ATTACHMENT 1

LIMERICK GENERATING STATION

UNITS 1 AND 2

Docket Nos. 50-352 50-353

License Nos. NPF-39 NPF-85

TECHNICAL SPECIFICATIONS CHANGE REQUEST NO. 95-15-0

ADDITIONAL AFFECTED PAGES

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PURGE - PURGING

1.31 PURGE or PURGING shall be the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

RATED THERMAL POWER

1.32 RATED THERMAL POWER shall be a total reactor core heat transfer rate to the reactor coolant of 3293 MWt.

REACTOR ENCLOSURE SECONDARY CONTAINMENT INTEGRITY REACTOR ENCLOSURE SECONDARY CONTAINMENT INTEGRITY shall exist when: 1.33

- All reactor enclosure secondary containment penetrations required to a. be closed during accident conditions are either:
 - Capable of being closed by an OPERABLE secondary containment 1. automatic isolation system, or
 - Closed by at least one manual valve, blind flange, slide gate 2. damper, or deactivated automatic valve secured in its closed position, except as provided in Table 3.6.5.2.1 of (b) Add Specification 3.6.5.2.1. Delete
- All reactor enclosure secondary containment hatches and blowout panels b. are closed and sealed.
- The standby gas treatment system is in compliance with the requirements C . of Specification 3.6.5.3.
- The reactor enclosure recirculation system is in compliance with the d. requirements of Specification 3.6.5.4.
- At least one door in each access to the reactor enclosure secondary е. containment is closed.
- The sealing mechanism associated with each reactor enclosure secondary f. containment penetration, e.g., welds, bellows, or O-rings, is OPERABLE.
- The pressure within the reactor enclosure secondary containment is Q . less than or equal to the value required by Specification 4.6.5.1.1a.

REACTOR PROTECTION SYSTEM RESPONSE TIME

1.34 REACTOR PROTECTION SYSTEM RESPONSE TIME shall be the time interval from when the monitored parameter exceeds its trip setpoint at the channel sensor until de-energization of the scram pilot valve solenoids. The response time may be measured by any series of sequential, overlapping or total steps such that the entire response time is measured. .

REFUELING FLOOR SECONDARY CONTAINMENT INTEGRITY

- 1.35 REFUELING FLOOR SECONDARY CUNTAINMENT INTEGRITY shall exist when:
 - All refueling floor secondary containment penetrations required to а. be closed during accident conditions are either:

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REFUELING FLOOR SECONDARY CONTAINMENT INTEGRITY (Continued)

- 1. Capable of being closed by an OPERABLE secondary containment automatic isolation system, or
- b. All refueling floor secondary containment hatches and blowout panels are closed and sealed.
- c. The standby gas treatment system is in compliance with the requirements of specification 3.6.5.3.
- d. At least one door in each access to the refueling floor secondary containment is closed.
- e. The sealing mechanism associated with each refueling floor secondary containment penetration, e.g., welds, bellows, or O-rings, is OPERABLE.
- f. The pressure within the refueling floor secondary containment is less than or equal to the value required by Specification 4.6.5.1.2a.

REPORTABLE EVENT

1.36 A REPORTABLE EVENT shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

ROD DENSITY

1.37 ROD DENSITY shall be the number of control rod notches inserted as a fraction of the total number of control rod notches. All rods fully inserted is equivalent to 100% ROD DENSITY.

SHUTDOWN MARGIN

1.38 SHUTDOWN MARGIN shall be the amount of reactivity by which the reactor is subcritical or would be subcritical assuming all control rods are fully inserted except for the single control rod of highest reactivity worth which is assumed to be fully withdrawn and the reactor is in the shutdown condition; cold, i.e. 68°F; and xenon free.

SITE BOUNDARY

- 1.39 The SITE BOUNDARY shall be that line as defined in Figure 5.1.3-1a.
- 1.40 (Deleted)

SOURCE CHECK

1.41 A SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.

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PURGE - PURGING .

1.31 PURGE or PURGING shall be the controlled process of discharging air or gas from a confinement to maintain temperature, pressure, humidity, concentration or other operating condition, in such a manner that replacement air or gas is required to purify the confinement.

RATED THERMAL POWER

1.32 RATED THERMAL POWER shall be a total reactor core heat transfer rate to the reactor coolant of 3458 MWt.

REACTOR ENCLOSURE SECONDARY CONTAINMENT INTEGRITY

- 1.33 REACTOR ENCLOSURE SECONDARY CONTAINMENT INTEGRITY shall exist when:
 - a. All reactor enclosure secondary containment penetrations required to be closed during accident conditions are either:
 - Capable of being closed by an OPERABLE secondary containment automatic isolation system, or
 - Closed by at least one manual valve, blind flange, slide gate damper or deactivated automatic valve secured in its closed position, except as provided in Table 3.6.5.2.1.
 Specification 3.6.5.2.1.
 - All reactor enclosure secondary containment hatches and blowout panels are closed and sealed.
 - c. The standby gas treatment system is in compliance with the requirements of Specification 3.6.5.3.
 - d. The reactor enclosure recirculation system is in compliance with the requirements of Specification 3.6.5.4.
 - e. At least one door in each access to the reactor enclosure secondary containment is closed.
 - f. The sealing mechanism associated with each reactor enclosure secondary containment penetration, e.g., welds, bellows, or O-rings, is OPERABLE.
 - g. The pressure within the reactor enclosure secondary containment is less than or equal to the value required by Specification 4.6.5.1.1a.

REACTOR PROTECTION SYSTEM RESPONSE TIME

1.34 REACTOR PROTECTION SYSTEM RESPONSE TIME shall be the time interval from when the monitored parameter exceeds its trip setpoint at the channel sensor until de-energization of the scram pilot valve solenoids. The response time may be measured by any series of sequential, overlapping or total steps such that the entire response time is measured.

REFUELING FLOOR SECONDARY CONTAINMENT INTEGRITY

1.35 REFUELING FLOOR SECONDARY CONTAINMENT INTEGRITY shall exist when:

a. All refueling floor secondary containment penetrations required to be closed during accident conditions are either:

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REFUELING FLOOR SECONDARY CONTAINMENT INTEGRITY (Continued)

- Capable of being closed by an OPERABLE secondary containment
 automatic isolation system, or
 - 2. Closed by at least one manual valve, blind flange, slide gate damper or deactivated automatic valve secured in its closed position, except as provided in Table 3.6.5.2.2. Specification 3.6.5.2.2.
- All refueling floor secondary containment hatches and blowout panels are closed and sealed.
- c. The standby gas treatment system is in compliance with the requirements of Specification 3.6.5.3.
- d. At least one door in each access to the refueling floor secondary containment is closed.
- e. The sealing mechanism associated with each refueling floor secondary containment penetration, e.g., welds, bellows, or O-rings, is OPERABLE.
- f. The pressure within the refueling floor secondary containment is less than or equal to the value required by Specification 4.6.5.1.2a.

REPORTABLE EVENT

1.36 A REPORTABLE EVENT shall be any of those conditions specified in Section 50.73 to 10 CFR Part 50.

ROD DENSITY

1.37 ROD DENSITY shall be the number of control rod notches inserted as a fraction of the total number of control rod notches. All rods fully inserted is equivalent to 100% ROD DENSITY.

SHUTDOWN MARGIN

1.38 SHUTDOWN MARGIN shall be the amount of reactivity by which the reactor is subcritical or would be subcritical assuming all control rods are fully inserted except for the single control rod of highest reactivity worth which is assumed to be fully withdrawn and the reactor is in the shutdown condition; cold, i.e. 68°F; and xenon free.

SITE BOUNDARY

- 1.39 The SITE BOUNDARY shall be that line as defined in Figure 5.1.3-1a.
- 1.40 (Deleted)

SOURCE CHECK

1.41 A SOURCE CHECK shall be the qualitative assessment of channel response when the channel sensor is exposed to a radioactive source.

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