTOL		1	02/02/85
<u>ANTICIPATED ACTIONS</u>				
53606	MILLSTONE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53689	MILLSTONE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53772	MILLSTONE 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53855	MILLSTONE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53990	MILLSTONE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54082	MILLSTONE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
49390	MILLSTONE UNIT 1 - ACTIVITY LIMITS			12/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: MILLSTONE 2

PLANT LOCATION: 5 MI SW OF NEW LONDON, CONN  
 DOCKET NUMBER: 050-00336  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: T. SHEDLOSKY

LICENSED POWER: 2/00 MWT  
 DESIGN POWER: 0370 MWE  
 NSSS VENDOR: COMB

PROJECT MANAGER: D. OSBORNE  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12318	MILLSTONE 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE		3	<u>COMPLETE</u> 01/31/84
42588	MILLSTONE 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	04/25/84
49741	MILLSTONE 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			05/23/84
03952	MILLSTONE 2 - TS SURVEILLANCE FOR MECHANICAL SNUBBERS		2	06/11/84
03969	MILLSTONE 2 - TS SURVEILLANCE FOR HYDRAULIC SNUBBERS		3	06/11/84

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
51107	MILLSTONE 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52245	MILLSTONE 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52774	MILLSTONE 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52855	MILLSTONE 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RIS COMPONENTS			
52936	MILLSTONE 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53017	MILLSTONE 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53080	MILLSTONE 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53133	MILLSTONE 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
49558	MILLSTONE 2 - REVIEW ALTERNATIVE APPENDIX I TECH SPECS			11/01/83
08949	MILLSTONE 2 - REVIEW OF ASYMMETRIC LOCA LOADS		2	11/30/83
51757	MILLSTONE 2 - THERMAL SHIELD FOLLOWUP ANALYSIS			01/15/84
08061	MILLSTONE 2 - CONTROL OF HEAVY LOADS (NUREG 0612)		2	09/30/84
48642	MILLSTONE 2 - EXEMPTION FROM APP. R. IIIG TECHNICAL			10/01/84
42493	MILLSTONE 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/01/84
42894	MILLSTONE 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	12/01/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47807	MILLSTONE 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 10/21/83
44104	MILLSTONE 2 - NUREG-0737 I.A.1.3., SHIFT MANNING			02/03/84
44246	MILLSTONE 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
46102	MILLSTONE 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	06/07/84
46893	MILLSTONE 2 - II K2.13 THERMAL MECHANICAL REPORT		1	06/11/84
44454	MILLSTONE 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/14/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
51177	MILLSTONE 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51257	MILLSTONE 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45148	MILLSTONE 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	02/28/84
45840	MILLSTONE 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/15/84
46318	MILLSTONE 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/01/84
48179	MILLSTONE 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
45959	MILLSTONE 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46030	MILLSTONE 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
44595	MILLSTONE 2 - RV AND SV TESTING		1	10/30/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## MILLSTONE 2

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
44864	MILLSTONE 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	TARGET 11/30/84
44316	MILLSTONE 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
48919	MILLSTONE 2 - BORIC ACID FLOWPATH TECH. SPEC. REVISION			COMPLETE 10/07/83
51997	MILLSTONE 2 - EVALUATION OF WELD FLAWS IN S.G. #1 TOP HEAD STEAM DOME			12/01/83
52573	MILLSTONE 2 - REVISED CONFIRMATORY ORDER			12/14/83
51351	MILLSTONE 2 - SLEEVING STEAM GENERATOR TUBES			12/30/83
51646	MILLSTONE 2 - NON-LOCA TRANSIENT ANALYSIS FOR OPERATION WITH 15.3% STEAM GENERATOR TUBEL PLUGGED			12/30/83
49062	MILLSTONE 2 - LOCA REANALYSES WITH 15.3% OF SG TUBES PLUGGED		3	12/30/83
49798	MILLSTONE 2 - RELOAD CYCLE 6			12/30/83
48063	MILLSTONE 2 - MEASUREMENT UNCERTAINTIES		3	01/06/84
53462	MILLSTONE NO 2 - REVISED RELOAD CALCULATIONS			01/27/84
52023	MILLSTONE 2 - ADMINISTRATIVE T.S. CHANGES PROVIDE SER			02/19/84
48383	MILLSTONE 2 - LIQUID WASTE EFFLUENTS			03/28/84
54511	MILLSTONE UNIT NO. 2 - TEMPORARY RELOCATION OF TSC			04/02/84
53512	MILLSTONE NO 2 - PT CURVE CHANGES			04/10/84
51969	MILLSTONE 2 - THERMAL SHIELD DAMAGE RESOLUTION			04/12/84
49061	MILLSTONE 2 - CEA POSITION INDICATION, SIT LEVEL AND CONTAINMENT INTEGRITY		2	05/16/84
53384	MILLSTONE NO. 2 - PREPARE CONFIRMATORY ORDERS TO GENERIC LETTER 82-33			06/14/84
<u>ACTIVE ACTIONS</u>				
51701	MILLSTONE 2 - DEGRADED GRID VOLTAGE PROCEDURES			TARGET
48644	MILLSTONE 2 : EXEMPTION FROM APP.R III G, SCHEDULAR			07/01/84
51574	MILLSTONE 2 - FOLLOWUP AMENDMENT OF IST PROGRAM RELATIVE TO PUMPS AND VALVES			08/31/84
55011	MILLSTONE UNIT NO 2 - RELIEF FROM INSERVICE INSPECTION REQUIREMENTS			09/30/84
54546	MILLSTONE 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54741	MILLSTONE UNIT 2 - STEAM GENERATOR LEAK RATE DETECTION			09/30/84
54399	MILLSTONE 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			10/01/84
48643	MILLSTONE 2 - EXEMPTION FROM APPENDIX R, ALTERNATE SAFE SHUTDOWN CAPABILITY			10/28/84
52339	MILLSTONE 2 - TECH. SPEC. CHANGE RE STEAM GENERATOR TUBE INSPECTION			10/31/84
52504	MILLSTONE NO. 2 - PRESSURIZER LEVEL BAND			12/30/84
49658	MILLSTONE 2 - REACTOR COOLANT PUMP TRIP			12/31/84
54199	MILLSTONE UNIT 2 - CYCLE 6 CONFIRMATIONS SUBMITTALS			04/30/85
55219	MILLSTONE UNIT NO. 2 - FUEL LEAKAGE			
<u>ANTICIPATED ACTIONS</u>				
53607	MILLSTONE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53690	MILLSTONE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53773	MILLSTONE 2- ITT'S 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53856	MILLSTONE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53920	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54000	MILLSTONE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54083	MILLSTONE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: MONTICELLO

PLANT LOCATION: 30 MI NW OF MINNEAPOLIS, MINN  
 DOCKET NUMBER: 050-00263  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: C. BROWN

LICENSED POWER: 1670 MWT  
 DESIGN POWER: 0545 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: V. ROONEY  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
43497	MONTICELLO - 10 CFR 50.48 AND 10 CFR 50 APPENDIX R FIRE PROTECTION REQUIREMENTS		1	<u>COMPLETE</u> 01/03/84
42083	MONTICELLO - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	01/06/84
43020	MONTICELLO - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	01/23/84
12306	MONTICELLO - HELB AND CONSEQUENTIAL SYSTEM FAILURE MONTICELLO		3	01/31/84
08062	MONTICELLO - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	03/19/84
11134	MONTICELLO - APP J-CONTAINMENT LEAK TESTING		2	06/03/84
46666	MONTICELLO - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/15/84

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
51108	MONTICELLO - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52246	MONTICELLO - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52775	MONTICELLO - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52856	MONTICELLO - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52937	MONTICELLO - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53018	MONTICELLO-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
51685	MONTICELLO NUREG 0737 TECH SPECS			03/01/84
42475	MONTICELLO - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	04/30/84
42577	MONTICELLO - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	09/30/84
43720	MONTICELLO - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		3	10/30/84
07942	MONTICELLO - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	06/30/85

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44105	MONTICELLO - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			<u>COMPLETE</u> 11/16/83
45841	MONTICELLO - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
48180	MONTICELLO -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/28/83
44455	MONTICELLO - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	03/12/84
44865	MONTICELLO - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	03/14/84
44247	MONTICELLO - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
44317	MONTICELLO - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	<u>TARGET</u>
51178	MONTICELLO - I.D.1 CONTROL ROOM DESIGN REVIEW			
51258	MONTICELLO - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45694	MONTICELLO - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	01/29/84
47857	MONTICELLO - NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS		1	03/30/84
44596	MONTICELLO - RV AND SV TESTING		1	03/30/84
45598	MONTICELLO - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	03/30/84
45149	MONTICELLO - NUREG-0737 III.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/29/84
46319	MONTICELLO - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/29/84
45960	MONTICELLO - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46031	MONTICELLO - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46103	MONTICELLO - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

MONTICELLO

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
43998	MONTICELLO - HIGH TEMP IN MAIN STEAM LINE TUNNEL			<u>COMPLETE</u> 11/18/83
51016	MONTICELLO - PLANT OPERATING PROCEDURES			12/30/83
52563	MONTICELLO - EXTENSION REQUEST PER 10CFR50.54(M)(2) ON LICENSED OPERATOR STAFFING REQUIREMENTS			12/30/83
48951	MONTICELLO - SOURCE RANGE MONITOR COUNT RATE		1	01/16/84
41061	MONTICELLO - DC POWER SUPPLY FAILURE EFFECT ECCS		2	01/31/84
49157	MONTICELLO - SHUTDOWN COOLING SUPPLY ISOLATION			02/02/84
49932	MONTICELLO - FOUR SECOND RPS TIME DELAY			04/03/84
49093	MONTICELLO - VENT AND DRAIN VALVES OF THE SDV SYSTEM			05/01/84
48457	MONTICELLO: REQUEST FOR SCHEDULAR RELIEF ON COMBUSTIBLE GAS CONTROL SYSTEMS			05/18/84
<u>ACTIVE ACTIONS</u>				
46510	MONTICELLO - ISI/IST PROGRAM-SECOND TEN YEAR INTERVAL		3	<u>TARGET</u> 05/29/83
52435	MONTICELLO - REVIEW OF SURVEILLANCE TECH RPT ON REACTOR VESSEL RADIATION FROM RAM			01/31/84
52204	MONTICELLO - REPLACEMENT OF RECIRCULATION PIPING REVIEW			03/30/84
53412	MONTICELLO - EXEMPTION FROM SCHEDULE FOR EMERGENCY PREPAREDNESS EXERCISE			03/30/84
53561	MONTICELLO - EXEMPTION ON SHIFT STAFFING RULE			03/30/84
52036	MONTICELLO - REVIEW OF MODIFICATIONS ON SRV LOW-LOW SETPOINT			03/30/84
49934	MONTICELLO - MAXIMUM ALLOWABLE LEAKAGE RATE INCREASE LA			03/30/84
51338	MONTICELLO - EXEMPTION REQUEST FROM FIXED SUPPRESSION IN THE CONTROL ROOM			03/30/84
49102	MONTICELLO - SURVEILLANCE REQUIREMENTS			04/30/84
49158	MONTICELLO - JET PUMP OPERABILITY		1	05/31/84
49931	MONTICELLO - SCREEN WASH/FIRE PUMP		1	06/29/84
49101	MONTICELLO - ALTERNATE ROD INJECTION			06/30/84
52039	MONTICELLO - REVIEW OF THE UNDERVOLTAGE RELAY TRIP EVENT		1	06/30/84
54748	MONTICELLO - CYCLE 11 RELOAD			06/30/84
54692	MONTICELLO - DEFER INTEGRATED LEAK TEST			08/01/84
55210	MONTICELLO - ARTS			08/30/84
48648	MONTICELLO - REVIEW OF LICENSEE'S RESPONSE TO UNRESOLVED FIRE PROTECTION SER ITEMS			09/01/84
51785	MONTICELLO - REVISED LICENSE AMENDMENT ON ENVIRONMENTAL TECHNICAL SPECIFICATION			09/28/84
51335	MONTICELLO - STATION BATTERY SYSTEM			09/29/84
49787	MONTICELLO - TURBINE BYPASS VALVES			09/29/84
49788	MONTICELLO - FEEDWATER/TURBINE TRIP INSTRUMENTATION			09/29/84
48646	MONTICELLO - REVISED LICENSE AMENDMENT ON SINGLE LOOP OPERATION			09/29/84
54547	MONTICELLO - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
55211	MONTICELLO - RHR INTERTIE LINE ADDITION			09/30/84 10/01/84
<u>ANTICIPATED ACTIONS</u>				
53608	MONTICELLO - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53691	MONTICELLO 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53774	MONTICELLO- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53857	MONTICELLO - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54001	MONTICELLO - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54084	MONTICELLO - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: NINE MILE POINT 1

PLANT LOCATION: 8 MI NE OF OSWEGO, NY  
 DOCKET NUMBER: 050-00220  
 ARCH/ENGINEER: NIAGARA  
 IE INSPECTOR: S. HUDSON

LICENSED POWER: 1850 MWT  
 DESIGN POWER: 0620 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. HERMANN  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
13155	NINE MILE POINT 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEMS VOLTAGES		2	<u>COMPLETE</u> 12/20/83
12307	NINE MILE POINT 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE NINE MILE POINT		3	01/31/84
43021	NINE MILE POINT 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	02/02/84
03029	NINE MILE POINT 1 - REQUESTED EXEMPTIONS FROM 10CFR50 APPENDIX J		2	02/22/84
51606	NINE MILE POINT 1 NUREG 0737 TECH SPECS			03/09/84
07445	NINE MILE POINT 1 - REVISION TO NDT CURVE (APP G)		8	04/18/84
08943	NINE MILE POINT 1 - RPS POWER SUPPLY		1	05/25/84
<u>ACTIVE ACTIONS</u>				
52247	NINE MILE POINT 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
51109	NINE MILE POINT 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52776	NINE MILE POINT 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52857	NINE MILE POINT 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52938	NINE MILE POINT 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53019	NINE MILE PT 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		1	05/30/84
42476	NINE MILE POINT 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/01/84
42578	NINE MILE POINT 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		2	06/30/84
42884	NINE MILE POINT 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		3	06/30/84
08118	NINE MILE POINT 1 - APPENDIX I TECH SPECS IMPLEMENTATION REVIEW		1	09/30/84
07943	NINE MILE POINT 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		2	09/30/84
08063	NINE MILE PT 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)			
<u>TAC # TAC DESCRIPTION MULTI-ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47954	NINE MILE POINT 1 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	<u>COMPLETE</u> 12/01/83
48181	NINE MILE PT. 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	01/03/84
45842	NINE MILE POINT 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/03/84
44456	NINE MILE POINT 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	04/17/84
46460	NINE MILE POINT 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	05/21/84
<u>ACTIVE ACTIONS</u>				
51179	NINE MILE POINT 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51259	NINE MILE POINT 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	03/31/84
46320	NINE MILE POINT 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	03/31/84
44248	NINE MILE POINT 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	03/31/84
44318	NINE MILE POINT 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	06/30/84
41032	NINE MILE-REACTOR VESSEL WATER LEVEL INSTRUMENTATION		1	06/30/84
44597	NINE MILE POINT 1 - RV AND SV TESTING		1	06/30/84
44366	NINE MILE POINT 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	06/30/84
45599	NINE MILE POINT 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	06/30/84
45695	NINE MILE POINT 1 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/30/84
45961	NINE MILE POINT 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NINE MILE POINT 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
46032	NINE MILE POINT 1 - N'REG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	<u>TARGET</u> 09/30/84
46104	NINE MILE POINT 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
07443	NINE MILE POINT 1 - ADDITION OF N2 ISOLATION VALVES		1	<u>COMPLETE</u> 11/30/83
53286	NINE MILE PT - 1 - TS CHANGE - FIRE PROTECTION			12/20/83
52687	NINE MILE PT. 1 - REVIEW - PROPOSED SRO MANNING AND, IF NEC., REQUEST FOR TIME EXTENSION			12/28/83
42294	NINE MILE POINT 1 - REACTOR BUILDING CRACK		1	12/29/83
07442	NINE MILE POINT 1 - SFP MODIFICATION		1	02/01/84
52684	NINE MILE 1 - TS CHANGE - ADMIN CONTROLS SECTION			03/20/84
52685	NINE MILE 1 - TS CHANGE - APPENDIX H-REPORTING REQUIREMENTS			04/18/84
52698	NINE MILE PT-1 - TECH SPECS - ISI, IST, AUG. ISI			04/18/84
53270	NINE MILE PA-1 TECH SPEC REVIEW AND INTERPRETATION MSIV INST CHANNEL			05/08/84
53447	NINE MILE PT-1 - AMENDMENT REQUEST TO INCREASE MCPR LIMITS			05/08/84
53389	NINE MILE POINT 1 - SCHEDULAR EXEMPTION APPENDIX R - HIGH RADIATION LOGIC CIRCUITING			05/08/84
54130	NINE MILE PT 1 - TS DELETION OF AUTOMATIC ISOLATION OF EMERG COND. ON HIGH RAD			05/08/84
54188	NINE MILE PT - 1-TS-ADD CHANNEL CAL. REQUIREMENTS FOR MSIV AND TURBINE STOP VALVE INSTRS.			05/08/84
52686	NINE MILE PT. 1 - EXEMPTION FROM HDROGEN RECOMBINER RULE			06/28/84

<u>ACTIVE ACTIONS</u>				
54436	NINE MILE PT. 1 - LIMITED CODE REVISION TO APPROVED ISI PROGRAM PER 10 CFR (2)(4)(IV)			<u>TARGET</u> 04/15/84
54187	NINE MILE PT - TS - ADMIN CONTROL SECTION, OVERTIME, ETC.			05/31/84
52683	NINE MILE 1 - TS CHANGE - RAD MUT INVENTORY REQUIREMENTS			05/31/84
54628	NINE MILE PT - 1 - EVALUATION OF STUB TUBE REPAIR			06/15/84
42424	NINE MILE POINT 1 - SNUBBER ADDITIONS AND DELETIONS		2	06/30/84
51334	NINE MILE PT. - UNIT-1 - PLANT SPECIFIC EVALUATION OF CORE SPRAY PERFORMANCE			06/30/84
54152	NINE MILE PT-1 - PLANT SPECIFIC APPENDIX J. REVIEW			07/01/84
55047	NINE MILE PT.-1, O.D. CRACK - LOOP 11 - CORE SPRAY PIPE			07/15/84
54751	NINE MILE 1 - TECH. SPEC. - TRIPLE LOW REACTOR WATER LEVEL SETPOINT			07/31/84
54752	NINE MILE - 1, TECH. SPEC. - ADS AND OVERPRESSURIZATION VALVE SET POINTS AND TESTING			07/31/84
54129	NINE MILE PT 1 - REVIEW OF RECOUPLING SCREEHOUSE REACTOR BLDG CODE			08/01/84
55050	NINE MILE PT-1 - FEEDWATER HEADER OD CRACK			08/01/84
54753	NINE MILE 1 - TECH. SPEC. - INT. OF DIESEL GENERATORS ON LOSS OF OR DEGRADED VOLTAGE			08/15/84
54210	NINE MILE PT. UNIT 1 - T.S. CHANGE FOR CAPSULE SURVEILLANCE PROGRAM PER APPENDIX H			08/31/84
55049	NINE MILE PT-1, MK-1 - CONTAINMENT VENT HEADER CRACKS			09/01/84
55048	NINE MILE PT-1 - TECH. SPEC. REMOTE SHUTDOWN PANALS			09/15/84
54548	NINE MILE POINT 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
53390	NINE MILE POINT 1 - ADDITIONAL EXEMPTION REQUEST FROM APPENDIX R (GENERIC LETTER 83-33)			09/30/84

<u>ANTICIPATED ACTIONS</u>				
54085	NINE MILE POINT 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			<u>INITIATION</u> 11/01/84
53609	NINE MILE POINT 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53692	NINE MILE POINT 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53775	NINE MILE PT 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53858	NINE MILE POINT 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84



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CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NINE MILE POINT 1

TAC #    TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

54002    NINE MILE POINT 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS

INITIATION

11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: NORTH ANNA 1

PLANT LOCATION: 40 MI NW OF RICHMOND, VA  
 DOCKET NUMBER: 050-00338  
 ARCH/ENGINEER: S&W  
 IE INSPECTOR: T. WEBSTER

LICENSED POWER: 2775 MWT  
 DESIGN POWER: 0907 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: L. ENGLE  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47152	NORTH ANNA 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 10/19/83
49742	NORTH ANNA 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		1	01/12/84
47112	NA-1: CONTROL OF HEAVY LOADS - NUREG 0612		2	05/25/84
<u>ACTIVE ACTIONS</u>				
51110	NORTH ANNA 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	<u>TARGET</u>
52248	NORTH ANNA 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52777	NORTH ANNA 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52858	NORTH ANNA 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52939	NORTH ANNA 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53020	NORTH ANNA 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
53081	NORTH ANNA 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53134	NORTH ANNA 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53181	NORTH ANNA 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42497	NORTH ANNA 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/83
42895	NORTH ANNA 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	07/30/83
13109	NORTH ANNA 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES		2	07/30/83
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44388	NORTH ANNA 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 10/09/83
45240	NORTH ANNA 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	10/19/83
45286	NORTH ANNA 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES			10/19/83
47955	NORTH ANNA 1 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	11/14/83
47808	NORTH ANNA 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/24/84
44457	NORTH ANNA 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/27/84
46461	NORTH ANNA 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	03/20/84
46894	NORTH ANNA 1 - II K2.13 THERMAL MECHANICAL REPORT		1	06/05/84
<u>ACTIVE ACTIONS</u>				
51180	NORTH ANNA 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51260	NORTH ANNA 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45150	NORTH ANNA 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION			
46321	NORTH ANNA 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
45843	NORTH ANNA 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/83
48182	NORTH ANNA 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/83
44598	NORTH ANNA 1 - RV AND SV TESTING		1	12/31/83
45962	NORTH ANNA 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46033	NORTH ANNA 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46105	NORTH ANNA 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44249	NORTH ANNA 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44319	NORTH ANNA 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NORTH ANNA 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
52472	NA-1: AMEND LICENSE TO INCORPORATE OLD DOMINIOM ELECTRIC COOPERATIVE AS PARTIAL OWNER OF NORTH ANNA, UNIT 1			11/18/83
52313	NA-1: REQUEST INTERP 10 CFR 20.202(B)(2) AND 20.203(B) REPOSTING OF RADIATION AREAS			01/27/84
53483	NA-1: REVISE TS TABLE 6.2.1 PER GENERIC LETTER 82-12			02/01/84
52671	NA-1: EXEMPTION REQUEST TO 10CFR50 APPENDIX R, CONTROL ROOM AREA			02/23/84
53526	NA-1: REVISE TS LIMITS FOR QUADRANT POWER TILT RATIO TO ALLOW FULL-CORE FLUX MAPPING			02/28/84
54158	NA-1: REVISE TS FOR RCS LEAKAGE DETECTION SYSTEM			02/28/84
52550	NA-1: INCREASE IN FIRE PROTECTION SURVEILLANCE REQUIREMENTS			02/28/84
52480	NA-1: AMEND TS IN TABLE 3.6.1 FOR CLARIFYING CHECK VALVE TESTING REQUIREMENTS			03/06/84
53251	NA-1: REVISE T.S. 3.3-3 FOR CONSISTENCY WITH NA-2 TS 3.3-3			03/06/84
52574	NA-1: REVISING TS BY CHANGING FRACTIONAL POWER MULTIPLIER FROM 0.2 TO 0.3 FOR 587.8 F TAV			03/13/84
49524	NA-1: AMEND LIC FOR INCREASING RCS TAV TO 587.8 (PHASE 2, POWER UPRATE)		2	03/13/84
53517	NA: 1 REVISE TS TO DESIGNATE PORV & SAFETY VALVES AS PART OF "RCS"			04/09/84
53262	NORTH ANNA 1 - EXTEND BURNUP TO 45,000 MWD/MTV			04/09/84
54267	NA-1: EXEMPTION REQUEST TO 10 CFR PART 50.44(C)(3)(III)			04/13/84
53519	NA-1: REVISE TS FOR TURBINE OVERSPEED PROTECTION OPERABILITY TO 31 DAYS			04/16/84
54296	NA-1: RELIEF REQUEST: INSERVICE TESTING - PUMPS AND VALVES			04/17/84
52474	NA-1: EVALUATE 5 YEAR REPORT ON SETTLEMENT AND GROUNDWATER LEVEL AT SERVICE WATER RESERVOIR		1	05/23/84
52476	NA-1: AMEND TS TERMINATING MEASUREMENTS FOR TORBIDITY & SUSPENDED SOLIDS FROM DRAIN SYS UNDER SER WATER PMP HSE		1	05/23/84
54465	NA-1: DELETE TECH SPEC REQUIREMENTS FOR SG SUPPORTS		1	06/07/84
54832	NA-1: APPENDIX R REANALYSIS EXEMPTION REQUESTS			06/19/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52053	NA-1: ADMIN CHANGES TO T.S. REORGANIZATION OF NUC OPS DEPT., QUALITY ASS, EMERG PLN & SECURITY DEPT			09/22/83
53252	NA-1: AMEND T.S. BY ADDING EXEMPTION FOR ENGINEERING EVAL FOR ANY FAILURE OF SG/RCP SUPPORT SNUBBERS			02/15/84
53528	NA-1: PM BRIEFING BOOK-HEADQUARTES OPERATION-CENTER			02/28/84
53521	NA-1: REVISE TS FOR ALLOWING 1 OUT OF 2 HX RECOMBINERS TO BEINOPERABLE			03/15/84
53522	NA-1: REVISE TS FOR REACTOR TRIP SYS INTERLOCK SETPOINT P-7 FOR 10% FULL LOAD TURBINE IMPULSE CHAMBER PRESSURE			03/20/84
54789	NA-1: ISSUE SER AND EIA FOR SPENT FUEL AND TRANSSHIPMENT HEARING			05/11/84
54472	NA-1: RELIEF FROM ASME XI VOL. EXH. ACC AND BORON TANK			05/16/84
49525	NA-1: EVALUATE NA MAIN TRANSFORMER FAULTS		1	05/30/84
54845	NORTH ANNA 1 - CORE RELOAD METHODOLOGY AUDIT			06/01/84
54500	NA-2: REVISE TS PER GEN.LTR. 83-30			06/15/84
54505	NA-1: REVISE TS FOR LER REPORTING PER 10CFR PART 50.72 & 50.73			06/15/84
53524	NA-1: REVISE TS FOR LCO FOR SURVEILLANCE IN MODES 3,4 AND 5			06/30/84
52478	NA-1: AMEND TS BY REDUCING LENGTH OF TIME FOR CORE SURVEILLANCE REPORT FROM 60 TO 30 DAYS		1	06/30/84
54549	NORTH ANNA 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54400	NORTH ANNA 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
52414	NA-1: INSERVICE TESTING (PUMPS & VALVES)			12/30/84
48868	NA-1: AMEND LICENSE FOR RECEIPT AND STORAGE OF SPENT FUEL FROM SURRY 1 & 2		1	12/31/84
48870	NA-1: AMEND T.S. FOR AUGMENTED SPENT FUEL STORAGE		1	12/31/84
49665	NORTH ANNA 1 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53610	NORTH ANNA 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53693	NORTH ANNA 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NORTH ANNA 1

<u>IAC #</u>	<u>IAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53776	NORTH ANNA 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			<u>INITIATION</u> 11/01/84
53859	NORTH ANNA 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53921	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54003	NORTH ANNA 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54086	NORTH ANNA 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: NORTH ANNA 2

PLANT LOCATION: 40 MI NW OF RICHMOND, VA  
 DOCKET NUMBER: 050-00339  
 ARCH/ENGINEER: S&W  
 IE INSPECTOR:

LICENSED POWER: 2775 MWT  
 DESIGN POWER: 0907 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: L. ENGLE  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47153	NORTH ANNA 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	<u>COMPLETE</u> 10/19/83
49743	NORTH ANNA 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			01/12/84
52249	NORTH ANNA 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			01/26/84
47113	NA-2: CONTROL OF HEAVY LOADS - NUREG-0612		2	05/25/84
<u>ACTIVE ACTIONS</u>				
51111	NORTH ANNA 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52778	NORTH ANNA 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52859	NORTH ANNA 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52940	NORTH ANNA 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53021	NORTH ANNA 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
53082	NORTH ANNA 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELAYED MODIFICATIONS			
53135	NORTH ANNA 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53182	NORTH ANNA 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	07/30/82
42540	NORTH ANNA 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	07/30/83
48331	NORTH ANNA 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION VOLTAGES			
TAC #	TAC DESCRIPTION	IMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44389	NORTH ANNA 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 10/09/83
45241	NORTH ANNA 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION			10/19/83
45287	NORTH ANNA 2 - NUREG-0737 II.K.3.2, REPOR? ON PORV FAILURES		1	10/19/83
47956	NORTH ANNA 2 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	11/14/83
45647	NORTH ANNA 2 - NUREG-0737 II.K.3.17, ECC SYSTEM OUTAGES		1	12/30/83
47809	NORTH ANNA 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/24/84
44458	NORTH ANNA 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/27/84
46462	NORTH ANNA 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	03/20/84
46895	NORTH ANNA 2 - II K2.13 THERMAL MECHANICAL REPORT		1	06/05/84
<u>ACTIVE ACTIONS</u>				
45151	NORTH ANNA 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION			<u>TARGET</u>
51181	NORTH ANNA 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51261	NORTH ANNA 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
44599	NORTH ANNA 2 - RV AND SV TESTING		1	10/31/82
46322	NORTH ANNA 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
45844	NORTH ANNA 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/83
48183	NORTH ANNA 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46			09/30/83
45963	NORTH ANNA 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46034	NORTH ANNA 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46106	NORTH ANNA 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44250	NORTH ANNA 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44320	NORTH ANNA 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NORTH ANNA 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
51540	NA-2: PLANT UPGRADE, PHASE 1, INCREASE T AVERAGE			10/19/83
52473	NA-2: AMEND LICENSE TO INCORPORATE OLD DOMINION ELECTRIC COOPERATIVE AS PARTIAL OWNER OF NORTH ANNA, UNIT 2			11/18/83
52314	NA-2: REQUEST INTERP 10 CFR 20.202(B)(2) AND 20.203(B) REPOSTING OF RADIATION AREAS			01/27/84
53484	NA-2: REVISE T.S. TABLE 6.2.1 PER GENERIC LETTER 82-12			02/01/84
52672	NA-2: EXEMPTION REQUEST TO 10CFR50 APPENDIX R, CONTROL ROOM			02/23/84
52551	NA-2: INCREASE IN FIRE PROTECTION SURVEILLANCE REQUIREMENTS			02/28/84
53527	NA-2: REVISE TS LIMITS FOR QUADRANT POWER TILT RATIO TO ALLOW TOLL-CORE FLUX-MAPPING			02/28/84
54159	NA-2: REVISE TS FOR RCS LEAKAGE DETECTION SYSTEM			02/28/84
52481	NA-2:AMEND TS IN TABLE 3.6.1 FOR CLARIFYING CHECK VALVE TESTING REQUIREMENTS			03/06/84
53518	NA: 2 REVISE TS TO DESIGNATE PORV & SAFETY VALVES AS PART OF "RCS"			04/09/84
53263	NORTH ANNA 2 - EXTEND BURNUP TO 45,000 MWD/MTU			04/09/84
54268	NA-2: EXEMPTION REQUEST TO 10 CFR PART 50.44(C)(3)(III)			04/13/84
53520	NA-2: REVISE TS FOR TURBINE OVERSPEED PROTECTION OPERABILITY TO 31 DAYS			04/16/84
54297	NA-2: RELIEF REQUEST: INSERVICE TESTING - PUMPS AND VALVES			04/17/84
52475	NA-2: EVALUATE 5 YEAR REPORT ON SETTLEMENT AND GROUNDWATER LEVEL AT SERVICE WATER RESERVOIR			05/23/84
52477	NA-2:AMEND TS TERMINATING MEASUREMENTS FOR TURBIDITY & SUSPENDED SOLIDS FROM DRAINSYS UNDER SER WATER PMP HSE			05/23/84
54464	NA-2: DELETE TECH. SPEC. REQUIREMENTS FOR SG SUPPORTS		1	06/07/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
46921	NORTH ANNA 2 - CONTROL ROOM HUMAN FACTORS REVIEW			10/01/82
52052	NA-2: ADMIN CHANGES TO T.S. REORGANIZATION OF NUC OPS DEPT. QUALITY ASS., EMERG PLN & SECURITY DEPT			09/22/83
46734	NA-2: FOLLOWUP ON JULY 3, 1981 TRANS FIRE RECURRING TRANS FAULTS		2	12/30/83
53253	NA-2: AMEND T.S. BY ADDING EXEMPTION FOR ENGINEERING EVAL. FOR FAILURE OF SG/RCP SUPPORT SNUBBERS			02/15/84
53529	NA-2: PM BRIEFING BOOK-HEADQUARTERS-OPERATION CENTER			02/28/84
53523	NA-2: REVISE TS FOR REACTOR TRIPS SYS INTERLOCK SETPOINT P-7 FOR 10% FULL LOAD TURBINE IMPULSE CHAMBER PRESSURE			03/20/84
54790	NA-2: ISSUE SER AND EIA FOR SPENT FUEL AND TRANSSHIPMENT HEARING			05/11/84
5473	NA-2: RELIEF FROM ASME XI VOL. EXM ACC AND BORON TANK			05/16/84
54846	NORTH ANNA 2 - CORE RELOAD METHODOLOGY AUDIT			06/01/84
54501	NA-2: REVISE TS PER GEN. LTR. 83-30			06/15/84
54506	NA-2: REVISE TS FOR LER REPORTING PER 10CFR PART 50.72 & 50.73			06/15/84
53525	NA-2: REVISE TS FOR LCO FOR SURVEILLANCE IN MODES 3, 4 AND 5			06/30/84
54833	NA-2: APPENDIX R REANALYSIS EXEMPTION REQUESTS			06/30/84
54165	NA-2: REG. REVIEW-VITAL AREA VALIDATION			06/30/84
52479	NA-2:AMEND TS BY REDUCING LENGTH OF TIME FOR CORE SURVEILLANCE REPORT FROM 30 TO 60 DAYS		2	06/30/84
54834	NA-2: REVISE TECH. SPEC. TO INDICATE VALVES TESTED ONLY FOR EVENT V			07/30/84
54550	NORTH ANNA 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54401	NORTH ANNA 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
52575	NA-2: REVISE TS BY CHANGING FRACTIONAL POWER MULTIPLIER FROM 0.2 TO 0.3 FOR 587.8 F TAV			11/30/84
49523	NA-2: AMEND LIC FOR INCREASE RCS TAV TO 587.8 (PHASE 2- POWER INCREASE)		2	11/30/84
48867	NA-2: AMEND LICENSE FOR RECEIPT AND STORAGE OF SPENT FUEL FROM SURRY 1 & 2		2	12/30/84
48869	NA-2: AMEND T.S. FOR AUGMENTED SPENT FUEL STORAGE			12/30/84
54858	NA-2: INSERVICE TESTING - PUMPS AND VALVES			12/30/84
49666	NORTH ANNA 2 - REACTOR COOLANT PUMP TRIP			02/01/85

ANTICIPATED ACTIONSINITIATION

DATA AS OF - 06/30/84

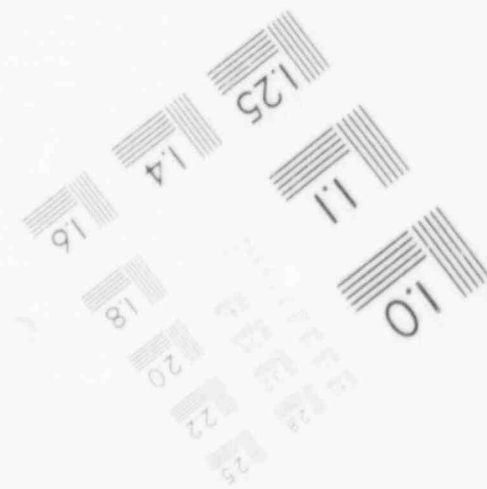
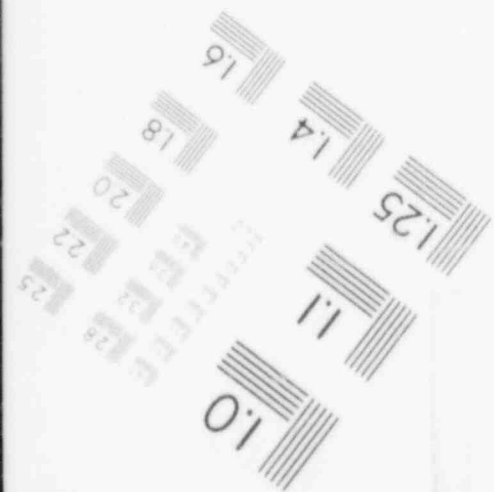
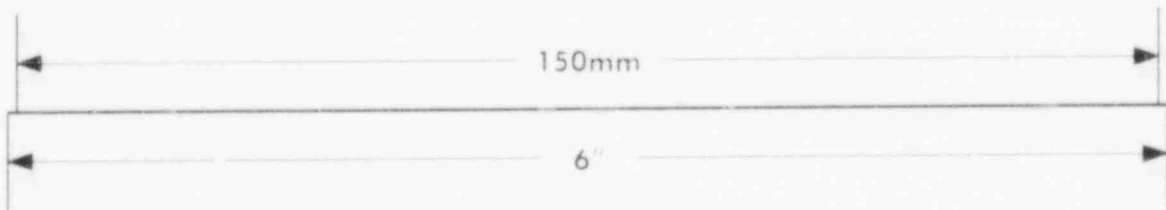
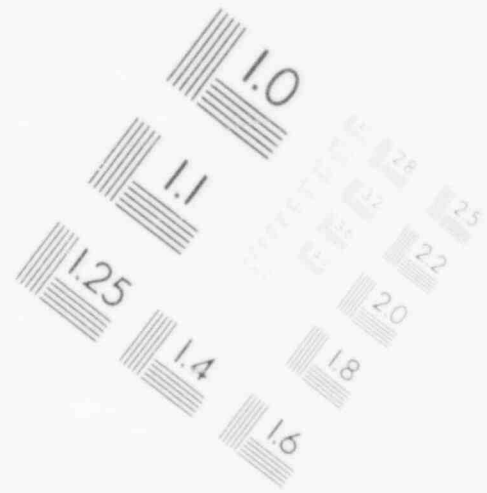
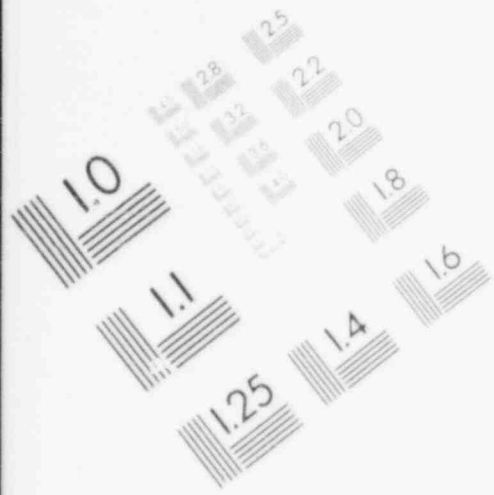
## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

NORTH ANNA 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53611	NORTH ANNA 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53694	NORTH ANNA 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53777	NORTH ANNA 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53860	NORTH ANNA 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53922	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54004	NORTH ANNA 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54087	NORTH ANNA 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

IMAGE EVALUATION  
TEST TARGET (MT-3)





DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: OCONEE 1

PLANT LOCATION: 30 MI W OF GREENVILLE, SC  
 DOCKET NUMBER: 050-00269  
 ARCH/ENGINEER: DUKE/BECH  
 IE INSPECTOR: J. BRYANT

LICENSED POWER: 2568 MWT  
 DESIGN POWER: 0887 MWF  
 NSSS VENDOR: B&W

PROJECT MANAGER: H. NICOLARAS  
 BRANCH CHIEF: J. STOLZ  
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08119	OCONEE 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	<u>COMPLETE</u> 01/16/84
12323	OCONEE 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE	OCONEE 1	3	01/31/84
<u>ACTIVE ACTIONS</u>				
52779	OCONEE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52860	OCONEE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52941	OCONEE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53022	OCONEE 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53083	OCONEE 1 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53136	OCONEE 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
49744	OCONEE 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		2	06/29/84
47183	OCONEE 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	07/31/84
42904	OCONEE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	09/30/84
43643	OCONEE 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	09/30/84
42509	OCONEE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (20-CLI-21)		1	03/31/85
51112	OCONEE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	03/31/85
52250	OCONEE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		2	09/30/85
<u>TMI ACTIONS</u>				
TAC #	TAC DESCRIPTION		PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44109	OCONEE 1 - NUREG-0737 I.A.1.3.1. SHIFT MANNING OVERTIME LIMITS		1	<u>COMPLETE</u> 10/28/83
44390	OCONEE 1 - NUREG-0737 II.B.1. RC. HIGH POINT VENTS		1	11/02/83
44530	OCONEE 1 - NUREG-0737 II.B.4.1. TRAINING FOR MITIGATING CORE		1	11/18/83
44180	OCONEE 1 - NUREG-0737 I.A.2.1.4. UPGRADING OF RO AND SRO TRAINING		1	11/18/83
45270	OCONEE 1 - NUREG-0737 II.K.3.1. AUTO PORV ISOLATION		1	11/22/83
45316	OCONEE 1 - NUREG-0737 II.K.3.2. REPORT ON PORV FAILURES		1	11/22/83
45152	OCONEE 1 - NUREG-0737 II.F.2.3. INADEQUATE CORE COOLING INSTRUMENTATION		1	11/23/83
44459	OCONEE 1 - NUREG-0737 II.B.3.2. POST ACCIDENT SAMPLING MODIFICATIONS		1	01/11/84
<u>ACTIVE ACTIONS</u>				
51182	OCONEE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	06/15/84
51262	OCONEE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		2	06/15/84
46463	OCONEE 1 - NUREG-0737 III.D.3.4. CONTROL ROOM HABITABILITY		1	09/30/84
45199	OCONEE 1 - NUREG-0737 II.K.2.13. THERMAL-MECHANICAL REPORT		1	09/30/84
45845	OCONEE 1 - NUREG-0737 II.K.3.30. SB LOCA OUTLINE		1	09/30/84
45964	OCONEE 1 - NUREG-0737 III.A.1.2. TECHNICAL SUPPORT CENTER		1	09/30/84
46035	OCONEE 1 - NUREG-0737 III.A.1.2. OPERATIONAL SUPPORT CENTER		1	09/30/84
46107	OCONEE 1 - NUREG-0737 III.A.1.2. EMERGENCY OPERATIONS FACILITY		1	09/30/84
46323	OCONEE 1 - NUREG-0737 III.A.2.2. METEOROLOGICAL DATA UPGRADE		1	10/30/84
44600	OCONEE 1 - RV AND SV TESTING		1	11/30/84
44251	OCONEE 1 - NUREG-0737 I.C.1.2.A. INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44321	OCONEE 1 - NUREG-0737 I.C.1.3A. ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
48184	OCONEE 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>TARGET</u> 09/30/85
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
51032	OCONEE 1 - EXEMPTION TO MULTIPLE CRISIS MANAGEMENT CENTER STAFF DRILLS			<u>COMPLETE</u> 01/06/84
48954	OCONEE 1 - ISI RELIEF REQUEST OF 10/4/82			02/14/84
51049	OCONEE 1 - STARTUP TRANSFORMERS AND ALLOWABLE DEGRADED CONDITIONS		2	03/02/84
52612	OCONEE 1 - REQUALIFICATION PROGRAM		2	03/15/84
51315	OCONEE 1 - REFUELING OUTAGE SURVEILLANCE REQUIREMENTS			03/28/84
54206	OCONEE 1 - ECCW TECH. SPEC. CHANGE			04/30/84
<u>ACTIVE ACTIONS</u>				
54144	OCONEE 1 - OVERTIME LIMITS			<u>TARGET</u>
48225	OCONEE 1 - EFW TORNADO PROTECTION		2	
53183	OCONEE 1 - ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
53341	OCONEE 1 - ISI/IST EXEMPTION			02/28/84
49568	OCONEE 1 - RCS LEAK TEST T.S. CHANGES		1	04/15/84
52674	OCONEE 1 - FIRE PROTECTION EXEMPTION (BARRIER/CABLE)			05/04/84
52406	OCONEE 1 - ISI RELIEF REQUEST OF 8/15/83		2	05/31/84
49827	OCONEE 1- SMALL BREAK LOCA PROCEDURES AND MAINTAINING PROPER SG LEVEL		2	06/01/84
55264	OCONEE UNIT 1 - 10CFR20.302 APPLICATION FOR SLUDGE DISPOSAL			06/27/84
52649	OCONEE 1 - FIPE PROTECTION EXEMPTION REQUEST (LIGHTING)			06/29/84
55085	MCGUIRE UNIT 1 - EVAL MCGUIRE RADIOACTIVE MAT'L DISPOSAL PROCEDURE PURSUANT TO IE CIRCULAR 81-07 GUIDANCE REF DUKE			06/29/84
51776	OCONEE 1 - EXEMPTION FOR LOCATION OF EOF		1	06/29/84
51894	OCONEE 1 - HYDROGEN ANALYZER PENETRATION T.S. CHANGES		1	06/29/84
51897	OCONEE 1 - B & PV CODE T.S. CHANGES		1	06/29/84
55012	OCONEE UNIT 1 - WASTE GAS HOLUDP TANKS TECH. SPEC. REVIEW			07/16/84
55015	OCONEE UNIT 1 - TECH. SPEC. REVIEW OF "REPORTABLE OCCURRENCE" REQS			07/31/84
52201	OCONEE 1 - CONFIRMATORY ORDER AMEND REQUEST 8/26/83		1	07/31/84
49974	OCONEE 1 - STA BACHELOR'S DEGREE OR EQUIVALENT			08/15/84
55056	OCONEE UNIT 1: DISPOSAL METHOD FOR VERY LOW LEVEL RADWASTE			08/20/84
54211	OCONEE 1 - ADMIN. T.S. CHANGE OF 2/13/84			08/31/84
54402	OCONEE 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54551	OCONEE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49659	OCONEE 1 - REACTOR COOLANT PUMP TRIP			09/30/84
49234	OCONEE 1 - IST PROGRAM REVISION AND REQUEST FOR RELIEF		1	10/10/84
<u>ANTICIPATED ACTIONS</u>				
53612	OCONEE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53695	OCONEE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53778	OCONEE 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53861	OCONEE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53923	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53955	OCONEE 1 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
54005	OCONEE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84

TECHNICAL ASSIGNMENT CONTROL SYSTEM  
CONDENSED MANAGEMENT REPORT

R-1208620-001

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(CONTINUATION)

OCONEE 1

IAC # IAC DESCRIPTION

PLANT SPECIFIC

54088 ANTICIPATED ACTIONS (CONTINUATION)

OCONEE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

PRIORITY

CRITICAL DATE

INITIATION  
11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: OCONEE 2

PLANT LOCATION: 30 MI W OF GREENVILLE, SC  
 DOCKET NUMBER: 050-00270  
 ARCH/ENGINEER: DUKE/BECH  
 IE INSPECTOR: J. BRYANT

LICENSED POWER: 2568 MWT  
 DESIGN POWER: 0887 MWE  
 NSSS VENDOR: B&W

PROJECT MANAGER: H. NICOLARAS  
 BRANCH CHIEF: J. TOLZ  
 LIC. ASSISTANT: R. INGRAM

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49617	OCONEE 2 - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION		1	<u>COMPLETE</u> 10/28/83
08120	OCONEE 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	01/16/84
12324	OCONEE 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE OCONEE 2		3	01/31/84
49745	OCONEE 2 - NUREG 0737 TECH SPECS (GENERIC LTR 82-16)		2	06/12/84
<u>ACTIVE ACTIONS</u>				
52780	OCONEE 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52861	OCONEE 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52942	OCONEE 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53023	OCONEE 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53084	OCONEE 2 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53137	OCONEE 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
5384	OCONEE 2 - ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42705	OCONEE 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/29/84
47184	OCONEE 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	07/31/84
43644	OCONEE 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	09/30/84
42510	OCONEE 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	03/31/85
51113	OCONEE 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	03/31/85
52251	OCONEE 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		2	09/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44110	OCONEE 2 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			<u>COMPLETE</u> 10/28/83
44391	OCONEE 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	11/02/83
44531	OCONEE 2 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	11/18/83
44181	OCONEE 2 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	11/18/83
45271	OCONEE 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	11/22/83
45317	OCONEE 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	11/22/83
45153	OCONEE 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	11/23/83
44460	OCONEE 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	01/11/84
45200	OCONEE 2 - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT		1	06/06/84
<u>ACTIVE ACTIONS</u>				
51183	OCONEE 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	<u>TARGET</u> 06/15/84
51263	OCONEE 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		2	06/15/84
46108	OCONEE 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46324	OCONEE 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
46464	OCONEE 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	09/30/84
45846	OCONEE 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
45965	OCONEE 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46036	OCONEE 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
44601	OCONEE 2 - RV AND SV TESTING		1	10/30/84



DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 2

TAC #	TAC DESCRIPTION	IMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
44252	OCONEE 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	TARGET 11/30/84
44322	OCONEE 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
48185	OCONEE 2 - NUREG-0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85
<u>PLANT SPECIFIC</u>				
TAC #	TAC DESCRIPTION		PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
52312	OCONEE 2 - TECH SPECS FOR CYCLE 7 RELOAD		1	COMPLETE 11/23/83
51075	OCONEE 2 - EXEMPTION TO MULTIPLE CRISIS MANAGEMENT CENTER STAFF DRILLS		2	01/06/84
51050	OCONEE 2 - STARTUP TRANSFORMERS AND ALLOWABLE DEGRADED CONDITIONS		2	02/14/84
48955	OCONEE 2 - ISI RELIEF REQUEST OF 10/4/82		2	03/15/84
52613	OCONEE 2 - REQUALIFICATION PROGRAM			04/30/84
54207	OCONEE 2 - ECCW TECH. SPEC. CHANGE			06/08/84
55006	OCONEE UNIT 2 - LETTER ON SNUBBER SURVEILLANCE SCHEDULE			
<u>ACTIVE ACTIONS</u>				
54145	OCONEE 2 - OVERTIME LIMITS		2	
48226	OCONEE 2 - EFW TORNADO PROTECTION			
55041	OCONEE 2, NEUTRON SOURCE DATA FOR FLUX REDUCTION VERIFICATION			02/28/84
53342	OCONEE 3 - ISI/IST EXEMPTION		1	04/15/84
49569	OCONEE 2 - RCS LEAK TEST T.S. CHANGES		2	04/20/84
52407	OCONEE 2 - ISI RELIEF REQUEST OF 8/15/83			05/04/84
52675	OCONEE 2 - FIRE PROTECTION EXEMPTION (BARRIER/CABLE)		2	06/01/84
49828	OCONEE 2 - SMALL BREAK LOCA PROCEDURES AND MAINTAINING PROPER SG LEVEL			06/27/84
55265	OCONEE UNIT 2 - 10CFR20.301 APPLICATION FOR SLUDGE DISPOSAL			06/29/84
52650	OCONEE 2 - FIRE PROTECTION EXEMPTION REQUEST (LIGHTING)			06/29/84
55086	MCGUIRE UNIT 2 - EVAL MCGUIRE RADIOACTIVE MAT'L DISPOSAL PROCEDURE PURSUANT TO IE CIRCULAR 81-07 GUIDANCE REF DUKE		1	06/29/84
51777	OCONEE 2 - EXEMPTION FOR LOCATION OF EOF		1	06/29/84
51895	OCONEE 2 - HYDROGEN ANALYZER PENETRATION T.S. CHANGES		1	06/29/84
51898	OCONEE 2 - B & PV CODE T.S. CHANGES			07/16/84
55013	OCONEE UNIT 2 - WASTE GAS HOLDUP TANKS TECH. SPEC. REVIEW			07/31/84
55016	OCONEE UNIT 2 - TECH. SPEC. REVIEW OF "REPORTABLE OCCURRENCE" REQS		1	07/31/84
52202	OCONEE 2 - CONFIRMATORY ORDER AMEND. REQUEST 8/26/83			08/15/84
49972	OCONEE 2 - STA BACHELOR'S DEGREE OR EQUIVALENT			08/20/84
55057	OCONEE UNIT 2: DISPOSAL METHOD FOR VERY LOW LEVEL RADWASTE			08/31/84
55160	OCONEE UNIT 2 - ISI FLAW EVALUATIONS			08/31/84
54212	OCONEE 2 - ADMIN. T.S. CHANGE OF 2/13/84			09/30/84
54403	OCONEE 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54552	OCONEE 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		2	09/30/84
49660	OCONEE 2 - REACTOR COOLANT PUMP TRIP		1	10/10/84
49235	OCONEE 2 - IST PROGRAM REVISION AND REQUEST FOR RELIEF			
<u>ANTICIPATED ACTIONS</u>				
53613	OCONEE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53695	OCONEE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53779	OCONEE 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53862	OCONEE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53924	OCONEE 2 - ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53956	OCONEE 2 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			<u>INITIATION</u> 11/01/84
54006	OCONEE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54089	OCONEE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: OCONEE 3

PLANT LOCATION: 30 MI W OF GREENVILLE, SC  
 DOCKET NUMBER: 050-00287  
 ARCH/ENGINEER: DUKE/BECH  
 IE INSPECTOR: J. BRYANT

LICENSED POWER: 2568 MWT  
 DESIGN POWER: 0887 MWE  
 NSSS VENDOR: B&W

PROJECT MANAGER: H. NICOLARAS  
 BRANCH CHIEF: J. STOLZ  
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08121	OCONEE 3 - APPENDIX I-ALARA		3	<u>COMPLETE</u> 01/16/84
12325	OCONEE 3 - HELB AND C/ SEQUENTIAL SYSTEM FAILURE OCONEE 3		3	01/31/84
49746	OCONEE 3 - NUREG 0737 TECH SPECS (GENERIC LTR 82-16)		2	06/12/84
<u>ACTIVE ACTIONS</u>				
52731	OCONEE 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52862	OCONEE 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52943	OCONEE 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53024	OCONEE 3 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53085	OCONEE 3 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53138	OCONEE 3 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53185	OCONEE 3 - ITEM 4.3 - AUTOMATIC AUTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		2	06/29/84
42906	OCONEE 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	07/31/84
47185	OCONEE 3 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		1	09/30/84
43645	OCONEE 3 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	03/31/85
42511	OCONEE 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	03/31/85
51114	OCONEE 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		2	09/30/85
52252	OCONEE 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-13)			
<u>TMI ACTIONS</u>				
TAC #	TAC DESCRIPTION		PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44111	OCONEE 3 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS		1	<u>COMPLETE</u> 10/23/83
44392	OCONEE 3 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	11/02/83
44182	OCONEE 3 - NUREG-0737 I.A.2.1.4, UPGRADING OF RO AND SRO TRAINING		1	11/18/83
44532	OCONEE 3 - NUREG-0737 II.B.4.1, TRAINING FOR MITIGATING CORE		1	11/18/83
45272	OCONEE 3 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	11/22/83
45318	OCONEE 3 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	11/22/83
45154	OCONEE 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	11/23/83
44461	OCONEE 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	01/11/84
45201	OCONEE 3 - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT		1	06/06/84
<u>ACTIVE ACTIONS</u>				
51184	OCONEE 3 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	<u>TARGET</u> 06/15/84
51264	OCONEE 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		2	06/15/84
45847	OCONEE 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
45966	OCONEE 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46037	OCONEE 3 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46109	OCONEE 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46325	OCONEE 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
46465	OCONEE 3 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	10/30/84
44602	OCONEE 3 - RV AND SV TESTING		1	11/30/84
44253	OCONEE 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OCONEE 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
44323	OCONEE 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	<u>TARGET</u> 11/30/84
48186	OCONEE 3 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85
<u>COMPLETED ACTIONS</u>				
51034	OCONEE 3 - EXEMPTION TO MULTIPLE CRISIS MANAGEMENT CENTER STAFF DRILLS			<u>COMPLETE</u> 01/06/84
52447	OCONEE 3 - APSR POSITION LIMITS		1	01/30/84
48956	OCONEE 3 - ISI RELIEF REQUEST OF 10/4/82			02/14/84
51051	OCONEE 3 - STARTUP TRANSFORMERS AND ALLOWABLE DEGRADED CONDITIONS		2	03/02/84
52614	OCONEE 3 REQUALIFICATION PROGRAM		2	03/15/84
54208	OCONEE 3 - ECCN TECH. SPEC. CHANGE			04/30/84
54205	OCONEE 3 - TECH SPECS FOR CYCLE 8 RELOAD			05/15/84
<u>ACTIVE ACTIONS</u>				
54146	OCONEE 3 - OVERTIME LIMITS			<u>TARGET</u>
48227	OCONEE 3 - EFW TORNADO PROTECTION		2	
53343	OCONEE 2 - ISI/IST EXEMPTION			02/28/84
49570	OCONEE 3 - RCS LEAK TEST T.S. CHANGES		1	04/15/84
52408	OCONEE 3 - ISI RELIEF REQUEST OF 8/15/83		2	04/20/84
52676	OCONEE 3 - FIRE PROTECTION EXEMPTION (BARRIER/CABLE)			05/04/84
49829	OCONEE 3- SMALL BREAK LOCA PROCEDURES AND MAINTAINING PROPER SG LEVEL		2	06/01/84
55266	OCONEE UNIT 3 - 10CFR20.302 APPLICATION FOR SLUDGE DISPOSAL			06/27/84
52651	OCONEE 3 - FIRE PROTECTION EXEMPTION REQUEST (LIGHTING)			06/29/84
51778	OCONEE 3 - EXEMPTION FOR LOCATION OF EOF		1	06/29/84
51896	OCONEE 3 - HYDROGEN ANALYZER PENETRATION T.S. CHANGES		1	06/29/84
51899	OCONEE 3 - B & PV CODE T.S. CHANGES		1	06/29/84
55014	OCONEE UNIT 3 - WASTE GAS HOLDUP TANKS TECH. SPEC. REVIEW			07/16/84
55017	OCONEE UNIT 3 - TECH. SPEC. REVIEW OF "REPORTABLE OCCURRENCE" REQS			07/31/84
52203	OCONEE 3 - CONFIRMATORY ORDER AMEND. REQUEST 8/26/83		1	07/31/84
49978	OCONEE 3 - STA BACHELOR'S DEGREE OR EQUIVALENT			08/15/84
55058	OCONEE UNIT 3: DISPOSAL METHOD FOR VERY LOW LEVEL RADWASTE			08/20/84
54213	OCONEE 3 - ADMIN. T.S. CHANGE OF 2/13/84			08/31/84
54404	OCONEE 3- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54553	OCONEE 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49661	OCONEE 3 - REACTOR COOLANT PUMP TRIP			09/30/84
49236	OCONEE 3 - IST PROGRAM REVISION AND REQUEST FOR RELIEF		1	10/10/84
<u>ANTICIPATED ACTIONS</u>				
53614	OCONEE 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53697	OCONEE 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53780	OCONEE 3- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53863	OCONEE 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53925	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
53957	OCONEE 3 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
54007	OCONEE 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54090	OCONEE 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



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TECHNICAL ASSIGNMENT CONTROL SYSTEM  
CONDENSED MANAGEMENT REPORT

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FACILITY: OYSTER CREEK 1

PLANT LOCATION: 9 MI S OF TOMS RIVER, NJ  
DOCKET NUMBER: 050-00219  
ARCH/ENGINEER: B&R  
IE INSPECTOR: C. CONGILL

LICENSED POWER: 1930 MWT  
DESIGN POWER: 0650 MWE  
NSSS VENDOR: GE

PROJECT MANAGER: J. LOMBARDO  
BRANCH CHIEF: D. CRUTCHFIELD  
LIC. ASSISTANT: H. SMITH

		MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
07944	OYSTER CREEK - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	01/13/84
46668	OYSTER CREEK - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/29/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52732	OYSTER CREEK 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52863	OYSTER CREEK 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52944	OYSTER CREEK 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53025	OYSTER CREEK 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			05/01/84
51115	OYSTER CREEK 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/01/84
51687	OYSTER CREEK NUREG 0737 TECH SPECS		1	07/15/84
42613	OYSTER CREEK - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		2	08/01/84
08439	OYSTER CREEK - MECHANICAL SNUBBERS		3	08/01/84
08440	OYSTER CREEK - HYDRAULIC SNUBBERS		1	08/15/84
42524	OYSTER CREEK - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		2	10/01/84
43717	OYSTER CREEK - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	11/01/84
42914	OYSTER CREEK - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		3	11/15/84
08100	OYSTER CREEK - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW			09/30/85
52253	OYSTER CREEK 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
		<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
44112	OYSTER CREEK - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS		1	11/28/83
45848	OYSTER CREEK - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/21/83
48187	OYSTER CREEK -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/21/83
45802	OYSTER CREEK - NUREG-0737 II.K.3.29, PERFORMANCE OF ISOLATION CONDENSER		1	02/01/84
44254	OYSTER CREEK - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
44603	OYSTER CREEK 1 - RV AND SV TESTING			06/19/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
46466	OYSTER CREEK - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	06/01/84
47960	OYSTER CREEK - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	06/01/84
51185	OYSTER CREEK 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			08/01/84
51265	OYSTER CREEK 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	08/01/84
46326	OYSTER CREEK - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	08/01/84
45155	OYSTER CREEK - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	08/01/84
45600	OYSTER CREEK - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/01/84
45696	OYSTER CREEK - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	08/01/84
45720	OYSTER CREEK - NUREG-0737 II.K.3.19, INTERLOCK RECIRC PUMP MODIFICATION		1	08/01/84
44324	OYSTER CREEK - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	08/01/84
44462	OYSTER CREEK - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFCATIONS		1	09/30/84
45967	OYSTER CREEK - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46038	OYSTER CREEK - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46110	OYSTER CREEK - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OYSTER CREEK 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
51304	OYSTER CREEK	- REVIEW LICENSEE'S ADMINISTRATIVE ORGANIZATION CHANGE REQUEST TO THE T.S.		<u>COMPLETE</u>
51726	OYSTER CREEK	- REVIEW REQUEST TO DEFER REPLACEMENT OF CORE SPRAY SPARGER FROM CYCLE 10 TO CYCLE 11 REFUELING OUTAGE	2	01/12/84
52460	OYSTER CREEK	- REVIEW TS CHANGE REQUEST TO THE NEUTRON MONITORING SYSTEM		01/26/84
51402	OYSTER CREEK	- REVIEW REQUEST FOR AMENDMENT TO ISOLATION CONDENSER TECH. SPECS.		01/30/84
43588	OYSTER CREEK	- NEW CORE SPRAY SPARGER DESIGN AND PIPING MODIFICATIONS		02/06/84
49393	OYSTER CREEK	- HYDROSTATIC LOADS	1	02/17/84
52337	OYSTER CREEK	- REVIEW REQUEST FOR APPROVAL OF PROPOSED DISPOSAL METHODS	1	02/23/84
52517	OYSTER CREEK	- REVIEW TS CHANGE REQUEST FOR SECONDARY CONTAINMENT INTEGRITY		03/08/84
52519	OYSTER CREEK	- REVIEW TS CHANGE REQUEST FOR SCRAM SYSTEM RELIABILITY		06/19/84
				06/20/84
<u>ACTIVE ACTIONS</u>				
54987	OYSTER CREEK	- ISOLATION CONDENSER PIPING CRACK FAILURE ANALYSIS		<u>TARGET</u>
52461	OYSTER CREEK	- REVIEW FIRE PROTECTION EXEMPTION REQUESTS		
51314	OYSTER CREEK	- REVIEW OF NO JET PUMP CORE SPRAY PERFORMANCE		06/01/84
52549	OYSTER CREEK	- REVIEW TS CHANGE REQUEST FOR ISI	1	06/01/84
52518	OYSTER CREEK	- REVIEW TS CHANGE REQUEST FOR HIGH DRYWELL PRESSURE TRIP SETPOINT		06/15/84
52336	OYSTER CREEK	- REVIEW FINAL ENVIRONMENTAL STATEMENT (FES) FOR POL/FTL CONVERSION		06/15/84
48787	OYSTER CREEK	- REVIEW REQUEST FOR INCREASED SPENT FUEL STORAGE CAPACITY		07/01/84
42646	OYSTER CREEK	- GENERAL ELECTRIC RELOAD FUEL APPLICATION	2	07/01/84
49392	OYSTER CREEK	- WIND LOADS	1	07/01/84
49394	OYSTER CREEK	- TORNADO MISSILE DAMAGE	1	08/01/84
53411	OYSTER CREEK	- REVIEW ADMINISTRATIVE TS CHANGE REQUEST FOR CLARITY & FORMAT ONLY	1	08/01/84
49395	OYSTER CREEK	- TURBINE MISSILE		08/01/84
49397	OYSTER CREEK	- EMERGENCY CONDENSER ISOLATION	1	08/01/84
49398	OYSTER CREEK	- SEISMIC DESIGN	1	08/01/84
49399	OYSTER CREEK	- DESIGN CODES AND STANDARDS	1	08/01/84
49400	OYSTER CREEK	- THERMAL OVERLOAD	1	08/01/84
49404	OYSTER CREEK	- WATER PURITY SPECIFICATION	1	08/01/84
49405	OYSTER CREEK	- EMERGENCY CONDENSER LOGIC TESTING	1	08/01/84
49406	OYSTER CREEK	- BUS TRANSFERS	1	08/01/84
49407	OYSTER CREEK	- RPS TESTING	1	08/01/84
49408	OYSTER CREEK	- NEUTRON MONITORING ISOLATION	1	08/01/84
49410	OYSTER CREEK	- BATTERY STATUS ALARMS	1	08/01/84
49411	OYSTER CREEK	- VENTILATION SYSTEMS	1	08/01/84
49412	OYSTER CREEK	- PRIMARY COOLANT ACTIVITY	1	08/01/84
49413	OYSTER CREEK	- MAIN STEAM ISOLATION VALVE MAINT.	1	08/01/84
49496	OYSTER CREEK	- FINAL RULE ON INTERIM REQUIREMENTS RELATED TO H2 CONTROL - REVIEW OF EXEMPTION REQUEST	1	08/01/84
52516	OYSTER CREEK	- REVIEW TS CHANGE REQUEST FOR GRID UNVOLTAGE PROTECTION SYSTEM AND STATION BATTERIES		08/01/84
11270	OYSTER CREEK	- INSERVICE TESTING (IST)	1	
11268	OYSTER CREEK	- FULL TERM OPERATING LICENSE	1	08/15/84
54554	OYSTER CREEK	1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		09/30/84
49401	OYSTER CREEK	- LEAKAGE DETECTION	1	09/30/84
				02/01/85
<u>ANTICIPATED ACTIONS</u>				
53615	OYSTER CREEK	1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		<u>INITIATION</u>
53698	OYSTER CREEK	1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		11/01/84
				11/01/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

OYSTER CREEK 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53781	OYSTER CREEK 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			<u>INITIATION</u> 11/01/84
53864	OYSTER CREEK 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54008	OYSTER CREEK 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54091	OYSTER CREEK 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: PALISADES

PLANT LOCATION: 5 MI S OF SOUTH HAVEN, MI  
 DOCKET NUMBER: 050-00255  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: B. JORGENSEN

LICENSED POWER: 2530 MWT  
 DESIGN POWER: 0805 MWE  
 NSSS VENDOR: COMB

PROJECT MANAGER: W. PAULSON  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47129	PALISADES - CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANTS (NUREG 0612)		2	<u>COMPLETE</u> 11/09/83
47186	PALISADES - NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	12/15/83
49747	PALISADES - NUREG 0737 TECH SPECS (GENERIC LTR 82-16)			03/02/84
42764	PALISADES - ESF RESET CONTROL DESIGN DEFICIENCY - I&E BULLETIN 80-06		3	03/07/84
46850	PALISADES - PWR MSLB WITH CONTINUED FEEDWATER ADDITION		2	04/11/84
42614	PALISADES - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	04/18/84
06779	PALISADES - FILTER TECH SPECS		3	05/22/84
<u>ACTIVE ACTIONS</u>				
51115	PALISADES - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>RGET</u>
42915	PALISADES - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
42517	PALISADES - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
42130	PALISADES - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		2	07/15/84
43052	PALISADES - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	07/31/84
08621	PALISADES - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	08/30/84
08148	PALISADES - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	11/15/84
52254	PALISADES - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			12/31/84
52783	PALISADES - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			12/31/84
52864	PALISADES - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			12/31/84
52945	PALISADES - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			12/31/84
53026	PALISADES - ITEM 3.1.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH SPECS - RTS COMPONENTS			12/31/84
53086	PALISADES - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			12/31/84
53139	PALISADES - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			12/31/84
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47810	PALISADES - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 10/19/83
45265	PALISADES - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	10/20/83
45311	PALISADES - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/20/83
44394	PALISADES - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	11/07/83
44873	PALISADES - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	04/18/84
44255	PALISADES - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
46896	PALISADES - II K2.13 THERMAL MECHANICAL REPORT		1	06/13/84
<u>ACTIVE ACTIONS</u>				
48188	PALISADES - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>TARGET</u> 06/30/84
45849	PALISADES - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
46327	PALISADES - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/30/84
44463	PALISADES - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
45156	PALISADES - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
44666	PALISADES - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION		1	07/31/84
44604	PALISADES - RV AND SV TESTING		1	08/31/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## PALISADES

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
45968	PALISADES - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	TARGET 09/30/84
46039	PALISADES - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46111	PALISADES - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
48448	PALISADES - NUREG 0737, II.K.3.25 POWER ON PUMP SEALS		1	11/30/84
44325	PALISADES - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES			06/30/85
51186	PALISADES - I.D.1 CONTROL ROOM DESIGN REVIEW			06/30/85
51266	PALISADES - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
49358	PALISADES - INTEGRATED ASSESSMENT SUPPLEMENT (POST SEP)			COMPLETE 11/07/83
52532	PALISADES - REVISIONS TO GUARD TRAIN. PLAN			11/07/83
53543	PALISADES - STEAM GENERATOR INSPECTION AND REPAIR			02/28/84
52290	PALISADES - T.S. CHANGE R.V. SURVEILLANCE SPECIMEN SCHEDULE			02/28/84
54298	PALISADES - VARIANCE FROM THE DISPLAY CRITERION FOR NOBLE GAS EFFLUENT MONITOR (II.F.J.1)			03/07/84
54477	PALISADES - PROPOSED T/S CHANGES - REVISED ACCIDENT & TRANSIENT ANALYSES DUE TO STEAM GENERATOR TUBE PLUGGING			04/12/84
53507	PALISADES - EXEMPTION REQUEST FROM REQUIREMENT TO PERFORM AIR LOCK LEAK TEST AT PA			05/14/84
52511	PALISADES - CHANGE IN BASIS FOR LIMITING SAFETY SYSTEM SETTINGS BASED ON REVISED ROD WITHDRAWAL ANALYSIS			06/07/84
54269	PALISADES - JUSTIFICATION OF XNE CORRELATION FOR PALISADES			06/07/84
54811	PALISADES - ANALYSIS OF AXIAL POWER DISTRIBUTION LIMITS (XN-NF-78-16, SUPPLEMENT 1)			06/11/84
53508	PALISADES - STEAM GENERATOR INSPECTION AND REPAIR			06/11/84
54166	PALISADES - DELETE SPARE HPSI PUMP FROM SECTION 5.33 OF TECH. SPECS.			06/19/84
55007	PALISADES - HYDROTEST RELIEF REQUEST			06/26/84
<u>ACTIVE ACTIONS</u>				
53557	PALISADES - REQUEST TO DELETE REQUIREMENTS OF NOVEMBER 9 1979 ORDER FOR MODIFICATION OF LICENSE		1	TARGET 06/30/84
42320	PALISADES - RELIEF REQUEST FOR INSERVICE PUMP TESTING PROGRAM			06/30/84
48725	PALISADES - SEISMIC REVIEW OF PIPING <2 1/2" & ELECTRICAL COMPONENTS		3	06/30/84
46574	PALISADES - DISCONNECTION OF BOTH REDUNDANT BATTERIES FROM DC BUSES			06/30/84
49327	PALISADES PLANT - PROPOSED TECHNICAL SPECIFICATION CHANGE REQUEST: SECTION 6 ADMIN CONTROLS			07/31/84
48726	PALISADES - DESIGN CODE ASSESSMENT		1	07/31/84
43269	PALISADES - ISI UPDATE SEC. XI 77578 ADDENDA DATED 11/12/80			07/31/84
53545	PALISADES - TECH. SPEC. CHANGE OPERABILITY OF LEAKAGE		1	07/31/84
11218	PALISADES - FULL TERM OPERATING LICENSE			09/30/84
55299	PALISADES - TECHNICAL SPECIFICATIONS FOR CONTAINMENT PURGE AND VENTILATION SYSTEMS			09/30/84
54555	PALISADES - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54795	PALISADES - ADDITION OF 408V DISTRIBUTION BUSES - TECH SPEC. CHANGE			09/30/84
54405	PALISADES - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		1	10/15/84
07733	PALISADES - HYDROGEN RECOMBINER TESTING REQUIREMENTS			10/30/84
54844	PALISADES - APPENDIX R EXEMPTION REQUEST			10/30/84
52490	PALISADES - MSIV COST/BENEFIT ANALYSIS OF TWO STEAM GENERATOR BLOWDOWN			02/01/85
49700	PALISADES - REACTOR COOLANT PUMP TRIP			
<u>ANTICIPATED ACTIONS</u>				
53616	PALISADES - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53699	PALISADES - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53782	PALISADES- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PALISADES

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53855	PALISADES - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			<u>INITIATION</u> 11/01/84
53926	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54009	PALISADES - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54092	PALISADES - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: PEACH BOTTOM 2

PLANT LOCATION: 19 MI S OF LANCASTER, PA  
 DOCKET NUMBER: 050-00277  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: A. BLOUGH

LICENSED POWER: 3293 MWT  
 DESIGN POWER: 1065 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: G. GEARS  
 BRANCH CHIEF: J. STOLZ  
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08503	PEACH BOTTOM 2 - CONTAINMENT LEAK TESTING - APP. J (GENERIC)		2	COMPLETE 03/01/84
07945	PEACH BOTTOM 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	03/06/84
11258	PEACH BOTTOM 2 - IST		1	03/30/84
42605	PEACH BOTTOM 2 - LONG TERM REV CONTAINMENT PURGE AND VENT (B-24)		1	04/25/84
08935	PEACH BOTTOM 2 - RPS POWER SUPPLY		1	06/21/84
<u>ACTIVE ACTIONS</u>				
52255	PEACH BOTTOM 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			TARGET
52784	APEACH BOTTOM 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52865	PEACH BOTTOM 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52946	PEACH BOTTOM 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53027	PEACH BOTTOM 2 - ITEM 3.1.3 - POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
51638	PEACH BOTTOM 2 - NUREG 0737 TECH SPECS			06/29/84
08501	PEACH BOTTOM 2 - SURV. OF MECHANICAL SNUBBERS		2	06/29/84
46671	PEACH BOTTOM 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/29/84
08044	PEACH BOTTOM 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/30/84
42922	PEACH BOTTOM 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	07/02/84
42503	PEACH BOTTOM 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/31/84
43728	PEACH BOTTOM 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	09/28/84
51117	PEACH BOTTOM 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			12/28/84
<u>COMPLETED ACTIONS</u>				
44605	PEACH BOTTOM 2 - RV AND SV TESTING		1	COMPLETE 11/04/83
44464	PEACH BOTTOM 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/08/84
45850	PEACH BOTTOM 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	02/28/84
48189	PEACH BOTTOM 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	02/28/84
48275	PEACH BOTTOM 2 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	05/14/84
45697	PEACH BOTTOM 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/05/84
44256	PEACH BOTTOM 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
51187	PEACH BOTTOM 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	TARGET 07/21/84
44874	PEACH BOTTOM 2 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	08/31/84
44326	PEACH BOTTOM 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	09/30/84
45969	PEACH BOTTOM 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46040	PEACH BOTTOM 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46112	PEACH BOTTOM 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46328	PEACH BOTTOM 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	12/14/84
45157	PEACH BOTTOM 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/29/84
51267	PEACH BOTTOM 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			12/30/84
45601	PEACH BOTTOM 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	

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## CONDENSED MANAGEMENT REPORT

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PEACH BOTTOM 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
48111	PEACH BOTTOM 1 - TECH SPECS RELATED TO IMPLEMENTATION OF NUREG-0737			<u>COMPLETE</u>
52371	PEACH BOTTOM 2 - ANNUAL EXERCISE EXEMPTION REQUEST		3	10/04/83
51784	PEACH BOTTOM 2 - JCO CONCERNING RHR PIPE CRACKS			11/03/83
51552	PEACH BOTTOM 2 - RECIRCULATION PIPE CRACKS			11/30/83
49194	PEACH BOTTOM 2 - GEOLOGICAL ASSESSMENT OF ALLEGED FAULTING AT SITE		1	11/30/83
51385	PEACH BOTTOM 2 - TS FOR PERIODIC LEAK INTEGRITY TESTING		3	12/06/83
52377	PEACH BOTTOM 2 - ALT. SHUTDOWN CAPABILITY ASSESSMENT			12/12/83
48469	PEACH BOTTOM 2 - ROD BLOCK SYSTEM TECH. SPEC. CHANGE		1	01/26/84
49603	PEACH BOTTOM 2 - DELETION OF APP. B WATER QUAL. REG. AND AQUATIC MONITORING		2	02/10/84
51404	PEACH BOTTOM 2 - NOISE MONITORING PROGRAM - APP. B TS AMENDMENT		3	02/24/84
53331	PEACH BOTTOM 2 - ONE-TIME EXEMPTION FROM LOCAL LEAK RATE TEST		2	02/24/84
53351	PEACH BOTTOM 2 - TS CHANGE TO PERMIT TEMP. BYPASS OF HYDROGEN TRIP			02/28/84
49031	PEACH BOTTOM 2 - MK 1 TORUS T.S. CHANGES LONG TERM PROGRAM		2	02/28/84
48874	PEACH BOTTOM 2 - REVIEW OF HEALTH PHYSICS ORGANIZATIONAL CHANGE & GENERAL ORG. CHANGES IN T.S.			03/13/84
51019	PEACH BOTTOM 2 - HPCI STEAM LINE LOW-PRESSURE		2	03/21/84
51354	PEACH BOTTOM 2 - FOUR MISC. TECH SPEC CHANGES		2	03/21/84
48404	PEACH BOTTOM 2 - DEGRADED GRID VOLTAGE TECH. SPECS.			03/28/84
53485	PEACH BOTTOM - 2 EXEMPTION REQUEST FROM ISI-CLASS II ECCS			04/11/84
				06/01/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
55059	PEACH BOTTOM 2: REVIEW OF PENETRATION SEAL TEST REPORTS AND PROGRAM			
55157	PEACH BOTTOM 2 - TS CHANGES INVOLVING MANAGEMENT REORGANIZATION			
55161	PEACH BOTTOM 2: TECH. SPEC. CHANGES RELATED TO PURGE/VENT VALVES			
55174	PEACH BOTTOM 2: JET PUMP INSTRUMENT NOZZLE CRACKS			
54617	PEACH BOTTOM 2 - EMERGENCY PREPAREDNESS EXERCISE EXEMPTION REQUEST			06/15/84
54488	PEACH BOTTOM 2: FIRE BARRIER PENETRATIONS T/S CHANGE			06/22/84
54367	PEACH BOTTOM 2 - TS CHANGE TO DELETE DRYWELL AIR MONITORING SYSTEM			06/29/84
52681	PEACH BOTTOM 2 - TS CHANGE FOR COOLANT LEAKAGE AND I-131 LIMITS			06/29/84
53273	PEACH BOTTOM 2 - TS CHANGE RELATED TO SDV AND RWCU INSTRUMENTATION			06/29/84
47537	PEACH BOTTOM 2 - CORE SPRAY PUMP DISCHARGE PRESSURE INTERLOCK		3	06/30/84
47539	PEACH BOTTOM 2 - REACTOR LEVEL SURVEILLANCE INSTRUMENTS		3	06/30/84
47375	PEACH BOTTOM 2 - SUPPRESSION CHAMBER HIGH LEVEL TRIP SETTING		3	06/30/84
47377	PEACH BOTTOM 2 - SURVEILLANCE REQS FOR THE PCIS/LPC* INTERLOCK PRESSURE SWITCH		3	06/30/84
54766	PEACH BOTTOM 2 - OPERABILITY OF PURGE/VENT VALVES			06/30/84
51741	PEACH BOTTOM 2 - REVIEW OF APP R, III F AND III M EXEMPTION REQUESTS		2	07/02/84
51017	PEACH BOTTOM 2 WETWELL/DRYWELL VACUUM BREAKERS		3	07/30/84
46787	PEACH BOTTOM 2 - TECH SPEC CHANGE RELATED TO VERIF. OF DRYWELL-TO-TORUS VAC BREAKER CLOSURE		3	07/31/84
46789	PEACH BOTTOM 2 - ENVIRONM TECH SPEC CHANGE RELATED TO A REPORTING PERIOD		3	07/31/84
54800	PEACH BOTTOM 2: REVIEW OF PROPOSED TECH. SPEC. REQUIREMENTS ON AIR SUPPLY SYSTEM LEAKAGE			07/31/84
54793	PEACH BOTTOM 2 - TS CHANGE ON REQUIREMENTS FOR TESTING EMERGENCY BATTERIES			08/03/84
47379	PEACH BOTTOM 2 - FUEL OIL SYSTEMS FOR STANDBY DIESEL GENERATORS		3	08/30/84
54828	PEACH BOTTOM 2: TECH. SPEC. CHANGES CONCERNING REACTOR WATER LEVEL SETPOINT			08/31/84
54830	PEACH BOTTOM 2 - TECH. SPEC. CHANGE ON AUDIT OF EMERG. PLAN PROCEDURES			08/31/84
54823	PEACH BOTTOM 2: REVISED APP. J. TECH. SPECS. REQUEST		2	09/03/84
54556	PEACH BOTTOM 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54359	PEACH BOTTOM 2: REVIEW OF STRUCTURAL STEEL			09/30/84
<u>ANTICIPATED ACTIONS:</u>				<u>INITIATION</u>
53617	PEACH BOTTOM 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84



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PEACH BOTTOM 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53700	PEACH BOTTOM 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			<u>INITIATION</u> 11/01/84
53783	PEACH BOTTOM 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53866	PEACH BOTTOM 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54010	PEACH BOTTOM 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54093	PEACH BOTTOM 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: PEACH BOTTOM 3

PLANT LOCATION: 19 MI S OF LANCASTER, PA  
 DOCKET NUMBER: 050-00278  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: A. BLOUGH

LICENSED POWER: 3293 MWT  
 DESIGN POWER: 1065 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: G. GEARS  
 BRANCH CHIEF: J. STOLZ  
 LIC. ASSISTANT: R. INGRAM

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08504	PEACH BOTTOM 3 - CONTAINMENT LEAK TESTING - APP. J (GENERIC)		2	<u>COMPLETE</u> 03/01/84
07946	PEACH BOTTOM 3 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	03/06/84
11257	PEACH BOTTOM 3 - IST		1	03/30/84
42606	PEACH BOTTOM 3 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B 74)		1	04/25/84
08956	PEACH BOTTOM 3 - RPS POWER SUPPLY		1	06/21/84
<u>ACTIVE ACTIONS</u>				
52256	PEACH BOTTOM 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA50-10)			<u>TARGET</u>
52785	PEACH BOTTOM 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52866	PEACH BOTTOM 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52947	PEACH BOTTOM 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53028	PEACH BOTTOM 3 - ITEM 3.1.3 - POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
51689	PEACH BOTTOM 3 - NUREG 0737 TECH SPECS			06/29/84
08500	PEACH BOTTOM 3 - SURV. OF MECHANICAL SNUBBERS		2	06/29/84
08122	PEACH BOTTOM 3 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/30/84
46672	PEACH BOTTOM 3 - IMPLEMENTATION OF NU CC-0313, REV. 1		2	06/30/84
42923	PEACH BOTTOM 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	07/02/84
42504	PEACH BOTTOM 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/31/84
43732	PEACH BOTTOM 3 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	09/28/84
51118	PEACH BOTTOM 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.17)			12/28/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44606	PEACH BOTTOM 3 - RV AND SV TESTING		1	<u>COMPLETE</u> 11/04/83
44465	PEACH BOTTOM 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/05/84
45851	PEACH BOTTOM 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	02/28/84
48190	PEACH BOTTOM 3 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	02/28/84
48276	PEACH BOTTOM 3 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	05/14/84
45698	PEACH BOTTOM 3 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	06/05/84
44257	PEACH BOTTOM 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
51188	PEACH BOTTOM 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
44875	PEACH BOTTOM 3 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	07/31/84
44327	PEACH BOTTOM 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	08/31/84
45970	PEACH BOTTOM 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46041	PEACH BOTTOM 3 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46113	PEACH BOTTOM 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46329	PEACH BOTTOM 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
45158	PEACH BOTTOM 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/14/84
51268	PEACH BOTTOM 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			12/29/84
45602	PEACH BOTTOM 3 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	12/30/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PEACH BOTTOM 3

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS:</u>				<u>COMPLETE</u>
48112	PEACH BOTTOM 2	- TECH SPECS RELATED TO IMPLEMENTATION OF NUREG-0737	3	10/04/83
52372	PEACH BOTTOM 3	- ANNUAL EXERCISE EXEMPTION REQUEST		11/03/83
49195	PEACH BOTTOM 3	- GEOLOGICAL ASSESSMENT OF ALLEGED FAULTING AT SITE	3	12/06/83
51386	PEACH BOTTOM 3	- TS FOR PERIODIC LEAK INTEGRITY TESTING		12/12/83
52378	PEACH BOTTOM 3	- ALT. SHUTDOWN CAPABILITY ASSESSMENT	2	01/26/84
48470	PEACH BOTTOM 3	- ROD BLOCK SYSTEM TECH. SPEC. CHANGE	2	02/10/84
49604	PEACH BOTTOM 3	- DELETION OF APP. B WATER QUAL. REG. AND AQUATIC MONITORING	3	02/24/84
51405	PEACH BOTTOM 3	- NOISE MONITORING PROGRAM - APP. B TS AMENDMENT	2	02/24/84
49032	PEACH BOTTOM 3	- MK 1 TORUS T.S. CHANGES LONG TERM PROGRAM		03/13/84
48875	PEACH BOTTOM 2	- REVIEW OF HEALTH PHYSICS ORGANIZATIONAL CHANGE & GENERAL ORG. CHANGES IN T.S.		03/21/84
51020	PEACH BOTTOM 3	- HPCI STEAM LINE LOW PRESSURE TECH. SPEC.	2	03/21/84
51355	PEACH BOTTOM 3	- FOUR MISC. TECH SPEC CHANGES	2	03/28/84
48405	PEACH BOTTOM 3	- DEGRADED GRID VOLTAGE TECH. SPECS.		04/11/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
55060	PEACH BOTTOM 3	- REVIEW OF PENETRATION SEAL TEST REPORTS AND PROGRAM		
55158	PEACH BOTTOM 3	- TS CHANGES INVOLVING MANAGEMENT REORGANIZATION		
55162	PEACH BOTTOM 3	- TECH. SPEC. CHANGES RELATED PURGE/VENT VALVES		
55175	PEACH BOTTOM 3	- JET PUMP INSTRUMENT NOZZLE CRACKS		
53274	PEACH BOTTOM 3	- TS CHANGE RELATED TO SDV AND RWCU INSTRUMENTATION		05/31/84
54618	PEACH BOTTOM 3	- EMERGENCY PREPAREDNESS EXERCISE EXEMPTION REQUEST		06/15/84
54489	PEACH BOTTOM 3	- FIRE BARRIER PENETRATIONS T/S CHANGE		06/22/84
54368	PEACH BOTTOM 3	- T/S CHANGE TO DELETE DRYWELL AIR MONITORING SYSTEM		06/29/84
52682	PEACH BOTTOM 3	- TS CHANGE FOR COOLANT LEAKAGE AND I-131 LIMITS		06/29/84
51018	PEACH BOTTOM 3	- WETWELL/DRYWELL VACUUM BREAKERS	3	06/30/84
47376	PEACH BOTTOM 3	- SUPPRESSION CHAMBER HIGH LEVEL TRIP SETTING	3	06/30/84
47378	PEACH BOTTOM 3	- SURVEILLANCE REQS FOR THE PCIS/LPCI INTERLOCK PRESSURE SWITCH	3	06/30/84
47538	PEACH BOTTOM 3	- CORE SPRAY PUMP DISCHARGE PRESSURE INTERLOCK	3	06/30/84
47540	PEACH BOTTOM 3	- REACTOR LEVEL SURVEILLANCE INSTRUMENTS	3	06/30/84
51742	PEACH BOTTOM 3	- REVIEW OF APP R, III F AND III M EXEMPTION REQUESTS	2	07/02/84
54801	PEACH BOTTOM 3	- REVIEW OF PROPOSED TECH. SPEC. REQUIREMENTS ON AIR SUPPLY SYSTEM LEAKAGE		07/31/84
46788	PEACH BOTTOM 3	- TECH SPEC CHANGE RELATED TO VERIF OF DRYWELL-TO-TORUS VAC. BREAKER CLOSURE	3	07/31/84
46790	PEACH BOTTOM 3	- ENVIRONM TECH SPEC CHANGE RELATED TO A REPORTING PERIOD	3	07/31/84
54794	PEACH BOTTOM 3	- TS CHANGE ON REQUIREMENTS FOR TESTING EMERGENCY BATTERIES		08/03/84
55123	TS CHANGE TO PERMIT OPERATION WITH INCREASED CORE FLOW			08/24/84
47380	PEACH BOTTOM 3	- FUEL OIL SYSTEMS FOR STANDBY DIESEL GENERATORS	3	08/30/84
54829	PEACH BOTTOM 3	- TECH. SPEC. CHANGES CONCERNING REACTOR WATER LEVEL SETPOINTS		08/31/84
54831	PEACH BOTTOM 3	- TECH. SPEC. CHANGE ON AUDIT OF EMERG. PLAN PROCEDURES		08/31/84
54824	PEACH BOTTOM 3	- REVISED APP. J TECH. SPECS. REQUEST		09/03/84
54557	PEACH BOTTOM 3	- TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NURFG-0737		09/30/84
54360	PEACH BOTTOM 3	- REVIEW OF STRUCTURAL STEEL		09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
54767	PEACH BOTTOM 3	- OPERABILITY OF PURGE/VENT VALVES		10/19/84
53618	PEACH BOTTOM 3	- ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		11/01/84
53701	PEACH BOTTOM 3	- ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		11/01/84
53784	PEACH BOTTOM 3	- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		11/01/84
53867	PEACH BOTTOM 3	- ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		11/01/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PEACH BOTTOM 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				<u>INITIATION</u>
54011	PEACH BOTTOM 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54094	PEACH BOTTOM 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: PILGRIM 1

PLANT LOCATION: 4 MI SE OF PLYMOUTH, MASS  
 DOCKET NUMBER: 050-00293  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: J. JOHNSON

LICENSED POWER: 1998 MW  
 DESIGN POWER: 0655 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: P. LEECH  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. MORRIS

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12185	PILGRIM 1 HIGH ENERGY LINE BREAK & CONSEQUENTIAL SYSTEM FAILURE		2	<u>COMPLETE</u> 01/31/84
42579	PILGRIM 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	04/11/84
46673	PILGRIM 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/20/84
<u>ACTIVE ACTIONS</u>				
51119	PILGRIM 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52257	PILGRIM 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52786	PILGRIM 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52867	PILGRIM 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52948	PILGRIM 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53029	PILGRIM 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
51690	PILGRIM 1 NUREG 0737 TECH SPECS			03/01/84
43804	PILGRIM - APPENDIX J EXEMPTION REQUESTS		3	04/30/84
10317	PILGRIM 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	04/30/84
03841	PILGRIM 1 - BWR FEEDWATER & CONTROL ROD DRIVE NOZZLE CRACKING		1	04/31/84
51643	PILGRIM - SUPPLEMENTAL REQUESTS FOR EXEMPTION REGARDING 10CFR 50.48 AND APPENDIX R			05/15/84
03123	PILGRIM 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/15/84
07948	PILGRIM 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	06/30/84
10825	PILGRIM 1 - BWR SINGLE LOOP OPERATION		3	06/30/84
42477	PILGRIM 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-2')		1	08/31/84
42885	PILGRIM 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	09/30/84
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
48191	PILGRIM 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>COMPLETE</u> 12/29/83
45852	PILGRIM 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/29/83
47625	PILGRIM 1 - NUREG 0737 ITEM II F1.4 CONTAINMENT PRESSURE INSTRUMENT		1	03/23/84
47696	PILGRIM 1 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	03/23/84
47767	PILGRIM 1 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	03/23/84
47858	PILGRIM 1 - NUREG - 0737 II. 3.22B, RCIC SUCTION MODIFICATIONS		1	06/26/84
<u>ACTIVE ACTIONS</u>				
51269	PILGRIM 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			<u>TARGET</u>
46330	PILGRIM 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	12/30/83
44607	PILGRIM 1 - RV AND SV TESTING		1	03/31/84
44328	PILGRIM 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	04/30/84
44876	PILGRIM 1 - NUREG-0737 II.E.4.2, CONTAINMENT ISOLATION DEPENDABILITY		1	06/30/84
45159	PILGRIM 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
45603	PILGRIM 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	06/30/84
44258	PILGRIM 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/30/84
47313	PILGRIM 1 - CORRECT PERFORMANCE OF OPERATING ACTIVITIES (API.C.6)		1	06/30/84
48271	PILGRIM - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	06/30/84
45971	PILGRIM 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## PILGRIM 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
46042	PILGRIM 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	<u>TARGET</u> 09/30/84
46114	PILGRIM 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44466	PILGRIM 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	09/30/84
51189	PILGRIM 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			12/30/84
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
49128	PILGRIM - APPENDIX R FIRE PROTECTION ALTERNATIVE SHUTDOWN REVIEW			<u>COMPLETE</u> 11/02/83
49909	PILGRIM - PROPOSED T.S. CHANGE TO ADD ACTION STATEMENT RE: JET PUMP FLOW MISMATCH			11/09/83
51888	PILGRIM - REQUEST TO EXPAND THE OPERATING REGION OF PILGRIM'S POWER/FLOW MAP			11/22/83
51560	PILGRIM - INCREASED SAFETY RELIEF VALVE SIMMER MARGIN TECH SPEC			12/29/83
52471	PILGRIM - SHIFT MANNING EXTENSION			12/30/83
48573	PILGRIM 1 - ASSESSMENT OF MANAGEMENT IMPROVEMENT PLAN			03/22/84
53423	PILGRIM - ALARA PROGRAM FOR PIPE REPLACEMENT			04/03/84
52688	PILGRIM - TS AMENDMENT CONCERNING APPROVAL OF PROCEDURES			04/24/84
52689	PILGRIM-T.S. AMENDMENT REFLECTING BEC ORGANIZATIONAL CHNGS			04/24/84
<u>ACTIVE ACTIONS</u>				
52438	PILGRIM - LONG TERM SCHEDULING PROGRAM			<u>TARGET</u> 03/31/84
52690	PILGRIM - TS CHANGE RE CRD WEEKLY SURVEILLANCE			05/15/84
52033	PILGRIM - REQUEST FRD SCHEDULAR EXTENSION FOR COMPLETION OF CLASS II & III HYDROTESTS			05/30/84
53416	PILGRIM - APPENDIX R EXEMPTION REQUESTS #11-15			05/31/84
54609	PILGRIM - TS CHANGE REMOVING DETAILS OF SECTION XI ISI PROGRAM			06/01/84
54691	PILGRIM - OPERABILITY OF NEW 8 INCH ISOLATION VALVES			06/15/84
48767	PILGRIM - TS CHANGE REQUEST TO ALLOW CONTROLLED EFFLUENT RELEASE FROM OTHER THAN RADWASTE FACILITY			06/15/84
51381	PILGRIM 1 - INSERVICE INSPECTION - 2ND TEN YEAR INTERVAL			06/30/84
54610	PILGRIM - TS CHANGES RELATIVE TO FIRE PROTECTION			06/30/84
54608	PILGRIM - TS CHANGE TO ALLOW INCREASED CORE FLOW			07/06/84
53217	PILGRIM-RELOAD 6 FOR CYCLE 7 OPERATION			07/06/84
53218	PILGRIM - RECIRCULATION SYSTEM INSPECTION AND REPAIR			08/05/84
52032	PILGRIM - INSERVICE TESTING PROGRAM AND REQUESTS FOR RELIEF - 2ND TEN YEAR INTERVAL			08/30/84
54558	PILGRIM 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53619	PILGRIM 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53702	PILGRIM 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53785	PILGRIM 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53868	PILGRIM 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54012	PILGRIM 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54095	PILGRIM 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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FACILITY: POINT BEACH 1

PLANT LOCATION: 15 MI N OF MANITOWOC, WISC  
DOCKET NUMBER: 850-00266  
ARCH/ENGINEER: BECH  
IE INSPECTOR: R. HAGUELICENSED POWER: 1518 MW  
DESIGN POWER: 0497 MWE  
NSSS VENDOR: WEST  
PROJECT MANAGER: T. COLBURN  
BRANCH CHIEF: J. MILLER  
LIC. ASSISTANT: P. KREUTZER

<u>IAC #</u>	<u>IAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47147	POINT BEACH 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	11/08/83
49748	POINT BEACH 1 - NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)*		1	12/28/83
12271	POINT BEACH 1 - ENVIRON. QUAL. OF CONTROL SYSTEMS POINT BEACH 1		3	01/31/84
07710	POINT BEACH 1 - ASYMMETRIC LOCA LOADS (GENERIC) (USI A-02)		2	02/01/84
07727	POINT BEACH 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL POWER PLANTS (NUREG 0612)		2	03/27/84
<u>ACTIVE ACTIONS</u>				
52787	POINT BEACH 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		1	TARGET
52868	POINT BEACH 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		1	
52949	POINT BEACH 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		1	
53030	POINT BEACH 1-ITEM 3.1.3-POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS		1	
53087	POINT BEACH 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		1	
53140	POINT BEACH 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDRG		1	
53186	POINT BEACH 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	05/30/84
42498	POINT BEACH 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
43247	POINT BEACH 1 - APPENDIX R FIRE PROTECTION REVIEW		1	07/31/84
07712	POINT BEACH 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	07/31/84
06828	POINT BEACH 1 - UPI ECCS INJECTION		3	07/31/84
51120	POINT BEACH 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	10/30/84
43669	POINT BEACH 1 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	10/31/84
52258	POINT BEACH 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MP#C-10)		3	07/30/85
<u>IMI ACTIONS</u>				
<u>IAC # IAC DESCRIPTION</u>				
<u>COMPLETED ACTIONS</u>				
47965	POINT BEACH 1 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	11/03/83
44487	POINT BEACH 1 - NUREG-0737 II.B.3.2. POST ACCIDENT SAMPLING MODIFICATIONS		1	11/03/83
47811	POINT BEACH 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/24/84
66897	POINT BEACH 1 - II K2.13 THERMAL MECHANICAL REPORT		1	06/06/84
44259	POINT BEACH 1 - NUREG-0737 I.C.1.2.A. INADEQUATE CORE COOLING GUIDELINES		1	06/25/84
<u>ACTIVE ACTIONS</u>				
45853	POINT BEACH 1 - NUREG-0737 II K.3.30. SB LOCA OUTLINE		1	TARGET
51270	POINT BEACH 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	07/31/84
46331	POINT BEACH 1 - NUREG-0737 III.A.2.2. METEOROLOGICAL DATA UPGRADE		1	08/01/84
44608	POINT BEACH 1 - RV AND SV TESTING		1	08/30/84
45972	POINT BEACH 1 - NUREG-0737 III.A.1.2. TECHNICAL SUPPORT CENTER		1	09/30/84
46043	POINT BEACH 1 - NUREG-0737 III.A.1.2. OPERATIONAL SUPPORT CENTER		1	09/30/84
46115	POINT BEACH 1 - NUREG-0737 III.A.1.2. EMERGENCY OPERATIONS FACILITY		1	09/30/84
44329	POINT BEACH 1 - NUREG-0737 I.C.1.3A. ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51190	POINT BEACH 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	04/30/85
48192	POINT BEACH 1 - NUREG 0737 II K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/30/85
45160	POINT BEACH 1 - NUREG-0737 II.F.2.3. INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/85

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

POINT BEACH 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
43929	POINT BEACH 1 - SPENT FUEL POOL POISON SURVEILLANCE AND PORV OPERABILITY		3	<u>COMPLETE</u> 10/05/83
51597	POINT BEACH 1 - ADMINISTRATIVE TECH SPEC CHANGES		3	10/06/83
40375	POINT BEACH 1 - LOSS OF VOLTAGE RELAY TIME DELAY		1	10/31/83
52693	POINT BEACH 1-RESPONSE TO SEN. GLENN RE: TRANSFER OF SPENT FUEL TO POINT BEACH - GREEN TICKET		1	11/28/83
53271	POINT BEACH 1 - AUXILIARY FEEDWATE AUTO INITIATION T.S		2	12/29/83
53345	POINT BEACH 1 - RCS VENTS SCHEDULAR EXEMPTION			12/30/83
52692	POINT BEACH 1 - RETURN TO POWER OPERATION AT EITHER 2200 PSIA		2	12/30/83
53212	POINT BEACH 1 - EQUIPMENT QUALIFICATION DEADLINE EXTENSION		1	01/03/84
52663	POINT BEACH 1 - ADMINISTRATIVE CHANGES TO LOC'S		2	02/02/84
52502	POINT BEACH 1 - SHIFT STAFFING EXEMPTION REQUEST		1	03/26/84
48762	POINT BEACH 1 - 2ND 10-YEAR INTERVAL ISI RELIEF		3	03/29/84
53400	POINT BEACH - SURVEILLANCE TESTS FOR INTEGRITY OF OF RECOVERY SYSTEMS		1	04/06/84
54509	POINT BEACH 1 - DEBRIS IN CRDM-STUCK CONTROL ROD			04/06/84
47494	POINT BEACH 1 - INSTRUMENT POWER SUPPLY TECH SPEC		2	04/30/84
54356	POINT BEACH 1 - CRDM SUPPORT PIN CRACKING		1	05/08/84
<u>ACTIVE ACTIONS</u>				
52365	POINT BEACH 1 - CONTAINMENT TYPE A TESTING (APPENDIX J)		2	<u>TARGET</u> 06/15/84
51042	POINT BEACH 1 - TIA, REGION III, AUX FW SYSTEM OPERABILITY		1	06/28/84
54446	POINT BEACH - REACTOR VESSEL HOT LEG NOZZLE SLAG INCLUSIONS		1	06/30/84
54200	POINT BEACH 1 - REPORTING REQUIREMENTS T.S. GL 83-43		2	06/30/84
54596	POINT BEACH 1 - NUREG-0737 ORDER MODIFICATION		1	06/30/84
54989	POINT BEACH 1 - CONTAINMENT TENDON SURVEILLANCE		1	07/31/84
54253	POINT BEACH 1 - REPEAL OF CONFIRMATORY ORDERS		3	07/31/84
54591	POINT BEACH 1 - HEAVY LOADS HANDLING T.S.		2	08/30/84
54592	POINT BEACH 1 - HYDRAULIC SNUBBER T.S.		2	08/30/84
54595	POINT BEACH 1 - ADMIN T.S.		1	08/30/84
51301	POINT BEACH 1 - INSERVICE TESTING		1	08/30/84
51550	POINT BEACH 1 - FIRE PROTECTION TECH SPECS		2	08/31/84
55292	POINT BEACH 1 - APPENDIX R EXEMPTION			08/31/84
54559	POINT BEACH 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		2	09/30/84
54406	POINT BEACH 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		2	09/30/84
49959	POINT BEACH 1 - WESTINGHOUSE OPTIMIZED FUEL DESIGN		3	09/30/84
49656	POINT BEACH 1 - REACTOR COOLANT PUMP TRIP		3	02/01/85
<u>ANTICIPATED ACTIONS</u>				
53620	POINT BEACH 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53703	POINT BEACH 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53786	POINT BEACH 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53869	POINT BEACH 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53927	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54013	POINT BEACH 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54096	POINT BEACH 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: POINT BEACH 2

PLANT LOCATION: 15 MI N OF MANITOWOC, WISC  
 DOCKET NUMBER: 050-00301  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: K. HAGUE

LICENSED POWER: 1518 MWT  
 DESIGN POWER: 0497 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: T. COLBURN  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KPFUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47148	POINT BEACH 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	COMPLETE 11/08/83
49749	POINT BEACH 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"		1	12/28/83
12272	POINT BEACH 2 - ENVIRON. QUAL. OF CONTROL SYSTEMS POINT BEACH 2		3	01/31/84
08158	POINT BEACH 2 - GENERIC - ASSYMETRIC LOCA LOADS (USI A-02)		2	02/01/84
08074	POINT BEACH 2-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	03/27/84
<u>ACTIVE ACTIONS</u>				
53141	POINT BEACH 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		1	TARGET
53187	POINT BEACH 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	
52788	POINT BEACH 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		1	
52869	POINT BEACH 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		1	
52950	POINT BEACH 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		1	
53031	POINT BEACH 2-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS		1	
53088	POINT BEACH 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		1	
48246	POINT BEACH 2 - APPENDIX R FIRE PROTECTION REVIEW		1	06/30/84
42499	POINT BEACH 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
08349	POINT BEACH 2 - UPI ECCS INJECTION		3	07/31/84
08150	POINT BEACH 2 - GENERIC - APPENDIX I		3	07/31/84
43670	POINT BEACH 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	10/31/84
51121	POINT BEACH 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	10/31/84
52259	POINT BEACH 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPAWC-10)		3	07/30/85
<u>TAC # TAC DESCRIPTION MULTI-PLANT PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47966	POINT BEACH 2 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	COMPLETE 11/03/83
44468	POINT BEACH 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/03/83
47812	POINT BEACH 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/24/84
46898	POINT BEACH 2 - II K2.13 THERMAL MECHANICAL REPORT		1	06/07/84
44260	POINT BEACH 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/25/84
<u>ACTIVE ACTIONS</u>				
51271	POINT BEACH 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	TARGET 07/30/84
45854	POINT BEACH 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	07/31/84
46332	POINT BEACH 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	08/01/84
44609	POINT BEACH 2 - RV AND SV TESTING		1	08/31/84
45973	POINT BEACH 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46044	POINT BEACH 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46116	POINT BEACH 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44330	POINT BEACH 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51191	POINT BEACH 2 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	04/30/85
43161	POINT BEACH 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/85
48193	POINT BEACH 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	06/30/85

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

POINT BEACH 2

IAC #	IAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
48930	POINT BEACH 2 - SPENT FUEL POOL POISON SURVEILLANCE AND PORV OPERABILITY		3	COMPLETE
51596	POINT BEACH 2 - ADMINISTRATIVE TECH SPEC CHANGES		3	10/05/83
48376	POINT BEACH 2 - LOSS OF VOLTAGE RELAY TIME DELAY		1	10/31/83
52694	POINT BEACH 2 - RESPONSE TO SEN GLENN RE: TRANSFER OF SPENT FUEL TO POINT BEACH - GREEN TICKET		1	11/28/83
52421	POINT BEACH 2 - COMPONENT COOLING WATER HEAT EXCHANGER T.S.		2	12/12/83
53272	POINT BEACH 2 - AUXILIARY FEEDWATER AUTO INITIATION T.S.		2	12/29/83
53345	POINT BEACH 2 - RCS VENTS SCHEDULAR EXEMPTION		1	12/30/83
53213	POINT BEACH 2 - EQUIPMENT QUALIFICATION DEADLINE EXTENSION		1	01/03/84
52664	POINT BEACH 2 - ADMINISTRATIVE CHANGES TO LCD'S		2	02/02/84
52503	POINT BEACH 2 - SHIFT STAFFING EXEMPTION REQUEST		1	03/26/84
49549	POINT BEACH 2 - 2ND 18-MONTH INTERVAL ISI RELIEF		3	03/29/84
53401	POINT BEACH 2 - SURVEILLANCE TESTS FOR INTEGRITY OF RECOVERY SYSTEMS		1	04/06/84
47495	POINT BEACH 2 - INSTRUMENT POWER SUPPLY T.S.		2	04/30/84
<u>ACTIVE ACTIONS</u>				
52655	POINT BEACH 2 - CONTAINMENT TYPE A TESTING (APPENDIX J)		2	TARGET
51013	POINT BEACH 2 - T1A, REGION III, AUX FW SYSTEM OPERABILITY		1	06/15/84
54597	POINT BEACH 2 - NUREG-0737 ORDER MODIFICATION		1	06/28/84
54201	POINT BEACH 2 - REPORTING REQUIREMENTS T.S. GL 83-43		2	06/30/84
54998	POINT BEACH 2 - CONTAINMENT VENDOR SURVEILLANCE		1	07/31/84
54598	POINT BEACH 2 - HEAVY LOADS HANDLING T.S.		2	08/30/84
54593	POINT BEACH 2 - HYDRAULIC SNUBBER T.S.		2	08/30/84
54594	POINT BEACH 2 - ADMIN T.S.		2	08/30/84
51502	POINT BEACH 2 - INSERVICE TESTING		1	08/30/84
51551	POINT BEACH 2 - FIRE PROTECTION TECH SPECS		2	08/31/84
55293	POINT BEACH 2 - APPENDIX R EXEMPTION REQUEST		2	08/31/84
54407	POINT BEACH 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		2	09/30/84
54560	POINT BEACH 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		2	09/30/84
49970	POINT BEACH 2 - WESTINGHOUSE OPTIMIZED FUEL DESIGN		3	09/30/84
49657	POINT BEACH 2 - REACTOR COOLANT PUMP TRIP		3	02/01/85
<u>ANTICIPATED ACTIONS</u>				
53624	POINT BEACH 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION
53704	POINT BEACH 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53787	POINT BEACH 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53870	POINT BEACH 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53928	ITEMS 4.2.3 & 4.2.4-PR .EMENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54014	POINT BEACH 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54097	POINT BEACH 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

TECHNICAL ASSIGNMENT CONTROL SYSTEM  
CONDENSED MANAGEMENT REPORT

PLANT LOCATION: 28 MI SE OF MINNEAPOLIS, MINN  
 DOCKET NUMBER: 850-00282  
 ARCH/ENGINEER: FPA  
 IE INSPECTOR: C. FEHRBAND

FACILITY: PRAIRIE ISLAND 1

LICENSED POWER: 1650 MW  
 DESIGN POWER: 0530 PWE  
 NSSS VENDOR: WEST  
 PROJECT MANAGER: D. DIANNI  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

IAC #	IAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
4599	PRAIRIE ISLAND UNIT NO 1 - FIRE PROTECTION EXEMPTION			COMPLETE 01/09/84
5289	PRAIRIE ISLAND 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			06/01/84
5287	PRAIRIE ISLAND 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			06/01/84
5295	PRAIRIE ISLAND 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			06/01/84
5303	PRAIRIE IS 1 - ITEM 3.1.3 - POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			06/01/84
5309	PRAIRIE ISLAND 1 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			06/01/84
5308	PRAIRIE ISLAND 1 - ITEM 4.3 - AUTOAM,IC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			06/01/84
<u>ACTIVE ACTIONS</u>				
5220	PRAIRIE ISLAND 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPARC-10)			TARGET
5122	PRAIRIE ISLAND 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (EG 1.97)		2	05/20/84
0873	PRAIRIE ISLAND 1 - REVIEW OF ASYMMETRIC LOCAL LOADS SUBMITTAL		1	07/15/84
4250	PRAIRIE ISLAND 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (81-CL1-21)			
5314	PRAIRIE ISLAND 1 - ITEMS 4.2.1 & 4.2.2- PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDS			
0835	PRAIRIE ISLAND 1 - WESTINGHOUSE UPPER PLENUM ECCS INJECTION - LONG RANGE		3	09/30/84
<u>IAC # IAC DESCRIPTION</u>				
<u>COMPLETED ACTIONS</u>				
4797	PRAIRIE ISLAND 1 - PLANT SHIELDING MODIFICATIONS (II.B.2-2)		1	COMPLETE 10/12/83
4469	PRAIRIE ISLAND 1 - NUREG-0737 II.B.3.2. POST ACCIDENT SAMPLING MODIFICATIONS		1	11/23/83
4783	PRAIRIE ISLAND 1 - NUREG-0737 ITEM II K2.17. POTENTIAL FOR VOIDING IN RCS		1	12/30/83
4639	PRAIRIE ISLAND 1 - II K2.13 THERMAL MECHANICAL REPORT		1	06/06/84
4426	PRAIRIE ISLAND 1 - NUREG-0737 I.C.1.2.A. INADEQUATE CORE COOLING GUIDELINES		2	06/07/84
<u>ACTIVE ACTIONS</u>				
4460	PRAIRIE ISLAND 1 - RV AND SV TESTING		2	TARGET
4633	PRAIRIE ISLAND 1 - NUREG-0737 III.A.2.2. METEOROLOGICAL DATA UPGRADE		1	
5192	PRAIRIE ISLAND 1 - I.B.1 CONTROL ROOM DESIGN REVIEW			
4512	PRAIRIE ISLAND 1 - NUREG-0737 II.F.2.3. INADEQUATE CORE COOLING INSTRUMENTATION		2	08/05/84
4597	PRAIRIE ISLAND 1 - NUREG-0737 III.A.1.2. TECHNICAL SUPPORT CENTER		1	09/30/84
4604	PRAIRIE ISLAND 1 - NUREG-0737 III.A.1.2. OPERATIONAL SUPPORT CENTER		1	09/30/84
4617	PRAIRIE ISLAND 1 - NUREG-0737 III.A.1.2. EMERGENCY OPERATIONS FACILITY		1	09/30/84
4433	PRAIRIE ISLAND 1 - NUREG-0737 I.C.1.3A. ABNORMAL TRANSIENT OPERATOR GUIDELINES		2	11/30/84
4585	PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.30. SB LOCA OUTLINE		1	09/30/85
4819	PRAIRIE ISLAND 1 - NUREG-0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85
5127	PRAIRIE ISLAND 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			10/30/86
<u>-IAC # IAC DESCRIPTION</u>				
<u>COMPLETED ACTIONS</u>				
				COMPLETE

PLANT SPECIFIC

CRITICAL DATE

COMPLETED ACTIONS

COMPLETE

DATA AS OF - 06/10/84

CONDENSED MANAGEMENT REPORT

(CONTINUATION)

PRAIRIE ISLAND 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
	COMPLETED ACTIONS (CONTINUATION)			COMPLETE
51908	PRAIRIE ISLAND 1 - ISI RELIEF REQUEST RCP CASING WELDS			10/12/83
51948	PRAIRIE ISLAND UNIT 1 FQ LIMIT CHANGE DUE TO REVISED LOCA ANA			12/28/83
52405	PRAIRIE ISLAND UNIT 1 ISI/IST 10YR EXTENSION			12/28/83
51485	PI-1 ISI 7 OPEN ISSUES			01/31/84
51931	PI-1 TS CHANGE REQUEST ON 0737 ITEMS & MISC CHANGES			02/21/84
54408	PRAIRIE ISLAND 1 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			03/27/84
53622	PRAIRIE ISLAND 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			06/01/84
53705	PRAIRIE ISLAND 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			06/01/84
53788	PRAIRIE IS 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			06/01/84
53871	PRAIRIE ISLAND 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			06/01/84
54015	PRAIRIE ISLAND 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			06/01/84
54098	PRAIRIE ISLAND 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			06/01/84
	ACTIVE ACTIONS			TARGET
55337	PI 1 - STEAM GENERATOR LEAK -015 GPM SE			06/21/84
53559	PRAIRIE ISLAND 1 FIRE PROTECTION EXEMPTION (2) BASED ON GL83-33			06/30/84
53456	PRAIRIE ISLAND UNIT 1 ISI/IST; 2ND 10 YEAR PROGRAM			07/15/84
51458	PRAIRIE ISLAND UNIT 1 TS CHANGE REQUEST RESPONDING TO GL83-37			09/01/84
47663	PRAIRIE ISLAND 1 - REACTOR COOLANT PUMP TRIP			02/01/85
	ANTICIPATED ACTIONS			INITIATION
53929	- ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84



DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: PRAIRIE ISLAND 2

PLANT LOCATION: 28 MI SE OF MINNEAPOLIS, MINN  
 DOCKET NUMBER: 050-00306  
 ARCH/ENGINEER: FPI  
 IE INSPECTOR: C. FEIERABEND

LICENSED POWER: 1650 MWT  
 DESIGN POWER: 0530 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: D. DIIANNI  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49950	PRAIRIE ISLAND UNIT NO 2 - FIRE PROTECTION 4 EXEMPTIONS			<u>COMPLETE</u> 01/09/84
52790	PRAIRIE ISLAND 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			06/01/84
52871	PRAIRIE ISLAND 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			06/01/84
52952	PRAIRIE ISLAND 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			06/01/84
53033	PRAIRIE IS 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			06/01/84
53090	PRAIRIE ISLAND 2 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			06/01/84
53189	PRAIRIE ISLAND 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			06/01/84
<u>ACTIVE ACTIONS</u>				
52261	PRAIRIE ISLAND 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
51123	PRAIRIE ISLAND 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
08732	PRAIRIE ISLAND 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	05/20/84
42501	PRAIRIE ISLAND 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/15/84
53143	PRAIRIE ISLAND 2 - ITEMS 4.2.1 & 4.2.2- PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			07/15/84
08471	PRAIRIE ISLAND 2 - WESTINGHOUSE UPPER PLENUM ECCS INJECTION - LONG RANGE		3	09/30/84
<u>TAC # TAC DESCRIPTION IMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47968	PRAIRIE ISLAND 2 - PLANT SHIELDING MODIFICATIONS		1	<u>COMPLETE</u> 10/12/83
44470	PRAIRIE ISLAND 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/23/83
47814	PRAIRIE ISLAND 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	12/30/83
46990	PRAIRIE ISLAND 2 - II K2.13 THERMAL MECHANICAL REPORT		1	06/06/84
44262	PRAIRIE ISLAND 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
44611	PRAIRIE ISLAND 2 - RV AND SV TESTING		1	<u>TARGET</u>
46334	PRAIRIE ISLAND 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	
51193	PRAIRIE ISLAND 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			
45975	PRAIRIE ISLAND 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46046	PRAIRIE ISLAND 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46118	PRAIRIE ISLAND 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44332	PRAIRIE ISLAND 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
45163	PRAIRIE ISLAND 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/05/85
45556	PRAIRIE ISLAND 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/85
48195	PRAIRIE ISLAND 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/85
51273	PRAIRIE ISLAND 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			01/01/87
<u>TAC # TAC DESCRIPTION PLANT SPECIFIC PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
<u>COMPLETE</u>				

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## PRAIRIE ISLAND 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS (CONTINUATION)</u>				<u>COMPLETE</u>
51905	PRAIRIE ISLAND UNIT 2 - ISI RELIEF REQUEST RCP CASING WELDS			10/12/83
51941	PRAIRIE IS UNIT 2 FQ LIMIT CHANGE DUE REVISED LOCA ANA			12/28/83
51484	PI-1 IST 7 OPEN ISSUES			01/31/84
51992	PI-2 TS CHANGE REQUEST ON 0737 ITEMS & MISCHANGE			02/21/84
54409	PRAIRIE ISLAND 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			03/27/84
53623	PRAIRIE ISLAND 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			06/01/84
53706	PRAIRIE ISLAND 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			06/01/84
53789	PRAIRIE IS 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			06/01/84
53872	PRAIRIE ISLAND 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			06/01/84
53930	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			06/01/84
54016	PRAIRIE ISLAND 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			06/01/84
54099	PRAIRIE ISLAND 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			06/01/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
53558	PRAIRIE ISLAND 2 FIRE PROTECTION EXEMPTION (2) BASED ON GL83-33			06/30/84
53457	PRAIRIE ISLAND UNIT NO 2 ISI/IST; 2ND 10 YR PROGRAM			07/15/84
53459	PRAIRIE ISLAND UNIT 2 TS CHANGE REQUEST RESPONDING TO GL83-37			09/01/84
49664	PRAIRIE ISLAND 2 - REACTOR COOLANT PUMP TRIP			02/01/85

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## CONDENSED MANAGEMENT REPORT

FACILITY: QUAD CITIES 1

PLANT LOCATION: 20 MI NE OF MOLINE, ILL  
 DOCKET NUMBER: 050-00254  
 ARCH/ENGINEER: S&L  
 IE INSPECTOR: N. CHRISSTIMOS

LICENSED POWER: 2511 MWT  
 DESIGN POWER: 0789 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. BEVAN  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08933	QUAD CITIES 1 - RPS POWER SUPPLY		1	<u>COMPLETE</u> 12/30/83
12310	QUAD CITIES 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE QUAD CITIES 1		3	01/31/84
08124	QUAD CITIES 1 - APP 1 TECH SPEC IMPLEMENTATION REVIEW		3	05/31/84
46669	QUAD CITIES 1 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/15/84
<u>ACTIVE ACTIONS</u>				
52262	QUAD CITIES 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52791	QUAD CITIES 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52872	QUAD CITIES 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52953	QUAD CITIES 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53034	QUAD CITIES 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
42478	QUAD CITIES 1 - ENVIRONMENTAL QUALIFICATIONS OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/30/83
42580	QUAD CITIES 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	07/30/83
42886	QUAD CITIES 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/31/83
51124	QUAD CITIES 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (PG 1.97)			12/01/83
51691	QUAD CITIES 1 NUREG 0737 TECH SPECS			03/01/84
07947	QUAD CITIES 1 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	06/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44400	QUAD CITIES 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 11/17/83
45857	QUAD CITIES 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/28/83
47859	QUAD CITIES 1 - NUREG - 0737 II.K.3.22B, RCIC SUCCTION MODIFICATIONS		1	12/29/83
44263	QUAD CITIES 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
44613	QUAD CITIES 1 - RV AND SV TESTING		1	06/29/84
<u>ACTIVE ACTIONS</u>				
46335	QUAD CITIES 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u> 06/01/83
45604	QUAD CITIES 1 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/25/83
45700	QUAD CITIES 1 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	08/30/83
45164	QUAD CITIES 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/83
48196	QUAD CITIES 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/83
48272	QUAD CITIES 1 - NUREG 0737 II.K.3.28, VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES		1	09/30/83
44333	QUAD CITIES 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83
51194	QUAD CITIES 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		2	06/30/84
51274	QUAD CITIES 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			06/30/84
45976	QUAD CITIES 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46047	QUAD CITIES 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52362	QUAD CITIES 1 - PROPOSED T.S. RE RPS POWER MONITORING SYSTEM			<u>COMPLETE</u> 02/14/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

QUAD CITIES 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
52318	QUAD CITIES 1 - T.S. AMENDMENT RE S/R VALVE POSITION INDICATION			02/24/84
53364	QUAD CITIES 1 - REQUESTED EXEMPTION FROM REQMTS OF APPX FOR ILRT			04/10/84
54304	QUAD CITIES 1 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
52145	QUAD CITIES UNIT 1 - IMPLEMENTATION OF FW NOZZLE AND CRD-RL NOZZLE WORK			04/16/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
55008	QUAD CITIES UNIT 1 - 125V DC BATTERY OPERABILITY		2	07/29/83
51520	QUAD CITIES 1 - ECONOMIC GENERATION CONTROL		3	07/30/83
49939	QUAD CITIES 1 - TIA-TS 48 VOLT D.C.			08/31/83
52080	QUAD CITIES 1 - T/A ON INSERVICE INSPECTION PROGRAM REQUIREMENTS			09/30/83
48301	QUAD CITIES 1 - PROPOSED TS CHANGES RE APP. H AND G METAL SURVEILLANCE & MIN TEMP REQ		3	09/30/83
51746	QUAD CITIES UNIT 1 CRD SCRAM TIME TESTING - TS CHANGE			09/30/83
52304	QUAD CITIES 1 - INSERVICE TESTING PROGRAM, PUMPS AND VALVES, 2ND DECADE			11/15/83
51744	QUAD CITIES UNIT 1 CHANGE TO TS ON CTMT VAC BKRS AND DG TEST RQMTS			11/30/83
52302	QUAD CITIES - 1 INSERVICE INSPECTION PROGRAM - 2ND DECADE			03/01/84
49196	QUAD CITIES 1 - CONTAINMENT OXYGEN CONTENT TS CHANGE		3	04/01/84
54361	QUAD CITIES-1 TECH SPEC CHANGES IN MAPLGR CURVES			06/01/84
54616	QUAD CITIES 1 - REQUEST TO RESCIND ORDER RELATED TO IEB 80-17-SDV MODIFICATIONS			06/01/84
55083	QUAD CITIES 1 - SCHEDULAR EXTENSION FOR E Q REQMTS			07/02/84
55277	QUAD CITIES 1 - CALIBRATION AND FUNCTIONAL TEST FREQUENCY OF INSTRUMENTS			07/18/84
55148	QUAD CITIES UNIT 1: HYDROGEN RECOMBINER, G.L.84-9 RESPONSE			08/01/84
54362	QUAD CITIES-1 TECH SPEC CHANGES FOR SCRAM DISCHARGE VOLUME MODS			08/01/84
54561	QUAD CITIES 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53624	QUAD CITIES 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53707	QUAD CITIES 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53790	QUAD CITIES 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53873	QUAD CITIES 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54017	QUAD CITIES 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54100	QUAD CITIES 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: QUAD CITIES 2

PLANT LOCATION: 20 MI NE OF MOLINE, ILL  
 DOCKET NUMBER: 050-00265  
 ARCH/ENGINEER: S&L  
 IE INSPECTOR: N. CHRISOYIMOS

LICENSED POWER: 2511 MWT  
 DESIGN POWER: 0789 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. BEVAN  
 BRANCH CHIEF: D. VASSALLO  
 LIC. ASSISTANT: S. NORRIS

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
08934	QUAD CITIES 2 - RPS POWER SUPPLY		1	<u>COMPLETE</u> 12/30/83
12311	QUAD CITIES 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE QUAD CITIES 2		3	01/31/84
08125	QUAD CITIES 2 - APP I TECH SPEC IMPLEMENTATION REVIEW		3	05/31/84
46670	QUAD CITIES 2 - IMPLEMENTATION OF NUREG-0313, REV. 1		2	06/15/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52263	QUAD CITIES 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52792	QUAD CITIES 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52873	QUAD CITIES 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52954	QUAD CITIES 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53035	QUAD CITIES 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
42479	QUAD CITIES 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/30/83
42581	QUAD CITIES 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	07/30/83
42887	QUAD CITIES 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/31/83
43725	QUAD CITIES 2 - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BWR SCRAM SYSTEM		2	10/31/83
51125	QUAD CITIES 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			12/01/83
51692	QUAD CITIES 2 NUREG 0737 TECH SPECS			03/01/84
07949	QUAD CITIES 2 - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION		1	06/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
44401	QUAD CITIES 2 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 11/17/83
47360	QUAD CITIES 2 - NUREG - 0737 II.K.3.22B, RCIC SUCTION MODIFICATIONS		1	12/28/83
45858	QUAD CITIES 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/23/83
44264	QUAD CITIES 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
44614	QUAD CITIES 2 - RV AND SV TESTING		1	06/29/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
44472	QUAD CITIES 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>TARGET</u> 09/30/82
46336	QUAD CITIES 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	05/01/83
45605	QUAD CITIES 2 - NUREG-0737 II.K.3.16, CHALLENGES AND FAILURES OF RELIEF VALVES		1	08/25/83
45701	QUAD CITIES 2 - NUREG-0737 II.K.3.18, ADS ACTUATION STUDY		1	08/30/83
45165	QUAD CITIES 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/83
48197	QUAD CITIES 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/83
44334	QUAD CITIES 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/83
51195	QUAD CITIES 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			06/30/84
51275	QUAD CITIES 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			06/30/84
45977	QUAD CITIES 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46048	QUAD CITIES 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
<u>COMPLETE</u>				

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## QUAD CITIES 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
52025	QUAD CITIES 2 - PROPOSED OPERATING LIMIT MCPR REDUCTION			01/24/84
52315	QUAD CITIES 2 - PROPOSED CHANGES IN SAFETY-RELIEF VALVE SETPOINTS			01/24/84
52363	QUAD CITIES 2 - PROPOSED T.S. RE RPS POWER MONITORING SYSTEM			02/14/84
52319	QUAD CITIES 2 - T.S. AMENDMENT RE S/R VALVE POSITION INDICATION			02/24/84
52094	QUAD CITIES 2 - LARGE PIPE CRACK			02/28/84
51747	QUAD CITIES UNIT 2 CRD SCRAM TIME TESTING - TS CHANGE			04/12/84
54305	QUAD CITIES 2 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
52146	QUAD CITIES UNIT 2 - IMPLEMENTATION OF FW NOZZLE AND CRD-RL NOZZLE WORK			04/16/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
55009	QUAD CITIES UNIT 2 - 125V DC BATTERY OPERABILITY			
51521	QUAD CITIES 2 - ECONOMIC GENERATION CONTROL		2	07/29/83
49940	QUAD CITIES 2 - TIA-TS 48 VOLT D.C.		3	07/30/83
52081	QUAD CITIES 2 - T/A ON INSERVICE INSPECTION PROGRAM REQUIREMENTS			08/31/83
48302	QUAD CITIES 2 - PROPOSED TS CHANGES RE APP. H & G, METAL SURVEILLANCE AND MIN. TEMP. REQ		2	09/30/83
51745	QUAD CITIES UNIT 2 CHANGE TO TS ON CTM, VAC BKRS AND DG TEST RQMTS			12/30/83
52303	QUAD CITIES 2- INSERVICE INSPECTION PROGRAM - 2ND DECADE			03/01/84
49197	QUAD CITIES 2 - CONTAINMENT OXYGEN CONTENT TS CHANGE		3	04/01/84
52305	QUAD CITIES 2 - INSERVICE TESTING PROGRAM, PUMPS AND VALVES, 2ND DECADE			05/01/84
55149	QUAD CITIES UNIT 2: HYDROGEN RECOMBINER, G.L. 84-9 RESPONSE			08/01/84
55019	QUAD CITIES 2 - LICENSEE PROPOSED INTEGRATED LEAK RATE TEST SCHEDULE			09/28/84
54562	QUAD CITIES 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53625	QUAD CITIES 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53708	QUAD CITIES 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53791	QUAD CITIES 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53874	QUAD CITIES 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54018	QUAD CITIES - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54101	QUAD CITIES 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: RANCHO SECO 1

PLANT LOCATION: 25 MI SE OF SACRAMENTO, CA  
 DOCKET NUMBER: 050-00312  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: H. CANTER

LICENSED POWER: 2772 MWT  
 DESIGN POWER: 0918 MWE  
 NSSS VENDOR: B&W

PROJECT MANAGER: S. MINER  
 BRANCH CHIEF: J. STOLZ  
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
03079	RANCHO SECO 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		1	<u>COMPLETE</u> 10/25/83
42601	RANCHO SECO - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	10/25/83
42918	RANCHO SECO - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	12/03/83
49616	RANCHO SECO - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION		1	12/28/83
03602	RANCHO SECO - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	01/16/84
12326	RANCHO SECO - HELB AND CONSEQUENTIAL SYSTEM FAILURE RANCHO SECO		3	01/31/84
12616	QUALITY ASSURANCE REQUIREMENTS REGARDING DIESEL GENERATOR FUEL OIL		3	01/31/84
08153	RANCHO SECO - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		1	02/22/84
<u>ACTIVE ACTIONS</u>				
52793	RANCHO SECO - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52874	RANCHO SECO - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52955	RANCHO SECO - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53036	RANCHO SECO-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53091	RANCHO SECO - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53144	RANCHO SECO - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
11288	RANCHO SECO - INSERVICE TESTING		1	06/28/84
06710	RANCHO SECO - REACTOR COOLANT SYSTEM OVERPRESSURIZATION PROTECTION		2	06/29/84
43042	RANCHO SECO - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	06/29/84
46855	RANCHO SECO - PWR MSLB WITH CONT'D FEEDWATER ADDITION		1	07/30/84
53190	RANCHO SECO - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		2	07/30/84
42507	RANCHO SECO - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	08/30/84
42126	RANCHO SECO - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		3	09/30/84
52264	RANCHO SECO - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/84
51126	RANCHO SECO 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	09/30/84
43606	RANCHO SECO - SNUBBER INSERVICE SURVEILLANCE TECHNICAL SPECIFICATIONS		3	11/15/84
47170	RANCHO SECO 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	01/31/85
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44673	RANCHO SECO - NUREG-0737 II.E.1.1, AFW SYSTEM EVALUATION		1	<u>COMPLETE</u> 10/05/83
45273	RANCHO SECO - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	10/26/83
45319	RANCHO SECO - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/26/83
45202	RANCHO SECO - NUREG-0737 II.K.2.13, THERMAL-MECHANICAL REPORT		1	06/05/84
44265	RANCHO SECO - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/08/84
<u>ACTIVE ACTIONS</u>				
44615	RANCHO SECO 1 - RV AND SV TESTING		1	<u>TARGET</u> 07/16/84
45166	RANCHO SECO - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/30/84
45859	RANCHO SECO - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	09/30/84
46265	RANCHO SECO 1 - III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE		1	09/30/84
46337	RANCHO SECO - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84

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(CONTINUATION)

RANCHO SECO 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
48198	RANCHO SECO - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>TARGET</u> 09/30/84
51196	RANCHO SECO 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/84
51276	RANCHO SECO 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	09/30/84
44335	RANCHO SECO - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
45978	RANCHO SECO - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	01/18/85
46049	RANCHO SECO - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	01/18/85
46121	RANCHO SECO - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	01/18/85
<u>COMPLETED ACTIONS</u>				
48907	RANCHO SECO - SEPTEMBER 30, 1982 ASLAB ORDER		1	<u>COMPLETE</u> 10/05/83
51330	RANCHO SECO - EXEMPTION REQUEST FROM NUREG 0737 ORDER		2	10/30/83
51432	RANCHO SECO - CORE BARREL BOLTS		1	11/28/83
49309	RANCHO SECO - REVIEW OF GENERIC HPI NOZZLE CRACKING IN B & W PLANTS		2	12/30/83
52641	RANCHO SECO - IMPLEMENTATION OF APPEAL BOARD DECISION (ALAB-746)			01/03/84
49241	RANCHO SECO - SPENT FUEL POOL EXPANSION		2	01/20/84
52292	RANCHO SECO - HIGH RANGE RADIATION MONITOR			03/16/84
52713	RANCHO SECO - SALP REPORT FOR 1983 (THRU NOV 30)			04/23/84
54441	RANCHO SECO - ISI INTERVAL CLARIFICATION		2	05/07/84
49575	RANCHO SECO - IAEA SAFEGUARDS INSPECTIONS		2	06/01/84
52561	RANCHO SECO - 10CFR50.54 SHIFT STAFFING EXTENSION REQUEST		1	06/08/84
43190	RANCHO SECO - AUXILIARY FEEDWATER SYSTEM UPGRADE		1	06/26/84
<u>ACTIVE ACTIONS</u> <u>TARGET</u>				
52606	RANCHO SECO - SUPPLEMENT NO 1 TO NUREG 0737 TECH SPEC AMEND			
54349	RANCHO SECO - INTEGRATED SCHEDULES			
54351	RANCHO SECO-SUPPLEMENT NO. 2 TO NUREG 0737 TECH SPECS (AMMEND #83 SUPPL #2)		1	
54352	RANCHO SECO-NUREG 0737 TECH SPEC - GL83-37 (AMMEND #100)		1	
54835	RANCHO SECO - REACTOR VESSEL HEAD VENT			
55033	RANCHO SECO - PRESSURIZED THERMAL SHOCK			
51696	RANCHO SECO - MINI PURGE SYSTEM		2	11/30/83
54487	RANCHO SECO - NNI FAILURE INVESTIGATION		2	05/30/84
54271	RANCHO SECO - CONTAINMENT RAD MONITORS SURVEILLANCE (AMEND 108)		2	05/30/84
51695	RANCHO SECO - ALLOWABLE LEAKAGE FOR REACTOR BUILDING SPRAY AND DECAY HEAT REMOVAL			06/29/84
49703	RANCHO SECO - TECH SPEC PRESSURE TEMP LIMITS			06/29/84
52307	RANCHO SECO - AUX FEEDWATER HEADER INSPECTION		2	06/29/84
52291	RANCHO SECO - ALTERNATE SHUTDOWN CAPABILITY			06/29/84
53329	RANCHO SECO - LOCAL LEAK RATE TESTING FOR PURGE AND 12 INCHES CONTAINMENT VALVES		2	06/29/84
49825	RANCHO SECO- SMALL BREAK LOCA PROCEDURES AND MAINTAINING PROPER SG LEVEL		2	06/30/84
53444	RANCHO SECO - APPENDIX R EXEMPTION REQUEST FOR AUX BUILDING		2	07/01/84
53388	RANCHO SECO - 12 "REACTOR BUILDING EQUALIZING VALVES OPERABILITY		2	07/29/84
53510	RANCHO SECO - CYCLE 7 RELOAD REPORT VOLUME 1			07/30/84
51697	RANCHO SECO - ORGANIZATIONAL CHANGES		2	07/30/84
49574	RANCHO SECO - REACTOR BUILDING POST TENSIONING TENDON SURVEILLANCE		2	08/28/84
52607	RANCHO SECO - NUCLEAR INSTRUMENT CALIBRATION		2	09/28/84
52656	RANCHO SECO - AFW SYSTEM INTERNAL MISSES			09/30/84
48129	RANCHO SECO - REACTOR COOLANT PUMP SEAL DAMAGE		3	09/30/84
54410	RANCHO SECO- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		2	09/30/84
42933	RANCHO SECO - ATMOSPHERIC DUMP VALVE CIRCUITRY MODIFICATION		1	09/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

RANCHO SECO 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>		<u>(CONTINUATION)</u>		<u>TARGET</u>
13099	RANCHO SECO - SEISMIC ASSESSMENT OF FOOTHILLS FAULT SYSTEM		3	09/30/84
42049	RANCHO SECO - LCO FOR DG OPERABILITY		1	09/30/84
53509	RANCHO SECO - NUREG 0630 IMPLEMENTATION		2	10/17/84
51007	RANCHO SECO - ISI LICENSE AMENDMENT		2	11/15/84
52605	RANCHO SECO - INSTRUMENT SURVEILLANCE REQUIREMENTS		2	11/15/84
49680	RANCHO SECO - REACTOR COOLANT PUMP TRIP		2	02/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53792	RANCHO SECO - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53875	RANCHO SECO - ITEM 3.2.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53931	-ITEMS 4.2.3 & 4.2.4 - PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS - LIFE TESTING & REPLACEMENT			11/01/84
53958	RANCHO SECO - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
54019	RANCHO SECO - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54102	RANCHO SECO - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84
53626	RANCHO SECO - ITEM 1.2 - POST TRIP REVIEW - DATA AND INFORMATION CAPABILITY			11/01/84
53709	RANCHO SECO - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: ROBINSON 2

PLANT LOCATION: 5 MI NW OF HARTSVILLE, SC  
 DOCKET NUMBER: 050-00261  
 ARCH/ENGINEER: EBASCO  
 IE INSPECTOR: S WEISS

LICENSED POWER: 2300 MWT  
 DESIGN POWER: 0700 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: G. PEQUA  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
43604	ROBINSON - EXEMPTION TO APPENDIX R FROM COLD SHUTDOWN WITHIN 72 HOURS		1	<u>COMPLETE</u> 11/22/83
43605	ROBINSON - EXEMPTION TO APPENDIX R ON CABLE PENETATION TEMPERATURE		1	11/25/83
42560	ROBINSON 2 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	12/05/83
49610	ROBINSON 2 - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION		1	12/05/83
12275	ROBINSON 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE H.B. ROBINSON 2		3	01/31/84
03522	ROBINSON 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
08511	ROBINSON 2 - TECH SPEC SURVEILLANCE REQMTS. FOR MECHANICAL SNUBBERS		2	03/25/84
12978	ROBINSON 2 - ADEQUACY OF STATION ELECTRIC. DISTR SYS. VOLTAGES		2	05/07/84
08080	ROBINSON 2 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	05/30/84
<u>ACTIVE ACTIONS</u>				
52265	ROBINSON 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52794	ROBINSON 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52875	ROBINSON 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52956	ROBINSON 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53037	ROBINSON 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH SPECS - RTS COMPONENTS			
53092	ROBINSON 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53145	ROBINSON 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53151	ROBINSON 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
08126	ROBINSON 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/29/83
49753	ROBINSON 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			06/25/84
51127	ROBINSON 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		1	06/29/84
42466	ROBINSON 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
43381	ROBINSON - EXEMPTION FOR FIRE PROTECTION OIL COLLECTION SYSTEM		2	07/00/84
43636	ROBINSON 2 - SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM		1	07/16/84
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47633	ROBINSON 2 - NUREG 0737 ITEM II F1.4 CONTAINMENT PRESSURE INSTRUMENT		1	<u>COMPLETE</u> 04/13/84
47704	ROBINSON 2 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	04/13/84
47775	ROBINSON 2 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	04/13/84
46901	ROBINSON 2 - II K2.13 THERMAL MECHANICAL REPORT		1	06/18/84
44689	ROBINSON 2 - NUREG-0737 II.E.1.2.1, AFW SAFETY GRADE AUTO INITIATION		1	06/20/84
44728	ROBINSON 2 - NUREG-0737 II.E.1.2.2, AFW SAFETY GRADE FLOW INDICATION		1	06/20/84
<u>ACTIVE ACTIONS</u>				
45167	ROBINSON 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	<u>TARGET</u>
45860	ROBINSON 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	
45979	ROBINSON 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	
46050	ROBINSON 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	
46122	ROBINSON 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	
46478	ROBINSON 2 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ROBINSON 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
44336	ROBINSON 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	<u>TARGET</u>
44474	ROBINSON 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/15/84
46338	ROBINSON 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
48199	ROBINSON 2 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
51197	ROBINSON 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/84
51277	ROBINSON 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
44616	ROBINSON 2 - RV AND SV TESTING		1	10/31/84
<u>PLANT SPECIFIC</u>				
<u>TAC #</u>	<u>TAC DESCRIPTION</u>		<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
43828	ROBINSON 2 - ALTERNATE SHUTDOWN SYSTEM		1	<u>COMPLETE</u> 11/22/83
53230	H.B. ROBINSON 2 - HEARING PREPARATION REPAIR			02/10/84
48877	H. B. ROBINSON - STEAM GENERATOR REPAIR		1	02/28/84
49438	H. B. ROBINSON #2 GENERIC LETTERS 82-17 & 82-23		1	02/28/84
52553	H.B. ROBINSON 2 - MISCL TECH SPECS			03/28/84
52632	H.B. ROBINSON 2 - REVISE TEC SPEC SURVEILLANCE REQUIREMENTS			03/30/84
51796	HB ROBINSON 2 - REVISE TEC SPECS TO BE CONSISTANT WITH PRESENT SA		3	04/13/84
51967	H.B. ROBINSON 2 - REVISE ISI TEC SPEC TO CORRECT ERROR		3	05/30/84
52335	H.B. ROBINSON 2 - STEAM GENERATOR TUBE INSPECTION			06/12/84
52616	H.B. ROBINSON 2 SAMPLING OF REACTOR VESSEL HEAD WELDS			06/28/84
<u>ACTIVE ACTIONS</u>				
52615	H.B. ROBINSON-2 PTS FLUX REDUCTION PROGRAM			<u>TARGET</u>
52696	H.B. ROBINSON 2 - T/S ORGANIZATION CHANGE		3	
52697	H. B. ROBINSON 2 - VENT T/S CHANGE CONTAINMENT PURGE			
49294	H.B. ROBINSON 2 - LOSS OF FEEDWATER TRANSIENT		2	
13024	ROBINSON 2 - ENVIRONMENTAL TECHNICAL SPECIFICATIONS		3	
11215	ROBINSON 2 - INSERVICE TESTING - PUMPS AND VALVES		1	
55303	H. B. ROBINSON 2 - BACKFEEDING SR BASSES THRU TRANSFORMERS			
53336	H. B. ROBINSON 2 - APPENDIX J TEST PROGRAM EXEMPTIONS			
54619	H.B. ROBINSON 2 - REPORTING REQUIREMENT TEC. SPECS. (6C 83-43)			
54202	H.B. ROBINSON-2 - APPENDIX R SEC III G COLD SHUTDOWN			05/31/84
52617	H.B. ROBINSON 2 HEATUP & COOLDOWN CURVE IS REVISION		2	06/01/84
53482	H.B. ROBINSON 2 AFW INITIATION T/S REVISION		3	06/15/84
53481	H.B. ROBINSON 2 USE OF 10 SPARE SPACES FOR FUEL STORAGE		2	06/20/84
53326	H. B. ROBINSON-2 - DEDICATED SHUTDOWN SYSTEM TECHNICAL SPECIFICATIONS			06/29/84
54994	H.B. ROBINSON - 2 EXEMPTION FROM 10CFR50 APPENDIX E SECTION IV.F.1.			06/29/84
54896	H.B. ROBINSON 2 - ABT FOR AFW INITIATION/FLOW			07/02/84
51966	H.B. ROBINSON 2 - REVISE ISI TEC SPECS TO INCORPORATE SECT XI OF ASME CWE			07/08/84
51339	H. B. ROBINSON 2 - EXEMPTION TO APPENDIX R, III.G.2		1	07/20/84
49223	H. B. ROBINSON 2 - REVIEW AFW PROTECTION FROM TORNADO MISSLES		3	07/20/84
52440	H.B. ROBINSON 2 - SECOND 10 YR ISI PROGRAM		1	07/20/84
54484	H. B. ROBINSON - 2 WASTE DISPOSAL PROCEDURE			07/20/84
54192	ROBINSON 2 - SEALING FACTORS - RADWASTE			07/20/84
51962	H.B. ROBINSON 2 - CHANGE TECH SPECS - DEDICATED SHUTDOWN SYSTEM		3	09/00/84
55156	H.B. ROBINSON 2 - MECH. DES. REPORT XN-NF-83-55			09/20/84
54411	ROBINSON 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54563	ROBINSON 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
53229	H.B. ROBINSON 2- FLUX REDUCTIOIN REPORT & CAPSULE T REPORT			09/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ROBINSON 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
52695	H.B. ROBINSON 2 - CYCLE 10 OPERATIONS			<u>TARGET</u>
49690	ROBINSON 2 - REACTOR COOLANT PUMP TRIP (GL 83 10D)		2	10/20/84 02/01/85
<u>ANTICIPATED ACTIONS</u>				
55155	H.B. ROBINSON 2 - MECH DES REPORT XN-NF-83-71			<u>INITIATION</u>
53627	ROBINSON 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			10/05/84
53710	ROBINSON 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53793	ROBINSON 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53876	ROBINSON 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53932	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54020	ROBINSON 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54103	ROBINSON 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: SALEM 1

PLANT LOCATION: 20 MI S OF WILMINGTON, DEL  
 DOCKET NUMBER: 050-00272  
 ARCH/ENGINEER: PUBSERVE  
 IE INSPECTOR: L. NORRHOLM

LICENSED POWER: 3338 MWT  
 DESIGN POWER: 1090 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: D. FISCHER  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12276	SALEM 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE SALEM 1		3	COMPLETE 01/31/84
08081	SALEM 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/22/84
53192	SALEM 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMNT FOR W AND B & W PLANTS			06/25/84
<u>ACTIVE ACTIONS</u>				
52795	SALEM 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			TARGET
52876	SALEM 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52957	SALEM 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53038	SALEM 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53093	SALEM 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53146	SALEM 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
52710	SALEM 1 - ADD SURVEILLANCE TESTING ON RTB'S			05/01/84
49754	SALEM 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			05/01/84
51128	SALEM 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)		3	05/01/84
08154	SALEM 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		2	09/30/84
08824	SALEM 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		1	09/30/84
42467	SALEM 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)			09/30/84
52266	SALEM 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
45247	SALEM 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	COMPLETE 10/06/83
45293	SALEM 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/06/83
44476	SALEM 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/04/83
47817	SALEM 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/30/84
44268	SALEM 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
46903	SALEMS 1 - II K2.13 THERMAL MECHANICAL REPORT		1	06/18/84
<u>ACTIVE ACTIONS</u>				
46340	SALEM 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	TARGET 06/01/84
46480	SALEM 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	06/01/84
45862	SALEM 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
45169	SALEM 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
45981	SALEM 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46052	SALEM 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46124	SALEM 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
48201	SALEM 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
44618	SALEM 1 - RV AND SV TESTING		1	09/30/84
51198	SALEM 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	09/30/84
44338	SALEM 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51278	SALEM 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			12/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SALEM 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52322	SALEM 1 - EXTENDED I.L.R.T INTERVAL FOR UNIT 1			<u>COMPLETE</u>
49296	SALEM 1 - MODIFY FUEL ENRICHMENT			10/31/83
52706	SALEM 1 - RADIOLOGICAL EFFLUENT CONTROLS			11/22/83
52708	SALEM 1 - NUCLEAR DEPT. REORGANIZATION			02/28/84
52323	SALEM 1 - REV. 0 TO TRAINING PROCEDURE TP-305			02/28/84
49269	SALEM UNIT 1 - REVISED ROD EXCHANGE METHODOLOGY			05/04/84
				06/29/84
<u>ACTIVE ACTIONS</u>				
52704	SALEM 1 - REVISE AIRLOCK TECH SPEC			<u>TARGET</u>
47102	SALEM 1 - NUCLEAR DEPARTMENT REORGANIZATION		2	01/20/84
52702	SALEM 1 - ADD MANUAL INITIATION OF AUX FEED			05/01/84
51831	SALEM 1 - 10" PRESSURE/VACUUM VALVE FOR CONTAINMENT VENTING VENTING			06/01/84
51570	SALEM 1 - AUTO SWITCHOVER MODIFICATION			06/01/84
48993	SALEM 1 RCS INVENTORY BALANCE			06/15/84
48991	SALEM 1 MSIV SOLENOID VALVES			06/15/84
47122	SALEM 1 - ENVIRONMENTAL PROTECTION PLAN		2	06/25/84
49908	SALEM - ATWS EVENT FOLLOWUP			07/01/84
53539	SALEM 1 - ADDITIONAL APPENDIX R EXEMPTIONS			07/31/84
54412	SALEM 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			08/01/84
54564	SALEM 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49669	SALEM 1 - REACTOR COOLANT PUMP TRIP			09/30/84
				02/01/85
<u>ANTICIPATED ACTIONS</u>				
53628	SALEM 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u>
53711	SALEM 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53794	SALEM 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53877	SALEM 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53933	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54021	SALEM 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54104	SALEM 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: SALEM 2

PLANT LOCATION: 20 MI S OF WILMINGTON, DEL  
 DOCKET NUMBER: 050-00311  
 ARCH/ENGINEER: PUBSERVE  
 IE INSPECTOR:

LICENSED POWER: 3411 MWT  
 DESIGN POWER: 1115 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: D. FISCHER  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
46815	SALEM 2 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	<u>COMPLETE</u> 06/22/84
53193	SALEM 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			06/25/84
<u>ACTIVE ACTIONS</u>				
52796	SALEM 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52877	SALEM 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52958	SALEM 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53039	SALEM 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53094	SALEM 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53147	SALEM 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
48327	SALEM 2 - APPENDIX I TECHNICAL SPECIFICATIONS REVIEW		3	05/01/84
49755	SALEM 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			05/01/84
51129	SALEM 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			05/01/84
52267	SALEM 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		1	09/30/84
42533	SALEM 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)			09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
45248	SALEM 2 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	<u>COMPLETE</u> 10/06/83
45294	SALEM 2 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/06/83
44477	SALEM 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	11/04/83
47818	SALEM 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/30/84
46904	SALEM 2 - II K2.13 THERMAL MECHANICAL REPORT		1	06/18/84
<u>ACTIVE ACTIONS</u>				
46341	SALEM 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u> 06/01/84
45863	SALEM 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
45170	SALEM 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
44619	SALEM 2 - RV AND SV TESTING		1	09/30/84
45982	SALEM 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46053	SALEM 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46125	SALEM 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
48202	SALEM 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
51199	SALEM 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/84
44269	SALEM 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44339	SALEM 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
51279	SALEM 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			12/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49295	SALEM 2 - MODIFY FUEL ENRICHMENT			<u>COMPLETE</u> 11/22/83

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SALEM 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS (CONTINUATION)</u>				<u>COMPLETE</u>
48938	SALEM UNIT 2 SURVEILLANCE OF SNUBBERS			01/27/84
52707	SALEM 2 - RADIOLOGICAL EFFLUENT CONTROLS			02/28/84
52709	SALEM 2 - NUCLEAR DEPT. REORGANIZATION			02/28/84
52324	SALEM 2 - REV. 0 TO TRAINING PROCEDURE TP-305			05/04/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54468	SALEM UNIT 2. SURVEILLANCE CAPSULE REPORT TIME EXTENSION			
46713	SALEM 2 - FIRE PROTECTION REVIEW OPEN ITEMS		3	04/20/84
52712	SALEM 2 - FIRE PROTECTION LICENSE CONDITIONS			04/20/84
52705	SALEM 2 - REVISE AIRLOCK TECH SPEC			04/30/84
52711	SALEM 2 - ADD SURVEILLANCE TESTING ON RTB'S			05/01/84
47103	SALEM 2 - NUCLEAR DEPARTMENT REORGANIZATION		3	05/11/84
52703	SALEM 2 - ADD MANUAL INITIATION OF AUX FEED			06/01/84
51882	SALEM 1 - 10" PRESSURE/VACUUM VALVE FOR CONTAINMENT VENTING			06/01/84
48994	SALEM 2 RCS INVENTORY BALANCE			06/15/84
51571	SALEM 2 - AUTO SWITCHOVER MODIFICATION			06/15/84
53541	SALEM 2 - CHANGE CONTROL ROOM POSITIVE PRESSURE TEST VALVE			06/15/84
48992	SALEM 2 MSIV SOLENOID VALVES			06/15/84
47123	SALEM 2 - ENVIRONMENTAL PROTECTION PLAN		3	06/25/84
53540	SALEM 2 - ADDITIONAL APPENDIX R EXEMPTIONS			07/01/84
54565	SALEM 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			08/01/84
54413	SALEM 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
46922	SALEM 2 - CONTROL ROOM HUMAN FACTORS REVIEW			09/30/84
47887	SALEM 2 - SUMP TEST REPORT		3	09/30/84
49670	SALEM 2 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53629	SALEM 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53712	SALEM 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53795	SALEM 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53878	SALEM 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53934	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54022	SALEM 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54105	SALEM 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84



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## CONDENSED MANAGEMENT REPORT

FACILITY: SAN ONOFRE 1

PLANT LOCATION: 5 MI S OF SAN CLEMENTE CA  
 DOCKET NUMBER: 050-00206  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: L. MILLER

LICENSED POWER: 1347 MWT  
 DESIGN POWER: 0436 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: E. MCKENNA  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
13178	SAN ONOFRE 1 - REACTOR VESSEL CAVITY SEAL RING MISSILE POTENTIAL		2	<u>COMPLETE</u> 02/01/84
42615	SAN ONOFRE 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	02/17/84
<u>ACTIVE ACTIONS</u>				
52268	SAN ONOFRE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52797	SAN ONOFRE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52878	SAN ONOFRE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
53040	SAN ONOFRE 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
53095	SAN ONOFRE 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53148	SAN ONOFRE 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53194	SAN ONOFRE 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		1	12/31/83
51130	SAN ONOFRE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
42916	SAN ONOFRE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/84
42516	SAN ONOFRE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
08082	SAN ONOFRE 1-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/30/84
08101	SAN ONOFRE 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/30/84
46518	SAN ONOFRE - FIRE PROTECTION TECH SPECS		1	06/30/84
42131	SAN ONOFRE 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		2	09/30/84
11231	SAN ONOFRE 1 - INSERVICE TESTING (IST)		1	09/30/84
49756	SAN ONOFRE 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"		1	09/30/84
TAC #	TAC DESCRIPTION	IMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47819	SAN ONOFRE 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 12/19/83
46482	SAN ONOFRE 1 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	02/03/84
44128	SAN ONOFRE 1 - NUREG-0737 I.A.1.3.1, SHIFT MANNING OVERTIME LIMITS			02/17/84
47637	SAN ONOFRE 1 - NUREG 0737 ITEM II F1.4 CONTAINMENT PRESSURE INSTRUMENT		1	04/16/84
47708	SAN ONOFRE 1 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	04/16/84
47779	SAN ONOFRE 1 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	04/16/84
44270	SAN ONOFRE 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
46906	SAN ONOFRE 1 - II K2.13 THERMAL MECHANICAL REPORT		1	06/13/84
<u>ACTIVE ACTIONS</u>				
46342	SAN ONOFRE 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u> 06/30/84
44478	SAN ONOFRE 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/30/84
45171	SAN ONOFRE 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/84
45864	SAN ONOFRE 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/84
45983	SAN ONOFRE 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46054	SAN ONOFRE 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46126	SAN ONOFRE 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
48203	SAN ONOFRE 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/84
44340	SAN ONOFRE 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
44620	SAN ONOFRE 1 - RV AND SV TESTING		1	12/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SAN ONOFRE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
51200	SAN ONOFRE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	<u>TARGET</u> 12/31/85
51280	SAN ONOFRE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM		1	12/31/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
48142	SAN ONOFRE 1 - OPERABILITY OF PWR SOURCES DURING SHUTDOWN & ELIM OF AUX SALT WTR PUMP FR OPER CONSIDERATION FOR SIS		2	<u>COMPLETE</u> 10/28/83
51312	SAN ONOFRE UNIT 1 - REVISED SECURITY PLAN		2	10/31/83
48753	SAN ONOFRE UNIT NO. 1 - MODIFICATION OF TECH. SPECS. REGARDING COOLANT ACTIVITY SAMPLING REQUIREMENTS		2	12/06/83
47835	SAN ONOFRE UNIT 1 - STEAM GENERATOR INSPECTION PROGRAM		1	02/07/84
52165	SAN ONOFRE UNIT NO. 1 - MODIFY TABLE 3.5.2-2 AND PORTIONS OF SECTION 5 OF TECH SPECS.			02/17/84
52352	SAN ONOFRE UNIT NO. 1 - TECH. SPEC. CHANGE - SPHERE PURGE AIR SUPPLY AND OUTLET VALVES			02/17/84
48035	SAN ONOFRE UNIT 1 - REVISION OF EQUATION TO CALCULATE RADIOACTIVE AIRBORNE EFFLUENT RELEASE RATE LIMIT		2	03/01/84
52630	SAN ONOFRE UNIT NO. 1 - REEVALUATION OF SPENT FUEL STORAGE RACKS			04/17/84
52351	SAN ONOFRE UNIT NO. 1 - REVISE TECH. SPECS. REGARDING BORON CONCENTRATION CHANGES			04/23/84
49685	SAN ONOFRE 1 - REACTOR COOLANT PUMP TRIP		2	05/01/84
54749	SAN ONOFRE 1 - RHR MAINTENANCE			05/03/84
53440	SAN ONOFRE 1 - PROPOSED CHANGE NG 129 - APPENDIX B T.S. 13 T.S. (12/5/83)			06/04/84
<u>ACTIVE ACTIONS</u>				
52959	SAN ONOFRE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			<u>TARGET</u>
54127	SAN ONOFRE 1 - AUXILIARY OPERATOR REQUIREMENTS			
54376	SAN ONOFRE 1 - INTEGRATED LIVING SCHEDULE			
54377	SAN ONOFRE 1 - LICENSE CONDITION FOR STEAM GENERATOR INSPECTION			
55027	SAN ONOFRE 1 - HEATUP/COOLDOWN CURVES			
52166	SAN ONOFRE UNIT NO. 1 - INVESTIGATION OF POTENTIAL DIESEL GENERATOR CRANKSHAFT PROBLEMS			05/31/84
55090	SAN ONOFRE 1 - TEMPORARY BATTERY			06/00/84
52350	SAN ONOFRE UNIT NO. 1 - APPENDIX J TECHNICAL SPECIFICATIONS			06/15/84
10853	SAN ONOFRE 1 - CHANGE TESTING OF STATION BATTERY SYSTEM		2	06/30/84
54991	SAN ONOFRE 1 - OVERPRESSURE MITIGATION SYSTEM TECHNICAL SPECIFICATION			07/15/84
55026	SAN ONOFRE - AUXILIARY FEEDWATER SYSTEMS T.S.			07/30/84
54393	SAN ONOFRE 1 - ORGANIZATION CHANGE			07/30/84
55179	SAN ONOFRE 1 - AUXILIARY SALT WATER COOLING PUMPS TS			08/15/84
54880	SAN ONOFRE 1 - DEDICATED SHUTDOWN SYSTEM FOR APPENDIX R			08/30/84
11232	SAN ONOFRE 1 - FULL TERM OPERATING LICENSE		1	09/01/84
51739	SAN ONOFRE 1: STATION DIST. VOLTAGE VERIF. TEST			09/30/84
54414	SAN ONOFRE 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54566	SAN ONOFRE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
53332	SAN ONOFRE UNIT NO. 1 - REVISIONS TO TECHNICAL SPECIFICATIONS 3.7, 4.4.B, 4.4.E AND 4.4.F			09/30/84
<u>ANTICIPATED ACTIONS</u>				
53630	SAN ONOFRE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53713	SAN ONOFRE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53796	SAN ONOFRE 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53879	SAN ONOFRE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SAN ONOFRE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				<u>INITIATION</u>
53935	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54023	SAN ONOFRE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54106	SAN ONOFRE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: SAN ONOFRE 2

PLANT LOCATION: 5 MI S OF SAN CLEMENTE, CA  
 DOCKET NUMBER: 050-00361  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: D. KIRSCH

LICENSED POWER: 3410 MWT  
 DESIGN POWER: 1070 MWE  
 NSSS VENDOR: COMB

PROJECT MANAGER: H. ROOD  
 BRANCH CHIEF: G. KNIGHTON  
 LIC. ASSISTANT: J. LEE

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49757	SAN ONOFRE 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			<u>COMPLETE</u>
51131	SAN ONOFRE 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			04/30/84
42528	SAN ONOFRE 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	05/04/84 05/25/84

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
52269	SAN ONOFRE 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52373	SAN ONOFRE 2 - CONTROL OF HEAVY LOADS (PHASE I)			
52798	SAN ONOFRE 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52879	SAN ONOFRE 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52960	SAN ONOFRE 2 - ITEMS 3.1.1 & 3.1.2 - POSI MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53041	SAN ONOFRE 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
53096	SAN ONOFRE 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53149	SAN ONOFRE 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
51664	SONGS 2 - RCS VOIDING - LIC. COND. 2.C(19)P (TMI II.K.2.17)			<u>COMPLETE</u>
54652	SAN ONOFRE 2 - NUREG 0737 III.A.1.2 TECHNICAL SUPPORT CENTER			03/08/84
54654	SAN ONOFRE 2 - NUREG-0737 III.A.1.2 EMERGENCY OPERATIONS FACILITY			04/26/84
54653	SAN ONOFRE 2 - NUREG-0737 III.A.1.2 OPERATIONAL SUPPORT CENTER			04/26/84
54980	SAN ONOFRE 2, II.K.2.13, THERMAL MECHANICAL REPORT			05/26/84 06/13/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
51281	SAN ONOFRE 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			<u>TARGET</u>
51588	SAN ONOFRE 2 - RV AND SV TESTING (II.D.1)			
49463	SAN ONOFRE 2 - INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING (II.F.2)			
51201	SAN ONOFRE 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51666	SONGS 2 - SBLOCA MODEL & ANALYSIS - LIC. COND. 2.C.19(R) - (TMI II.K.3.31)			05/26/84

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
51773	SONGS 2 - SAMP EVALUATION			<u>COMPLETE</u>
51833	SONGS 2 - CHG T.S. 3.6.1.4 BASES - PCN 29			10/01/83
51838	SONGS 2 - CHG T.S. SEC. 7 - NAT. CIRC A SERIES TESTS-PCN 7 - AMDT 4			10/01/83
51837	SONGS 2 - CHG T.S. TABLE 3.8-2 MOV OVLOAD BYPASS PCN 2 - AMDT 4			10/05/83
51830	SONGS 2 - REVIEW PASS MONTHLY REPORTS LIC COND. 2.C.(19)I			10/15/83
51839	SONGS 2 - TECH SPEC CORRECTIONS - PCN 10 AMDT 4			10/15/83
51840	SONGS 2 - T.S. SEC 6, ADMIN.-PCN 11-AMDT 4			10/22/83
51841	SONGS 2 - T.S. 4.6.1.3 - PCN 13 - AMDT 4			10/22/83
51842	SONGS 2 - T.S. TABLE 3.12-1 - PCN 12 AMDT 4			10/22/83
51846	SONGS 2 - LIC COND. 2(C)8- PCN 22 - AMDT5			10/22/83 10/24/83



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SAN ONOFRE 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
51845	SONGS 2 - FIRE PROTECTION T.S. - PCN 24 -AMDT 7			10/27/83
51843	SONGS 2 - CHG T.S. 3.3.3.9 - PCN 28 - AMDT 6			10/30/83
51844	SONGS 2 - CHG PASS DATE - PCN 42 - AMDT 8			11/05/83
51626	SONGS 2 - CHG T.S. 2.2.1, 3.2.4, 4.2.4.4, CHG DNBR PENALTY FACTORS (PC33)			11/09/83
54284	SONGS 2 - ISSUE ORDER CONFIRMING E.P. COMMITMENTS			02/23/84
54263	SONGS 2 - REVIEW EMERGENCY PREPAREDNESS EXEMPTION REQUEST			02/28/84
51657	SONGS 2 - WASTE SOLIDIFICATION PROCESS CONTROL PROGRAM - LICENSE CONDITION 2.C.(16)			04/24/84
51828	SONGS 2 - REVIEW EOF TMI III A.2. - LIC COND. 2.C.(19)S			04/26/84
51832	SONGS 2 - CHG T.S. 6.9.1.1.3.6 - PCN 21			04/30/84
51821	SONGS 2 - CHG SURV INTERNAL TO 25 MOS			04/30/84
52626	SONGS 2/3 - EVALUATE T.S. CHANGE ON OVERTIME PLANT STAFF - PCN 37			04/30/84
54782	SONGS 2 - DELETE L.C.2.C(19)F - PCN 82			05/01/84
54784	SONGS 2 - CHG PURGE LIMIT TO 3000 HRS - PCN 144			05/01/84
51826	SONGS 2 - REVIEW 6/30/83 PORV SUBMITTAL LIC COND 2.C(24)			05/04/84
51662	SONGS 2 - RCS VENTS - LICENSE CONDITION 2.C.19(H) - (TMI II.B.1)			05/04/84
51614	SONGS 2 - CHG. T.S. 4.4.5.2.2.C/D, CLARIFY RCPB VALVE TEST REQUIREMENTS (PC9)			05/04/84
51621	SONGS 2 - CHG T.S. 6.12.3, CLARIFY ACCESS CONTROL TO HIGH RAD. AREAS IN CONTAINMENT (PC44)			05/04/84
52527	SONGS 2 - RTB FAILURES DURING SURVEILLANCE TEST			05/04/84
51835	SONGS 2 - CHG T.S. 4.5.2.F - PCN 64 - HPSI REQTS			05/04/84
51836	SONGS 2 - DELETE T.S. 3.0.4 - PCN 54			05/04/84
51649	SONGS 2 - HIGH BURNUP FISSION GAS RELEASE - LICENSE CONDITION 2.C.(6)			05/07/84
52624	SONGS 2 AND 3 TRIP BREAKER UY ARMATURE JAMMING EVALUATE, ETC			05/13/84
54282	SONGS 2 - EVALUATE RTB SHUNT DEVICE INOPERABILITY			05/13/84
54510	SONGS 3 - EVALUATE LOSS OF CONTAINMENT SPRAY EVENT OF 3-15-84			05/17/84
51661	SONGS 2 - SPECIAL L.P.TESTS - LICENSE CONDITION 2.C.(19)G - (TMI I.G.1)			05/29/84
51647	SONGS 2 - CONTAINMENT TENDON SURVEILLANCE - LICENSE CONDITION 2.C.(4)			05/30/84
54635	SAN ONOFRE 2 - CONTAINMENT ISOLATION VALVES			05/30/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
55120	SONGS 2 - INSERVICE TESTING PER 10CFR50.55A			04/30/83
51316	SAN ONOFRE 2 - HIGH FREQUENCY OF NUISANCE ALARMS			05/20/83
51424	SAN ONOFRE 2 - IN-CONTAINMENT H2 MONITORS			06/25/83
51613	SONGS 2-CHG T.S. TABLE 4.3-2, CHANGE ESFAS SUBGROUP RELAY SURVEILLANCE INTERVAL FROM 6 MOS TO 18 MOS (PC4)			06/25/83
51623	SONGS 2 - CHG T.S. 3.3.2(TABLE 3.3.-5) AND 4.7.1.2.1.A, CHG RESPONSE TIME LIMIT FOR AFW (PC36)			06/25/83
51616	SONGS 2 - CHG. T.S.3.4.1.4.1, ALLOW BOTH TRAINS SDCS TO BE OUT IF ONE RCP IS ON-MODE 5 ONLY (PC15)			06/25/83
51618	SONGS 2 - CHG T.S.2.2.1, TABLE 2.2-1 TO CLARIFY SETPOINT VERIFICATION PROCEDURE, RC FLOW-LOW (PC-34)			06/25/83
51619	SONGS 2 - CHG T.S. 6.8.2/6.8.3 TO MODIFY APPROVAL AUTHORITY FOR PROCEDURE CHANGES (PC43)			06/25/83
51627	SONGS 2 - CHG T.S. 3/4.3.3.7, 3/4.7.8.2 TO REFLECT EXTRA FIRE PROTECTION EQPT. (PC72)			06/25/83
51653	SONGS 2 - CONTROL SYSTEM FAILURES DUE TO HELDA OR EQPT. FAILURES. LIC. COND. 2.C.(12)			06/26/83
51834	SONGS 2 - EDIT T.S. TO AGREE WITH UNIT 3 - PCN 56			07/21/83
52567	SONGS 2 BORON FLOWPATH ADEQUACY/ISOLATION			11/10/83
54314	SONGS 2 - REVIEW REVISED FHA - ISSUE SER			04/25/84
54638	SAN ONOFRE 2 - CRITICALITY ACCIDENT REQMT'S			05/01/84
54816	SONGS 2 - AMEND. LIC. PER IAEA FACILITY AGREEMENT			05/25/84
54257	SONGS 2 - EVALUATE CPC THERMAL POWER DECALIBRATION			05/30/84
54694	SONGS 2 - PCN 8 - REVISE TECH. SPEC. 3.6-1 - CONT. ISO. VALVES			06/12/84
54697	SONGS - 2-PCN 41 - TECH. SPEC. 3/4.2.8 - PRESSURIZER			06/12/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SAN ONOFRE 2

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54699	SONGS 2-PCN 75 - TECH. SPEC. 4.8.1.1.2.C - FUEL OIL RETEST			06/12/84
54700	SONGS 2-PCN 83 - TECH. SPEC. FIG. 6.2-1 - SCE ORG. CHART			06/12/84
54702	SONGS 2-PCN 84 - REVISE E.Q. LIC. COND. 2.C.(5)			06/12/84
54705	SONGS 2-PCN 85 - TECH. SPEC. TABLE 3.3-2, RPIS RESP. TIME			06/12/84
54706	SONGS 2-PCN 89 - TECH. SPEC. 3/4.4.4 - S.G. INSPECTION			06/12/84
54708	SONGS 2-PCN 91 - DELETE TECH. SPEC. 4.8.1.2.D.6 PER 83-30			06/12/84
54710	SONGS 2-PCN 97 - TECH. SPEC. 3/4.6.4-H2 MONITOR SURV.			06/12/84
54712	SONGS 2-PCN 98 - F.P. LIC. COND. 2.C.(14)			06/12/84
54714	SONGS 2-PCN 99 - TECH. SPEC. TABLE 3.3-3 - CPIS ACTION			06/12/84
54716	SONGS 2-PCN 100 - TECH. SPEC. 3/4.3.3.1 - RAD. MON. ALARM			06/12/84
54718	SONGS 2-PCN 101 - TECH. SPEC. 3/4.3.3.6 - ACC. MON. INST. TIME			06/12/84
54720	SONGS 2-PCN 102 - TECH. SPEC. 3/4.3.3.9 - EFFLUENT MON			06/12/84
54722	SONGS 2-PCN 103 - TECH. SPEC. 3/4.11 - NUREG 0472 CHANGES			06/12/84
54724	SONGS 2-PCN 109 - TECH. SPEC. 4.2.3.C - BURNUP - 31 DAYS			06/12/84
54726	SONGS 2-PCN 110 - TECH. SPEC. 4.3.3.2.4.A - CHANNEL CHECK			06/12/84
54728	SONGS 2-PCN 131 - TECH. SPEC. 3.3.3.8 - SALT WATER DILUTION			06/12/84
54730	SONGS 2-PCN 135 - TECH. SPEC. TABLE 3.3-4 - TGIS SETPOINTS			06/12/84
54733	SONGS 2-PCN 145 - CHG. F.P. LIC. COND. 2.C.(14)			06/12/84
54735	SONGS 2-PCN 146 - CHG. F.P. TECH. SPECS.			06/12/84
54738	SONGS 2-PCN 155 VENTS - TECH. SPEC. 3/4.4.10			06/12/84
54780	SONGS 2 - CHG. OSRC REVIEW OF PCP ODCM/ETC - PCN-79			06/18/84
54817	SONGS 2 - EVALUATE OPERATING EXPERIENCE			06/25/84
54812	SONGS 2 - REVIEW FISSION GAS PRESSURE ANALYSIS			06/25/84
54819	SONGS 2 - PCN 138 - TECH. SPEC. 3.1.3.1/6/7 - CHANGE TECH. SPEC. PER REVIEW CPC SOFTWARE			06/25/84
54860	SONGS 2 - PCN 108 - TECHNICAL SPECIFICATION 3.8-1(TBL) CONTAINMENT PENETRATION OVERCURRENT PROTECTION			06/30/84
54862	SONGS 2 - PCN 111 - TECH. SPEC. 3.11.2 - AUX. BOILER RADWASTE, INCINERATE WASTE OIL			06/30/84
54864	SONGS 2 - PCN 114 - TECHNICAL SPECIFICATION 4.6.1.6 - CONTAINMENT TENDON SURVEILLANCE			06/30/84
54866	SONGS 2 - NPF 119 - TECH. SPEC. B 3.2.3 - AZIMUTHAL POWERTILT			06/30/84
54868	SONGS 2 - PCN 123 - TECH. SPEC. 2.0 - DNBR LOW-BASIS			06/30/84
54870	SONGS 2 - PCN 125 - TECH. SPEC. 3.6.2.2 - SPRAY CHEMICAL REQUIRMENTS			06/30/84
54872	SONGS 2 - PCN 127 - TECH. SPEC. 3.7.10 - LCD FOR EMERGENCY CHILLED WATER SYSTEM			07/01/84
54874	SONGS 2 - PCN 134 - TECH. SPEC. 2.2.2 - CPC ADDRESSABLE CONSTANTS			07/01/84
54876	SONGS 2 - PCN-130 - L.C.2.C(4) - CONTAINMENT TENDON SURVEILLANCE			07/01/84
55113	SONGS 2 - PCN 137- T.S. 3/4.8.1.2- CHG LCO			09/01/84
54148	SONGS 2 - IN SERVICE INSPECTION, SER INPUT			09/01/84
54567	SAN ONOFRE 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
55116	SONGS 2 - MPA A-17(R.G.1.97) REVIEW RESPONS			10/01/84
55117	SONGS 2 - 1984 SALP EVALUATION			10/01/84
51847	SONGS 2 - AFW PUMP LUBE OIL COOLERS - LIC. COND. 2.C(25)			11/01/84
51659	SONGS 2 - EVALUATE EMERGENCY PROCEDURES/GUIDELINES-LICENSE CONDITION 2.C.(19).D - (TMI-I.C.1)			11/20/84
49691	SAN ONOFRE 2 - REACTOR COOLANT PUMP TRIP			02/01/85
51655	SONGS 2 - TURBINE DISC INSPECTION - LICENSE CONDITION 2.C.(15)			05/26/85
51656	SONGS 3 - TURBINE DISC INSPECTION - LICENSE CONDITION 2.C.(13)			05/26/85
51651	SONGS 2 - LOW TEMPERATURE OVERPRESSURE PROTECTION - LICENSE CONDITION 2.C.(7)			05/26/88
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53631	SAN ONOFRE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53714	SAN ONOFRE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53797	SAN ONOFRE 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SAN ONOFRE 2

<u>IAC #</u>	<u>IAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53880	SAN ONOFRE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			<u>INITIATION</u> 11/01/84
53936	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54024	SAN ONOFRE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54107	SAN ONOFRE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: SAN ONOFRE 3

PLANT LOCATION: 5 MI S OF SAN CLEMENTE, CA  
 DOCKET NUMBER: 050-00362  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: H. CANTER

LICENSED POWER: 3390 MWT  
 DESIGN POWER: 1080 MWE  
 NSSS VENDOR: COMB

PROJECT MANAGER: H. ROOD  
 BRANCH CHIEF: G. KNIGHTON  
 LIC. ASSISTANT: J. LEE

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
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COMPLETED ACTIONS

51132	SAN ONOFRE 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>COMPLETE</u>
42529	SAN ONOFRE 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	05/04/84 05/25/84

ACTIVE ACTIONS

12135	SAN ONOFRE 3, EMERGENCY PLAN REVIEW		1	<u>TARGET</u>
52270	SAN ONOFRE 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52374	SAN ONOFRE 3 - CONTROL OF HEAVY LOADS (PHASE I)			
52799	SAN ONOFRE 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52880	SAN ONOFRE 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52961	SAN ONOFRE 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53042	SAN ONOFRE 3-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS			
53097	SAN ONOFRE 3 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53150	SAN ONOFRE 3 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
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COMPLETED ACTIONS

54757	SAN ONOFRE-3 - NUREG-0737 III.A.1.2 TECHNICAL SUPPORT CENTER			<u>COMPLETE</u>
54758	SAN ONOFRE 3 - NURGE-0737 III.A.1.2 OPERATIONAL SUPPORT CENTER			04/26/84
54759	SAN ONOFRE-3 - NUREG-0737 III.A.1.2 EMERGENCY OPERATIONS FACILITY			04/26/84
54981	SAN ONOFRE 3, II.K.2.13, THERMAL MECHANICAL REPORT			05/10/84

ACTIVE ACTIONS

51202	SAN ONOFRE 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51282	SAN ONOFRE 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
51589	SAN ONOFRE 3 - RV AND SV TESTING (II.D.1)			
51667	SONGS 3 - SBLOCA MODEL AND ANALYSIS - LIC. COND. 2.C.(17)I - (TMI II.K.3.31)			05/26/84
51663	SONGS 3 - I.C.C. INSTRUMENTATION - LIC. COND. 2.C.(17)H - (TMI II.F.2)			05/26/85

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
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COMPLETED ACTIONS

51772	SONGS 3 - SALP EVALUATION			<u>COMPLETE</u>
51628	SONGS 3 - CHG T.S. 3/4.3.3.7, 3/4.7.8.2 TO REFLECT EXTRA FIRE PROTECTION EQPT (PC72)			10/01/83
51831	SONGS 3 - REVIEW PASS MONTHLY REPORTS - LIC COND. 2.C(17)D			10/12/83
51823	SONGS 3 - T.S. TABLE 3.3-5 PCN 60 AMDT 1			10/15/83
52308	SONGS 3 - DELAY LOSS OF OFFSITE POWER TEST			11/05/83
54285	SONGS 3 - ISSUE ORDER CONFIRMING E.P. COMMITMENTS			11/15/83
54264	SONGS 3 - REVIEW EMERGENCY PREPAREDNESS EXEMPTION REQUEST			02/23/84
51665	SONGS 3 - 48 HP. TEST OF AFW PUMPS - LIC. COND. 2.C.(17).F - (TMI II.E.1.1)			02/28/84
54785	SONGS 3 - CHG PURGE LIMIT TO 3000 HRS - PCN 144			03/08/84
52560	SONGS 3 STARTUP TESTING - EVALUATE CHANGES			03/13/84
51658	SONGS 3 - WASTE SOLIDIFICATION PROCESS CONTROL PROGRAM - LICENSE CONDITION 2.C.(14)			04/01/84 04/24/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SAN ONOFRE 3

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
51829	SONGS 3 - REVIEW EOF - TMI III.A.2. - LIC COND. 2.C.(17)J			04/26/84
52625	SONGS 2/3 - EVALUATE T.S. CHANGE ON OVERTIME LIMITS FOR PLANT STAFF			04/30/84
51822	SONGS 3 - CHG SURV INTERVAL TO 25MOS			04/30/84
54783	SONGS 2 - DELETE L.C.2.C.(17)F - PCN 82			05/01/84
51650	SONGS 3 - HIGH BURNUP FISSION GAS RELEASE - LICENSE CONDITION 2.C.(6)			05/04/84
51615	SONGS 3-CHG. T.S. 4.4.5.2.2 C/D, CLARIFY RCPB VALVE TEST REQUIREMENTS (PC9)			05/04/84
51622	SONGS 3 - CHG T.S. 6.12.3, CLARIFY ACCESS CONTROL TO HIGH RAD AREAS IN CONTAINMENT (PC44)			05/04/84
51824	SONGS 3 - T.S. 4.5.2.F-HPSI PUMP REQTS PCN 64			05/04/84
51825	SONGS 3 - DELETE T.S. 3.0.4- PCN 54			05/04/84
51827	SONGS 3 - REVIEW 6/30/83 PORV SUBMITTAL LIC COND 2.C.(19)			05/04/84
52627	SONGS 3 - TRIP BREAKER UV ARMATURE JAMMING - EVALUATE ETC.			05/13/84
54283	SONGS 3 - EVALUATE RTB SHUNT DEVICE INOPERABILITY			05/13/84
51625	SONGS 3 - CHG T.S.2.2.1, 3.2.4, 4.2.4.4, CHG DNBR PENALTY FACTORS (PC33)			05/18/84
51648	SONGS 3 - CONTAINMENT TENDON SURVEILLANCE - LICENSE CONDITION 2.C.(4)			05/30/84
54636	SAN ONOFRE 3 - CONTAINMENT ISOLATION VALVES			05/30/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
51317	SAN ONOFRE 3 - HIGH FREQUENCY OF NUISANCE ALARMS			04/30/83
51425	SAN ONOFRE 3 - IN-CONTAINMENT H2 MONITORS			05/20/83
51624	SONGS 3 - CHG T.S. 3.3.2 (TABLE 3.3-5) AND 4.7.1.2.1.A, CHG RESPONSE TIME LIMIT FOR AFW (PC36)			06/25/83
51617	SONGS 3 - CHG T.S. 3.4.1.4.1, ALLOW BOTH TRAINS SDCS TO BE OUT IF ONE RCP IS ON-MODE 5 ONLY (PC15)			06/25/83
51620	SONGS 3 - CHG T.S. 6.8.2/6.8.3 TO MODIFY APPROVAL AUTHORITY FOR PROCEDURE CHANGES (PC43)			06/25/83
51654	SONGS 3 - CONTROL SYS. FAILURES DUE TO HELBA OR EQPT. FAILURES - LIC. COND. 2.C.(10)			06/26/83
54637	SAN ONOFRE 3 - CRITICALITY ACCIDENT REQUIREMENT			05/01/84
54256	SONGS 3 - EVALUATE CPC THERMAL POWER DECALIBRATION			05/30/84
54695	SONGS 3 - PCN 8 - REVISE TECH. SPEC. 3.6.1 - CONT. ISO. VALVES			06/12/84
54696	SONGS 3 - PCN 41 - TECH. SPEC. 3/4.2.8 PRESSURIZER			06/12/84
54698	SONGS 3-PCN 75 - TECH. SPEC. 4.8.1.1.2.C - FUEL OIL RETEST			06/12/84
54701	SONGS 3-PCN 83 - TECH. SPEC. FIG. 6.2-1 - SCE ORG. CHART			06/12/84
54703	SONGS 3-PCN 84 - REVISE E.Q. LIC. COND. 2.C.(5)			06/12/84
54704	SONGS 3-PCN 85 - TECH. SPEC. TABLE 3.3-2, RPIS RESP. TIME			06/12/84
54707	SONGS 3-PCN 89 - TECH. SPEC. 3/4.4.4 - S.G. INSPECTION			06/12/84
54709	SONGS 3-PCN 91 - DELETE TECH. SPEC. 4.8.1.2.D.6 PER 83-30			06/12/84
54713	SONGS 3-PCN 98 - F.P. LIC. COND. 2.C.(14)			06/12/84
54715	SONGS 3-PCN 99 - TECH. SPEC. TABLE 3.3-3 - CPIS ACTION			06/12/84
54717	SONGS 3-PCN 100 - TECH. SPEC. 3/4.3.3.1 - RAD. MON. ALARMS			06/12/84
54719	SONGS 3-PCN 101 - TECH. SPEC. 3/4.3.3.6 - ACC. MON. INST. TIME			06/12/84
54721	SONGS 3-PCN 102 - TECH. SPEC. 3/4.3.3.9 - EFFLUENT MON			06/12/84
54723	SONGS 3-PCN 103 - TECH. SPEC. 3/4.11 - NUREG 0472 CHANGES			06/12/84
54725	SONGS 3-PCN 109 - TECH. SPEC. 4.2.3.C - BURNUP - 31 DAYS			06/12/84
54727	SONGS 3-PCN 110 - TECH. SPEC. 4.3.3.2.4.A - CHANNEL CHECK			06/12/84
54729	SONGS 3-PCN 131 - TECH. SPEC. 3.3.3.8 - SALT WATER DILUTION			06/12/84
54731	SONGS 3-PCN 135 - TECH. SPEC. TABLE 3.3-4 - TGIS SETPOINTS			06/12/84
54732	SONGS 3-PCN 140 - TECH. SPEC. 4.5.2 - DELETE HV-0396			06/12/84
54734	SONGS 3-PCN 145 - CHG. F.P. LIC. COND. 2.C.(12)			06/12/84
54736	SONGS 3-PCN 146 - CHG. F.P. TECH. SPECS.			06/12/84
54737	SONGS 3-PCN 155 VENTS - TECH. SPEC. 3/4.4.10			06/12/84
54781	SONGS 3 - CHG OSRC REVIEW OF PCP/ODCM/ETC - PCN79			06/18/84
54711	SONGS 3-PCN 97 - TECH. SPEC. 3/4.6.4 - H2 MONITOR SURV.			06/21/84

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## CONDENSED MANAGEMENT REPORT

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SAN ONOFRE 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54313	SONGS 3 - REVIEW FISSION GAS PRESSURE ANALYSIS			06/25/84
54818	SONGS 3 - EVALUATE OPERATING EXPERIENCE			06/25/84
54820	SONGS 3 - PCN 138 - TECH. SPEC. 3.1.3.1/6/7 - CHANGE TECH. SPEC. PER REVIEW CPC SOFTWARE			06/25/84
54861	SONGS 2 - PCN 108 - TECHNICAL SPECIFICATION 3.8-1(TBL) CONTAINMENT PENETRATION OVERCURRENT PROTECTION			06/30/84
54863	SONGS 3 - PCN 111 - TECHNICAL SPECIFICATION 3.11.2 - AUX. BOILER RADWASTE, INCINERATE WASTE OIL			06/30/84
54865	SONGS 3 - PCN 114 - TECH. SPEC. 4.6.1.6 - CONTAINMENT TENDON SURVEILLANCE			06/30/84
54867	SONGS 3 - NPF 119 - TECH. SPEC. B3.2.3 - AZIMUTHAL POWER TILT			06/30/84
54869	SONGS 2 - PCN 123 - TECH. SPEC. 2.0 - DNBR LOW-BASIS			06/30/84
54871	SONGS 2 - PCN 125 - TECH. SPEC. 3.6.2.2 - SPRAY CHEMICAL REQUIRMENTS			06/30/84
54873	SONGS 2 - PCN 127 - TECH. SPEC. 3.7.10 - LCO FOR EMERGENCY CHILLED WATER SYSTEM			07/01/84
54875	SONGS 2 - PCN 134 - TECH. SPEC. 2.2.2 - CPC ADDRESSABLE CONSTANTS			07/01/84
54877	SONGS 2 - PCN-130 - L.C.2.C(4) - CONTAINMENT TENDON SURVEILLANCE			07/01/84
55119	SONGS 3- CHG T.S. TABLE 4.3-2 (PCN4) ESFAS SUBGROUP RELAYS			07/05/84
54815	SONGS 3 - REVIEW REVISED FHA - ISSUE SER			07/25/84
55114	SONGS 3 - PCN 137- T.S. 3/4.8.1.2 - CHG LCO			09/01/84
54149	SONGS 3 - IN SERVICE INSPECTION - SER INPUT			09/01/84
54568	SAN ONOFRE 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
55115	SONGS 3 - MPA A-17(R.G. 1.97) REVIEW RESPONSE			10/01/84
55118	SONGS 3 - 1984 SALP EVALUATION			10/01/84
49692	SAN ONOFRE 3 - REACTOR COOLANT PUMP TRIP			02/01/85
51660	SONGS 3 - EMERGENCY PROCEDURES/GUIDELINES-LICENSE CONDITION 2.C.(17)A - (TMI I.C.1)			05/26/85
51848	SONGS 3 - AFW LUBE OIL COOLERS - LIC COND. 2.C.(20)			06/01/85
55121	SONGS 3 - INSERVTC TESTING PER 10CFR 50.550			06/05/85
51652	SONGS 3 - LOW TEMPERATURE OVERPRESSURE PROTECTION - LICENSE CONDITION 2.C.(7)			05/26/88
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53632	SAN ONOFRE 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53715	SAN ONOFRE 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53798	SAN ONOFRE 3- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53881	SAN ONOFRE 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53937	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54025	SAN ONOFRE 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54108	SAN ONOFRE 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: SEQUOYAH 1

PLANT LOCATION: 9.5 MI NE OF CHATTANOOGA, TN  
 DOCKET NUMBER: 050-00327  
 ARCH/ENGINEER: TVA  
 IE INSPECTOR: F. FORD

LICENSED POWER: 3411 MWT  
 DESIGN POWER: 1148 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: C. STAHL  
 BRANCH CHIEF: E. ADENSAM  
 LIC. ASSISTANT: M. DUNCAN

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51759	SEQUOYAH 1 - NATURAL CIRCULATION COOLDOWN			<u>COMPLETE</u> 12/30/83
<u>ACTIVE ACTIONS</u>				
52271	SEQUOYAH 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52800	SEQUOYAH 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52881	SEQUOYAH 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52962	SEQUOYAH 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53043	SEQUOYAH 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53098	SEQUOYAH 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53151	SEQUOYAH 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53195	SEQUOYAH 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & V PLANTS			
42535	SEQUOYAH 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	06/30/84
47064	SEQUOYAH 1/2 - FIRE PROTECTION		1	09/09/84
49758	SEQUOYAH 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			10/10/84
47130	SEQUOYAH 1 - CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANTS (NUREG 0612)		2	12/12/84
51133	SEQUOYAH 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/06/86
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51283	SEQUOYAH 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			<u>COMPLETE</u> 12/30/83
44408	SEQUOYAH 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	12/30/83
46271	SEQUOYAH 1 - III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE MEET RULE		1	12/30/83
46343	SEQUOYAH 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	12/30/83
44479	SEQUOYAH 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	03/03/84
51203	SEQUOYAH 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			05/30/84
47820	SEQUOYAH 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	05/30/84
<u>ACTIVE ACTIONS</u>				
44621	SEQUOYAH 1 - RV AND SV TESTING		1	<u>TARGET</u> 02/15/83
45250	SEQUOYAH 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	02/15/83
45865	SEQUOYAH 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/83
44341	SEQUOYAH 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	06/30/83
48204	SEQUOYAH 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/83
45172	SEQUOYAH 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/00/83
45984	SEQUOYAH 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46055	SEQUOYAH 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46127	SEQUOYAH 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51451	SEQUOYAH 1 - CHEMICAL FEED LINE CHANGE			<u>COMPLETE</u> 12/30/83

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(CONTINUATION)

SEQUOYAH 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
51970	SEQUOYAH 1 - DG SURVEILLANCE REQ'DS			12/30/83
51466	SEQUOYAH 1 - ADMINISTRATIVE CHANGE TO RADIOLOGICAL EMERGENCY PLAN			12/30/83
51467	SEQUOYAH 1 - LIMITING CONDITION REVISION TO CO2 SYSTEM			12/30/83
51470	SEQUOYAH 1 - ADMINISTRATIVE CHANGE TO CENTRAL ROOM COMMAND			12/30/83
51760	SEQUOYAH 1 - TEMPORARY TECH SPEC CHANGE TO PERFORMING SHUTDOWN BOARD MODIFICATIONS			12/30/83
51763	SEQUOYAH 1 - VOLUME REDUCTION SYSTEM TO PROCESS			12/30/83
52130	SEQUOYAH UNIT 1 - DNB PARAMETER CHANGES			12/30/83
53391	SEQUOYAH 1 - POST ACCIDENT SAMPLING			02/02/84
53393	SEQUOYAH 1 - MOV THERMAL OVERLOAD PROTECTION			02/02/84
53403	SEQUOYAH 2 - ASME CODE RELIEF FOR HYDROSTATIC TESTING			02/02/84
52661	SEQUOYAH 1 - DG TIMERS			02/05/84
54214	SEQUOYAH UNIT 1 - RV MATERIAL ANALYSIS			03/00/84
54273	SEQUOYAH 1 - SLAP REVIEWS			03/03/84
54274	SEQUOYAH 2 - SALP REVIEWS			03/03/84
53334	SEQUOYAH-UNIT 1 - AFW PUMP ENDURANCE TESTS			03/03/84
52125	SEQUOYAH UNIT 1 - REVISION TO FUSE SURVEILLANCE REQUIREMENTS			03/10/84
52659	SEQUOYAH 1 - SNUBBER SPECIFICATIONS			05/28/84
51457	SEQUOYAH 1 - LOW TEMPERATURE OVERPRESSURE PROTECTION			05/30/84
51459	SEQUOYAH 1 - DIESEL GENERATOR RELIABILITY			05/30/84
47360	SEQUOYAH 1/2 - TECHNICAL SPECIFICATION REVISIONS		3	05/30/84
54277	SEQUOYAH 1 - BRITISH FUEL ASSEMBLIES			05/30/84
54278	SEQUOYAH 2 - BRITISH FUEL ASSEMBLIES			05/30/84
46917	SEQUOYAH 1 - CONTROL ROOM HUMAN FACTORS REVIEW			05/30/84
49171	SEQUOYAH 1 CONTAINMENT AIR TEMPERATURE AND PURGE LINES			06/27/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52134	SEQUOYAH UNIT 1 - CONTAINMENT ISOLATION VALVES			09/26/83
52128	SEQUOYAH UNIT 1 - PRESSURIZER SPRAY NOZZLE			01/00/84
54884	SEQUOYAH UNIT 1 - EMERGENCY EXERCISE EXEMPTION REQUEST			05/21/84
54275	SEQUOYAH 1 - IST REVIEW			06/30/84
54276	SEQUOYAH 2 - IST PROGRAM			06/30/84
52132	SEQUOYAH UNIT 1 - H2 IGNITER SURVEILLANCE			06/30/84
51463	SEQUOYAH 1 - ROD DROP PROTECTION			09/00/84
47065	SEQUOYAH 1/2 TASK INTERFACE AGREEMENT(81-15)		3	09/00/84
53494	SEQUOYAH 1 - CONTAINMENT PURGE LINES			09/00/84
52195	SEQUOYAH UNIT 1 - SUBCOOLING MARGIN MONITOR			09/09/84
54415	SEQUOYAH 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54569	SEQUOYAH 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49694	SEQUOYAH 1 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53633	SEQUOYAH 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53716	SEQUOYAH 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53799	SEQUOYAH 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53882	SEQUOYAH 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53938	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54026	SEQUOYAH 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84



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SEQUOYAH 1

TAC #    TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

54109 SEQUOYAH 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

INITIATION

11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: SEQUOYAH 2

PLANT LOCATION: 9.5 MI NE OF CHATTANOOGA, TN  
 DOCKET NUMBER: 050-00328  
 ARCH/ENGINEER: TVA  
 IE INSPECTOR: F. JAPE

LICENSED POWER: 3411 MWT  
 DESIGN POWER: 1148 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: C. STAHL  
 BRANCH CHIEF: E. ADENSAM  
 LIC. ASSISTANT: M. DUNCAN

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51790	SEQUOYAH 2 - NATURAL CIRCULATION COOLDOWN			<u>COMPLETE</u> 12/30/83
<u>ACTIVE ACTIONS</u>				
52272	SEQUOYAH 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52801	SEQUOYAH 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52882	SEQUOYAH 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52963	SEQUOYAH 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53044	SEQUOYAH 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53099	SEQUOYAH 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53152	SEQUOYAH 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53196	SEQUOYAH 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			09/15/83
51442	SEQUOYAH 2 - CONTROL OF HEAVY LOADS			09/30/83
49759	SEQUOYAH 2 -"NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			09/30/83
51134	SEQUOYAH 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			07/01/84
51528	SEQUOYAH 2 - REACTOR COOLANT PUMP TRIP (II.K.3.5)			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51449	SEQUOYAH 2 - RCS HIGH POINT VENTS (II.B.1)			<u>COMPLETE</u> 12/30/83
51455	SEQUOYAH 2 - EMERGENCY PLAN UPRAGE (III.A.2.1)			12/30/83
<u>ACTIVE ACTIONS</u>				
51456	SEQUOYAH 2 - FEEDBACK OF OPERATING EXPERIENCE (I.C.S)			<u>TARGET</u>
51450	SEQUOYAH 2 - POST ACCIDENT SAMPLING (II.B.3)			
51451	SEQUOYAH 2 - RV AND SV TESTING			
51452	SEQUOYAH 2 - IODINE/PARTICULATE (II.F.1.2)			
51453	SEQUOYAH 2 - SB LOCA OUTLINE (II.K.3.30)			
51454	SEQUOYAH 2 - TECHNICAL SUPPORT CENTER (III.A.1.2)			
51444	SEQUOYAH 2 - POTENTIAL FOR VOIDING IN RCS (II.K.2.17)			
51445	SEQUOYAH 2 - COMPLIANCE WITH 10CFR 50.46 (II.K.3.31)			
51447	SEQUOYAH 2 - INADEQUATE CORE COOLING GUIDELINES (I.C.1.2.A)			
51448	SEQUOYAH 2 - ABNORMAL TRANSIENT OPERATOR GUIDELINES (I.C.1.3)			
51204	SEQUOYAH 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/83
51284	SEQUOYAH 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/83
48205	SEQUOYAH 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46			09/30/83
49458	SEQUOYAH 2 - INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING (II.F.2)			09/30/83
54984	SEQUOYAH 2, II.K.2.13, THERMAL MECHANICAL REPORT			06/30/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51971	SEQUOYAH 2 - FUEL RELOAD			<u>COMPLETE</u> 12/12/83

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SEQUOYAH 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u> (CONTINUATION)				<u>COMPLETE</u>
51761	SEQUOYAH 2 - TEMPORARY TECH SPEC CHANGE TO PERFORM SHUTDOWN BOARD MODIFICATIONS			12/30/83
51762	SEQUOYAH 2 - VOLUME REDUCTION SYSTEM TO PROCESS LOW-LEVEL RADIOACTIVE WASTE			12/30/83
52126	SEQUOYAH UNIT 2 - REVISION TO FUSE SURVEILLANCE REQUIREMENTS			12/30/83
52131	SEQUOYAH UNIT 2 - DNB PARAMETER CHANGES			12/30/83
51458	SEQUOYAH 2 - LOW TEMPERATURE OVERPRESSURE PROTECTION			12/30/83
51460	SEQUOYAH 2 - DIESEL GENERATOR RELIABILITY			12/30/83
51465	SEQUOYAH 2 - ADMINISTRATIVE CHANGE TO RADIOLOGICAL EMERGENCY PLAN			12/30/83
51468	SEQUOYAH 2 - LIMITING CONDITION REVISION TO CO2 SYSTEM			12/30/83
51469	SEQUOYAH 2 - ADMINISTRATIVE CHANGE TO PHYSICAL SECURITY PLAN			12/30/83
51046	SEQUOYAH 2 - UHI ACCUMULATOR WATER LEVEL SETPOINT AND TOLERANCES			12/30/83
52662	SEQUOYAH 2 - DG TIMERS			03/03/84
53335	SEQUOYAH-UNIT 2 - AFW PUMP ENDURANCE TESTS			03/03/84
53392	SEQUOYAH 2 POST ACCIDENT SAMPLING			03/03/84
53394	SEQUOYAH 2 - MOV THERMAL OVERLOAD PROTECTION			03/03/84
53404	SEQUOYAH 1 - ASME CODE RELIEF FOR HYDROSTATIC TESTING			03/03/84
51440	SEQUOYAH 2 - ENVIRONMENTAL QUALIFICATION			05/05/84
46918	SEQUOYAH 2 - CONTROL ROOM HUMAN FACTORS REVIEW			05/30/84
51972	SEQUOYAH 2 - DG SURVEILLANCE REG'DS			05/30/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
48254	SEQUOYAH 2 - TECH. SPEC. CHANGES ON PLANT ORGANIZATION			07/01/83
52129	SEQUOYAH UNIT 2 - PRESSURIZER SPRAY NOZZLE			09/26/83
53495	SEQUOYAH 2 - CONTAINMENT PURGE LINES			03/15/84
52133	SEQUOYAH UNIT 2 - H2 IGNITER SURVEILLANCE			06/30/84
51441	SEQUOYAH 2 - R & D H2 CONTROL			06/30/84
52135	SEQUOYAH UNIT 2 - CONTAINMENT ISOLATION VALVES			09/09/84
52196	SEQUOYAH UNIT 2 - SUBCOOLING MARGIN MONITOR			09/09/84
52660	SEQUOYAH 2 - SNUBBER SPECIFICATIONS			09/09/84
49172	SEQUOYAH 2 CONTAINMENT AIR TEMPERATURE AND PURGE LINES			09/09/84
54416	SEQUOYAH 2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54570	SEQUOYAH 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53634	SEQUOYAH 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53717	SEQUOYAH 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53800	SEQUOYAH 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53883	SEQUOYAH 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53939	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54027	SEQUOYAH 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54110	SEQUOYAH 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: ST LUCIE 1

PLANT LOCATION: 12 MI SE OF FT. PIERCE, FLA  
 DOCKET NUMBER: 050-00335  
 ARCH/ENGINEER: EBASCO  
 IE INSPECTOR: C. FEIERABEND

LICENSED POWER: 2700 MWT  
 DESIGN POWER: 0830 MWE  
 NSSS VENDOR: COMB

PROJECT MANAGER: D. SELLS  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
12319	ST. LUCIE 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE ST. LUCIE 1		3	<u>COMPLETE</u> 01/31/84
49760	ST. LUCIE 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			03/02/84
51325	ST. LUCIE 1 - THERMAL SHIELD DAMAGE			03/14/84
<u>ACTIVE ACTIONS</u>				
52273	ST. LUCIE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
51135	ST. LUCIE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52802	ST. LUCIE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52883	ST. LUCIE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52964	ST. LUCIE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53045	ST. LUCIE 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53100	ST. LUCIE 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53153	ST. LUCIE 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
11108	ST. LUCIE 1 - FIRE PROTECTION SER SUPPLEMENT		1	05/01/84
43852	ST LUCIE 1 - EXEMPTION FROM APP. R SECT. 111G		1	05/01/84
43853	ST LUCIE 1 - EXEMPTION FROM APPENDIX R SECT. 111A		1	05/01/84
42900	ST. LUCIE 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	05/01/84
08292	ST LUCIE 1 - 10CFR 50.55 A(6) INSERVICE TESTING		1	05/01/84
08923	ST. LUCIE 1 - REVIEW OF ASYMETRIC LOCA LOADS SUBMITTAL		2	06/01/84
08083	ST LUCIE 1 -CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	06/01/84
42492	ST. LUCIE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	07/01/84
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
47816	ST. LUCIE 1- NUREG 0737 ITEM II.K.2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u> 10/05/83
45266	ST. LUCIE 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	10/06/83
45312	ST. LUCIE 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	10/06/83
46905	ST. LUCIE 1 - II.K.2.13 THERMAL MECHANICAL REPORT		1	06/06/84
44267	ST. LUCIE 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
45168	ST. LUCIE 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	<u>TARGET</u> 10/30/83
46339	ST. LUCIE 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	11/30/83
48200	ST. LUCIE 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	11/30/83
45861	ST. LUCIE 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	01/30/84
51205	ST. LUCIE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			06/01/84
51285	ST. LUCIE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			06/01/84
45980	ST. LUCIE 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	10/30/84
46051	ST. LUCIE 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	10/30/84
46123	ST. LUCIE 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	10/30/84
44617	ST LUCIE 1 - RV AND SV TESTING		1	10/30/84
44337	ST. LUCIE 1 - NUREG-0737 I.C.1.3.1, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ST LUCIE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
52556	ST LUCIE 1 - LICENSE AMENDMENT DEALING WITH MINIMUM SHIFT CREW COMPOSITION			02/24/84
49853	ST. LUCIE 1 - LIC. AMDT. FOR LINEAR HEAT RATE CHANGE			03/01/84
49483	ST. LUCIE 1 - LIC. AMDT. FOR CYCLE 6 RELOAD			03/01/84
48640	ST. LUCIE 1 - TRANSFER TO EXXON FUEL			03/01/84
52410	ST. LUCIE 1 - LICENSE AMENDMENT TO DELETE CONTAINMENT LEAKAGE PATH TEST FOR TAP 2 OF PENETRATION 25			03/14/84
52299	ST LUCIE 1 - LICENSE AMENDMENT TO MODIFY PEAKING FACTORS AND PEAKING PENALTIES			04/25/84
43069	ST. LUCIE 1 - FIRE PROTECTION TECH SPEC		3	05/01/84
53538	ST. LUCIE - LICENSE AMENDMENT TO DELETE APPENDIX B-PART 1 FROM THE TECHNICAL SPECIFICATIONS			05/16/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
55046	ST. LUCIE 1 AMENDS TECHNICAL SPECIFICATIONS TO MAKE THEM MORE CONSISTENT WITH UNIT NO. 2			10/30/83
51044	ST LUCIE 1 - LIC AMDT FOR RELIEF FROM TS REQmnt			11/01/83
52031	ST. LUCIE 1 - CONTAINMENT PURGE VALVE OPERABILITY			11/15/83
48641	ST. LUCIE 1 : EXEMPTION FROM APPENDIX R, III G SCHEDULAR			12/15/83
48924	ST. LUCIE 1 - DIESEL GENERATOR SURVEILLANCE REQ.			01/16/84
52558	ST. LUCIE 1 - ALTERNATE SHUTDOWN SUBMITTAL.			01/30/84
48795	ST. LUCIE 1 - REANALYSES OF RCP SEIZED ROTOR AND LOSS OF ALL NON-EMERGENCY AC POWER			06/01/84
48622	ST. LUCIE 1 - 10 CFR 50.55 A(6) - INSERVICE INSPECTION		1	06/01/84
43241	ST. LUCIE 1 - STATION BLACKOUT		1	06/01/84
54417	ST. LUCIE 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 0737 NUREG 073 ITEM II.B.1			09/30/84
54571	ST. LUCIE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			02/01/85
49693	ST. LUCIE 1 - REACTOR COOLANT PUMP TRIP			
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53635	ST. LUCIE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53718	ST. LUCIE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53801	ST. LUCIE 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53884	ST. LUCIE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53940	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54028	ST. LUCIE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54111	ST. LUCIE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: ST LUCIE 2

PLANT LOCATION: 12 MI SE OF FT. PIERCE, FLA  
 DOCKET NUMBER: 050-00389  
 ARCH/ENGINEER: EBASCO  
 IE INSPECTOR: C. FEIERABEND

LICENSED POWER: 2560 MWT  
 DESIGN POWER: 0804 MWE  
 NSSS VENDOR: COMB

PROJECT MANAGER: D. SELLS  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
52274	ST. LUCIE 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			<u>TARGET</u>
52803	ST. LUCIE 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52884	ST. LUCIE 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52965	ST. LUCIE 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53046	ST. LUCIE 2 - ITEM 3.1.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH SPECS - RTS COMPONENTS			
53101	ST. LUCIE 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53154	ST. LUCIE 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
51136	ST. LUCIE 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
54982	ST. LUCIE 2, II.K.2.13, THERMAL MECHANICAL REPORT			<u>COMPLETE</u> 06/06/84
<u>ACTIVE ACTIONS</u>				
51605	ST. LUCIE 2 - RV AND SV TESTING			<u>TARGET</u>
51206	ST. LUCIE 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51286	ST. LUCIE 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51493	ST. LUCIE 2 - INSERVICE INSPECTION PROGRAM			<u>COMPLETE</u> 10/06/83
51782	ST. LUCIE 2 - TURBINE OVERSPEED PROTECTION: VALVE TESTING CYCLES			12/20/83
51491	ST. LUCIE 2 - REVIEW DETAILED CONTROL RM. DESIGN REPORT			02/09/84
52557	ST. LUCIE 2 - LICENSE AMENDMENT DEALING WITH MINIMUM SHIFT CREW COMPOSITION			02/24/84
51421	ST. LUCIE 2 - FLAME IMPINGEMENT SHIELDS INSIDE CONTAINMENT			03/27/84
51532	ST. LUCIE 2 - BARRIER FOR HIGH ENERGY EQUIP			03/27/84
54266	ST. LUCIE 2 - LICENSE AMDT TO MODIFY ISI REQUIREMENT FOR TWO MECHANICAL SNUBBERS			05/03/84
51422	ST. LUCIE 2 - STEAM GENERATOR PRESSURE/TEMPERATURE LIMITATION			06/23/84
<u>ACTIVE ACTIONS</u>				
52559	ST. LUCIE 2 - INSERVICE INSPECTION AND TESTING FOR CLASS 1,2, AND 3 COMPONENTS			<u>TARGET</u>
55044	MAINE YANKEE, NEUTRON SOURCE DATA FOR FLUX REDUCTION VERDICATION			
55151	ST. LUCIE 2: AMENDS TECHNICAL SPECIFICATIONS AS A RESULT OF CYCLE 2 RELOAD/REFUELING			
51474	ST. LUCIE 2 - EVALUATE FP & L REPORT ON COLD SHUTDOWN REQR.			05/06/83
51488	ST. LUCIE 2 - ENVIRONMENTAL QUAL OF MECH. & ELECT. EQUIP			05/20/83
51489	ST. LUCIE 2 - REVIEW AND APPROVED REVISED PHYS. SECURITY PLAN			05/20/83
51492	ST. LUCIE 2 - REVIEW ANAL. OF CONTINUOUS CONT. PURGE SYST.			05/20/83
51864	ST. LUCIE 2 - TS ADMINISTRATION AND EDITORIAL CHANGES - I L-83-349, 6/17/83			08/31/83
51530	ST. LUCIE 2 - EMERGENCY RESPONSE CAPABILITY			10/30/83
51529	ST. LUCIE 2 - PASS CORE DAMAGE EST. PROCEDURE			12/30/83
51490	ST. LUCIE 2 - REVIEW INADEQUATE CORE COOL. REPT.			03/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ST LUCIE 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
53387	ST LUCIE 2 - AMENDS FIGURES IN TECHNICAL SPECIFICATIONS ASSOCIATED WITH AXIAL SHAPE INDEX			04/15/84
54513	ST. LUCIE 2 - PRESSURIZER HEATER TRANSFORMER BARRIER DESIGN			05/25/84
53428	ST. LUCIE 2 - FIRE PROTECTION, COMPLIANCE WITH APPENDIX R			06/01/84
54485	ST. LUCIE - 2, BORON MIXING TEST REPORT - CEN-259			06/15/84
54745	ST LUCIE 2 - TURBINE DISC INTEGRITY			06/30/84
54746	ST. LUCIE 2 - FIRE PROTECTION FEATURES			07/31/84
54463	ST. LUCIE 2 - LICENSE AMDT. TO RERACK THE SPENT FUEL POOL TO INCREASE THE SPENT FUEL STORAGE CAPACITY			08/10/84
54572	ST. LUCIE 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
51533	ST. LUCIE 2 - AXIAL GROWTH			12/30/84
51531	ST. LUCIE 2 - HEAVY LOADS			12/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53636	ST. LUCIE 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53719	ST. LUCIE 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53802	ST. LUCIE 2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53885	ST. LUCIE 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53941	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54029	ST. LUCIE 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54112	ST. LUCIE 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: SUMMER 1

PLANT LOCATION: 26 MI NW OF COLUMBIA, SC  
 DOCKET NUMBER: 050-00395  
 ARCH/ENGINEER: GIL  
 IE INSPECTOR:

LICENSED POWER: 2775 MWT  
 DESIGN POWER: 0900 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: J. HOPKINS  
 BRANCH CHIEF: E. ADENSAM  
 LIC. ASSISTANT: M. DUNCAN

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
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COMPLETED ACTIONS

53197	SUMMER 1 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			<u>COMPLETE</u> 11/23/83
49761	SUMMER 1 - NUREG 0737 TECH SPECS (GENERIC LTR 82-16)			04/20/84

ACTIVE ACTIONS

51137	SUMMER 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52275	SUMMER 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52376	SUMMER 1 - CONTROL OF HEAVY LOADS (PHASE I)			
52804	SUMMER 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52885	SUMMER 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52966	SUMMER 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53047	SUMMER 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53102	SUMMER 1 - REACTOR TRIP SYSTEM RELIABILITY- VENDOR RELATED MODIFICATIONS			
53155	SUMMER 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
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COMPLETED ACTIONS

53549	V.C. SUMMER - VOIDING IN THE RCS NUREG-0737 II.K.2.17			<u>COMPLETE</u> 02/15/84
54983	SUMMER 1, II.K.2.13, THERMAL MECHANICAL REPORT			06/22/84

ACTIVE ACTIONS

51207	SUMMER 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51287	SUMMER 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
51606	SUMMER 1 - RV AND SV TESTING			
49464	SUMMER 1 - INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING (II.F.2)			09/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
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COMPLETED ACTIONS

51356	V.C. SUMMER - FIRE SUPPRESSION SYSTEMS			<u>COMPLETE</u> 10/12/83
52054	V.C. SUMMER - ORGANIZATIONAL CHANGES			10/31/83
49975	V.C. SUMMER - SEISMIC CONFIRMATORY PROGRAM			11/16/83
52093	SUMMER 1 - RELIEF FROM ASME CODE/SECTION XI HYDROSTATIC TESTING REQUIREMENTS FOR SUMMER #1 FEEDWATER SYSTEM			11/21/83
49208	V.C. SUMMER - ATTCH. 5 MISC			12/23/83
52057	V.C. SUMMER - RADIATION MONITORS - T.S. REQTS			01/05/84
52058	V.C. SUMMER - T.S. SURVEILLANCE - SERVICE WATER			02/03/84
53553	V.C. SUMMER - GL81-27 REMINDER			02/06/84
53552	V.C. SUMMER - EMERG. PLANNING EXERCISE EXEMPTION			03/20/84
52055	V.C. SUMMER - T.S. SURVEILLANCE TENDONS			04/16/84
53281	V.C. SUMMER - FWIV TECH. SPECS.			04/16/84
52338	V.C. SUMMER - CONTROL ROOM AIR FLOW - T.S 3.7.6			04/25/84
51630	V.C. SUMMER - 25 MILE BACKUP EOF			05/09/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SUMMER 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS (CONTINUATION)</u>				<u>COMPLETE</u>
51568	V. C. SUMMER - REACTOR TRIP BREAKERS			05/15/84
49539	V. C. SUMMER - REQUALIFICATION PROGRAM			05/30/84
54899	V. C. SUMMER - STM. GEN. INSPECTION PORTS (L.C. 13)			06/07/84
52059	V. C. SUMMER - GL 82-33 RESPONSE			06/15/84
55159	V. C. SUMMER - SHORT DURATION ILRT			05/25/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
55216	V. C. SUMMER - DIESEL GENERATORS - L.C.2.C.(19)			
55298	V. C. SUMMER - RCS FLOW			
55095	V. C. SUMMER - TURBINE/RX TRIP; P-9 INTERLOCK			
55096	V. C. SUMMER - OPERATOR REQUAL. PROCEDURE REVIEW			
53551	V. C. SUMMER - GL83-37 RESPONSE - HYDROGEN MONITOR T.S.			
53317	V. C. SUMMER - SPENT FUEL POOL RERACK AND NEW 4.3% ENRICHED FUEL RACKS			
53406	V. C. SUMMER - AUTO SHUT TRIP TECH SPECS			
52413	V. C. SUMMER - SEISMIC MONITORING PROGRAM			
53550	V. C. SUMMER - RX BLDG SUMP ISO VALVES-CONT. PENET. COND. OVERCURRENT PROT. DEVICES			
49976	V. C. SUMMER - ISI/PUMP RELIEF REQUESTS			06/30/84
52056	V. C. SUMMER - OVERPOWER & OVERTEMP. DELTA T SETPTS			07/30/84
52124	V. C. SUMMER - L. C. (10) LOW TEMP. OVER PRESS. PROT. SYS.			07/30/84
49203	V. C. SUMMER - SW INTAKE STRUCTURE			07/30/84
53282	V. C. SUMMER - T.S. MICH. SNUBBERS			08/30/84
53283	V. C. SUMMER - RBCU FAN MOTORS EDDY CURR TENT BRAKE- CONT. PENET. COND. OVER CURRENT PROT. DEVICES			08/30/84
54467	V. C. SUMMER - RHR SYS. POWER LOCKOUTS (L.C. 15)			08/31/84
49209	V. C. SUMMER - TYPE C TESTS			12/30/84
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
55217	V. C. SUMMER - THERMAL SLEEVES, L.C.2.C.(7)			08/06/84
53637	SUMMER 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53720	SUMMER 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53803	SUMMER 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53886	SUMMER 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53942	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54030	SUMMER 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54113	SUMMER 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: SURRY 1

PLANT LOCATION: 17 MI NW OF NEWPORT NEWS, VA  
 DOCKET NUMBER: 050-00280  
 ARCH/ENGINEER: SAW  
 IE INSPECTOR: D. BURKE

LICENSED POWER: 2441 MWT  
 DESIGN POWER: 0788 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: D. NEIGHBORS  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12976	SURRY 1 - ADEQUACY OF STATION ELECTRIC. DISTR. SYS VOLTAGES		2	<u>COMPLETE</u> 10/14/83
47167	SURRY 1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	10/19/83
08617	SURRY 1 - FILTER TECH SPECS		3	01/17/84
11091	SURRY 1 - FIRE PROTECTION SER SUPPLEMENT		1	01/17/84
08521	SURRY 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
12277	SURRY 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE SURRY 1		3	02/02/84
11974	SURRY 1 - REACTOR CAVITY SEAL RING GENERIC ISSUE		2	02/10/84
49762	SURRY 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			03/28/84
08084	SURRY 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	05/16/84
08102	SURRY 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/19/84

TAC #	TAC DESCRIPTION	IMT ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
52805	SURRY 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52886	SURRY 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52967	SURRY 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53048	SURRY 1 - ITEM 3.1.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH SPECS - RTS COMPONENTS			
53103	SURRY 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53156	SURRY 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDRING			
53198	AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42468	SURRY 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/31/83
08620	SURRY 1 - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT		2	02/01/84
51138	SURRY 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/01/84
42867	SURRY 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/31/84
52276	SURRY 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85

TAC #	TAC DESCRIPTION	IMT ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47978	SURRY 1 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	<u>COMPLETE</u> 10/31/83
47821	SURRY 1 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/09/84
44480	SURRY 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/27/84
46907	SURRY 1 - II K2.13 THERMAL MECHANICAL REPORT		1	06/15/84

TAC #	TAC DESCRIPTION	IMT ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
48206	SURRY 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	<u>TARGET</u> 12/31/83
45173	SURRY 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/31/83
45866	SURRY 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/31/83
44622	SURRY 1 - RV AND SV TESTING		1	03/31/84
45985	SURRY 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46056	SURRY 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
51208	SURRY 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/84
51288	SURRY 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
46128	SURRY 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SURRY 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
46344	SURRY 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	<u>TARGET</u> 09/30/84
44272	SURRY 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44342	SURRY 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84
<u>PLANT SPECIFIC</u>				
<u>TAC #</u>	<u>TAC DESCRIPTION</u>		<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
51369	SURRY 1 - CONTAINMENT ISOLATION VALVES TECH SPECS			<u>COMPLETE</u> 11/14/83
48968	SURRY 1 - ISI SECOND TEN YEAR INTERVAL			01/24/84
52293	SURRY 1 - REVISE SNUBBER TECH SPECS			01/31/84
52295	SURRY 1 - ADD HEALTH PHYSICS SUPERVISOR TO T.S.6.4.B.1.B			01/31/84
52358	SURRY 1 - REDUCE BORDON CONCENTRATION IN BIT			02/24/84
49552	SURRY 1 - ADDENDUM TO FIRST TEN YEAR ISI - RELIEF REQUEST			02/28/84
53260	SURRY 1 - EXTEND BURNUP TO 45,000 MWD/MTU			04/09/84
43115	SURRY 1 - FIRE PUMP TECH SPEC 4.18.B.1.F(2)		1	04/20/84
54626	SURRY 1 - ASME XI RELIEF (3/26/84) -HYDROSTATIC TEST			05/17/84
<u>ACTIVE ACTIONS</u>				
53535	SURRY 1 - REGULATORY EFFECTIVENESS REVIEW			<u>TARGET</u> 12/31/83
51371	SURRY 1 - REACTOR VESSEL COOLANT LEVEL MONITOR			02/01/84
48118	SURRY 1 - REORGANIZATION			03/01/84
49144	SURRY 1 - 1ST SECOND TEN YEAR INTERVAL			03/15/84
43113	SURRY 1 - HIGH ENERGY BREAK INSPECTION PROGRAM T.S. 4.15-A		1	04/01/84
53533	SURRY 1 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES (FOLLOWUP ON B-48)			04/01/84
53359	SURRY 1 - ASME XI RELIEF REQUEST (12/83)			04/01/84
49493	SURRY 1 - CHARGING PUMP TECH SPECS			06/01/84
54847	SURRY 1 - CORE RELOAD METHODOLOGY AUDIT			06/01/84
54742	SURRY 1 - AFMS - CHANGE OPERABLE TO AVAILABLE			06/30/84
53492	SURRY 1 - LIFT ORDER TO SHOW CAUSE DATED MARCH 13, 1979			06/30/84
55065	SURRY 1 - EMERGENCY EXERCISE EXEMPTION			07/01/84
54418	SURRY 1 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54573	SURRY 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49148	SURRY 1 - DRY CASK INDEPENDENT SPENT FUEL STORAGE INSTALLATION			12/31/84
49681	SURRY 1 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				
53638	SURRY 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53721	SURRY 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53804	SURRY 1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53887	SURRY 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53943	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54031	SURRY 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54114	SURRY 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: SURRY 2

PLANT LOCATION: 17 MI NW OF NEWPORT NEWS, VA  
 DOCKET NUMBER: 050-00281  
 ARCH/ENGINEER: S&W  
 IE INSPECTOR: D. BURKE

LICENSED POWER: 2441 MWT  
 DESIGN POWER: 0788 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: D. NEIGHBORS  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
12977	SURRY 2 - ADEQUACY OF STATION ELECTRIC. DISTR. SYS. VOLTAGES		2	<u>COMPLETE</u> 10/14/83
47168	SURRY 2 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	10/19/83
11092	SURRY 2 - FIRE PROTECTION SER SUPPLEMENT		1	01/17/84
08616	SURRY 2 - FILTER TECH SPECS		3	01/17/84
08520	SURRY 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
12278	SURRY 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE SURRY 2		3	02/02/84
11975	SURRY 2 - REACTOR CAVITY SEAL RING GENERIC ISSUE		2	02/10/84
49763	SURRY 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"		2	03/28/84
03085	SURRY 2 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	05/16/84
08103	SURRY 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	06/19/84
<u>ACTIVE ACTIONS</u>				
52806	SURRY 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
52887	SURRY 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52968	SURRY 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53049	SURRY 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53104	SURRY 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53157	SURRY 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53199	SURRY 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42469	SURRY 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	12/31/83
08619	SURRY 2 - FUEL HANDLING ACCIDENT INSIDE CONTAINMENT		2	02/01/84
51139	SURRY 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			06/01/84
42868	SURRY 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	08/31/84
52277	SURRY 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		1	09/30/85
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
47979	SURRY 2 - PLANT SHIELDING MODIFICATIONS (II.B.2.2)		1	<u>COMPLETE</u> 10/31/83
47822	SURRY 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/09/84
44481	SURRY 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	02/27/84
46908	SURRY 2 - II K2.13 THERMAL MECHANICAL REPORT		1	06/15/84
44273	SURRY 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/28/84
<u>ACTIVE ACTIONS</u>				
44343	SURRY 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	<u>TARGET</u> 08/15/83
45174	SURRY 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	12/31/83
45867	SURRY 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	12/31/83
48207	SURRY 2 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	12/31/83
44623	SURRY 2 - RV AND SV TESTING		1	03/31/84
45986	SURRY 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46057	SURRY 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46129	SURRY 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SURRY 2

TAC #	TAC DESCRIPTION	IMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
46345	SURRY 2 - NUREG-0737 III.A.2.2. METEOROLOGICAL DATA UPGRADE		1	TARGET 09/30/84
51209	SURRY 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			09/30/84
51239	SURRY 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			09/30/84
<u>PLANT SPECIFIC</u>				
<u>COMPLETED ACTIONS</u>				
51370	SURRY 2 - CONTAINMENT ISOLATION VALVES TECH SPECS			COMPLETE 11/14/83
52294	SURRY 2 - REVISE SNUBBER TECH SPECS			01/31/84
52296	SURRY 2 - ADD HEALTH PHYSICS SUPERVISOR TO TECH SPEC 6.4.B.1.B			01/31/84
52359	SURRY 2 - REDUCE BORON CONCENTRATION IN BIT			02/24/84
49553	SURRY 2 - ADDENDUM TO FIRST TEN YEAR ISS - REL. OF REQUEST			02/28/84
53261	SURRY 2 - EXTEND BURNUP TO 45,000 MWD/MTU			04/09/84
43116	SURRY 2 - FIRE PUMP TECH SPEC 4.18.B.1.F(2)		1	04/20/84
51887	SURRY 2 - RELIEF FROM IWA - 24--(A) OF ASME SECTION XI			05/07/84
54799	SURRY 2 - ASME XI RELIEF (4/12/84) - HYDROSTATIC TEST			05/17/84
54848	SURRY 2 - CORE RELOAD METHODOLOGY AUDIT			06/19/84
<u>ACTIVE ACTIONS</u>				
53536	SURRY 2 - REGULATORY EFFECTIVENESS REVIEW			TARGET 12/31/83
51372	SURRY 2 - REACTOR VESSEL COOLANT LEVEL MONITOR			02/01/84
48119	SURRY 2 - REORGANIZATION			03/01/84
49143	SURRY 2 - ISI - SECOND TEN YEAR INTERVAL			03/01/84
49145	SURRY 2 - IST - SECOND TEN YEAR INTERVAL			03/01/84
43114	SURRY 2 - HIGH ENERGY BREAK INSPECTION PROGRAM TECH SPEC 4.15-A		1	03/01/84
53360	SURRY 2 - ASME XI RELIEF REQUEST (12/83)			04/01/84
53534	SURRY 2 - ADEQUACY OF STATION ELECTRIC DISTRIBUTION SYSTEM VOLTAGES (FOLLOWUP OF B-48)			04/01/84
49494	SURRY 2 - CHARGING PUMP TECH SPECS			06/01/84
54743	SURRY 2 - AFWS - CHANGE OPERABLE TO AVAILABLE			06/30/84
55066	SURRY 2 - EMERGENCY EXERCISE EXEMPTION			07/01/84
54419	SURRY 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54574	SURRY 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
49149	SURRY 2 - DRY CASK INDEPENDENT SPENT FUEL STORAGE INSTALLATION			12/31/84
49682	SURRY 2 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				
53639	SURRY 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53722	SURRY 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53805	SURRY 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53888	SURRY 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53944	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54032	SURRY 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54115	SURRY 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: SUSQUEHANNA 1

PLANT LOCATION: 7 MI NE OF BERWICK, PA  
 DOCKET NUMBER: 050-00387  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: G. RHOADES

LICENSED POWER: 3293 MWT  
 DESIGN POWER: 1065 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. PERCH  
 BRANCH CHIEF: A. SCHWENCER  
 LIC. ASSISTANT: E. HYLTON

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
51360	SUSQUEHANNA 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52278	SUSQUEHANNA 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52807	SUSQUEHANNA 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52888	SUSQUEHANNA 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52969	SUSQUEHANNA 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53050	SUSQUEHANNA 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
53321	MCGUIRE UNIT 2: NUREG-C737, II.F.2.3 INADEQUATE CORE COOLING INSTRUMENTATION; REF: TAC #45146			<u>COMPLETE</u> 03/30/84
<u>ACTIVE ACTIONS</u>				
54584	SUSQUEHANNA 1 - II.D.1 - RV & SV TESTING			<u>TARGET</u>
51210	SUSQUEHANNA 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			
51290	SUSQUEHANNA 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
49461	SUSQUEHANNA 1 - INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING (II.F.2)			
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49595	SUSQUEHANNA 1 - REVISION 6 TO EMERGENCY PLAN			<u>COMPLETE</u> 10/31/83
49642	SUSQUEHANNA 1 - EXPAND POWER/FLOW MAP OPERATING AREA			11/02/83
49914	SUSQUEHANNA 1 - FIRE DETECTION INSTRUMENTATION T/S TABLE 3.3.7.9-1			11/02/83
52496	SUSQUEHANNA 1 - PRIMARY CONTAINMENT ISOLATION VALVE SIGNALS - RHR/RWCU			11/18/83
52585	SUSQUEHANNA 1 - ESW PUMP LOAD SEQUENCE TIMERS			12/12/83
52151	SUSQUEHANNA 1 - MONTHLY GRAB SAMPLES			12/12/83
52150	SUSQUEHANNA 1 - FIRE HOSE STATIONS			12/12/83
54157	SUSQUEHANNA 1 - EMERGENCY PLANNING EXERCISE - EXEMPTION REQUEST			03/21/84
53325	SUSQUEHANNA-1 - STANDBY GAS TREATMENT SYSTEM - TECHNICAL SPECIFICATION 3.6.5 AND 4.6.5			03/23/84
53232	SUSQUEHANNA 1 - SRV INPLANT TEST			03/23/84
54633	SUSQUEHANNA 1 - EXEMPTION FROM STATE AND LOCAL PARTICIPATION IN ANNUAL EMERGENCY DRILL			04/24/84
51980	SUSQUEHANNA 1 - PROPOSED TRANSCO PIPELINE			04/27/84
51716	SUSQUEHANNA 1 - SIMULATED LOSS OF OFFSITE POWER TEST			05/14/84
51958	SUSQUEHANNA 1 - LICENSE AMENDMENT TO INCLUDE CHANGES TO PHYSICAL SECURITY PLAN			05/14/84
53323	SUSQUEHANNA-1 - L/C 2.C.28 (B) IMI ITEM I.G.1			05/14/84
52495	SUSQUEHANNA 1 - SNUBBER INSERVICE VISUAL INSPECTION			05/23/84
52586	SUSQUEHANNA 1 - ULTIMATE HEAT SINK T/S 4.7.1.3			05/23/84
52717	SUSQUEHANNA 1 - PRIMARY CONTAINMENT ISOLATION VALVE SIGNALS - SHUTDOWN COOLING			05/23/84
51983	SUSQUEHANNA 1 - COMMON ELECTRICAL POWER SOURCES RE MULTIPLE CONTROL SYSTEMS FAILURE			05/25/84
54614	SUSQUEHANNA 1 - HIGH ENERGY LINE BREAKS			05/25/84
49256	SUSQUEHANNA 1 - SAMPLE TEST OF PROTECTIVE FUSES			06/04/84
48743	SUSQUEHANNA 1 - PROPOSED AMENDMENT 1 TO NPF-14			06/04/84
49878	SUSQUEHANNA 1 - CHANGE TO DIESEL GENERATOR SURVEILLANCE REQUIREMENT			06/04/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

SUSQUEHANNA 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
				<u>TARGET</u>
<u>ACTIVE ACTIONS</u>				
49137	SUSQUEHANNA 1 - MOLDED CASE CIRCUIT BREAKERS			
55132	SUSQUEHANNA 1 - CONTAINMENT SYSTEMS - PROPOSED AMENDMENT 43			
55133	SUSQUEHANNA 1 - PLANT SYSTEMS - PROPOSED AMENDMENT 43			
55134	SUSQUEHANNA 1 - ELECTRICAL POWER SYSTEMS PROPOSED AMENDMENT 43			
53324	SUSQUEHANNA-1; REPORTING REQUIREMENTS L/C 2.G.			
53564	SUSQUEHANNA 1 - REVISION 7 TO EMERGENCY PLAN			
52149	SUSQUEHANNA 1 - EQUIPMENT QUALIFICATION FOR SDV PIPE BREAK ENVIRONMENT			
55135	SUSQUEHANNA 1 - REFUELING OPERATIONS - PROPOSED AMENDMENT 43			
55136	SUSQUEHANNA 1 - SPECIAL TEST EXCEPTIONS - PROPOSED AMENDMENT 43			
55137	SUSQUEHANNA 1 - RADIOACTIVE EFFLUENTS - PROPOSED AMENDMENT 43			
55138	SUSQUEHANNA 1 - RADIOLOGICAL ENVIRONMENTAL MONITORING - PROPOSED AMENDMENT 43			
55139	SUSQUEHANNA 1 - DESIGN FEATURES - PROPOSED AMENDMENT 43			
49596	SUSQUEHANNA 1 - REVISION 31 TO FS R			
49597	SUSQUEHANNA 1 - REVISION 32 TO FSAR			
52048	SUSQUEHANNA 1 - MAIN STEAM LINE RADIATION - HIGH SETPOINT			
51932	SUSQUEHANNA 1 - INSERVICE INSPECTION PROGRAM			
51508	SUSQUEHANNA 1 - REACTOR VESSEL MATERIAL SURVEILLANCE PROGRAM - WITHDRAWAL SCHEDULE			
54755	SUSQUEHANNA-1 - SPRAY POND OPERATING LIMITS			
54949	SUSQUEHANNA 1 - PHYSICAL SECURITY PLAN - CHANGE S			
54967	SUSQUEHANNA-1 FEEDWATER BYPASS LEAKAGE			
55021	SUSQUEHANNA 1 - THERMAL HYDRAULIC STABILITY			
55038	SUSQUEHANNA 1 - EXEMPTION TO GENERAL DESIGN CRITERIA 56			
55124	SUSQUEHANNA 1 - SPECIAL REPORTING REQUIREMENTS - PROPOSED AMENDMENT 41			
55125	SUSQUEHANNA 1 - ADMIN CHANGES TO TECH SPECS PROPOSED AMENDMENT 42			
55126	SUSQUEHANNA 1 - DEFINITIONS - PROPOSED AMENDMENT 43			
55127	SUSQUEHANNA 1 - SAFETY LIMITS AND LIMITING SAFETY SYSTEM SETTINGS - PROPOSED AMENDMENT 43			
55128	SUSQUEHANNA 1 - REACTIVITY CONTROL SYSTEMS - PROPOSED AMENDMENT 43			
55129	SUSQUEHANNA 1 - INSTRUMENTATION - PROPOSED AMENDMENT 43			
55130	SUSQUEHANNA 1 - REACTOR COOLANT SYSTEM - PROPOSED AMENDMENT 43			
55131	SUSQUEHANNA 1 - EMERGENCY CORE COOLING SYSTEMS - PROPOSED AMENDMENT 43			
<u>ANTICIPATED ACTIONS</u>				
53640	SUSQUEHANNA 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53723	SUSQUEHANNA 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53806	SUSQUEHANNA 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53889	SUSQUEHANNA 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54033	SUSQUEHANNA 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54116	SUSQUEHANNA 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: SUSQUEHANNA 2

PLANT LOCATION: 7 MI NE OF BERWICK, PA  
 DOCKET NUMBER: 050-00388  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: G. RHOADES

LICENSED POWER: 000 MW  
 DESIGN POWER: 1052 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. PERCH  
 BRANCH CHIEF: A. SCHWENCER  
 LIC. ASSISTANT: E. HYLTON

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
54585	SUSQUEHANNA 2 - II.D.1 - RV & SV TESTING			<u>TARGET</u>
55275	SUSQUEHANNA 2 - I.D.1 CONTROL ROOM DESIGN REVIEW - MPA F-08			

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
54634	SUSQUEHANNA 2 - EXEMPTION FROM STATE AND LOCAL PARTICIPATION IN ANNUAL EMERGENCY DRILL			<u>COMPLETE</u>
54765	SUSQUEHANNA 2 - INITIAL TEST PROGRAM			04/24/84
54612	SUSQUEHANNA 2 - COMMON ELECTRICAL POWER SOURCES RE MULTIPLE CONTROL SYSTEMS FAILURE			05/17/84
54613	SUSQUEHANNA 2 - HIGH ENERGY LINE BREAKS			05/25/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
54950	SUSQUEHANNA 2 - PHYSICAL SECURITY PLAN - CHANGE 5			<u>TARGET</u>
54968	SUSQUEHANNA-2 - FEEDWATER BYPASS LEAKAGE			
55022	SUSQUEHANNA 2 - THERMAL HYDRAULIC STABILITY			
55089	SUSQUEHANNA - EXEMPTION TO GENERAL DESIGN CRITERIA 56			
54754	SUSQUEHANNA - 2, SPRAY POND OPERATING LIMITS			
54756	SUSQUEHANNA-2 - NITROGEN MAKEUP SYSTEM			



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## CONDENSED MANAGEMENT REPORT

FACILITY: THREE MILE ISLAND 1

PLANT LOCATION: 10 MI SE OF HARRISBURG, PA  
 DOCKET NUMBER: 050-00289  
 ARCH/ENGINEER: GIL  
 IE INSPECTOR: R. CONTE

LICENSED POWER: 2535 MW  
 DESIGN POWER: 0819 MWE  
 NSSS VENDOR: B&W

PROJECT MANAGER: J. VANVLIT  
 BRANCH CHIEF: J. STOLZ  
 LIC. ASSISTANT: R. INGRAM

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
51733	THREE MILE ISLAND 1 - NUREG-0630 CLADDING MODELS FOR LOCA ANALYSIS			<u>COMPLETE</u> 10/23/83
49615	TMI-1 - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION			12/27/83
42602	THREE MILE ISLAND 1 - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	01/27/84
08603	THREE MILE ISLAND 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	03/27/84
49764	TMI-1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			05/10/84
47157	TMI-1 NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)		2	06/05/84

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
11286	THREE MILE ISLAND 1 - TS FOR HYDRAULIC SNUBBERS		3	<u>TARGET</u>
42127	THREE MILE ISLAND 1 - DECAY HEAT REMOVAL CAPABILITY TECH SPECS		2	
51361	TMI-1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52808	TMI-1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52889	TMI-1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52970	TMI-1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53051	TMI-1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53105	TMI-1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53158	TMI-1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53200	TMI-1 - ITEM 4.3 - AUTOAMIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
11238	THREE MILE ISLAND 1 - FIRE PROTECTION SER SUPPLEMENT		1	05/15/84
42513	THREE MILE ISLAND 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	05/30/84
47043	THREE MILE ISLAND 1 - TECHNICAL SPECIFICATIONS DEFINING OPERABILITY FOR SAFETY SYSTEMS		2	06/15/84
47131	TMI-1 - CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANTS (NUREG 0612)		2	09/28/84
52279	TMI-1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/23/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
44411	THREE MILE ISLAND 1 - NUREG-0737 II.B.1, RCS HIGH POINT VENTS		1	<u>COMPLETE</u> 10/17/83
45274	THREE MILE ISLAND 1 - NUREG-0737 II.K.3.1, AUTO PORV ISOLATION		1	03/05/84
40320	THREE MILE ISLAND 1 - NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES		1	03/05/84
46274	THREE MILE ISLAND 1- III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET RULE		1	03/09/84
48284	THREE MILE ISLAND 1 - NUREG-0737 II.B.3.2 POST ACCIDENT SAMPLING		1	03/22/84
48283	THREE MILE ISLAND 1 - NUREG-0737 I.C.1.3.A. ATOG		1	03/28/84
47641	THREE MILE ISLAND 1 - NUREG 0737 ITEM II F1.4 CONTAINMENT PRESSURE INSTRUMENT		1	04/20/84
47712	THREE MILE IS. 1 - NUREG 0737 ITEM II F1.5 CONTAINMENT WATER LEVEL MONITOR		1	04/20/84
47783	THREE MILE IS. 1 - NUREG 0737 ITEM II F1.6 CONTAINMENT HYDROGEN MONITOR		1	04/20/84
54979	TMI-1, II.K.2.13 FOLLOW ON, THERMAL MECH. RPT (GENERIC REVIEW)			06/05/84

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				
51211	TMI-1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51291	TMI-1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
48286	THREE MILE ISLAND 1 - NUREG-0737 II.K.3.30 SBLOCA METHODS		1	

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

THREE MILE ISLAND 1

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
<u>(CONTINUATION)</u>				
46287	THREE MILE ISLAND 1 - NUREG-0737 III.A.2.2 METEOROLOGICAL DATA UPGRADE		1	
45987	THREE MILE ISLAND 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	
46058	THREE MILE ISLAND 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	
46130	THREE MILE ISLAND 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	
47049	THREE MILE ISLAND 1 - NUREG-0737 III.D.3.4 CONTROL ROOM HABITABILITY		1	
44626	THREE MILE ISLAND 1 - RV AND SV TESTING		1	05/30/84
48208	THREE MILE IS. 1 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	08/30/84
47048	THREE MILE ISLAND 1 - NUREG-0737 II.F.2 INST. TO DETECT ICC		1	09/28/84
TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
52008	THREE MILE ISLAND 1 - EFW FLOW INDICATION REVIEW (CERTIFICATION ITEM)			10/01/83
47894	THREE MILE ISLAND 1 - TSCR 109-HYDROGEN RECOMBINER		3	10/05/83
51968	TMI 1 - EFFLUENT DISCHARGE MONITOR RELOCATION			10/14/83
49620	THREE MILE ISLAND 1 - TSCR NO. 120- RAISING HPI & LPI BYPASS SETPOINTS			10/19/83
51865	TMI-1 CYCLE 5 RELOAD RE-REVIEW			10/28/83
51855	TMI - 1 GPU/D & W LAWSUIT REVIEW			10/29/83
52451	TMI-1 TSCR-97			11/07/83
51740	TMI 1: STATION DIST VOLTAGE VERIF. TEST			01/19/84
52667	TMI-1 - POST ACCIDENT SHIELDING ALTERNATE			01/27/84
53516	THREE MILE ISLAND 1 - NUREG 0737 IMPLEMENTATION REVIEW			02/17/84
47047	THREE MILE ISLAND 1 - APPENDIX I REVIEW CORRECTIONS			03/06/84
53250	THREE MILE ISLAND 1 - ALTERNATE SOLUTION PLANT SHIELDING			03/14/84
53363	THREE MILES ISLAND 1 - FIRE-PROOF CABLE TEST PROGRAM REVIEW			03/15/84
53562	THREE MILE ISLAND 1 - RESPONSE TO COMMISSION ORDER ON DESIGN ISSUES			03/19/84
51527	TMI - ACCEPTANCE REVIEW OF INDEPENDENT AUDITOR FC <sup>2</sup> TRAINING PROGRAMS			04/09/84
54502	THREE MILE ISLAND 1-COMMISSION REVIEW OF DESIGN ISSUE (REPLY FILING)			04/13/84
53249	THREE MILE ISLAND 1 - EFW FLOW INDICATOR			04/25/84
54599	THREE MILE ISLAND 1 - NUREG-0737. ITEM II.E.1.2 REVIEW			04/25/84
46925	TMI-1 - CONTROL ROOM HUMAN FACTORS REVIEW			04/26/84
48838	THREE MILE ISLAND UNIT 1 - TSCR 116 - RX BLDG PURGE		2	04/26/84
53362	THREE MILE ISLAND 1 - EFW FLOW TEST REVIEW			04/30/84
52668	TMI-1 - MANAGER OF TRAINING, TMI (TSCR134)			05/03/84
49955	THREE MILE ISLAND 1 - TSCR 124 SNUBBER			05/11/84
51057	THREE MILE ISLAND 1- MAIN CONDENSER OFF-GAS IODINE SAMPLER			05/11/84
46929	TMI-1 APPENDIX J REVIEW			05/11/84
43815	TMI-1 TSCR 99 ADMINISTRATIVE CHANGES		3	05/11/84
54244	THREE MILE ISLAND 1 - HEARING-RELATED LICENSE CONDITIONS			05/31/84
48253	THREE MILE ISLAND 1 - APPENDIX R EXEMPTION REVIEW		1	06/04/84
49271	THREE MILE ISLAND 1 - ADDITIONAL WATER STORAGE CAPACITY			06/11/84
54246	THREE MILE ISLAND 1 - BYPRODUCT, SOURCE OR SPECIAL NUCLEAR MATERIAL (TSCR 133)			06/12/84
52006	THREE MILE ISLAND 1 - SUBCOOLING MARGIN INSTRUMENT STRING ERROR (CERTIFICATION ITEM)			06/14/84
52489	THREE MILE ISLAND 1 - FIRE BRIGADE TRAINING			06/20/84
54420	TMI-1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			06/21/84
54245	THREE MILE ISLAND 1 - RCS VENTS (TSCR-137)			06/21/84
52387	THREE MILE ISL-1 LONG TERM EFW MODS (NUREG 0737 II.E.1.1)			06/21/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
52568	THREE MILE ISLAND 1 - TSCR 38 - FUEL CASK DROP			

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

THREE MILE ISLAND 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE TARGET
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
53302	THREE MILE ISLAND 1 - PRE-OPERATIONAL TESTING REVIEW			
49831	THREE MILE ISLAND 1 - TSCR-123 STEAM GENERATOR REPAIR TECHNICAL SPECIFICATIONS			
51412	THREE MILE ISLAND 1 - TSCR 110 AUX BLDG/FHB EXHAUST FLOW			
54504	THREE MILE ISLAND 1 - BACK-UP INCORE THERMOCOUPLE OPERABILITY (TSCR 130)		2	
54500	THREE MILE ISLAND 1 - EMERGENCY FEEDWATER AUTO-INITIATION (I.I.E. 1.2 PART 1)			
53295	THREE MILE ISLAND 1 - STEAM GENERATOR HEARINGS			
55111	THREE MILE ISLAND 1 - STAFF ACTION ON FEHA FINDINGS (1983 EXERCISE)			
55234	TMI-1 - TSCR #135 - WASTE GAS HOLD-UP - HYDROGEN/OXYGEN LIMITS (IN 81-27)			
55040	TMI-1, NEUTRON SOURCE DATA FOR FLUX REDUCTION VERIFICATION			
54243	THREE MILE ISLAND 1 - MANAGEMENT INTEGRITY (CERT ITEM 64)			
47434	TMI-1 STEAM GENERATOR TUBE LEAKS		2	
46948	TMI-1 TSCR 104 SURVEILLANCE CAPSULES		3	
49619	THREE MILE ISLAND 1 - TSCR NO. 114 - POST-ACCIDENT MONITORING INSTRUMENTATION			05/15/84
52003	THREE MILE ISLAND 1 - FIRE PROTECTION SCHEDULAR EXTENSION (9/30/83)			05/15/84
53385	THREE MILE ISLAND 1 - TSCR 97, REV 1 FIRE PROTECTION CHANGES			05/30/84
52007	THREE MILE ISLAND 1 - BACKUP INCORE THERMOCOUPLE REVIEW (CERTIFICATION ITEM)			05/30/84
53511	THREE MILE ISLAND 1 - UCS 2.206 PETITION			05/30/84
52589	THREE MILE ISLAND 1 - TSCR 126 - SURVEILLANCE STANDARDS			05/31/84
53386	THREE MILE ISLAND 1 - FIRE AREA VS ZONE EXEMPTION REQUEST			05/31/84
53399	THREE MILE ISLAND 1 - TSCR 116, REV 1, CONTAINMENT PURGING			06/15/84
47050	THREE MILE ISLAND 1 - 1ST REVIEW		1	06/15/84
55112	THREE MILE ISLAND 1 - STAFF RESPONSE TO COMMISSION ORDER ON ALAB-772			06/22/84
55039	TMI - MANAGEMENT INTEGRITY ISSUES REVIEW			06/30/84
55107	THREE MILE ISLAND 1 - QUALIFICATION OF NDE PERSONNEL			07/15/84
49698	TMI-1 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				
55302	THREE MILE ISLAND 1 - NUREG-0680, SUPPLEMENT 5			07/02/84
55301	THREE MILE ISLAND 1 - FIRE PROOF CABLE TEST REPORT REVIEW			07/02/84
53724	TMI-1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53807	TMI-1 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53890	TMI-1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING - CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53945	-ITEMS 4.2.3 & 4.2.4 - PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS - LIFE TESTING & REPLACEMENT			11/01/84
53959	TMI-1 - ITEM 4.4 - IMPROVEMENTS IN MAINTENANCE AND TEST PROCEDURES FOR B & W PLANTS			11/01/84
54034	TMI-1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54117	TMI-1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84
53641	TMI-1 - ITEM 1.2 - POST TRIP REVIEW - DATA AND INFORMATION CAPABILITY			11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: TROJAN

PLANT LOCATION: 42 MI N OF PORTLAND, ORE  
 DOCKET NUMBER: 050-00344  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: G. JOHNSTON

LICENSED POWER: 3411 MWT  
 DESIGN POWER: 1130 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: C. TRAMMELL  
 BRANCH CHIEF: J. MILLER  
 LIC. ASSISTANT: P. KREUTZER

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
12279	TROJAN - ENV. QUAL. OF CONTROL SYSTEMS TROJAN TROJAN		3	<u>COMPLETE</u>
42594	TROJAN - LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)		1	01/31/84 06/01/84
<u>ACTIVE ACTIONS</u>				
42502	TROJAN - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	<u>TARGET</u>
51362	TROJAN - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			
52809	TROJAN - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52890	TROJAN - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52971	TROJAN - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53052	TROJAN-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53106	TROJAN - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53159	TROJAN - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53201	TROJAN - ITEM 4.3 - AUTOAMTC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
12369	TROJAN - SEISMIC RESTRAINTS/WALL PROBLEM		1	06/30/83
06596	TROJAN - ANALYSIS OF ASYMMETRIC LOCA LOADS AND EFFECT ON REACTOR VESSEL SUPPORTS		1	09/30/83
07721	TROJAN - APPENDIX I TECH SPECS IMPLEMENTATION REVIEW REVIEW		1	09/30/83
10141	TROJAN - TECH SPEC SURVEILLANCE PER FOR MECHANICAL SNUBBERS		1	09/30/83
11298	TROJAN - HYDRAULIC SNUBBERS UPGRADE TECH SPECS		3	09/30/83
49765	TROJAN - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			09/30/83
47448	TROJAN - 3 LOOP OPERATION (N-1)		3	01/01/85
52280	TROJAN - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47823	TROJAN - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	<u>COMPLETE</u>
44482	TROJAN - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	01/26/84
46909	TROJAN - II K2.13 THERMAL MECHANICAL REPORT		1	06/05/84
44274	TROJAN - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
43091	TROJAN - NUREG-0737, III.A.1.2, TECHNICAL SUPPORT CENTER		1	<u>TARGET</u>
46059	TROJAN - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	
46131	TROJAN - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	
46346	TROJAN - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	
51212	TROJAN - I.D.1 CONTROL ROOM DESIGN REVIEW			
51292	TROJAN - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
45175	TROJAN - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	06/30/83
45868	TROJAN - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	06/30/83
48209	TROJAN -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46			12/01/83
44625	TROJAN - RV AND SV TESTING		1	10/31/84
44344	TROJAN - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84



DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

TROJAN

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
51820	TROJAN - REVISED TESTING OF MSIVS AND PZR SAFETY VALVES			<u>COMPLETE</u> 11/01/83
52076	TROJAN - DELETE BORIC ACID TRANSFER PUMPS AND PZR HEATERS FROM APP.R SCOPE (SAFE SHUTDOWN)		3	11/01/83
42996	TROJAN - SURVEILLANCE SPECIMEN-CAPSULE U ANALYSIS (WCAP - 9469)			11/14/83
49630	TROJAN - REVISE PARA. 2.A OF NPF-1 TO REFERENCE UPDATED FSAR			11/22/83
51002	TROJAN - ISI RELIEF REQUEST DATED 3/3/83			12/19/83
52590	TROJAN - ON-SITE & OFF-SITE ORGANIZATIONAL CHANGES LCA 96 DTD 10-4-83			12/22/83
53344	TROJAN - SCHEDULE CHANGE FOR NUREG-0737 ITEMS II.F.1.1 AND II.F.1.2			02/21/84
52077	TROJAN - APP.R EXEMPTION - RCP LUBE OIL COLLECTION SYSTEM			03/05/84
48653	TROJAN - IST RELIEF REQUEST DTD 7-23-82			03/14/84
52591	TROJAN - REVISED FLOW TEST FOR SPRAY ADDITIVE TANK LCA 95. PGE 10-14-83			03/20/84
51953	TROJAN - REVIEW CLOW TRICENTRIC 8" BUTTERFLY VALVES FOR ACCEPT AS CONTAINMENT ISOL. VALVES (PGE LTR 1-25-83)			06/01/84
52088	TROJAN - SPENT FUEL POOL RE-RACK (RE-RACK II)			06/08/84
<u>ACTIVE ACTIONS</u>				
53396	TROJAN - COMBINE STA AND SENIOR OPERATOR			<u>TARGET</u>
53563	TROJAN - DELETE BORON INJECTION TANK			
54218	TROJAN - LCA 99. REVISED CAPSULE SCHEDULE; CONTAINMENT ISOL. VALVES; CONTAINMENT PEAK PRESS.			
54454	TROJAN - HIGHER FUEL BURNUP			
54750	TROJAN - CYCLE 7 WITH SS RODS. PGE 4-6-84			
54972	TROJAN - LCA 102 TESTING SEALED SOURCES. PGE 4-27-84			
54993	TROJAN - DELETE A DG TEST PER GL 83-30 LCA 105. PGE 4-26-84			
55274	TROJAN - LCA 106. ADD A TOLERANCE BAND TO HYDROGEN MIXING SYSTEM FLOW			
55297	TROJAN - LCA 107. CORRECT BORIC ACID STORAGE TANK VOLUMES. PGE 6-22-84			
49636	TROJAN - MOUNT ST. HELENS - SPIRIT LAKE DAM			
55003	TROJAN - MISC. FIRE PROTECTION TECH. SPEC. CHANGES LCA 103. PGE 4-27-84			
53357	TROJAN - EQ EXTENSIONS PGE LETTER DATED 6-1-84			02/15/84
54992	TROJAN - LCA 101. SHUTDOWN MARGIN VERIFICATION WITH INOPERABLE RODS			07/01/84
54575	TROJAN - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54421	TROJAN- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
52523	TROJAN - MOUNT ST. HELENS SURVEILLANCE FY84			09/30/84
49697	TROJAN - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				
53642	TROJAN 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53725	TROJAN - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53808	TROJAN- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53891	TROJAN - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53946	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54035	TROJAN - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54118	TROJAN 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: TURKEY POINT 3

PLANT LOCATION: 25 MI S OF MIAMI, FLA  
 DOCKET NUMBER: 050-00250  
 ARCH/ENGINEER: BECH  
 IE INSPECTOR: T. PEEBLES

LICENSED POWER: 2200 MW  
 DESIGN POWER: 0693 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: D. MCDONALD  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
03784	TURKEY POINT 3 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS		2	<u>COMPLETE</u> 10/14/83
08088	TURKEY PT 3 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	11/01/83
49611	TURKEY POINT 3 - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION			11/14/83
12280	TURKEY POINT 3 - HELB AND CONSEQUENTIAL SYSTEM FAILURE TURKEY POINT 3		3	02/02/84
08782	TURKEY POINT 3 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/06/84
48552	TURKEY POINT 3 FIRE PROTECTION (APPEND R) EXEMPTION REQUEST			03/27/84
49415	TURKEY POINT 3 - BLOCKING SI SIGNAL DURING COOLDOWN			04/13/84
43849	TURKEY POINT 3 - SAFE SHUTDOWN APPENDIX R TO SDV-1V COUPLING		1	04/16/84
08127	TURKEY POINT 3 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	04/23/84
<u>ACTIVE ACTIONS</u>				
52891	TURKEY POINT 3 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			<u>TARGET</u>
52972	TURKEY POINT 3 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53053	TURKEY PT 3-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53107	TURKEY POINT 3 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53160	TURKEY POINT 3 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53202	TURKEY POINT 3 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
10057	TURKEY POINT 3 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE		2	06/29/84
49766	TURKEY POINT 3 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"			06/29/84
42470	TURKEY POINT 3 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	08/17/84
42869	TURKEY POINT 3 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	09/07/84
51363	TURKEY POINT 3 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			12/14/84
52281	TURKEY POINT 3 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TMI ACTIONS</u>				
<u>COMPLETED ACTIONS</u>				
44483	TURKEY POINT 3 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	<u>COMPLETE</u> 11/08/83
46487	TURKEY POINT 3 - NUREG-0737 III.D.3.4, CONTROL ROOM HABITABILITY		1	11/25/83
47824	TURKEY POINT 3 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/06/84
44275	TURKEY POINT 3 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
45869	TURKEY POINT 3 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	08/10/84
45176	TURKEY POINT 3 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	09/07/84
45988	TURKEY POINT 3 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46060	TURKEY POINT 3 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46132	TURKEY POINT 3 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
46347	TURKEY POINT 3 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	09/30/84
46910	TURKEY PT 3 - II K2.13 THERMAL MECHANICAL REPORT		1	09/30/84
51213	TURKEY POINT 3 - I.D.1 CONTROL ROOM DESIGN REVIEW			10/26/84
51293	TURKEY POINT 3 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			10/26/84
44626	TURKEY POINT 3 - RV AND SV TESTING		1	10/31/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

## TURKEY POINT 3

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				
44345	TURKEY POINT 3 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	<u>TARGET</u> 11/30/84
48210	TURKEY POINT 3 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	10/27/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52493	TURKEY POINT UNIT 3 - TECH SPEC AMEND - RESIDUAL HEAT REMOVAL PUMPS			<u>COMPLETE</u> 10/14/83
51768	TURKEY POINT 3 - TECH SPEC AMEND REQUIRING MONTHLY SAFETY SYSTEM WALKDOWNS			10/26/83
52040	TURKEY POINT 3 - SPENT FUEL STORAGE FACILITY EXPANSION			11/16/83
51793	TURKEY POINT 3 - FUEL DESIGN TECH SPEC AMENDMENT FOR OFA AND WABA			12/09/83
52142	TURKEY POINT 3 - REQUEST TO INCREASE THE FH TECH SPEC LIMIT			12/23/83
51766	TURKEY POINT 3 - APPENDIX B TECH SPEC DELETION REQUEST MONITORING PROGRAM			01/04/84
08810	TURKEY POINT 3 - SINGLE FAILURE		1	02/22/84
52500	TURKEY POINT UNIT 3 - ISI RELIEF REQUEST			03/01/84
54280	TURKEY POINT 3 - SCHEDULAR EXEMPTION APP R			03/21/84
54292	TURKEY POINT 3 - ISI EXEMPTION REQUEST			04/25/84
54241	TURKEY POINT 3 - REACTOR VESSEL MATERIAL DATA			04/26/84
<u>ACTIVE ACTIONS</u>				
52810	TURKEY POINT 3 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			<u>TARGET</u>
54826	TURKEY POINT 3 - ELECT DISTRIBUTION DESIGN REVIEW			
54956	TURKEY POINT UNIT 3 - PROPOSED TECHNICAL SPECIFICATION FOR CONTROL OF HEAVY LOADS			06/08/84
52384	TURKEY POINT 3 - DELETION REQUEST FOR APPENDIX B GROUND WATER MONITORING REQUIREMENTS			06/22/84
53452	TURKEY POINT 3 DELETION OF APP B ENVIRONMENTAL PROTECTION LIMITS			06/22/84
53409	TURKEY POINT 3 ISI RELIEF REQUEST			06/29/84
54851	TURKEY POINT 3 - PROPOSED TECH. SPEC. - ELECTRICAL			06/29/84
55167	TURKEY POINT UNIT 3: SCHEDULES FOR ALTERNATE SHUTDOWN AND APPENDIX R			06/29/84
54675	TURKEY POINT UNIT 3 - FUEL STORAGE LINEAR LOADING INCREASE			07/01/84
54428	TURKEY POINT UNIT 3 - NEUTRON SOURCE DATA CY9			07/10/84
54677	TURKEY POINT UNIT 3 - ISI/IST PROPOSED TECH. SPEC.			08/15/84
49133	TURKEY POINT 3 - REVISED 1ST PROGRAM			08/15/84
49917	TURKEY POINT 3 - CONTAINMENT VENT/PURGE VALVE OPERABILITY			09/07/84
54973	TURKEY POINT UNIT 3 - INSERVICE INSPECTION (ISI) SECOND 10-YEAR SUMMARY REPORT			09/14/84
54480	TURKEY POINT UNIT 3 SPENT FUEL POOL EXPANSION			09/15/84
55165	TURKEY POINT UNIT 3: RECEIVED TECH. SPEC. TABLE OF SAFETY RELATED SNUBBERS			09/21/84
54576	TURKEY POINT 3 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54422	TURKEY POINT 3- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
49678	TURKEY POINT 3 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				
53643	TURKEY POINT 3 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53726	TURKEY POINT 3 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53809	TURKEY PT 3- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53892	TURKEY POINT 3 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53947	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54036	TURKEY POINT 3 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84

DATA AS OF - 06/30/84

C O N D E N S E D   M A N A G E M E N T   R E P O R T

(CONTINUATION)

TURKEY POINT 3

TAC #   TAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

INITIATION

54119   TURKEY POINT 3 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

11/01/84



DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: TURKEY POINT 4

PLANT LOCATION: 25 MI S OF MIAMI, FLA  
 DOCKET NUMBER: 050-00251  
 ARCH/ENGINEER: B<sup>F</sup>CH  
 IE INSPECTOR: I. PEEBLES

LICENSED POWER: 2200 MWt  
 DESIGN POWER: 0693 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: D. MCDONALD  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
08735	TURKEY POINT 4 - TECH SPEC SURVEILLANCE REQUIREMENTS FOR MECHANICAL SNUBBERS		2	COMPLETE 10/14/83
08089	TURKEY PT 4 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	11/01/83
49612	TURKEY POINT 4 - PLANS FOR PREVENTING EXCEEDING PTS SCREENING CRITERION			11/17/83
12281	TURKEY POINT 4 - HELB AND CONSEQUENTIAL SYSTEM FAILURE TURKEY POINT 4		3	02/02/84
08783	TURKEY POINT 4 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/06/84
48553	TURKEY POINT 4 FIRE PROTECTION (APPEND R) EXEMPTION REQUEST			03/27/84
49416	TURKEY POINT 4 - BLOCKING SI SIGNAL DURING COOLDOWN			04/13/84
43850	TURKEY POINT 4 SAFETY SHUTDOWN		1	04/16/84
08128	TURKEY POINT 4 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	04/23/84
<u>ACTIVE ACTIONS</u>				
52811	TURKEY POINT 4 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			TARGET
52892	TURKEY POINT 4 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52973	TURKEY POINT 4 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53108	TURKEY POINT 4 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53161	TURKEY POINT 4 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53203	TURKEY POINT 4 - ITEM 4.3 - AUTOMATIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
10058	TURKEY POINT 4 - POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE		2	06/29/84
49767	TURKEY POINT 4 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"			06/29/84
42471	TURKEY POINT 4 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	03/17/84
42870	TURKEY POINT 4 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	09/07/84
51364	TURKEY POINT 4 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			12/14/84
52282	TURKEY POINT 4 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			09/30/85
<u>TAC # TAC DESCRIPTION TMI ACTIONS PRIORITY CRITICAL DATE</u>				
<u>COMPLETED ACTIONS</u>				
44484	TURKEY POINT 4 - NUREG-0737 II.B.3.2. POST ACCIDENT SAMPLING MODIFICATIONS		1	COMPLETE 11/03/83
46488	TURKEY POINT 4 - NUREG-0737 III.D.3.4. CONTROL ROOM HABITABILITY		1	11/25/83
47825	TURKEY POINT 4 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/06/84
46911	TURKEY PT 4 - II K2.13 THERMAL MECHANICAL REPORT		1	06/06/84
44276	TURKEY POINT 4 - NUREG-0737 I.C.1.2.A. INADEQUATE CORE COOLING GUIDELINES		1	06/07/84
<u>ACTIVE ACTIONS</u>				
44627	TURKEY POINT 4 - RV AND SV TESTING		1	TARGET 10/31/83
45070	TURKEY POINT 4 - NUREG-0737 II.K.3.30. SB LOCA OUTLINE		1	08/10/84
45177	TURKEY POINT 4 - NUREG-0737 II.F.2.3. INADEQUATE CORE COOLING INSTRUMENTATION		1	03/31/84
45989	TURKEY POINT 4 - NUREG-0737 III.A.1.2. TECHNICAL SUPPORT CENTER		1	09/30/84
46061	TURKEY POINT 4 - NUREG-0737 III.A.1.2. OPERATIONAL SUPPORT CENTER		1	09/30/84
46133	TURKEY POINT 4 - NUREG-0737 III.A.1.2. EMERGENCY OPERATIONS FACILITY		1	09/30/84
46348	TURKEY POINT 4 - NUREG-0737 III.A.2.2. METEOROLOGICAL DATA UPGRADE		1	09/30/84
51214	TURKEY POINT 4 - I.D.1 CONTROL ROOM DESIGN REVIEW			10/26/84
51294	TURKEY POINT 4 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			10/26/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

TURKEY POINT 4

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
44346	TURKEY POINT 4 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	TARGET 11/30/84
43211	TURKEY POINT 4 - NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	10/27/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
52494	TURKEY POINT UNIT 4 - TECH SPEC AMEND - RESIDUAL HEAT REMOVAL PUMPS			COMPLETE 10/14/83
51769	TURKEY POINT 4 TECH SPEC AMEND REQUIRING MONTHLY SAFETY SYSTEM WALKDOWNS			10/26/83
52041	TURKEY POINT 4 SPENT FUEL STORAGE FACILITY EXPANSION			11/16/83
51794	TURKEY POINT 4 - FUEL DESIGN TECH SPEC AMENDMENT FOR DPA & WABA			12/09/83
52143	TURKEY POINT 4 - REQUEST TO INCREASE FN TECH SPEC LIMITS			12/23/83
51767	TURKEY POINT 4 - APPENDIX B TECH SPEC DELETIONS REQUEST - MONITORING PROGRAM			01/04/84
03812	TURKEY POINT 4 - SINGLE FAILURE IN BACKUP LINE (ECCS)		1	02/22/84
52501	TURKEY POINT UNIT 4 - ISI RELIEF REQUEST			03/01/84
54281	TURKEY POINT 4 - SCHEDULAR EXEMPTION APP R			03/21/84
54293	TURKEY POINT 4 - ISI EXEMPTION REQUEST			04/25/84
54242	TURKEY POINT 4 - REACTOR VESSEL MATERIAL DATA			04/26/84
<u>ACTIVE ACTIONS</u>				
53054	TURKEY PT 4-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			TARGET
54827	TURKEY POINT 4 - ELECT DISTRIBUTION DESIGN REVIEW			
55035	TURKEY POINT UNIT 4 - NEUTRON SOURCE DATA UNIT 4 CIR 9			
55042	TURKEY POINT UNIT 4			
54957	TURKEY POINT UNIT 4 - PROPOSED TECHNICAL SPECIFICATION FOR CONTROL OF HEAVY LOADS			06/08/84
52385	TURKEY POINT 4 - DELETION REQUEST FOR APP. B G AND WATER MONITORING REQUIREMENTS			06/22/84
53453	TURKEY POINT 4 - DELETION OF APP. B. ENVIRONMENTAL PROTECTION LIMITS			06/22/84
53410	TURKEY POINT 4 ISI RELIEF REQUEST			06/29/84
54352	TURKEY POINT 4 - PROPOSED TECH. SPEC. - ELECTRICAL			06/29/84
55168	TURKEY POINT UNIT 4: SCHEDULES FOR ALTERNATE SHUTDOWN AND APPENDIX R MODIFICATIONS			06/29/84
54676	TURKEY POINT UNIT 4 - FUEL STORAGE LINEAR LOADING INCREASE			07/01/84
54678	TURKEY POINT UNIT 4 - ISI/IST PROPOSED TECH. SPEC.			08/15/84
49936	TURKEY POINT 4 - REVISED 1ST PROGRAM			08/15/84
49919	TURKEY POINT 4 - CONTAINMENT VENT/PURGE VALVE OPERABILITY			09/07/84
54974	TURKEY POINT UNIT 4 - INSERVICE INSPECTION (ISI) SECOND 10-YEAR SUMMARY REPORT			09/14/84
54481	TURKEY POINT UNIT 4 SPENT FUEL POOL EXPANSION			09/15/84
55166	TURKEY POINT UNIT 4: RECEIVED TECH. SPEC. TABLE OF SAFETY RELATED SNUBBERS			09/21/84
54577	TURKEY POINT 4 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54423	TURKEY POINT 4- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
49679	TURKEY POINT 4 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				
53644	TURKEY POINT 4 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			INITIATION 11/01/84
53727	TURKEY POINT 4 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53810	TURKEY PT 4- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53893	TURKEY POINT 4 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53948	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84

DATA AS OF - 06/30/84

(CONTINUATION)

TURKEY POINT 4

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
54037	TURKEY POINT 4 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			<u>INITIATION</u> 11/01/84
54120	TURKEY POINT 4 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

PLANT LOCATION: 5 MI S OF BRATTLEBORO, VT PROJECT MANAGER: V. ROONEY  
 DOCKET NUMBER: 850-00271 BRANCH CHIEF: D. VASSALLO  
 ARCH/ENGINEER: EBLJCO LIC. ASSISTANT: S. SHEPPARD  
 IE INSPECTOR: M. RAYMOND

FACILITY: VERMONT YANKEE 1  
 LICENSED POWER: 1593 MW  
 DESIGN POWER: 0514 MWE  
 NSSS VENDOR: GE

MULTI-PLANT  
 HELB AND CONSEQUENTIAL SYSTEM FAILURE VERMONT YANKEE  
 LONG TERM REVIEW CONTAINMENT PURGE AND VENT (B-24)  
 IMPLEMENTATION OF NUREG-3313, REV 1  
 CONTROL OF HEAVY LOADS AT NUCLEAR POWER PLANTS (NUREG 0612)  
 RPS FOWER SUPPLY  
 FEEDWATER & NOZZLE CRACKING -NUREG 0619 IMPLEMENTATION

ACTIVE ACTIONS  
 51365 VERMONT YANKEE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)  
 52283 VERMONT YANKEE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)  
 52812 VERMONT YANKEE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES  
 52893 VERMONT YANKEE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS  
 52914 VERMONT YANKEE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR  
RECOMMENDATIONS  
 - RTS COMPONENTS  
 1-ITEM 3.1.3 -POST-MAINT TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS  
 VERMONT YANKEE - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION  
 VERMONT YANKEE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW  
 VERMONT YANKEE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11  
 VERMONT YANKEE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CUI-21)  
 VERMONT YANKEE - SAFETY CONCERNS ASSOCIATED WITH PIPE BREAKS IN THE BUR SCRAM SYSTEM  
 51693 VERMONT YANKEE NUREG 0737 TECH SPECS  
 08820 VERMONT YANKEE - HYDRAULIC SNUBBERS  
 10059 VERMONT YANKEE - POTENTIAL EQUIPMENT FAILURES ASSOCIATED DEGRADED GRID VOLTAGE  
 10428 VERMONT YANKEE - MECHANICAL SNUBBERS

IAC # IAC DESCRIPTION

COMPLETED ACTIONS  
 45871 VERMONT YANKEE - NUREG-0737 II.K.3.30, 38 LOCA OUTLINE  
 45872 VERMONT YANKEE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 19 CFR 50.46  
 46278 VERMONT YANKEE 1 - III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET PULE  
 46277 VERMONT YANKEE - NUREG-0737 I.C.1.2.A. INADEQUATE CORE COOLING GUIDELINES

ACTIVE ACTIONS  
 51215 VERMONT YANKEE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW  
 51295 VERMONT YANKEE 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM  
 45170 VERMONT YANKEE - NUREG-0737 II.F.2.3. INADEQUATE CORE COOLING INSTRUMENTATION  
 45805 VERMONT YANKEE - NUREG-0737 II.K.3.16. CHALLENGES AND FAILURES OF RELIEF VALVES  
 46349 VERMONT YANKEE - NUREG-0737 III.A.2.2. METEOROLOGICAL DATA UPGRADE  
 46628 VERMONT YANKEE 1 - II.D.1 RV AND SV TESTING  
 46347 VERMONT YANKEE - NUREG-0737 I.C.1.3A. ABNORMAL TRANSIENT OPERATOR GUIDELINES  
 48274 VERMONT YANKEE - NUREG 0737 II.K.3.28. VERIFY QUALIFICATION OF ACCUMULATORS ON ADS VALVES  
 45702 VERMONT YANKEE - NUREG-0737 II.K.3.18. ADS ACTUATION STUDY  
 44485 VERMONT YANKEE - NUREG-0737 II.B.3.2. POST ACCIDENT SAMPLING MODIFICATIONS  
 45990 VERMONT YANKEE - NUREG-0737 III.A.1.2. TECHNICAL SUPPORT CENTER

IAC #	IAC DESCRIPTION	PRIORITY	CRITICAL DATE
51365	VERMONT YANKEE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)	3	01/31/84
52283	VERMONT YANKEE 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)	1	05/22/84
52812	VERMONT YANKEE 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES	2	06/15/84
52893	VERMONT YANKEE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS	2	06/15/84
52914	VERMONT YANKEE 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR	1	06/27/84
53855	VERMONT YANKEE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)	1	12/01/83
08129	VERMONT YANKEE - MARK I CONTAINMENT LONG TERM PROGRAM IMPLEMENTATION	3	12/01/83
42030	VERMONT YANKEE - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW	2	12/30/83
42400	VERMONT YANKEE - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11	1	12/30/83
43724	VERMONT YANKEE - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CUI-21)	1	12/30/83
51693	VERMONT YANKEE NUREG 0737 TECH SPECS	3	03/01/84
08820	VERMONT YANKEE - HYDRAULIC SNUBBERS	2	12/30/84
10059	VERMONT YANKEE - POTENTIAL EQUIPMENT FAILURES ASSOCIATED DEGRADED GRID VOLTAGE	2	12/30/84
10428	VERMONT YANKEE - MECHANICAL SNUBBERS	2	12/30/84

IAC #	IAC DESCRIPTION	PRIORITY	CRITICAL DATE
45871	VERMONT YANKEE - NUREG-0737 II.K.3.30, 38 LOCA OUTLINE	1	01/04/84
45872	VERMONT YANKEE -NUREG 0737 II.K.3.31 COMPLIANCE WITH 19 CFR 50.46	1	01/04/84
46278	VERMONT YANKEE 1 - III.A.2.1 EMERGENCY PLAN UPGRADE TO MEET PULE	1	02/22/84
46277	VERMONT YANKEE - NUREG-0737 I.C.1.2.A. INADEQUATE CORE COOLING GUIDELINES	1	06/07/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

VERMONT YANKEE 1

TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
46062	VERMONT YANKEE - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	TARGET 09/30/84
46134	VERMONT YANKEE - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
53340	VERMONT YANKEE	EMERGENCY PLAN DRILL		COMPLETE 11/10/83
52014	VERMONT YANKEE	- DELETE TECH SPEC PLAN REQUIREMENT FOR ANNUAL EMERGENCY DRILL		11/10/83
52090	VERMONT YANKEE	- WITHDRAWAL OF I.A.1.3 COMMITMENT		12/12/83
48897	VERMONT YANKEE	- STA AS FIRE BRIGADE LEADER		01/30/84
51993	VERMONT YANKEE	REACTOR VESSEL PRESSURE TEMPERATURE CURVES		03/13/84
54290	VERMONT YANKEE	- REACTOR VESSEL NED PRESSURE TEMPERATURE CURVES		03/13/84
52638	VERMONT YANKEE	- 10/25/83 2.206 PETITION		04/16/84
52102	VERMONT YANKEE	- FOLLOWON REQUIREMENTS RELATED TO RECOMBINERS		05/22/84
<u>ACTIVE ACTIONS</u>				
51700	VERMONT YANKEE	- DEGRADED GRID VOLTAGE PROCEDURES		TARGET
48671	VERMONT YANKEE	- SUPPRESSION POOL TEMPERATURE		10/30/83
52026	VERMONT YANKEE	- HCU SEISMIC QUALIFICATION		12/30/83
52639	VERMONT YANKEE	- FIRE PROTECTION EXEMPTIONS DUE TO AUDIT		12/30/83
43078	VERMONT YANKEE	- CURRENT EVENTS	3	12/30/83
43308	VERMONT YANKEE	- RELOAD ANALYSIS METHODS	1	12/30/83
52109	VERMONT YANKEE	- SNUBBER CHANGES		12/31/83
53436	VERMONT YANKEE	- NEW ISI PROGRAM		05/01/84
53437	VERMONT YANKEE	- NEW IST PROGRAM		05/01/84
54288	VERMONT YANKEE	- SHOCK SUPPRESSORS - CHANGE 117		05/05/84
54289	VERMONT YANKEE	- HPCI AUTOMATIC SUCTION TRANSFER		05/05/84
54182	VERMONT YANKEE	- MAIN STEAM LINE LO PRESSURE ISOLATION SETPOINT		06/01/84
54287	VERMONT YANKEE	- REMOVE AUTOMATIC AIR DUMP		06/05/84
54496	VERMONT YANKEE	- FIRE PROTECTION DEFICIENCIES		06/30/84
54747	VERMONT YANKEE	- MSIV HIGH FLOW SETPOINT		06/30/84
55091	VERMONT YANKEE	- CYCLE II USE OF FROSSTY		07/01/84
55122	VERMONT YANKEE	EMERGENCY EXERCISE EXEMPTION REQUEST		07/01/84
55164	VERMONT YANKEE	- DIESEL GENERATOR OPERABILITY		07/02/84
55097	VERMONT YANKEE	- FSAR UPDATE EXEMPTION		07/15/84
55079	VERMONT YANKEE	- FIRE PROTECTION SCHEDULAR EXTENSION		07/30/84
55291	VERMONT YANKEE	- SDV AIR DUMP REMOVAL		08/01/84
54486	VERMONT YANKEE	- WELD INSPECTION AND REPAIR		08/03/84
54578	VERMONT YANKEE 1	- TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737		09/30/84
<u>ANTICIPATED ACTIONS</u>				
53645	VERMONT YANKEE 1	- ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY		INITIATION 11/01/84
53728	VERMONT YANKEE 1	- ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS		11/01/84
53811	VERMONT YANKEE 1	- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS		11/01/84
53894	VERMONT YANKEE 1	- ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS		11/01/84
54038	VERMONT YANKEE 1	- ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS		11/01/84

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C O N D E N S E D   M A N A G E M E N T   R E P O R T

(CONTINUATION)

VERMONT YANKEE 1

IAC #   IAC DESCRIPTION

PLANT SPECIFIC

PRIORITY

CRITICAL DATE

ANTICIPATED ACTIONS (CONTINUATION)

INITIATION

54121 VERMONT YANKEE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES

11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: WASHINGTON NUCLEAR 2

PLANT LOCATION: 30 MI NW OF RICHLAND, WASH  
 DOCKET NUMBER: 050-00397  
 ARCH/ENGINEER: B&R  
 IE INSPECTOR:

LICENSED POWER: 3323 MWT  
 DESIGN POWER: 1100 MWE  
 NSSS VENDOR: GE

PROJECT MANAGER: R. AULUCK  
 BRANCH CHIEF: A. SCHWENGER  
 LIC. ASSISTANT: E. HYLTON

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
54536	WNP-2 - II.D.1 - RV & SV TESTING			<u>TARGET</u>

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
54186	WNP-2 - TECH SPEC AUDIT			<u>COMPLETE</u>
54838	WNP-2, POST ACCIDENT SAMPLING		1	05/30/84 05/30/84

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u>				
55279	WNP 2 - CONTROL ROOM CHILLERS (L.C. #21)			<u>TARGET</u>
55280	WNP 2 - HYDRODYNAMIC LOADS (L.C. #23)			
55281	WNP 2 - EMERGENCY PLANNING PROGRAM (L.C. #24)			
55282	WNP 2 - OFF-SITE EMERGENCY PREPAREDNESS (L.C. #25)			
55283	WNP 2 - P.T. 33.0-B TEST			
55284	WNP 2 - ASME CODE CASE N-342			
55285	WNP 2 - ISI PROGRAM			
54997	WNP-2; CONTAINMENT INERTING			
54645	WNP-2, DRYWELL & SUPPRESSION CHAMBER H2 RECOMBINER SYSTEM		1	06/01/84
54646	WNP-2, ELECTRICAL EQUIPMENT PROTECTIVE DEVICES		1	07/08/84
54647	WNP-2, M-O VALVES THERMAL OVERLOAD PROTECTION		1	07/08/84
54648	WNP-2, ADMINISTRATIVE CONTROLS		1	07/08/84
54642	WNP-2 - FIRE DETECTION INSTRUMENTATION		1	07/08/84
54643	WNP-2 - RADIOACTIVE LIQUID EFFLUENT MONITORING SYSTEM		1	07/08/84
54644	WNP-2 - ECCS		1	07/08/84
54641	WNP-2, INITIAL TEST PROGRAM (LC-3)		1	08/15/84
54620	WNP-2 - D.C. SOURCES SURVEILLANCE REQUIREMENTS		1	08/15/84
54442	WNP-2, REPORTING REQUIREMENTS OF 10 CFR 50		1	08/15/84
54425	WPPS-2- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1		3	08/31/84
54443	WNP-2, MRSC. TECH. SPEC. CHANGES			09/30/84
54640	WNP-2 - DETAIL CONTROL ROOM DESIGN REVIEW (LC-16)		1	09/31/84
55288	WNP 2 - EMERGENCY RESPONSE CAPABILITY (LC #16)		3	12/30/85 12/30/85

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS</u>				
53647	WPPS-2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u>
53730	WPPS-2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53813	WPPS-2- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84 11/01/84
53896	WPPS-2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
54040	WPPS-2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54123	WPPS-2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT REPORT

FACILITY: YANKEE-ROWE 1

PLANT LOCATION: 25 MI NE OF PITTSFIELD, MASS  
 DOCKET NUMBER: 050-00029  
 ARCH/ENGINEER: S&W  
 IE INSPECTOR: J. COLLINS

LICENSED POWER: 0600 MWT  
 DESIGN POWER: 0175 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: P. ERICKSON  
 BRANCH CHIEF: D. CRUTCHFIELD  
 LIC. ASSISTANT: H. SMITH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
47145	YANKEE ROWE 1	NATURAL CIRCULATION COOLDOWN (GENERIC LTR 81-21)	2	COMPLETE 10/25/83
07259	YANKEE ROWE	- INSERVICE INSPECTION PROGRAM TO MEET 10 CFR 50.55A	1	01/03/84
10147	YANKEE ROWE	- REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL	2	02/01/84
43672	YANKEE ROWE	- SEISMIC QUALIFICATION OF THE AUXILIARY FEEDWATER SYSTEM	1	02/23/84
53204	YANKEE ROWE 1	- ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		04/13/84
<u>ACTIVE ACTIONS</u>				
43626	YANKEE ROWE	- APPENDIX R-ALTERNATE SAFE SHUTDOWN REVIEW	1	TARGET 07/01/84
42526	YANKEE ROWE	- ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)	1	07/31/84
42917	YANKEE ROWE	- MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11	2	07/31/84
08091	YANKEE ROWE 1	-CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)	2	08/30/84
49768	YANKEE ROWE 1	- "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 OR 83-02)"	3	08/30/84
10060	YANKEE ROWE	- POTENTIAL EQUIPMENT FAILURES ASSOCIATED WITH GRID VOLTAGE	2	09/30/84
52313	YANKEE ROWE 1	- ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES		02/01/85
52894	YANKEE ROWE 1	- ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS		02/01/85
52975	YANKEE ROWE 1	- ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS		02/01/85
53056	YANKEE ROWE 1	-ITEM 3.1.3-POST-MAINTENANCE TESTING-CHNGS TO TECH SPECS-RTS COMPONENTS		02/01/85
53109	YANKEE ROWE 1	- REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS		02/01/85
52284	YANKEE ROWE 1	- CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)		09/30/85
51366	YANKEE ROWE 1	- INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)	3	12/31/85
53162	YANKEE ROWE 1	- ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINT PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING		02/01/86
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
45179	YANKEE ROWE	- NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION	1	COMPLETE 10/24/83
45256	YANKEE ROWE	- NUREG-0737 II.K.3.1, AUTO PORV ISOLATION	1	12/13/83
45302	YANKEE ROWE	- NUREG-0737 II.K.3.2, REPORT ON PORV FAILURES	1	12/13/83
47826	YANKEE ROWE	- NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS	1	12/22/83
44065	YANKEE ROWE	- NUREG-0737 I.A.1.1 STA LONG TERM REQUIREMENTS		05/14/84
44278	YANKEE ROWE	- NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES	1	06/07/84
46912	YANKEE ROWE 1	- II K2.13 THERMAL MECHANICAL REPORT	1	06/13/84
<u>ACTIVE ACTIONS</u>				
46350	YANKEE ROWE	- NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE	1	TARGET 06/01/84
45991	YANKEE ROWE	- NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER	1	06/01/84
45872	YANKEE ROWE	- NUREG-0737 II.K.3.30, SB LOCA OUTLINE	1	06/30/84
51296	YANKEE ROWE 1	- I.D.2. SAFETY PARAMETER DISPLAY SYSTEM	1	08/01/84
48213	YANKEE ROWE	-NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46	1	09/30/84
46063	YANKEE ROWE	- NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER	1	09/30/84
46135	YANKEE ROWE	- NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY	1	09/30/84
44629	YANKEE ROWE 1	- RV AND SV TESTING	1	10/31/84
44348	YANKEE ROWE	- NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES	1	11/30/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

YANKEE-ROWE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>IMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS (CONTINUATION)</u>				
51216	YANKEE ROWE 1 - I.D.1 CONTROL ROOM DESIGN REVIEW		1	<u>TARGET</u> 12/31/84
44486	YANKEE ROWE - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	08/15/86
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49043	YANKEE: RECOMBINER EXEMPTION REQUEST			<u>COMPLETE</u> 16/05/83
49264	YANKEE - LOW POWER PHYSICS TEST PROBLEMS		3	10/05/83
52522	YANKEE - PROCESS CONTROL PROGRAM			11/02/83
51004	YANKEE - ISI RELIEF: PERSONNEL QUALIFICATION REQMTS		3	01/17/84
52114	YANKEE - TECH SPEC CHANGE RELATING TO CYCLE DEPENDENT VARIABLES			02/09/84
51890	YANKEE - REACTOR VESSEL ISI			02/28/84
51850	YANKEE - SPENT FUEL POOL RACK SEISMIC DESIGN			03/30/84
54216	YANKEE ROWE - CODE 17 REFUELING			06/05/84
<u>ACTIVE ACTIONS</u>				
54447	YANKEE ROWE - REQUEST FOR EXEMPTION - HYDROGEN RECOMBINER TAPS			<u>TARGET</u> 04/30/84
54842	YANKEE - CHANGES TO TECH. SPEC. DUE TO REFUELING MODIFICATIONS			05/30/84
51812	YANKEE - VENTILATION ISSUES			05/30/84
51810	YANKEE - ELECTRICAL AND INTERLOCK ISSUES			06/01/84
52113	YANKEE - CONTAINMENT SYSTEMS SURVEILLANCE & VALVE LISTING CHANGES			06/09/84
55154	YANKEE ROWE - HYDROGEN RECOMBINER CAPABILITY - SCHEDULAR EXEMPTION			06/30/84
51889	YANKEE - REANALYSIS OF MAIN STEAM LINE RUPTURE EVENT-CYCLE XVI			07/01/84
51804	YANKEE - APPENDIX R HOT SAFE SHUTDOWN SYSTEM			07/15/84
54655	YANKEE - FUEL POOL TECH. SPEC. 4.9.7.2			08/01/84
54215	YANKEE ROWE - T.S. CHANGES - CHANGE 139 - SUPPLEMENT 5			08/01/84
51808	YANKEE - PIPE BREAK EFFECTS ISSUES			08/01/84
51806	YANKEE - CLASSIFICATION OF STRUCTURES, COMPONENTS AND SYSTEMS			08/01/84
46544	YANKEE-ROWE: T.S. CHANGES - CHANGE 139, SUPPLEMENT 4			08/31/84
51358	YANKEE - LTOP - APP. G CURVE SHIFT		3	09/30/84
48784	YANKEE ROWE - ISI/IST REVIEW		3	09/30/84
51903	YANKEE - INTEGRATED ASSESSMENT REPORT SUPPLEMENT			09/30/84
54579	YANKEE ROWE 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
54424	YANKEE ROWE 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
51809	YANKEE - WATER CHEMISTRY ISSUES			10/01/84
54466	YANKEE - APPENDIX K EXEMPTION REQUEST			10/15/84
51805	YANKEE HYDROLOGIC ISSUES			10/30/84
51433	YANKEE ROWE - PROBABILISTIC SAFETY STUDY		3	10/31/84
51705	YANKEE ROWE - DEGRADED GRID VOLTAGE PROCEDURES			11/30/84
51811	YANKEE - CONTAINMENT ISSUES			12/31/84
52437	YANKEE - COORDINATION OF HARRIMAN DAM REVIEW WITH FERC			12/31/84
52515	YANKEE - CONFIRMATION OF SEP ACTION ITEMS			01/31/85
49655	YANKEE ROWE 1 - REACTOR COOLANT PUMP TRIP		3	02/01/85
51807	YANKEE - INTEGRATED STRUCTURAL ANALYSIS			06/06/85
<u>ANTICIPATED ACTIONS</u>				
53646	YANKEE ROWE 1 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			<u>INITIATION</u> 11/01/84
53729	YANKEE ROWE 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53812	YANKEE ROWE 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

YANKEE-ROWE 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ANTICIPATED ACTIONS (CONTINUATION)</u>				
53895	YANKEE ROWE 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			<u>INITIATION</u> 11/01/84
53949	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54039	YANKEE ROWE 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54122	YANKEE ROWE 1 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL TESTING - DIVERSE TRIP FEATURES			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: ZION 1

PLANT LOCATION: 40 MI N OF CHICAGO, ILL  
 DOCKET NUMBER: 050-00295  
 ARCH/ENGINEER: S&L  
 IE INSPECTOR: J. WATERS

LICENSED POWER: 3250 MWT  
 DESIGN POWER: 1040 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: J. NORRIS  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>MULTI-PLANT</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
49769	ZION 1 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			<u>COMPLETE</u> 11/01/83
12282	ZION 1 - HELB AND CONSEQUENTIAL SYSTEM FAILURE ZION 1		3	01/31/84
08872	ZION 1 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
08092	ZION 1 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	03/22/84
<u>ACTIVE ACTIONS</u>				
51367	ZION 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52285	ZION 1 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52814	ZION 1 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52895	ZION 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52976	ZION 1 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53057	ZION 1-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53110	ZION 1 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53163	ZION 1 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53205	ZION 1 - ITEM 4.3 - AUTOAMTIC ACUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS			
42871	ZION 1 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		2	06/30/83
08130	ZION 1 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	09/30/83
06431	ZION 1 - N-1 LOOP OPERATION		3	01/01/84
11228	ZION 1 - HYDRAULIC SNUBBERS UPGRADE TECH SPECS		3	03/30/84
08864	ZION 1 - MECHANICAL SNUBBERS		2	03/30/84
42472	ZION 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)		1	03/30/85
<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>TMI ACTIONS</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
46913	ZION 1 - II K2.13 THERMAL MECHANICAL REPORT		1	<u>COMPLETE</u> 10/26/83
47827	ZION 1 - NUREG-0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/16/84
44487	ZION 1 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/22/84
<u>ACTIVE ACTIONS</u>				
51217	ZION 1 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51297	ZION 1 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
46351	ZION 1 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
48214	ZION 1 -NUREG 0737 II.K.3.31 COMPLIANCE WITH 10 CFR 50.46		1	09/30/83
45180	ZION 1 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	11/30/83
45873	ZION 1 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	03/30/84
45992	ZION 1 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46064	ZION 1 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46136	ZION 1 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44630	ZION 1 - RV AND SV TESTING		1	10/31/84
44279	ZION 1 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44349	ZION 1 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ZION 1

TAC #	TAC DESCRIPTION	PLANT SPECIFIC	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				<u>COMPLETE</u>
42306	ZION 1 - CONTROL ROOM - HUMAN FACTORS ENGINEERING (ORDER 2/29/80 ITEM E.2)		1	11/21/83
52116	ZION 1 - REVIEW AND APPROVE OR DISAPPROVE SECT.2.5 OF TOPICAL REPORT: "BENCHMARK OF PWR NUCLEAR DESIGN METHODS"			12/02/83
52620	ZION 1 - ROD SWAP ANALYSIS METHOD			12/07/83
52288	ZION 1 - T.S. CHANGE FOR OPTIMIZED FUEL ASSEMBLY			12/23/83
52525	ZION 1 - T.S. CHANGE RE PREVENTIVE MAINTENANCE ON SWING DIESEL GENERATOR			03/12/84
53464	ZION 1 TS CHANGE REPVENTIVE MAINTENANCE ON SWING GENERATE (REVISION)			03/12/84
51306	ZION 1 - OPERABILITY OF VENT/PURGE VALVES			04/03/84
54306	ZION 1 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
52118	ZION 1 - T.S. CHANGE TO CONT. LEAKAGE TESTS AND ADD T.S FOR AIRLOCK LEAKAGE TESTS			04/18/84
52380	ZION 1 - TS CHANGE RE COOLANT CHEMISTRY AND SPECIFIC ACTIVITY			04/19/84
54365	ZION - 1 ONE TIME EXEMPTION FROM STATE AND COUNTY PANTIC. IN ANNUAL EMERG. EXERCISE			04/19/84
<u>ACTIVE ACTIONS</u>				<u>TARGET</u>
54929	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-5			
54931	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM D-1			
54933	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM D-2			
54935	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM E-1			
54937	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM E-2			
54939	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-3			
54941	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM F-4			
54943	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM F-5			
55267	ZION 1 - LEAK RATE TESTING - 2.206 PETITION			
54911	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-6			
54913	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-7			
54915	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-1			
54917	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-3			
54919	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-4			
54921	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-5			
54923	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-6			
54925	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-3			
54927	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-4			
54176	ZION 1 - RECONSIDERATION OF FEB. 29, 1980, ORDER			
54203	ZION 1 - SCHEDULAR EXEMPTION FROM 10CFR 50.48(C)4			
54160	ZION 1 - ITEM B.6 OF 1980 ORDER			
52185	ZION 1 - T.S. CHANGES RE AUXILIARY ELECTRIC POWER AND ECCS			
54901	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-1			
54903	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-2			
54905	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-3			
54907	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-4			
54909	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-5			
49601	ZION 1 - MOD TO 1980 ORDER - AFWS			05/30/83
48095	ZION 1 - OPERATION WITH ROD URGENT FAILURE ALARM		3	05/30/83
48103	ZION 1 - BORATION FLOW PATHS HEAT TRACING		3	06/30/83
42146	ZION 1 - ATWS INSTRUMENTATION MODIFICATIONS (ORDER 2/29/80 ITEM C.4)		1	06/30/83
42836	ZION 1 - FAILURE MODE EFFECTS ANALYSIS (2/29/80 ORDER ITEM F.4)		1	09/01/83
51901	ZION 1 - 6/27/83 ISI REQUEST			09/01/83
48692	ZION 1 - LICENSE CONDITION FOR TECH SPECS IN EMERG			09/30/83
41044	ZION 1 - RESIDUAL RISKS		1	02/28/84
13021	ZION 1 - STANDARD TECH SPEC LCO'S FOR TASK ACTION PLAN ITEM F.1 (F)(3)		1	03/15/84



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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ZION 1

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54162	ZION 1 - REVIEW OF REVISED LICENSED OPERATOR REQUALIFICATION TOPICAL REPORT			03/30/84
54426	ZION 1- RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54580	ZION 1 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-0737			09/30/84
52100	ZION 1 - ISI AND IST PROGRAM, SECOND INTERVAL			12/31/84
49683	ZION 1 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53648	ZION ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53731	ZION 1 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53814	ZION 1- ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53897	ZION 1 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53950	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54041	ZION 1 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54124	ZION ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84

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## CONDENSED MANAGEMENT REPORT

FACILITY: ZION 2

PLANT LOCATION: 40 MI N OF CHICAGO, ILL  
 DOCKET NUMBER: 050-00304  
 ARCH/ENGINEER: S&L  
 IE INSPECTOR: J. WATERS

LICENSED POWER: 3250 MWT  
 DESIGN POWER: 1040 MWE  
 NSSS VENDOR: WEST

PROJECT MANAGER: J. NORRIS  
 BRANCH CHIEF: S. VARGA  
 LIC. ASSISTANT: C. PARRISH

TAC #	TAC DESCRIPTION	MULTI-PLANT	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
49770	ZION 2 - "NUREG 0737 TECH SPECS (GENERIC LETTER 82-16 82-16 OR 83-02)"			<u>COMPLETE</u> 11/01/83
12283	ZION 2 - HELB AND CONSEQUENTIAL SYSTEM FAILURE ZION 2		3	01/31/84
08873	ZION 2 - REVIEW OF ASYMMETRIC LOCA LOADS SUBMITTAL		2	02/01/84
08093	ZION 2 - CONTROL OF HEAVY LOADS OVER SPENT FUEL POOL (NUREG 0612)		2	03/22/84
<u>ACTIVE ACTIONS</u>				
51368	ZION 2 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)			<u>TARGET</u>
52286	ZION 2 - CONTROL OF HEAVY LOADS - PHASE II (FOLLOWUP OF MPA#C-10)			
52815	ZION 2 - ITEM 1.1 - POST TRIP REVIEW PROGRAM DESCRIPTION AND PROCEDURES			
52896	ZION 2 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS			
52977	ZION 2 - ITEMS 3.1.1 & 3.1.2 - POST MAINTENANCE TESTING PROCEDURES/VENDOR RECOMMENDATIONS - RTS COMPONENTS			
53058	ZION 2-ITEM 3.1.3 -POST-MAINTENANCE TESTING-CHANGES TO TECH SPECS-RTS COMPONENTS			
53111	ZION 2 - REACTOR TRIP SYSTEM RELIABILITY - VENDOR RELATED MODIFICATIONS			
53164	ZION 2 - ITEMS 4.2.1 & 4.2.2 - PREVENTATIVE MAINTENANCE PROGRAM FOR REACTOR TRIP BREAKERS - MAINTENANCE & TRENDING			
53206	ZION 2 - ITEM 4.3 - AUTOAMTIC ACTUATION OF SHUNT TRIP ATTACHMENT FOR W AND B & W PLANTS		2	06/30/83
42872	ZION 2 - MASONRY WALL DESIGN. RESPONSE TO IE BULLETIN 80-11		3	09/30/83
08131	ZION 2 - APPENDIX I TECH SPEC IMPLEMENTATION REVIEW		3	01/01/84
08844	ZION 2 - N-1 LOOP OPERATION		2	03/30/84
08865	ZION 2 - MECHANICAL SNUBBERS		3	03/30/84
11229	ZION 2 - HYDRAULIC SNUBBERS UPGRADE TECH SPECS		1	03/30/85
42473	ZION 2 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)			
TAC #	TAC DESCRIPTION	TMI ACTIONS	PRIORITY	CRITICAL DATE
<u>COMPLETED ACTIONS</u>				
46914	ZION 2 - II K2.13 THERMAL MECHANICAL REPORT		1	<u>COMPLETE</u> 10/26/83
47828	ZION 2 - NUREG 0737 ITEM II K2.17 POTENTIAL FOR VOIDING IN RCS		1	01/16/84
44488	ZION 2 - NUREG-0737 II.B.3.2, POST ACCIDENT SAMPLING MODIFICATIONS		1	06/22/84
<u>ACTIVE ACTIONS</u>				
51218	ZION 2 - I.D.1 CONTROL ROOM DESIGN REVIEW			<u>TARGET</u>
51298	ZION 2 - I.D.2. SAFETY PARAMETER DISPLAY SYSTEM			
46352	ZION 2 - NUREG-0737 III.A.2.2, METEOROLOGICAL DATA UPGRADE		1	06/01/83
44631	ZION 2 - RV AND SV TESTING		1	10/31/83
45181	ZION 2 - NUREG-0737 II.F.2.3, INADEQUATE CORE COOLING INSTRUMENTATION		1	11/30/83
45874	ZION 2 - NUREG-0737 II.K.3.30, SB LOCA OUTLINE		1	03/30/84
45993	ZION 2 - NUREG-0737 III.A.1.2, TECHNICAL SUPPORT CENTER		1	09/30/84
46065	ZION 2 - NUREG-0737 III.A.1.2, OPERATIONAL SUPPORT CENTER		1	09/30/84
46137	ZION 2 - NUREG-0737 III.A.1.2, EMERGENCY OPERATIONS FACILITY		1	09/30/84
44280	ZION 2 - NUREG-0737 I.C.1.2.A, INADEQUATE CORE COOLING GUIDELINES		1	11/30/84
44350	ZION 2 - NUREG-0737 I.C.1.3A, ABNORMAL TRANSIENT OPERATOR GUIDELINES		1	11/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ZION 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>COMPLETED ACTIONS</u>				
42307	ZION 2 - CONTROL ROOM - HUMAN FACTORS ENGINEERING (ORDER 2/29/80 ITEM E.2)		1	<u>COMPLETE</u> 11/21/83
52117	ZION 2 - REVIEW AND APPROVE OR DISAPPROVE SECT.2.5 OF TOPICAL REPORT: "BENCHMARK OF PWR NUCLEAR DESIGN METHODS"			12/02/83
52621	ZION 2 ROD SWAP ANALYSIS METHOD			12/07/83
52289	ZION 2 - T.S. CHANGE FOR OPTIMIZED FUEL ASSEMBLY			12/23/83
52526	ZION 2 - T.S. CHANGE RE PREVENTIVE MAINTENANCE ON SWING DIESEL GENERATOR			03/12/84
53465	ZION 2 - TS CHANGE RE PREVENTIVE MAINTENANCE ON GIVING GENERATOR (REVISION)			03/12/84
51307	ZION 2 - OPERABILITY OF VENT/PURGE VALVES			04/03/84
54307	ZION 2 - CONTROL RAD. CONT. MATERIALS - DEMINIMUS LEVELS			04/13/84
54366	ZION-2 - ONE TIME EXEMPTION FROM STATE AND COUNTY PENTIC. IN ANNUAL ENERG. EXERCISE			04/16/84
52119	ZION 2 - T.S. CHANGE TO CONT. LEAKAGE TESTS AND ADD T.S. FOR AIRLOCK LEAKAGE TESTS			04/18/84
52381	ZION 2 - TS CHANGE RE COOLANT CHEMISTRY AND SPECIFIC ACTIVITY			04/19/84
<u>ACTIVE ACTIONS</u>				
52186	ZION 2 - T.S. CHANGE RE AUXILIARY ELECTRIC POWER AND ECCS			<u>TARGET</u>
54177	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER			
54204	ZION 2 - SCHEDULAR EXEMPTION FROM 10CFR 50.48(C)4			
54161	ZION 2 - ITEM B.6 OF 1980 ORDER			
54902	ZION 1 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-1			
54904	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-2			
54906	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-3			
54908	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-4			
54910	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-5			
54912	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-6			
54914	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM A-7			
54916	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-1			
54936	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM E-1			
54938	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM E-2			
54940	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM F-2			
54942	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM F-4			
54944	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM F-5			
55025	ZION 2 - SCHEDULAR EXEMPTION FROM 10 CFR 50.49(G) EQ			
54918	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-3			
54920	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-4			
54922	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-5			
54924	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM B-6			
54926	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-3			
54928	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-4			
54930	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM C-5			
54932	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM D-1			
54934	ZION 2 - RECONSIDERATION OF FEB. 29, 1980 ORDER ITEM D-2			
49602	ZION 2 - MOD TO 1980 ORDER - AFWS			05/30/83
48104	ZION 2 - BORATION FLOW PATHS HEAT TRACING		3	06/30/83
42147	ZION 2 - ATWS INSTRUMENTATION MODIFICATIONS (ORDER 2/29/80 ITEM C.4)		1	06/30/83
42837	ZION 2 - FAILURE MODE EFFECTS ANALYSIS (2/29/80 ORDER ITEM F.4)		1	09/01/83
51902	ZION 2 - 6/27/83 ISI REQUEST			09/01/83
48693	ZION 2 - LICENSE CONDITION FOR TECH SPECS IN EMERG			09/30/83
41045	ZION 2 - RESIDUAL RISKS		1	02/28/84
13022	ZION 2 - STANDARD TECH SPEC LCO'S FOR TASK ACTION PLAN ITEM F 1 (F)(3)		1	03/15/84
54163	ZION 2 - REVIEW OF REVISED LICENSED OPERATOR REQUALIFICATION TOPICAL REPORT			03/30/84

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## CONDENSED MANAGEMENT REPORT

(CONTINUATION)

ZION 2

<u>TAC #</u>	<u>TAC DESCRIPTION</u>	<u>PLANT SPECIFIC</u>	<u>PRIORITY</u>	<u>CRITICAL DATE</u>
<u>ACTIVE ACTIONS</u> (CONTINUATION)				<u>TARGET</u>
54427	ZION 2 - RCS VENTS TECHNICAL SPECIFICATIONS NUREG 073 ITEM II.B.1			09/30/84
54581	ZION 2 - TECHNICAL SPECIFICATIONS COVERED BY GENERIC LETTERS 83-36, 83-37 FOR NUREG-073			09/30/84
52101	ZION 2 - ISI AND IST PROGRAM, SECOND INTERVAL			12/31/84
49684	ZION 2 - REACTOR COOLANT PUMP TRIP			02/01/85
<u>ANTICIPATED ACTIONS</u>				<u>INITIATION</u>
53649	ZION 2 - ITEM 1.2 - POST TRIP REVIEW-DATA AND INFORMATION CAPABILITY			11/01/84
53732	ZION 2 - ITEM 2.2 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - ALL SR COMPONENTS			11/01/84
53815	ZION 2 - ITEMS 3.2.1 & 3.2.2 - POST-MAINT TESTING PROCEDURES AND VENDOR RECOMMENDATIONS - ALL SR COMPONENTS			11/01/84
53898	ZION 2 - ITEM 3.2.3 - POST-MAINTENANCE TESTING -CHANGES TO TECH. SPEC. - ALL SR COMPONENTS			11/01/84
53951	-ITEMS 4.2.3 & 4.2.4-PREVENTATIVE MAINT. PROGRAM FOR REACTOR TRIP BREAKERS-LIFE TESTING & REPLACEMENT			11/01/84
54042	ZION 2 - ITEMS 4.5.2 & 4.5.3 - REACTOR TRIP SYSTEM FUNCTIONAL TESTING - ALTERNATIVES & TEST INTERVALS			11/01/84
54125	ZION 2 - ITEM 4.5.1 - REACTOR SYSTEM FUNCTIONAL CAPABILITY			11/01/84



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## CONDENSED MANAGEMENT SUMMARY REPORT

FACILITY	COMPLETE	ACTIVE	ANTICIPATED
ARKANSAS 1	22	50	8
ARKANSAS 2	29	39	7
BEAVER VALLEY 1	29	50	7
BIG ROCK POINT 1	26	47	6
BROWNS FERRY 1	33	59	6
BROWNS FERRY 2	29	53	6
BROWNS FERRY 3	27	70	6
BRUNSWICK 1	37	46	7
BRUNSWICK 2	50	49	7
CALLAWAY 1	0	30	0
CALVERT CLIFFS 1	29	41	7
CALVERT CLIFFS 2	28	32	8
COOK 1	16	32	7
COOK 2	19	33	9
COOPER STATION	14	29	6
CRYSTAL RIVER 3	17	51	8
DAVIS-BESSE 1	24	59	8
DIABLO CANYON 1	0	41	7
DRESDEN 1	1	2	0
DRESDEN 2	22	49	6
DRESDEN 3	25	41	6
DUANE ARNOLD	30	35	6
FARLEY 1	31	42	7
FARLEY 2	22	54	7
FITZPATRICK	20	41	6
FORT CALHOUN 1	19	37	7
FORT ST VRAIN	23	37	6
GINNA	32	64	7
GRAND GULF 1	23	96	6
HADDAM NECK	15	62	7
HATCH 1	21	42	6
HATCH 2	24	39	6
HUMBOLDT BAY 3	1	2	0
INDIAN POINT 2	22	44	7
INDIAN POINT 3	17	44	7
KEWAUNEE	16	42	7
LA CROSSE	23	50	6
LASALLE 1	9	16	7
LASALLE 2	5	5	6
MAINE YANKEE	28	49	9
MCGUIRE 1	10	28	7
MCGUIRE 2	11	26	7
MILLSTONE 1	25	58	7
MILLSTONE 2	27	39	7
MONTICELLO	22	47	6
NINE MILE POINT 1	26	43	6
NORTH ANNA 1	31	43	7
NORTH ANNA 2	30	46	7
OCONEE 1	16	50	8
OCONEE 2	20	50	8

DATA AS OF - 06/30/84

## CONDENSED MANAGEMENT SUMMARY REPORT

FACILITY	COMPLETE	ACTIVE	ANTICIPATED
OCONEE 3	19	47	8
OYSTER CREEK 1	17	59	6
PALISADES	27	45	7
PEACH BOTTOM 2	30	49	6
PEACH BOTTOM 3	25	49	7
PILGRIM 1	18	45	6
POINT BEACH 1	25	42	7
POINT BEACH 2	23	40	7
PRAIRIE ISLAND 1	24	22	1
PRAIRIE ISLAND 2	24	21	0
QUAD CITIES 1	14	39	6
QUAD CITIES 2	17	35	6
RANCHO SECO 1	25	60	8
ROBINSON 2	25	56	8
SALEM 1	15	38	7
SALEM 2	12	42	7
SAN ONOFRE 1	22	47	7
SAN ONOFRE 2	45	75	7
SAN ONOFRE 3	32	70	7
SEQUOYA 1	33	35	7
SEQUOYA 2	22	38	7
ST LUCIE 1	16	39	7
ST LUCIE 2	9	32	7
SUMNER 1	22	30	8
SURRY 1	23	40	7
SURRY 2	25	38	7
SUSQUEHANNA 1	24	40	6
SUSQUEHANNA 2	4	8	0
THREE MILE ISLAND 1	51	52	10
TROJAN	18	45	7
TURKEY POINT 3	24	43	7
TURKEY POINT 4	25	43	7
VERMONT YANKEE 1	18	51	6
WASHINGTON NUCLEAR 2	2	23	6
YANKEE-ROWE 1	20	53	7
ZION 1	18	68	7
ZION 2	18	66	7
TOTAL	1,887	3,759	560

SECTION VI  
MULTIPLANT ACTION DESCRIPTION  
FOR  
SELECTED ACTIVE ISSUES

Multi-Plant Action No. A-02

Title: Appendix A - ALARA

Priority: 1

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. F. Stolz

Lead Project Manager: G. Gears

Lead Technical Review Branch: Effluent Treatment Systems Branch

Problem:

Implementation of Technical Specifications (TSs) called for in 50.36a and Appendix I to 10 CFR 50.

Background:

Appendix I was published in 1975 and Model TSs were sent to all licensees on 7/78, 11/78 and 7/79.

Current Status:

Action has been completed on 22 operating plants: Prairie Island (2), Cook (2), Monticello (1), Yankee Rowe (1), Beaver Valley (1), North Anna (2), Ginna (1), Brunswick (2), Fort St. Vrain (1), Oconee (3), Rancho Seco (1), St. Lucie (1), Turkey Point (2) and Indian Point (2).

Thirty-eight TERs and SERs have been received from FRC and EG&G to date. Forthcoming actions will primarily involve completion of TERs, updated SERs, negotiated TSs and DL issuance of license amendments. DL is proceeding to expedite issuance of 18 TS packages it has already received in FY83 and FY84. Forty-three licensing actions have been scheduled for completion in FY84.



Actions Remaining:

1. Licensee to submit final TS packages, ODCM and PCP.
2. Contractor provide NRR with updated SERs and negotiated TS.
3. DL issue license amendment.

Status Updated: June 14, 1984

Multi-Plant Action No. A-04

Title: Containment Leak Testing - Appendix J

Priority: 1

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: G. Dick

Lead Technical Review Branch: CSB

Problem:

Some operating plants may not be in full compliance with Appendix J to 10 CFR Part 50.

Background:

Appendix J to 10 CFR Part 50 was published after many nuclear power plants had either received their operating license or their containments had reached an advanced state of design of construction. Accordingly, some plants may not be in full compliance with Appendix J. Licensee submittals, including exemption requests and proposed Technical Specifications need to be reviewed.

Current Status:

Franklin Research Center performed the review of the 47 remaining plants in this program under contract. All 46 TERS have been received from Franklin Research Institute. No TER is expected for Dresden 1 which is on hold. All 46 SERs have been forwarded to DL by CSB. Thirty-one licensing actions have been completed.

The licensee for Nine Mile Point 1 has submitted a revised package. An SER is expected from CSB in 3rd Q FY84. CSB reviews may be required for revised submittals for Peach Bottom 2/3, Pilgrim 1, and Hatch 1.

Actions Remaining:

ORPMs are working on the remaining SERs in preparation for issuance.

Status Updated: April 4, 1984.

Multi-Plant Action No. A-14

Title: 10 CFR 50.55a(g) Inservice Testing

Priority: 1

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: S. Varga

Lead Project Manager: P. Tam

Lead Technical Review Branch: MEB, R-I, R-II, R-III, R-IV, R-V

Problem:

Licenseses for boiling or pressurized water-cooled nuclear power facilities are required by 10 CFR 50.55a(g) to have an inservice testing program that meets the requirements of Section XI of the ASME Boiler and Pressure Vessel Code and Addenda (the Code). If the licensee determines that it is impractical to meet certain requirements of the Code, justification must be provided to the Commission, which can grant relief from specific Code requirements or impose alternate requirements. MPA A-14 is only concerned with these relief requests.

Background:

Staff review is currently several years behind originally planned schedules. The staff has previously given interim implementation guidance to the licensees by letter. In general, this guidance authorized the licensees to implement their IST plans as proposed pending completion of review by the staff. In some cases the period for which the proposed plans and interim implementation authorization would apply has expired and the licensees have made subsequent submittals before the staff completed review of the original plan.

Current Status:

The backlog of IST relief requests is being reviewed by the above named organizations. After the current backlog is cleared, reviews of requests for relief from Code requirements will be conducted on a plant-specific basis with a schedule keyed to testing program update requirements.



Actions Remaining:

SERs to be written - 6; licensing actions to be completed - 14.

Status Updated: June 14, 1984.

Multi-Plant Action No. A-17

Title: Instrumentation to Follow the Course of an Accident (R.G. 1.97)

Priority: \_\_\_\_\_

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: J. Shea

Lead Technical Review Branch: ICSB, RSB, AEB, ASB, ETSB, CPS, CSB, HFEB, RAB

Statement of Problem:

A lesson learned from the TMI-2 accident is that better information to aid the operators in preventing and mitigating the consequences of reactor accidents is needed. Regulatory Guide 1.97, Rev. 2 deals with this subject area, and its implementation is included in Supplement 1 to NUREG-0737.

Background:

Regulatory Guide 1.97 provides the minimum design criteria for permanently installed instrumentation used to provide the operator with information that may be necessary to perform his role in bringing the plant to and maintaining it in a safe condition following the accidents identified in the Design Basis. Regulatory Guide 1.97 describes methods acceptable to the staff for complying with the Commission's regulations to provide instrumentation to monitor plant variables and systems during and following an accident in a light-water cooled nuclear power plant. Involved are General Design Criteria (10 CFR 50):

- No. 13 "Instrumentation and Control"
- No. 16 "Monitoring Radioactive Releases"
- No. 19 "Control Room"

Current Status:

Operating reactor licensees, operating license applicants and holders of construction permits were requested by Generic Letter 82-33 to provide schedules for implementing the basic requirements of Rev. 2 R.G. 1.97 by April 15, 1983. Most of the reports have been received and are being evaluated.

Actions Required:

Confirmatory Orders have been issued for all plants except Fort St. Vrain, committing the utilities to firm implementation dates.

Status Updated: July 2, 1984

Multi-Plant Action No. B-04

Title: Reactor Vessel Overpressure Protection

Priority: 2

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. F. Stolz

Lead Project Manager: A. W. DeAgazio

Lead Technical Review Branch: RSB\*

Problem:

Prevent overpressurizing the reactor vessel when at low temperature. Applicable to PWRs only.

Background:

A number of events have demonstrated that administrative controls alone are not adequate to prevent low-temperature overpressure events at PWRs. All PWRs were requested to provide an overpressure protection system that is to be used when the plant is in a low-temperature shutdown condition.

Current Status:

All plants implemented their system with preliminary NRC approval followed by post implementation review of (a) electrical aspects; (b) reactor system aspects; and (c) Technical Specifications. Four plants remain for which licensing action is required. Technical reviews for all plants are complete however open issues remain to be resolved for one plant. Administrative action only is required to close the other three licensing actions.

Actions Remaining:

PM to resolve open issues on the one plant. PM to issue SER and close out remaining actions.

Status Updated: June 12, 1984



Multi-Plant Action No. B-05 (USI A-42)

Title: Stress Corrosion Cracking - Implementation Of NUREG-0313, Rev. 1

Priority: High

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: D. Vassallo

Lead Project Manager: R. J. Clark

Lead Technical Review Branch: ORAB, MTEB

Problem:

Leaks and cracks in the heat-affected zones (HAZs) of welds in BWR austenitic stainless steel piping and components have been observed since the mid-1960's. All the cracks have been attributed to intergranular stress corrosion cracking (IGSCC). The revised purpose of this task is to ensure that all BWRs have adequate programs - as defined in NUREG-0313, Rev. 1 - to: (1) design systems constituting the reactor coolant system pressure boundary (RCSPB) to minimize the potential for IGSCC, (2) select materials for the RCSPB resistant to IGSCC, (3) control the water chemistry environment so as to reduce the potential for intergranular corrosion, (4) detect potential cracking through augmented inspections before it develops into a through-wall leak, and (5) detect possible RCSPB leakage before it becomes significant by adopting more stringent limits on allowable unidentified leakage.

Background:

In 1975, NRC (AEC) formed a Pipe Cracking Study Group (PCSG) to investigate the cause of cracking in BWR piping. In July 1977 the staff issued an implementation document, NUREG-0313, based on recommendations of the PCSG. As a result of cracking in large diameter pipes and in nozzles which had not been observed prior to 1975, a new PCSG was activated in September 1978. In October 1979 Revision 1 to NUREG-0313 was issued for comment; this contained the latest recommendations of the Study Group. Based on industry comments, the document was modified and reissued. In view of the revised staff acceptance criteria, the previous reviews were terminated (TACs 08207 through 08230 plus 11263). New TACs were issued (TACs 46651-46674) to cover the revised review effort.

Current Status:

In February 1981, NUREG-0313, Rev. 1 was issued to all holders of BWR operating licenses or construction permits and to all applicants for operating licenses. By July 1, 1981 the applicants/licensees were to provide their program for replacement of service sensitive lines and welds, their program for augmented inservice inspection, their program for improving the water chemistry environment and incorporation of adequate leak detection capability. These submittals were reviewed by EG&G Idaho under a technical assistance contract. While these reviews were underway, extensive cracks were found in the summer of 1982 in the large diameter recirculation system piping at Nine Mile Point Unit No. 1. This led to issuance of IE Bulletin 82-03 and 83-02, as well as some orders, requiring extensive inspections of service-sensitive welds in RCSPB piping. These inspection requirements were more extensive than the requirements in NUREG-0313, Rev. 1. Also, licensees of BWRs that found indications of cracks were required by amendments or orders to adopt more stringent limits on unidentified leakage. In effect, the requirements in NUREG-0313, Rev. 1 have been superseded by more recent NRC requirements. To close out this multiplant action, Safety Evaluations are scheduled to be issued in the third quarter of FY84 documenting the reviews that were performed and how the requirements of NUREG-0313, Rev. 1 have been implemented for each plant.

Actions Remaining:

MTEB provided Safety Evaluations for the 24 BWRs covered by this MPA to DL in May 1984. Letters have been or will be issued with the Safety Evaluations to utilities in June 1984 closing out action.

Status Updated: June 12, 1984

Multi-Plant Action No. B-17

Title: Tech Spec Surveillance for Hydraulic Snubbers

Priority: 1

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. R. Miller

Lead Project Manager: Leon B. Engle

Lead Technical Review Branch: See Current Status Below

Problem:

Operability of snubbers for LWRs is required to provide assurance that the structural integrity of the reactor coolant system and all other related systems is maintained during and following a seismic or other event initiating dynamic loads. OPERABILITY is verified by an Inservice Inspection and Testing Surveillance program specified in plant Technical Specifications (TSs). Recent operating experience has indicated the need for changes, clarifications and improvements in the Inservice Surveillance Requirements for hydraulic snubbers.

Background:

By letter dated November 20, 1981, the NRC requested that all power reactor licensee's (except SEP) incorporate the model NRC TS in plant specific Appendix A TS. A similar request was sent to SEP operating licensees on March 23, 1981. The NRC TS require that the visual inspection frequency be based upon maintaining a constant level of snubber protection to systems. In order to provide assurance of hydraulic snubber functional reliability, a representative sampling of plant installed snubbers must be functionally tested during plant shutdowns at 18 month intervals. The required sampling provides a confidence level of 95% that 90% to 100% of plant specific snubbers will be OPERABLE within acceptance limits.

Current Status:

In October 1982, management decided to transfer the remaining B-17 actions to the Regions.

The official transfer package to the Regions for B-17 was completed on March 1983. Region I and V contracted the initial review to outside consultants. Completion dates for FY83 slipped into FY84. To date Turkey Point 3&4 and Fort St. Vrain have been completed. SERs have been received for Peach Bottom 2&3 and Millstone Unit 2. Final action for Peach Bottom 2&3 has slipped but should be completed by June 30, 1984. An SER has been received for Millstone Unit 2. Amendment requests have been received for Arkansas Units 1&2 and Region IV is scheduled to produce SER no later than July 30, 1984. Twenty eight reactor units need to be completed before B-17 is complete. The Regions are being requested to expedite reviews.

Status Updated: June 18, 1984



Multi-Plant Action No. B-22

Title: Tech Spec Surveillance for Mechanical Snubbers

Priority: 1

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. R. Miller

Lead Project Manager: Leon B. Engle

Lead Technical Review Branch: See Current Status Below

Problem:

OPERABILITY of snubbers for LWRs is required to provide assurance that the structural integrity of the reactor coolant system and all other related systems is maintained during and following a seismic or other event initiating dynamic loads. Operability is verified by an Inservice Inspection and Testing Surveillance program specified in plant Technical Specifications (TS). Prior to initiation of MB-22, mechanical snubbers had not been included in operating plant surveillance programs. Recent operating experience has indicated that mechanical snubbers require OPERABILITY requirements similar to those required for hydraulic snubbers.

Background:

By letter dated November 20, 1981, the NRC requested that all power reactor licensee's (except SEP) incorporate the model NRC TS for mechanical snubbers in plant specific Appendix A TS. A similar request was sent to SEP operating licensees on March 23, 1981. The NRC TS require that the visual inspection frequency be based upon maintaining a constant level of snubbers protection to systems. In order to provide assurance of mechanical snubber functional reliability, a representative sampling of plant installed snubbers must be functionally tested during plant shutdowns at 18 month intervals. The required sampling provides a confidence level of 95% that 90% to 100% of plant specific snubbers will be OPERABLE within acceptance limits.

Current Status:

In October 1982, management decided to transfer the remaining B-22 actions to the Regions.

The official transfer package to the Regions for B-22 was completed in March, 1983. Region I and V contracted the initial review to an outside consultant. The additional time required for this review impacted scheduled completion dates for FY83 and FY84. To date Big Rock Point, Brunswick 1&2, Fort St. Vrain, and Turkey Point 3&4 have been completed. Final action for PeachBottom 2&3 should be completed by June 30, 1984. An SER has been received for Millstone Unit 2. Arkansas 1&2 SERs are scheduled no later than July 30, 1984. Twenty one reactor units need to be completed before B-22 is complete. The Regions are being requested to expedite reviews.

Status Updated: June 13, 1984

Multi-Plant Action No. B-23

Title: Potential Equipment Failures Associated with Degraded Grid Voltage

Priority: 1

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: R. Dudley

Lead Technical Review Branch: Power Systems Branch

Problem:

Potential exists for supplying safety and non-safety equipment with voltage outside the range for which it was designed for longer than transient periods. Such conditions could result in safety-related equipment being unavailable to mitigate the consequences of an accident.

Background:

On July 20, 1976, Northeast Nuclear Energy Company (NNECo) reported that, following a trip of Millstone Unit no. 2 on July 5, 1976, several motors powered from 480 volt (v) motor control centers failed to start as required. The failure of the 480 v motors to start was traced to blown control power fuses on the individual motor controllers. These controllers received control power through 480 v/120 v transformers within the controller.

NNECo's investigation disclosed that, as a result of the plant trip, the grid voltage dropped from 352 kv to 333 kv. This voltage drop, in conjunction with additional voltage drops associated with the transformers involved, reduced the control power and voltage within individual 480 v controllers to a voltage which was insufficient to actuate the main line controller contactors. As a result, when the motors were signalled to start, the control power fuses were blown. Subsequent testing by NNECo showed that the contactors required at least 410 v to function properly.

NNECo concluded that under similar low voltage conditions, the operability of 480 v Engineered Safety Feature equipment could not be assured.

NNECO's immediate corrective action was to raise the setpoint of the Engineered Safeguards Actuation System (ESAS) "loss of power" under-voltage relays to assure that the plant would be separated from the grid and that emergency power system (dual) operation would be initiated before the control voltage fell below that required for contactor operation. A trip of the under-voltage relays causes the emergency buses to be de-energized and a load shed signal to strip the emergency buses, the diesel generators to start and power the emergency buses, and required safety related loads to sequence start on the buses.

On July 21, 1976, NNECO reported that the earlier correction action taken was no longer considered appropriate because during starting of a circulating water pump, the voltage dropped below the new ESAS undervoltage relay setting. This de-energized the emergency buses, caused load shedding to occur, started the diesel generators and began sequencing loads onto the emergency buses in accordance with the design. However, during sequencing of the loads onto the buses the voltage again dropped below the undervoltage relay setting which caused the load shed signal to strip the buses. The result was energized emergency buses with no loads supplied.

#### Current Status:

All utilities operating plants (BWR, PWR, HTGR) licensed prior to 1976 must provide under-voltage relays which separate the plant from the grid (in case of degraded grid voltage) and activate the plant emergency power system. The current DL schedule requires this item to be completed in FY84. Technical review is provided by the Power Systems Branch, with contractor technical support from Lawrence Livermore Lab and EG&G.

- 67 - operating plants covered by B-23
- 67 - SERs completed (3 of which have open items)
- 61 - Licensing Actions completed

#### Actions Remaining:

Complete the 3 remaining SERs. Complete the 6 remaining licensing actions.

Status Updated: June 14, 1984



Multi-Plant Action No. B-24\*  
\*B-20 integrated into B-24 on 8/25/80

Title: Venting and Purging Containment While At Full Power and Effect of LOCA

Priority: High

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: Steven A. Varga

Lead Project Manager: Edward A. Reeves

Lead Technical Review Branch: CSB/DSI, M. Fields

Problem:

Purging and/or venting of reactor containment buildings with reactor coolant temperature above 200°F may result in site boundary doses exceeding 10 CFR 50 Part 100 limits if purge or vent valves fail to shut during a LOCA. The lead branch is CSB, however assisted by EQB/DE - R. Wright, RAB/DSI - K. Dempsey, and ORAB/DL - T. Alexion.

Background:

Events at Millstone 1 and Salem 1 (where the automatic high radiation closure of purge valves was overridden by operators) resulted in a generic letter being sent to all licensees of operating reactors in November 1978 and a report to Congress (#78-05). NRC advised licensees to take action to minimize purge and vent operations and to review and respond to SRP 6.2.4 and BTP CSB 6-4. In October 1979 a Staff Interim Position was developed. Licensees were requested to conform to it pending completion of our long-term review. In August 1980 generic item B-20 Leakage of Resilient Seals for purge and vent valves was integrated into the B-24 issue. During 1981 Technical Specifications (e.g., lock closed inoperable valves, only purge for safety related reasons, and seal leakage tests) were developed and included in guidance to the licensees.

Current Status:

1. Based on EQB/DE safety evaluations, we sent requests for valve qualification data and thirteen 50.54(f) letters to thirty licensees where insufficient data had been provided. Ten of the licensees closed the valves; others provided data which EQB/DE provided twelve memoranda or SEs to DL. The Quarterly Review Meeting schedules of April 20, 1984 indicate all reviews will be completed in the third quarter.

2. Technical reviews are completed for the electrical override concerns, for the radiological consequences concerns and for compliance with BTP CSB 6-4. Valve operability reviews are near completion. Plant specific reviews of the few remaining open issues and completion of Technical Specifications are underway. A total of 258 plant safety evaluations have been issued for this MPA alone.
3. Most of remaining action is with DL Project Managers who should expedite closeout actions and finalize Technical Specifications per lead PM memoranda of June 19, 1981, January 4, 1984, and MPA Quarterly Review memorandum dated April 20, 1984.

Status Updated: June 15, 1984

Multi-Plant Action No. B-25

Title: Feedwater and CRD Nozzle Cracking

Priority: \_\_\_\_\_

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: R. Gilbert

Lead Technical Review Branch: MTEB

Problem:

BWR feedwater nozzles and control rod drive nozzles have developed cracks due to thermal fatigue.

Background:

The generic issue has been resolved by the 11/13/80 issuance of NUREG-0619 which provided licensees of BWRs with acceptable methods to resolve the issue on a plant-by-plant basis.

Current Status:

Twenty-two plants have committed to follow the staff's guidance and have received letters confirming that their proposed action is acceptable. The licensing action is complete on these plants. Each will submit a post-implementation report following completion of their proposed modification.

An SER from MTEB has been received for the remaining units. However, some rewriting of the SER has been necessary. It is anticipated that the issue will be closed out by the end of FY84.

Actions Remaining:

Complete the review on the remaining plant. Confirm commitments to follow NUREG-0619 guidance or accept the licensee's proposed deviations from the NUREG suggestions.

Status Updated: June 13, 1984

Multi-Plant Action No. B-41

Title: Fire Protection - Appendix R, 10 CFR 50 Compliance

Priority: High

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: T. Wambach

Lead Technical Review Branch: CEB, ASB, PSB, ICSB, RSB

Problem:

Protection of safe, shutdown capability from the adverse effects of fire in any area of the plant. During the OL review of plants licensed prior to July 1976, no in-depth review was performed in this area.

Background:

Fire at Browns Ferry Nuclear Plant in March 1975 disabled redundant safety divisions and normal safe shutdown equipment. NUREG-0050.

Current Status:

10 CFR 50.48 and Appendix R to 10 CFR 50 became effective February 17, 1981. The licensees for 69 units licensed prior to January 1, 1979 have made submittals describing (1) plans and schedules to meet Appendix R including requests for exemption from the schedules of 10 CFR 50.48, (2) design descriptions of alternative, safe shutdown modifications, where those have been proposed, and (3) exemption requests from the technical requirements of Appendix R. We have completed the review and licensing actions for the exemption requests on 59 of the units. There were no exemption requests from 5 of the units. Technical review for 7 units is ongoing for second round submittals resulting from the denial of the original exemption requests. The technical review of exemption requests for 3 units have been completed and are awaiting licensing action in DL. Design descriptions of modifications for alternative, safe shutdown were submitted for all but 5 units. The modifications for 64 units have been approved. Revised proposals have been received for 9 units. These are being reviewed as plant specific items and are not included under this MPA.



Actions Remaining:

CMEB must complete the review of exemption requests for the 7 units which are not yet completed. DL must complete the licensing action for the 3 units for which the technical review has been completed and for the 7 units above when the technical review is completed. ASB is completed with this MPA.

Status Updated: July 10, 1984

Multi-Plant Action No. B-48

Title: Adequacy of Station Electric Distribution Voltage

Priority: 2

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: R. Dudley

Lead Technical Review Branch: Power Systems Branch

Problem:

This review is intended to determine if the onsite distribution system in conjunction with the offsite power sources has sufficient capacity and capability to automatically start and operate all required safety loads within the equipment voltage rating.

Background:

At Arkansas Nuclear 1, loss of offsite power indicated that the electric distribution systems were inadequate for two unit operation. All utilities were asked to review their distribution systems to see if voltages were adequate.

Current Status:

69 - operating plants covered by B-48  
69 - SERs completed (6 of which have open items)  
61 - Licensing Actions completed

Actions Remaining:

Complete the remaining 6 SERs. Complete the remaining 7 licensing actions

Status Updated: June 14, 1984

Multi-Plant Action No. B-57

Title: Decay Heat Removal Capability

Priority: 2

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. F. Stolz

Lead Project Manager: A. DeAgazio

Lead Technical Review Branch: RSB for six plants

Problem:

To assure redundancy in decay heat removal capability by incorporation of requirements into the Technical Specification.

Background:

Various events at PWRs that have occurred during shutdown conditions have indicated that administrative controls to assure decay heat removal capability have been inadequate. All PWR licensees were requested (in a letter dated June 11, 1980) to incorporate requirements for redundancy into their TSs.

Current Status:

Acceptance criteria are: (1) redundancy in decay heat removal capability must be provided for all modes or temperature ranges of operation, particularly in Modes 4, 5 and 6, and (2) adequate surveillance requirements must be provided. A total of 6 plants remain to be reviewed.

Actions Remaining:

Questions have been issued to licensees for 5 plants. Review responses and resolve outstanding issues. Complete SER for remaining plant.

Status Updated: June 14, 1984



Multi-Plant Action No. B-59

Title: Masonry Wall Design

Priority: 2

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. R. Miller

Lead Project Manager: C. Trammell

Lead Technical Review Branch: SGEB

Problem:

Inadequate design criteria for masonry walls. Applies to all plants.

Background:

Problem was discovered at Trojan during reviews associated with IEB 79-02 (anchor bolts) and IEB 79-14 (as-built verification). A masonry wall was found which supported numerous safety-related pipes which was not adequate to resist seismic forces. This resulted in the issuance of IE Bulletin 80-11 on May 8, 1980. NRR (SEB) has been assigned the responsibility to review licensee's responses to this Bulletin.

Current Status:

Franklin Research Center has been contracted to review most licensee submittals. Trojan is being reviewed by SEB. Dresden 1 and Humboldt Bay are currently in a shutdown status, and will be reviewed by SEB if/when needed. Review schedule and acceptance criteria for review has been established.

Actions Remaining:

44 plants remain to be completed. 26 finished to date. FRC review is active. Q-1s issued to all licensees.

Status Updated: April 12, 1984.

Multi-Plant Action No. B-60

Title: Environmental Qualification of Electric Equipment for Nuclear Power Plants

Priority: 1

Cognizant Assistant Director: F. J. Miraglia

Cognizant Branch Chief: G. Holahan

Lead Project Manager: Rudy O. Karsch

Lead Technical Review Branch: Equipment Qualification Branch

Problem:

All plants are required to meet 10 CFR 50.49 concerning 10 CFR 50, Appendix A, GDC 4.

Background:

The issue originated from a 1977 petition from the Union of Concerned Scientists on fire protection and equipment qualification. The multiplant action arose from the Commission Memorandum and Order CLI 80-21. Technical resolution of acceptance criteria were produced from Unresolved Safety Issue A-24.

Current Status:

Technical Evaluation Reports (TERs) for all of the seventy-one (71) operating plants have been issued by Franklin Research Center.

The Equipment Qualification Branch has issued all seventy-one (71) Safety Evaluation Reports. The Division of Licensing has issued all of the seventy-one (71) attached with TERs to the licensees.

The equipment qualification rule, 10 CFR 50.49, was issued on January 21, 1983, Federal Register Volume 48, No. 15, pp 2729-2734, and has been effective since February 22, 1983. The rule requires that each holder of an operating license issued prior to February 22, 1983, shall, by May 20, 1983, identify the electrical equipment important to safety within the scope of the rule already qualified and complete final equipment qualification for the remaining electrical equipment important to safety within the scope of the rule by the end of the second refueling outage after March 31, 1983, or by March 31, 1985, whichever is earlier.

The staff has completed a series of meetings with the licensee to discuss their responses to the SERs including the information for items in various categories for which justification for continued operation was not previously submitted to the NRC. SERs closing out the remaining open items are currently in preparation.

Actions Remaining:

Future actions required to complete the MPA will be predicated on the responses by the licensees to the SERs. The licensees are being requested (1) to provide, within ten (10) days of the receipt of the SER from the NRC, the resolution for the unqualified equipment if any, (2) to reaffirm the justification for continued operation, and within thirty (30) days of the receipt of the SER, submit information for equipment items neither considered qualified nor unqualified for which justification for continued operation was not previously submitted to the NRC, and (3) to provide the schedule for accomplishing proposed corrective action by May 20, 1983 as required by Paragraph (g) of 10 CFR 50.49.

An implementation plan needs to be developed which will establish the ground rules for evaluating the licensee's justification for continued operation.

Status Updated: June 18, 1984

Multi-Plant Action No. B-65

Title: Safety Concerns Associated With Pipe Breaks in BWR Scram System

Priority: High

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: D. Vassallo

Lead Project Manager: V. Rooney

Lead Technical Review Branch: ASB

Problem:

Assess the overall risk to long-term core cooling from postulated pipe break in Scram Discharge Volume; investigate the mitigation capability for the postulated event; establish generic recommendations for correcting any deficiencies identified.

Background:

Draft NUREG-0785 issued April 3, 1981 by AEOD described certain safety concerns associated with postulated pipe breaks in the BWR Scram System. NRC letter dated April 10, 1981 to all BWR licensees requested a generic evaluation of the concerns within 45 days and a plant specific review within 120 days. The generic evaluation was submitted by GE on behalf of the BWR owners by letter dated April 30, 1981.

Current Status:

NUREG-0803 was issued to BWR licensees and applicants on 08/31/81. Plant specific licensee responses to concerns identified in this NUREG were evaluated together with a BWROG submittal. In response to a July 25, 1983 letter, the BWROG provided additional information by letter dated November 18, 1983. On February 23, 1984 the staff met with the BWROG and identified information needed to close out this issue generically.

Actions Remaining:

Review BWROG report dated May 10, 1984 and take appropriate closeout action.

Status Updated: June 13, 1984.



Multi-Plant Action No. B-66

Title: Natural Circulation Cooldown

Priority: Medium

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: J. Miller

Lead Project Manager: K. Heitner

Lead Technical Review Branch: RSB (R. Jones)

Problem:

All PWRs must review plant cooldown on natural circulation to determine if voiding will occur in the reactor vessel.

Background:

St. Lucie Unit 1, while cooling down on natural circulation due to a component cooling water malfunction, developed a steam space in the upper head of the reactor vessel. Core cooling was not lost, however, complex situation was deemed an undesirable challenge to the operator.

Current Status:

Generic Letter 81-21, issued May 5, 1981, requested:

1. demonstration that natural circulation cooldown by procedure should not result in reactor vessel voiding,
2. verification that supplies of condensate grade aux feedwater are sufficient, and
3. aspects of training and procedures dealing with this subject.

All licensees have responded.

Westinghouse Owners Group Generic Report (OG-57) dated 04-20-81.

Plant specific review for leadcase (St. Lucie) is complete.

Safety Evaluations have been completed for all Westinghouse and Combustion Engineering plants. Licensing action is completed for all but Diablo Canyon.

Actions Remaining:

Review has been completed for TMI-1. The staff must complete the review for remaining 7 B&W plants. A new round of questions was sent to the licensees, in order to resolve the question at the proper cooldown rate for the remaining B&W plants.

Status Updated: June 11, 1984

Multi-Plant Action No. B-69

Title: PWR Main Steam Line Break with Continued Feedwater Addition

Priority: High (IE Bulletin 80-04)

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: J. Miller

Lead Project Manager: E. Tourigny

Lead Technical Review Branch: RSB, ICSB/DSI

Problem:

A licensee submitted a report to NRC identifying a deficiency in the original main steam line break (MSLB) analysis for their plant. They determined that, if the auxiliary feedwater system continued to supply feedwater at runout conditions to the affected steam generator, containment design pressure would be exceeded in about 10 minutes. The long term effect of the water supplied by the auxiliary feedwater system also had not been considered in prior analyses.

Background:

Virginia Electric and Power Company (VEPCO) submitted a report to the Nuclear Regulatory Commission (NRC) dated September 7, 1979, identifying a deficiency in the original main steam line break (MSLB) analysis for North Anna Power Station, Units 3 and 4. Stone and Webster Engineering Corporation re-analyzed the containment pressure response following a MSLB and determined that, if the auxiliary feedwater system continued to supply feedwater at runout conditions to the affected steam generator, containment design pressure would be exceeded in about 10 minutes. The long term effect of the water supplied by the auxiliary feedwater system had not been considered in prior analyses.

On October 1, 1971, IE Information Notice No. 79-24 was sent to all holders of operating licenses and construction permits, informing them of VEPCO's finding. The Consumer Power Company of Michigan (Palisades, Unit 1 facility) conducted an accident analysis review pursuant to the Information Notice and found that with offsite power available, the condensate pumps would feed the affected steam generator at an excessive rate. This was not considered in their analysis of the MSLB accident.

Because of the deficiencies that were identified in the original MSLB analyses, IE Bulletin No. 80-04 was issued on February 8, 1980. The IE Bulletin requested licensees and certain applicants of PWR facilities to assess the effect of continued feedwater or condensate addition on the containment response and reactor core reactivity response to a MSLB accident. The IE Bulletin was sent to all licensed PWR facilities and to those PWR facilities nearing licensing (Diablo Canyon, McGuire, North Anna 2, Salem 2, and Sequoyah).

Current Status:

All reviews are complete except for Indian Point 2 and Rancho Seco. These reviews should be completed by Jun 30, 1984.

Status Updated: June 12, 1984



Multi-Plant Action No. B-72

Title: NUREG-0737 Tech. Specs. (Generic Letter 82-16 and 83-02)

Priority: 2

Cognizant Assistant Director: F. J. Miraglia

Cognizant Branch Chief: G. Holahan

Lead Project Manager: C. Patel

Lead Technical Review Branch: Regions for 82-16 responses, ORAB for most 83-02 responses

Problem:

To establish Technical Specifications for TMI (NUREG-0737) issues that were originally scheduled for completion by December 31, 1981.

Background:

Generic Letters 82-16 and 83-02 requested that each PWR and BWR licensee review their facilities Technical Specifications (TSs) with respect to the guidance provided. Licensees were requested to propose TSs for the NUREG-0737 issues identified in 82-16 and 83-02. Responses have been received from all licensees.

Current Status:

Regional review of PWR responses to 82-16 is complete except for Arkansas 1 and 2, Crystal River, Fort Calhoun, Indian Point 1 and 2, San Onofre 1, 2 and 3, Summer 1, Turkey Point 3 and 4, and Yankee Rowe. All reviews by regions are scheduled for completion by August 30, 1984 or earlier. The regions have also completed reviews of two BWR responses to 83-02 and will also complete the review of the Cooper response to 83-02 by June 15, 1984.

ORAB is reviewing all but the above three BWR responses to 83-02. ORAB has completed their review of 83-02 responses for all plants.

MFA B-72 (continued)

- 2 -

Project Managers have closed out B-72 for 34 plants with the balance scheduled for close out by September 30, 1984 or earlier.

Status Updated: June 12, 1984

Multi-Plant Action No. B-75

Title: Cracking of Core Barrel Bolts In B&W Reactors

Priority: 1

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: J. Stolz

Lead Project Manager: N. Prasad Kadambi

Lead Technical Review Branch: MTEB, MEB, RSB

Problem:

Ultrasonic testing and laboratory results indicate that the A-286 core barrel bolts may crack prematurely in the head-to-shank transition area. Justification for continued operation is required for reactors which cannot demonstrate integrity in a sufficient number of core barrel bolts.

Background:

The core barrel bolts (upper and lower) support the full weight of the core in the B&W reactor design. The main problem appears to exist in the upper core barrel bolts, although a significantly smaller number of lower core barrel bolts also show indications of failure. On May 6, 1983, the B&W Owners Group reported to NRC staff on the results of their efforts to determine the nature, extent, cause and consequences of the bolt cracking problem. The objective of the presentations was to provide justification for continued operation in the cases of the units ANO-1, Oconee 1, 2 and 3, and Davis Besse 1. Also, they sought to provide assurance that a sufficient understanding has been achieved of the problem to assure public health and safety for all B&W units with the introduction of proposed new elements into their ISI and surveillance programs.

Current Status:

The B&W Owners Group has submitted to NRC a report based on the May 6, 1983 meeting (BAW 1784, May 1983). Since then, all the operating B&W reactors have inspected the core barrel bolts. The broken bolts which have been found beyond those reported in BAW 1784 are two upper core barrel bolts in Oconee 3 reported in March 1984.

Actions Remaining:

The B&W Owners Group's Internal Bolting Task Force is scheduled to submit a comprehensive report on the bolting problem in July 1984. The report is expected to propose long term solutions including consideration of materials selection, assembly procedures, ISI, surveillance, etc. The report will use the data from all the examinations performed so far. The action required to complete this MPA is for NRC staff to review the report when submitted and determine if the recommendations constitute a generic solution to the problem.

Status Updated: June 14, 1984



Multi-Plant Action No. B-76

Title: Item 1.1 - Post-Trip Review - Program Description and Procedure

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: A. Schwencer

Lead Project Manager: A. Bournia

Lead Technical Review Branch: LQB

Problem:

Licensees shall describe their program for ensuring that unscheduled reactor shutdowns are analyzed and that a determination is made that the plant can be restarted safely.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Item 1.1 Generic Letter 83-28 concerns post-trip review procedures.

Current Status:

Sixty-one percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: June 14, 1984

Multi-Plant Action No. B-77

Title: Item 2.1 - Equipment Clarification and Vendor Interface - RTS Components

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: A. Schwencer

Lead Project Manager: R. Auluck

Lead Technical Review Branch: ICSB

Problem:

Licensees shall confirm that all components whose functioning is required to trip the reactor are identified as safety-related on documents, procedures, and information handling systems used in the plant to control safety-related activities, including maintenance, work orders, and parts replacement. In addition, for these components, licensees and applicants shall establish, implement and maintain a continuing program to ensure that vendor information is complete, current and controlled throughout the life of the plant.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Item 2.1 of Generic Letter 83-28 concerns equipment classification and vendor interface for reactor trip system components.

Current Status:

Thirty-two percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: June 14, 1984

Multi-Plant Action No. B-78

Title: Items 3.1.1 and 3.1.2 - Post Maintenance Testing Procedures and Vendor Recommendations - RTS Components

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: E. Adensam

Lead Project Manager: J. Hopkins

Lead Technical Review Branch: Regions

Problem:

Licensees shall submit the results of their review of test and maintenance procedures and Technical Specifications to assure that post-maintenance operability testing of safety-related components in the reactor trip system is required to be conducted and that the testing demonstrates that the equipment is capable of performing its safety functions before being returned to service. Licensees shall submit the results of their check of vendor and engineering recommendations to ensure that any appropriate test guidance is included in the test and maintenance procedures or the Technical Specifications, where required.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Items 3.1.1 and 3.1.2 of Generic Letter 83-28 concern post-maintenance testing procedures and vendor recommendations for reactor trip system components.

Current Status:

Fifty percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: June 21, 1984

Multi-Plant Action No. B-79

Title: Item 3.1.3 - Post-Maintenance Testing - Changes to Tech Specs - RTS Components

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: E. Adensam

Lead Project Manager: J. Hopkins

Lead Technical Review Branch: ICSB (J. Calvo)

Problem:

Licensees shall identify, if applicable, any post-maintenance test requirements in existing Technical Specifications which can be demonstrated to degrade rather than enhance safety. Appropriate changes to these test requirements, with supporting justification, shall be submitted for staff approval.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Item 3.1.3 of Generic Letter 83-28 concerns changes to the Technical Specifications regarding post-maintenance testing for reactor trip system components.

Current Status:

Seventy percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: April 30, 1984.



Multi-Plant Action No. B-80

Title: Item 4.1 - Reactor Trip System Reliability - Vendor Related Modifications

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: E. Adensam

Lead Project Manager: J. Hopkins

Lead Technical Review Branch: Regions

Problem:

Licensees shall review all vendor-recommended reactor trip breaker modifications to verify that either: (1) each modification has, in fact, been implemented; or (2) a written evaluation of the technical reasons for not implementing a modification exists.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Item 4.1 of Generic Letter 83-28 concerns vendor related modifications for reactor trip breakers.

Current Status:

Eighty-one percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Update: April 30, 1984.

Multi-Plant Action No. B-81

Title: Items 4.2.1 and 4.2.2 - Preventative Maintenance and Trending Program for Reactor Trip Breakers

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: A. Schwencer

Lead Project Manager: R. Perch

Lead Technical Review Branch: EQB (R. Wright)

Problem:

Licensees shall describe their preventative maintenance and surveillance program to ensure reliable reactor trip breaker operation. The program shall include a planned program of periodic maintenance, including lubrication, housekeeping, and other items recommended by the equipment supplier, and trending of parameters affecting operation and measured during testing to forecast degradation of operability.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Items 4.2.1 and 4.2.2 of Generic Letter 83-28 concern a preventative maintenance and trending program for reactor trip breakers.

Current Status:

Twenty-three percent of the November 5, 1983 responses are complete and ready for technical review by the staff.

Status Updated: June 14, 1984

Multi-Plant Action No. B-82

Title: Item 4.3 - Automatic Actuation of Shunt Trip Attachment for W and B&W Plants

Priority: High

Cognizant Assistant Director: T. Novak

Cognizant Branch Chief: B. Youngblood

Lead Project Manager: J. Holonich

Lead Technical Review Branch: ICSB (J. Calvo)

Problem:

Licenseses of Westinghouse and B&W plants shall modify their reactor trip systems to provide automatic actuation of the breaker shunt trip attachments. The shunt trip attachment shall be considered safety related.

Background:

Generic Letter 83-28 was issued on July 8, 1983. The letter identified staff positions stemming from the review of the Salem ATWS Events. These positions are related to reactor trip system reliability and general management capability. The letter requested licensees and applicants to furnish, no later than November 5, 1983 (120 days from July 8, 1983), the status of current conformance with the positions, and the licensees' plans and schedules for any needed improvements for conformance with the positions. Item 4.3 of Generic Letter 83-28 concerns automatic actuation of the shunt trip attachment for Westinghouse and B&W plants.

Current Status:

Eleven SERs issued. All but 1 with open items. Eighteen other facility responses have been received and are ready for technical review by the staff. There are 42 facilities affected by this MPA.

Status Updated: June 18, 1984

Multi-Plant Action No. B-83

Title: NUREG-0737 Tech. Specs. (Generic Letter 83-36 and 83-37)

Priority: 2

Cognizant Assistant Director: F. J. Miraglia

Cognizant Branch Chief: G. Holahan

Lead Project Manager: C. Patel

Lead Technical Review Branch: ORAB

Problem:

To establish Technical Specifications for TMI (NUREG-0737) issues that were originally scheduled for completion after December 31, 1981.

Background:

Generic Letters 83-36 and 83-37 requested that each PWR and BWR licensee review their facilities Technical Specifications (TSs) with respect to the guidance provided. Licensees were requested to propose TSs for the NUREG-0737 issues identified in 83-36 and 83-37. Responses have been received from many licensees.

Current Status:

ORAB is reviewing the responses from licensees. ORAB is planning to complete the review of majority of the responses by end of July 1984. About 25 licensees have not submitted the response to these generic letters. These licensees will be requested to provide the responses as soon as possible.

Status Updated: June 12, 1984



Multi-Plant Action No. C-04

Title: ESF Filter Tech. Specs.

Priority: 3

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: J. Lombardo

Lead Technical Review Branch: ETSB

Problem:

Some licensees have not submitted T.S. regarding ESF filters which comply with the position of R.G. 1.52.

Background:

Letters sent out in 1976-1977 to all licensees. Most have been completed.

Current Status:

All actions completed.

Actions Remaining:

None.

Status Updated: June 12, 1984

Multi-Plant Action No. C-07

Title: Fuel Handling Accident Inside Containment

Priority: 2

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: J. Lombardo

Lead Technical Review Branch: \_\_\_\_\_

Problem:

By letter dated January 3, 1977 to Commissioner Gilinsky, Mr. Robert Pollard recommended that NRR review the potential consequences of a postulated fuel handling accident at each operating plant because this accident had not been considered during licensing of the plants.

Background:

Letters were sent to licensees of 39 operating plants in January 1977 requesting that they provide a detailed evaluation of the potential consequences of this accident. Letters were not sent to licensees of 24 operating plants because this accident had been evaluated in their SER.

Current Status:

There are two plants which are not complete.

Actions Remaining:

The Project Manager is pursuing resolution of the remaining issues.

Status Updated: June 12, 1984

Multi-Plant Action No. C-10

Title: Control of Heavy Loads At Nuclear Power Plants

Priority: Phase I, High

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: S. Varga

Lead Project Manager: D. Neighbors

Lead Technical Review Branch: ASB

Problem:

Dropped Heavy Loads may impact spent fuel, fuel in the core, or equipment that may be required to achieve safe shutdown and continued decay heat removal for PWRs and BWRs. If sufficient spent fuel or fuel in the core were damaged and if the fuel is highly radioactive, the potential releases of material could result in off-site doses that exceed 10 CFR Part 100 limits.

Background:

USI - Task A-36 was established to examine staff licensing criteria, the adequacy of measures in effect at operating plants, and to recommend necessary changes to assure the safe handling of Heavy Loads. NUREG-0612 provides the results of the review and includes the task groups recommendation.

Current Status:

1. NUREG-0612 was issued to all licensees by letter dated December 22, 1980. The letter requested: interim actions to be completed in 90 days, a Phase I Action (report, confirmation and justification) in six months and Phase II (specific requirements) in nine months.
2. The criteria contained in NUREG-0612 is used for the acceptance criteria. The contractor is Franklin Research Center (FRC). FRC provides the TER's to the lead technical group ASB.
3. Fifty actions have been closed out at this time.

MPA C-10 (continued)

- 2 -

Actions Remaining:

Project Managers expedite licensees' responses necessary to close out open items as discussed in draft TER issued.

Status Updated: June 15, 1984



Multi-Plant Action No. C-11

Title: Reactor Protection System (RPS) Power Supplies

Priority: \_\_\_\_\_

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: D. Vassallo

Lead Project Manager: M. Thadani

Lead Technical Review Branch: PSB

Problem:

Design deficiencies have been identified in the BWR RPS MG set regulators that could adversely affect the operability of the RPS if (1) a sequence of undetected single component failures were to occur, or (2) a seismic event occurred that resulted in the postulated component malfunctions.

Background:

The design deficiency was first identified during the Hatch 2 licensing process. By letter dated August 7, 1978, the other BWRs were directed to review their designs and submit a report. That letter also directed that a surveillance program be implemented. By letter dated September 24, 1980 the licensees were requested to: (1) commit to installing a Class 1E system, (2) provide a schedule for completion of the modification, and (3) provide a schedule for submitting design information and proposed Technical Specifications. All licensees responded satisfactorily.

Current Status:

Six licensing actions remain to be completed on this MPA. Four SERs were received during the second quarter of FY84. Three SEs are scheduled for issuance in the third quarter of FY84. The SER for Browns Ferry unit 1 is not complete and requires input from the licensee expected in FY84. The two remaining actions for Browns Ferry Unit 2 and 3 await TVA information scheduled to be provided in FY85 and beyond. Project Manager is aware of the problem with TVA.

MPA C-11 (continued)

- 2 -

Actions Remaining:

1. Review licensee's designs (post-implementation)
2. Review licensee's proposed TS changes
3. Issue license amendment that revises TS

Status Updated: June 12, 1983

Multi-Plant Action No. C-14

Title: Seismic Qualification of Auxiliary Feedwater Systems

Priority: 1

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: G. Holahan

Lead Project Manager: J. T. Beard

Lead Technical Review Branch: ASB

Problem:

Since TMI-2, great attention has been focused on the ability of PWRs to reliably provide for shutdown decay heat removal at normal system pressure. The steam generator is the preferred and proven choice for accomplishing a safe shutdown of the plant. Therefore, the auxiliary feedwater systems (AFW) should satisfy the seismic design criteria for the Safe Shutdown Earthquake (SSE), consistent with other safety-related systems in the plant. However, only those facilities licensed recently have been reviewed against the NRC Standard Review Plan which treats the AFW as an Engineered Safety Feature.

Background:

In August 1980, the NRC completed a preliminary study of the risk of a seismically induced loss of decay heat removal for plants without a seismically qualified AFW. One of the conclusions was that the increase in risk during the next three years is sufficiently small to allow plant operations to continued. Another conclusion was that plants should be surveyed to confirm the extent of thw BWR plants without seismically qualified AFW.

In February 1981, the NRC developed and embarked on Multi-Plant Action Plan C-14, "Seismic Qualification of Auxiliary Feedwater Systems". An objective to this plan is to increase, to the extent practicable, the capability of those plants without seismically qualified AFW to withstand earthquakes up to the SSE level. This Multi-Plant plan is described in an NRC memorandum to H. Denton dated February 20, 1981. Generic Letter 81-14 requested licensees: (1) to identify, via a walkdown of the AFW system, deficiencies ammenable to simple remedial actions, and (2) to review the plants to determine the extent and areas where significant improvements may be needed. The NRC will then take action to require licensees to make improvements such as re-analysis and/or modifications, for those plants which cannot demonstrate reasonable assurance of the functioning of the AFW system following an earthquake of SSE magnitude.

Current Status:

Under technical assistance contract FIN A-0429, Lawrence Livermore National Laboratory (LLNL) has reviewed the licensees' responses to Generic Letter 81-14 for the 47 PWRs involved. It was then necessary to generate Requests for Additional Information for almost every plant, in order to rectify deficiencies in the responses or to obtain technical clarification of the responses. LLNL documented the seismic capabilities of the AFW's in plant-specific Technical Evaluation Reports (TERs), which serve as the technical basis for NRR licensing actions.

During FY83, LLNL completed all the TERs. In some cases, additional communications with the licensees have been necessary to complete a Safety Evaluation Report (SER). In FY81, SERs for two plants were completed; in FY82, for 20 plants; in FY83 for 16 plants.

LLNL has produced a preliminary draft report for this project which provides a summary and overall evaluation of the seismic capabilities of AFW systems. The project report (NUREG/CR-3271) has been reviewed by the NRC and a final draft developed. LLNL has delivered a camera-ready copy of the overall project report. This action fulfilled the last of the contractual requirements.

Actions Remaining:

The NRC has to continue discussions with licensees for the last nine plants and complete these SERs. These actions are presently scheduled to be completed in the last quarter of FY84.

Status Updated: April 20, 1984



Multi-Plant Action No. C-15

Title: Control of Heavy Loads At Nuclear Power Plants

Priority: Phase II, Medium

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: S. Varga

Lead Project Manager: D. Neighbors

Lead Technical Review Branch: ASB

Problem:

Dropped Heavy Loads may impact spent fuel, fuel in the core, or equipment that may be required to achieve safe shutdown and continued decay heat removal for PWRs and BWRs. If sufficient spent fuel or fuel in the core were damaged and if the fuel is highly radioactive, the potential releases of radioactive material could result in off-site doses that exceed 10 CFR Part 100 limits.

Background:

USI - Task A-36 was established to examine staff licensing criteria, the adequacy of measures in effect at operating plants, and to recommend necessary changes to assure the safe handling of Heavy Loads. NUREG-0612 provides the results of the review and includes the task groups recommendation.

Current Status:

(See C-10 also)

Twelve sites (20 plants) have been selected as a pilot program to determine if significant problems exist for Phase II. As a result of this program, a determination will be made whether or not to review the remainder.

Actions Remaining:

1. Project Managers expedite licensees' responses necessary to close out open items as discussed in draft TER for 12 sites as they are issued.
2. Two TERs are being issued at this time.

Status Updated: June 15, 1984

Multi-Plant Action No. D-01

Title: Mark I Containment Program

Priority: 1

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: D. Vassallo

Lead Project Manager: B. Siegel

Lead Technical Review Branch: CSB - Load Audit Review  
MEB - Structural Audit Review

Problem:

Since the original design of the Mark I containment system additional loading conditions resulting from the dynamic effects of drywell air and steam being rapidly forced into the suppression pool during a postulated LOCA and from suppression pool response to SRV operation generally associated plant transient operating conditions have been identified, the objective of this program is to restore the originally intended design safety margins for each Mark I containment system.

Background:

A Mark I Owners Group (OG) with GE the lead technical organization was formed to quantify the hydrodynamic loads and assess the effects of these loads on the Mark I containment structure. The OG divided this task into a short term (STP) and long term program (LTP).

The objectives of the STP were to verify that each Mark I containment system would maintain its integrity and functional capability when subjected to the most probable loads induced by a postulated design-basis LOCA, and to verify that licensed Mark I BWR facilities would continue to operate safely, without endangering the health and safety of the public, while a methodical, comprehensive LTP was being conducted. The staff in the "Mark I Containment Short-Term Program Safety Evaluation Report", NUREG-0408 concluded that sufficient margin of safety had been demonstrated to assure the functional performance of the containment system and, therefore, any undue risk to the health and safety of the public was precluded.

The objectives of the LTP were to establish design-basis loads that are appropriate for the anticipated life of each Mark I BWR facility (40 years), and to restore the originally intended design-safety margins for each Mark I containment system. The principle thrust of this program has been the development of generic methods for the definition of

suppression pool hydrodynamic loading events and the associated structural assessment techniques for the Mark I configuration. The generic analysis techniques are intended to be used to perform a plant-unique analysis (PUA) for each Mark I facility. The staff in the "Mark I Containment Long-Term Program Safety Evaluation Report", NUREG-0661 concluded that the load definition procedures utilized by the BWR Owners Group, as modified by the staff's requirements provide conservative estimates of these loading conditions and that the structural acceptance criteria are consistent with the requirements of the applicable codes and standards.

#### Current Status:

The licensees have designed and are in the process of installing modifications to meet the Commission's Order date for each operating plant. More than half of the 22 plants affected have completed these modifications. The licensees have prepared and submitted plant-unique analysis reports which provide the bases for these modifications for staff review. Technical assistance contracts have been awarded to Franklin Research Center (FRC) and Brookhaven National Laboratory (BNL) to perform an audit review of the licensee's PUA. CSB has lead technical review responsibility for the BNL load audit review and MEB for the FRC structure audit review. Technical Evaluation Reports from the contractors have been received for nine of the 22 plants. Some licensees have submitted Technical Specifications which are being reviewed by the staff.

#### Actions Remaining:

1. Post implementation audit of PUA by Franklin Institute and BNL on Technical Assistance Contracts.
2. Issuance of SER to each licensee on acceptability of Mark I LTP modifications and analyses for their plant.
3. Technical Specification changes resulting from Mark I modifications.

Status Updated: June 14, 1984



Multi-Plant Action No. D-05

Title: Plant UPI Model Problem

Priority: 3

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: J. Miller

Lead Project Manager: D. DiIanni

Lead Technical Review Branch: RSB

Problem:

Only two-loop PWRs are affected. All Westinghouse (W) two-loop plants have a Low Pressure Safety Injection System (LPSI) which injects water directly into the Upper Plenum of the Reactor Vessel (UPI). This is used to counter the consequences of a single failure for two-loop units.

However, the currently approved ECCS evaluation models for two-loop plants are based on an ECCS system in which HPSI is directly to the cold leg of the Reactor Coolant System and because of this may be considered as not meeting the requirements of 10 CFR Appendix K Section II.5.

Background:

This has been an outstanding issue for two-loop plants since the implementation of Regulations for an approved ECCS system. The issue became a MPA during late 1977 and 1978 when an "interim" model was developed to review the potential effect of UPI against the conventional model.

Current Status:

Six facilities are affected. W two-loop UPI plants are presently operating on the bases of a number of approaches being taken to determine allowable peaking coefficients in an interim manner.

A letter has been sent to each of the four utilities informing them that a Safety Evaluation will be sent out which documents our efforts and concerns. The letter recommends that the utilities provide a plan of action or respond to our concerns in 90 days. The peaking factor penalty will be reevaluated based on the 90 day response.

MPA D-05 (continued)

- 2 -

Actions Remaining:

A Safety Evaluation will be prepared by RSB by June 30, 1984 for transmittal to the licensees.

Status Updated: June 18, 1984

Multi-Plant Action No. D-10

Title: Asymmetric LOCA Loads On Primary Reactor Coolant System

Priority: Medium

Cognizant Assistant Director: F. Miraglia

Cognizant Branch Chief: D. Crutchfield

Lead Project Manager: J. Shea

Lead Technical Review Branch: MEB, MTEB, CPB, CSB

Problem:

Asymmetric loads resulting from rupture of the primary piping could potentially damage reactor vessel support internals and other primary system components. Damage to the reactor internals could impair core coolability.

Background:

In January 1978, all pressurized water reactor (PWR) licensees were required by the NRC to provide an assessment of the adequacy of their reactor vessel supports and other affected structures to withstand combinations of responses to asymmetric loss-of-coolant accident (LOCA) loads and the Safe Shutdown Earthquake (SSE). This potential safety problem was subsequently identified as an unresolved safety issue Task Action Plan A-2.

Current Status:

All PWR plant assessments for asymmetric loads have been received and are currently being evaluated by staff and contractor EG&G. As a basis for this evaluation, staff is implementing criteria developed as a part of the Task Action Plan A-2. These criteria are included in NUREG-0609. Concurrently some PWR licensees have performed a study of "leak before break". The "leak before break" concept was reviewed by CRGR on September 28, 1983 and received written CRGR approval several days later. By Generic Letter 84-04 dated 2/1/84, PWR licensees were informed of the staff evaluation of the WOG "Leak Before Break" concept. The staff concluded that an acceptable technical basis has been provided so that asymmetric blowdown loads resulting from double ended pipe breaks in main coolant loop piping need not be considered as a design basis as long as acceptable leakage detection systems were operable.

MPA D-10 (continued)

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Actions Remaining:

Complete the 4 CE SERs (4th qtr. FY84). Resume evaluation of 8 remaining non owner group submittals.

Status Updated: June 13, 1984



Multi-Plant Action No. D-17

Title: Definition of Operable

Priority: Medium

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. F. Stolz

Lead Project Manager: Sydney Miner

Lead Technical Review Branch: None

Problem:

Clarification of the term "OPERABLE" as it applies to support system outages or multiple outages or redundant components.

Background:

In a letter dated April 10, 1980, an explanation of the problem plus accompanying illustrate examples of recommended changes contained in the associated bases, were delineated to all licensees. The licensees were required to submit proposed changes to tech specs applicable to their plant and to implement procedures designed to assure compliance with the specifications containing new requirements.

Current Status:

1. Acceptance Criteria

The acceptance criteria that LCO should be incorporated in the Technical Specification which requires that the plants be placed in cold shutdown if redundant ECCS loops become inoperable either due to failures in the ECCS loops or the diesel generators.

2. Important Schedule Considerations

For the remaining 11 licensing actions 10 are scheduled to be completed this fiscal year and one the first quarter of next fiscal year.

3. Review Responsibilities

This MPA is being handled completely within the Project Management Organization. Ten reviews were assigned to a contractor. All of the reviews have been completed and final TERs issued.

Actions Required:

Remaining Milestones

- (a) Approval of remaining changes to the Technical Specifications of the remaining plants.
- (b) Issuance of the SER and the License Amendments.

Status Updated: June 14, 1984

Multi-Plant Action No. E-04

Title: BWR Single Loop Operation

Priority: Low

Cognizant Assistant Director: G. Lainas

Cognizant Branch Chief: D. Vassallo

Lead Project Manager: R. Clark

Lead Technical Review Branch: RSB, CPB

Problem:

The standard BWR Technical Specifications do not permit a plant with two recirculation loops to operate either with one loop or in the natural circulation mode. Fifteen operating BWRs have requested approval to operate at reduced power levels with a single loop if problems develop with a jet pump, valve, seals, flow controller, etc., in the other loop. Most of the potential safety reviews can more appropriately be resolved on a generic basis.

Background:

Single loop operation at limited power levels had previously been authorized for 6 BWRs for short periods of time (one week to two months). During the fall of 1978 while Browns Ferry Unit 1 was operating on a single loop, an increase in instrument noise was noted above 60% power, which could be indicative of flow and flux variations. Based on this operating experience, the staff has concluded that BWRs can safely operate at at least 50% power by natural circulation (i.e., without forced flow from any recirculation loop). Accordingly, as an interim measure, six plants have been authorized to operate, if necessary, at power levels of 50% with a single recirculation loop while the generic concern of flow stability at higher power levels during single loop operation are being resolved.

Current Status:

With respect to operation at power levels above 50% power, a meeting was held October 22, 1981, with all BWR licensees and GE to discuss GE's evaluation of Browns Ferry's and other BWR operating experience with single loop operation. RSB and CPB have prepared a draft list of questions which we discussed in a meeting with GE on January 21, 1982. The issues regarding operation at power levels above 50% are not likely to be resolved in FY83. In the interim, the effort is being directed approving operation at 50% power.

MPA E-04 (continued)

- 2 -

Actions Remaining:

Complete SE inputs by CPB for operation at 50% power and issue revised Technical Specifications.

Status Updated: December 8, 1982

Multi-Plant Action No. E-05

Title: Westinghouse N-1 Loop Operation

Priority: Low

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: S. Varga

Lead Project Manager: D. Wigginton

Lead Technical Review Branch: RSB

Problem:

All Westinghouse plants have a provision in their license that prohibits operation with less than all loops (N) in service.

Background:

It may be desirable to operate plants with one loop out of service. Present restrictions in the license do not permit this type of operation.

Current Status:

Only three Westinghouse plants, Beaver Valley and Zion (1 and 2), have submitted proposed amendments for N-1 loop operation. Beaver Valley is lead plant and most of the review has been completed. Trojan, a plant without loop isolation valves, has expressed an interest in submitting an application.

Actions Remaining:

NRC to complete review for core thermodynamic, procedures, initial tests, and write SER on Beaver Valley. Zion review to follow. Other Westinghouse plants will probably submit similar applications if Beaver Valley is approved.

Status Updated: June 13, 1984



Multi-Plant Action No. G-1

Title: Reactor Coolant Pump Trip

Priority: 1

Cognizant Assistant Director: G. C. Lainas

Cognizant Branch Chief: J. R. Miller

Lead Project Manager: D. H. Jaffe

Lead Technical Review Branch: Reactor Systems Branch

Problem:

The implementation of MPA G-1 involves the selection of a post LOCA RCP operation strategy (auto-trip, manual trip, or no trip) based upon calculations using approved analytic models. This strategy is to be implemented via operator training and procedures.

Background:

The NRC, its licensees, and the PWR vendors have been evaluating the reactor coolant pump (RCP) trip issue since the accident at TMI. The technical understanding of the industry and the requirements of NRC on this issue have changed twice in that period. As a result, there have been extensive studies to better understand the dynamic response of all classes of PWRs to small break LOCAs. Although some confirmatory information is still to be received concerning some models, we conclude that the analytical models are sufficiently reliable to be used by licensees to choose their own best method for RCP operation upon indication that a LOCA has occurred.

Current Status:

Generic Letters 83-10a through 83-10f were issued on February 8, 1983 to licensees and applicants (PWR vendor-specific) providing the NRC's position on reactor coolant pump trip. A 60-day response was requested (50.54f). All responses to GL 83-10 have been received. The members of the CE, Westinghouse and B&W Owners Groups have submitted generic analyses to satisfy the requirements of GL 83-10. Maine Yankee has submitted plant-specific analyses to support their response to GL 83-10. We are presently awaiting endorsements of these reports by the Owners Groups together with implementation plans. We are also awaiting plant specific submittals from Yankee Rowe, Haddam Neck, and San Onofre Unit 1 facilities. We have issued requests for additional information to the Owners Group.

Actions Remaining:

1. Review generic and plant specific RCP trip calculations.
2. Review licensee's endorsements of Owners Groups' generic reports, including commitments on training and implementation.
3. Receive and review responses to requests for additional information.
4. Issue generic or plant specific SERs with letter indicating approval of training and implementation schedule.

Status Updated: June 14, 1984

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14 ABSTRACT (200 words or less)

The Operating Reactors Licensing Actions Summary is designed to provide the Management of the Nuclear Regulatory Commission (NRC) with an overview of licensing actions dealing the operating power and nonpower reactors.

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