GPU NUCLEAR CORPORATION OYSTER CREEK NUCLEAR GENERATING STATION

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Facility Operating License No. DPR-16

Technical Specification Change Request No. 229 Docket No. 50-219

Applicant submits, by this Technical Specification Change Request No. 229, to the Oyster Creek Nuclear Generating Station Operating License, a change to page 4.

BY

J. J. Barton Vice President and Director Oyster Creek

Sworn and Subscribed to before me this 26th day of Oct., 1995.

GERALDINE E. LEVIN NOTARY PUBLIC OF NEW JERSEY My Commission Expires 56-58-3000

Geraldine Levie

A Notary Public of NJ

1. LICENSE CHANGE REQUEST (TSCR) No. 229

GPU Nuclear requests that the following changed replacement page be inserted into existing Facility Operating License (FOL) No. 216:

Replace existing page 4 with the attached revised page 4.

II. REASON FOR CHANGE

By Amendment No. 70 to License No. DPR-216, GPUN was required to perform an inspection of the core spray sparger and repair assemblies at each refueling outage, to submit results and provide an evaluation of the safety significance of any new or progressing indications, and to obtain NRC authorization before the plant is restarted from the refueling outage. GPUN believes that sufficient data has accumulated to warrant a change of the administrative requirements in license condition 2.C.(5).

III. SAFETY EVALUATION JUSTIFYING CHANGE

During the 1978 refueling outage at the OCNGS, a scheduled inservice inspection of the reactor internals identified and confirmed the existence of a crack at azimuth 208° in the upper sparger. Although structural and hydraulic analyses indicated that the cracked sparger was adequate for continued operation, a mechanical support was installed. Additional clamp assemblies were installed on both the upper and lower spargers during the 1980 refueling outage. In the Safety Evaluation Report (SER) from the office of Nuclear Reactor Regulation (NRR) supporting Amendment #47, dated May 15, 1980, the staff found the licensee's design and installation of the repair bracket assemblies were in accordance with currently accepted engineering practices.

A 1983 reinspection of the spargers and annulus piping utilizing enhanced video techniques and UT inspection of accessible areas disclosed no indications except the through-wall crack at 208° identified and clamped in 1978. This inspection became the baseline against which subsequent inspections have been compared. GPUN believes that initial NRC concern about the propagation of new cracks has been alleviated by the installation of repair brackets and the positive results (no sparger indications) from the six successive inspections in 1983, 1986, 1988, 1991, 1992 and 1994. During the 1992 inspection a small leak was detected in the annulus piping which GPUN judged to be a weld defect and not a crack. The NRC Safety Evaluation Report (SER) issued February 5, 1993 concluded that Oyster Creek could be safely operated without repairing the leak. Continued inspection of the sparger and the annulus piping was required.

By amending license condition 2.C.(5) such that the core spray sparger and annulus piping are now part of the scheduled inservice inspection of reactor internals, the scope and frequency of the inspections are not modified. The visual inspections of

the spargers and annulus piping for the remainder of the 120 month inservice inspection interval will be done in accordance with paragraphs IWA 2211 and IWA 2213 of the ASME B&PV Code, Section XI, 1988 Edition. Thus, the inspection methodology and acceptance criteria are unchanged. Inspection results and an evaluation of the safety significance any new or progressing indications will continue to be provided to the NRC within 60 days of the end of the outage. What would be eliminated is the administrative process associated with obtaining separate NRC approvals for inspection methods, for reviewing inspection results and for obtaining restart authorization at the end of each refueling outage. In addition, a wording change has also been made to the phrase "once per 24 months". This phrase now reads "once per refueling outage". The phrase had been modified in the process of extending to 24 month cycles. With the issuance of Amendment 144, which defines "refueling outage" for the purpose of designating frequency of testing and surveillance, the original phrase can be reinstated.

IV. NO SIGNIFICANT HAZARDS CONSIDERATION

1. State the basis for the determination that the proposed activity will or will not decrease the margin of safety.

Inspection of the core spray sparger will continue on the same frequency and with the same scope to ensure the integrity of the components. Therefore, the margin of safety will not be reduced.

2. State the basis for the determination that the activity will or will not increase the probability of occurrence or the consequences of an accident.

This change is administrative in nature and does not modify the function of any components. The condition of the sparger has not changed since 1978 and the inspections will continue to be performed in the same manner. The amending of this license condition will not, therefore, increase the probability of occurrence or the consequences of an accident.

3. State the basis for the determination that the proposed activity does or does not create a possibility of an accident or malfunction of a different type than any previously identified in the SAR.

This change is administrative in nature and does not affect the function or purpose of any component. Amending the licensing condition to eliminate the docketing of the inspection methods and results would not create a possibility of an accident or malfunction of a different type than any previously identified in the SAR.

V. IMPLEMENTATION

GPUN requests that the amendment authorizing this license change be effective upon issuance.

The licensee may make changes to the approved fire protection program without prior approval of the Commission only if those changes would not adversely affect the ability to achieve and maintain safe shutdown in the event of a fire.

- (4) The licensee shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans, including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The plans, which contain Safeguards Information protected under 10 CFR 73.21, are entitled: "Oyster Creek Nuclear Generating Station Physical Security Plan," with revisions submitted through July 6, 1988; "Oyster Creek Nuclear Generating Station Training and Qualification Plan," with revisions submitted through June 24, 1986; and "Oyster Creek Nuclear Generating Station Safeguards Contingency Plan," with revisions submitted through June 24, 1986. Changes made in accordance with 10 CFR 73.55 shall be implemented in accordance with the schedule set forth therein.
- (5) Inspections by a method acceptable to the NRC, of all accessible surfaces and welds of both core spray spargers and repair assemblies at least once per refueling outage, will be performed so that meaningful comparisons of any indications with previous inspections can be made. Results of the inspections, along with an evaluation of the safety significance of any new or progressing indications, will be provided to the Commission's staff within 60 days of the end of the refueling outage.

(6) Long Range Planning Program

The revised "Plan for the Long-Range Planning Program for the Oyster Creek Nuclear Generating Station" (the Plan) submitted by GPUN letter C321-94-2140 dated September 26, 1994, is approved.

 The Plan shall be followed by the licensee from and after November 28, 1994.

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Amendment No. 173,

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