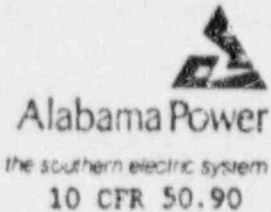


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W. G. Halston, III
Senior Vice President
Nuclear Operations

February 26, 1991



Docket Nos: 50-348
50-36A

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

Gentlemen:

Joseph M. Farley Nuclear Plant - Units 1 and 2
Steam Generator (SG) Tube Support Plate (TSP)
Alternate Plugging Criteria (APC)

In order to preclude unnecessarily removing significant numbers of SG tubes from service, Alabama Power Company has evaluated the possibility of utilizing a revised plugging criteria for SG tube cracks which are contained within the boundaries of the TSP. This evaluation has resulted in the development of the TSP APC.

The need for an APC became evident during a recent Unit 2 outage. The program of eddy current inspections performed at Farley Unit 2 during the seventh refueling outage included full length bobbin coil probing of the available tubes. With no application of a voltage amplitude criterion, numerous tubes found to exhibit degradation suggestive of outer diameter stress corrosion cracking (ODSCC) were plugged. This resulted in plugging 243 tubes, 30 in SG A, 64 in SG B, and 149 in SG C due to TSP degradation.

The scope of application of the TSP APC will be limited to those cracks contained within the TSP boundaries which are the result of ODSCC. All other portions of the tube will be covered by the current Technical Specifications. The APC utilizes the eddy current bobbin voltage and phase angle indicated depth as an indication of crack severity within the TSP. Tubes with an indicated depth greater than or equal to 50% will be plugged if eddy current bobbin voltage exceeds 3 volts. Tubes with an indicated depth less than 50% will be plugged if eddy current bobbin voltage exceeds 8 volts. The APC was developed to ensure margins against tube burst per Regulatory Guide 1.121 and to minimize leakage at normal operating conditions. The criteria was also developed to ensure that radiological results of a steam line break accident do not exceed a small fraction of the 10CFR100 limits. Details of the development of the APC are contained in WCAP-12871, "J. M. Farley Units 1 and 2 SG Tube Plugging Criteria for ODSCC at Tube Support Plates."

150000

9102280 117 XA

3/17/92

Change: NCC PDC
NSIC

Ltr. Encl.
1 INP
1 NP

ADD!

The proposed changed pages to Unit 1 and 2 Technical Specifications are provided in Attachment 1. Alabama Power Company has determined the proposed changes do not involve a significant hazards consideration. In accordance with 10CFR50.92, a significant hazards consideration evaluation is provided in Attachment 2. The technical justification supporting the proposed changes is provided in Attachment 3 as:

1. WCAP-12871-J. M. Farley Units 1 and 2 SG Tube Plugging Criteria for ODSCC at Tube Support Plates (Proprietary).
2. WCAP-12872-J. M. Farley Units 1 and 2 SG Tube Plugging Criteria for ODSCC at Tube Support Plates (Non-Proprietary).

Also enclosed are a Westinghouse authorization letter, CAW-91-130, accompanying affidavit, Proprietary Information Notice, and Copyright Notice.

As WCAP-12871 contains information proprietary to Westinghouse Electric Corporation, it is supported by an affidavit signed by Westinghouse, the owner of the information. The affidavit sets forth the basis on which the information may be withheld from public disclosure by the Commission and addresses with specificity the considerations listed in paragraph (B)(4) of Section 2.790 of the Commission's regulations.

Accordingly, it is respectfully requested that the information which is proprietary to Westinghouse be withheld from public disclosure in accordance with 10 CFR Section 2.790 of the Commission's regulations.

Correspondence with respect to the copyright or proprietary aspects of the items listed above or the supporting Westinghouse affidavit should reference CAW-91-130 and should be addressed to Mr. R. P. Mazza, Manager, Operating Plant Licensing Support, Westinghouse Electric Corporation, P. O. Box 355, Pittsburgh, Pennsylvania 15230-0355.

Alabama Power Company requests the NRC review and approve this proposed change by April 25, 1991.

Alabama Power Company's Plant Operations Review Committee has reviewed the proposed changes and the Nuclear Operation Review Board will review the changes at a future meeting. A copy of this proposed change is being sent to Dr. C. E. Fox, the Alabama State Designee, in accordance with 10CFR50.91(B)(1)

If there are any questions, please advise.

Respectfully submitted,
ALABAMA POWER COMPANY

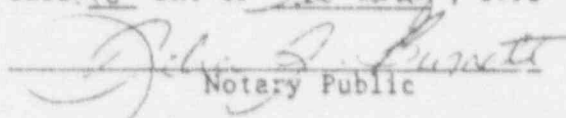

W. G. Hairston, III

WGH,III/REM:maf2239
Attachments

cc: Mr. S. D. Ebner
Mr. S. T. Hoffman
Mr. G. F. Maxwell
Dr. C. E. Fox

SWORN TO AND SUBSCRIBED BEFORE ME

THIS 26th DAY OF February, 1991


Notary Public

My Commission Expires: 9-14-94

Attachment 1

Revised Technical Specification Pages

Unit 1

<u>Page</u>		
3/4	4-9	Replace
3/4	4-10	Replace
3/4	4-11	Replace
3/4	4-12	Replace
3/4	4-13	Replace
3/4	4-13a	Insert
3/4	4-13b	Insert
3/4	4-17	Replace
B3/4	4-3	Replace
B3/4	4-3a	Replace
B3/4	4-4	Replace
B3/4	4-5	Replace
B3/4	4-5a	Insert

Unit 2

3/4	4-9	Replace
3/4	4-10	Replace
3/4	4-11	Replace
3/4	4-12	Replace
3/4	4-13	Replace
3/4	4-13a	Replace
3/4	4-13b	Insert
3/4	4-13c	Insert
3/4	4-17	Replace
B3/4	4-3	Replace
B3/4	4-3a	Replace
B3/4	4-4	Replace
B3/4	4-5	Replace
B3/4	4-5a	Insert

REACTOR COOLANT SYSTEM

3/4.4.6 STEAM GENERATORS

LIMITING CONDITION FOR OPERATION

3.4.6 Each steam generator shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more steam generators inoperable, restore the inoperable generator(s) to OPERABLE status prior to increasing T_{sv} above 200°F.

SURVEILLANCE REQUIREMENTS

4.4.6.0 Each steam generator shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

4.4.6.1 Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.4-1.

4.4.6.2 Steam Generator Tube # Sample Selection and Inspection - The steam generator tube minimum sample size, inspection result classification, and the corresponding action required shall be as specified in Table 4.4-2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.4.6.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.4.6.4. The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators. Implementation of the steam generator tube/tube support plate elevation alternate plugging criteria requires a 100% bobbin probe inspection. An inspection using a rotating pancake coil (RPC) probe is required for all outer diameter stress corrosion cracking (OD SCC) indications exceeding a 1.5 volt bobbin coil signal amplitude. The RPC results are to be evaluated to establish that the principal indications can be characterized as OD SCC. When applying the exceptions of 4.4.6.2.a through 4.4.6.2.c, previous defects or imperfections in the area repaired by sleeving are not considered an area requiring reinspection. The tubes selected for these inspections shall be selected on a random basis except:

- a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas.

#When referring to a steam generator tube, the sleeve shall be considered a part of the tube if the tube has been repaired per Specification 4.4.6.4.a.9.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- b. The first sample of tubes selected for each inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:
1. All nonplugged tubes that previously had detectable wall penetrations greater than 20%.
 2. Tubes in those areas where experience has indicated potential problems.
 3. At least 3% of the total number of sleeved tubes in all three steam generators or all of the sleeved tubes in the generator chosen for the inspection program, whichever is less. These inspections will include both the tube and the sleeve.
 4. A tube inspection (pursuant to Specification 4.4.6.4.a.8) shall be performed on each selected tube. If any selected tube does not permit the passage of the eddy current probe for a tube or sleeve inspection, this shall be recorded and an adjacent tube shall be selected and subjected to a tube inspection.
- c. The tubes selected as the second and third samples (if required by Table 4.4-2) during each inservice inspection may be subjected to a partial tube inspection provided:
1. The tubes selected for these samples include the tubes from those areas of the tube sheet array where tubes with imperfections were previously found.
 2. The inspections include those portions of the tubes where imperfections were previously found.

The results of each sample inspection shall be classified into one of the following three categories:

<u>Category</u>	<u>Inspection Results</u>
C-1	Less than 5% of the total tubes inspected are degraded tubes and none of the inspected tubes are defective.
C-2	One or more tubes, but not more than 1% of the total tubes inspected are defective, or between 5% and 10% of the total tubes inspected are degraded tubes.
C-3	More than 10% of the total tubes inspected are degraded tubes or more than 1% of the inspected tubes are defective.

Note: In all inspections, previously degraded tubes or sleeves must exhibit significant (greater than 10%) further wall penetrations to be included in the above percentage calculations.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

4.4.6.3 Inspection Frequencies - The above required inservice inspections of steam generator tubes shall be performed at the following frequencies:

- a. The first inservice inspection shall be performed after 6 Effective Full Power Months but within 24 calendar months of initial criticality. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. If two consecutive inspections following service under AVT conditions, not including the preservice inspection, result in all inspection results falling into the C-1 category or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of once per 40 months.
- b. If the results of the inservice inspection of a steam generator conducted in accordance with Table 4.4-2 at 40 month intervals fall in Category C-3, the inspection frequency shall be increased to at least once per 20 months. The increase in inspection frequency shall apply until the subsequent inspections satisfy the criteria of Specification 4.4.6.3.a; the interval may then be extended to a maximum of once per 40 months.
- c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-2 during the shutdown subsequent to any of the following conditions:
 1. Primary-to-secondary tubes leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.4.7.2.
 2. A seismic occurrence greater than the Operating Basis Earthquake.
 3. A loss-of-coolant accident requiring actuation of the engineered safeguards.
 4. A main steam line or feedwater line break.
- d. Tubes in which the tube support plate elevation alternate plugging criteria have been applied shall be inspected during all future refueling outages.

SURVEILLANCE REQUIREMENTS (Continued)

4.4.6.4. Acceptance Criteria

a AS used in this Specification:

1. Imperfection means an exception to the dimensions, finish or contour of a tube or sleeve from that required by fabrication drawings or specifications. Eddy-current testing indications below 20% of the nominal wall thickness, if detectable, may be considered as imperfections.
2. Degradation means a service-induced cracking, wastage, wear or general corrosion occurring on either inside or outside of a tube or sleeve.
3. Degraded Tube means a tube, including the sleeve if the tube has been repaired, that contains imperfections greater than or equal to 20% of the nominal wall thickness caused by degradation.
4. % Degradation means the percentage of the tube or sleeve wall thickness affected or removed by degradation.
5. Defect means an imperfection of such severity that it exceeds the plugging or repair limit. A tube or sleeve containing a defect is defective.
6. Plugging or Repair Limit means the imperfection depth at or beyond which the tube shall be repaired (i.e., sleeved) or removed from service by plugging and is greater than or equal to 40% of the nominal tube wall thickness. For a tube that has been sleeved with a mechanical joint sleeve, through wall penetration of greater than or equal to 31% of sleeve nominal wall thickness in the sleeve requires the tube to be removed from service by plugging. For a tube that has been sleeved with a welded joint sleeve, through wall penetration greater than or equal to 37% of sleeve nominal wall thickness in the sleeve between the weld joints requires the tube to be removed from service by plugging. Also, this definition does not apply for tubes experiencing outer diameter stress corrosion cracking confirmed by bobbin probe inspection to be within the thickness of the tube support plates. See 4.4.6.4.a.11 for the plugging limit for use within the thickness of the tube support plate.
7. Unserviceable describes the condition of a tube or sleeve if it leaks or contains a defect large enough to affect its structural integrity in the event of an Operating Basis Earthquake, a loss-of-coolant accident, or a steam line or feedwater line break as specified in 4.4.6.3.c, above.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

8. Tube Inspection means an inspection of the steam generator tube from the point of entry (hot leg side) completely around the U-bend to the top support of the cold leg. For a tube that has been repaired by sleeving, the tube inspection should include the sleeved portion of the tube. For a tube in which the tube support plate alternate plugging criteria have been applied, the inspection will include all the hot leg intersections and all cold leg intersections down to, at least, the level of the last crack indication.
9. Tube Repair refers to mechanical sleeving, as described by Westinghouse report WCAP-11178 Rev. 1, or laser welded sleeving, as described by Westinghouse report WCAP-12672, which is used to maintain a tube in service or return a tube to service. This includes the removal of plugs that were installed as a corrective or preventive measure.
10. Preservice Inspection means an inspection of the full length of each tube in each steam generator performed by eddy current techniques prior to service to establish a baseline condition of the tubing. This inspection shall be performed after the field hydrostatic test and prior to initial POWER OPERATION using the equipment and techniques expected to be used during subsequent inservice inspections.
11. The following bobbin coil signal amplitude and indicated depth criteria (Tube Support Plate Alternate Plugging Criteria) are used for dispositioning a steam generator tube for continued service that is experiencing outer diameter initiated stress corrosion cracking confined within the thickness of the tube support plates:
 1. A tube can remain in service if the signal amplitude of a crack indication is less than or equal to 3.0 volts, regardless of the depth of tube wall penetration, if, as a result, the projected end of cycle distribution of crack indications is verified to result in primary to secondary leakage less than 55.0 gpm in the faulted loop (less than 150 gpd in the intact loops) during a postulated steam line break event. This would ensure acceptable radiological conditions following a SLB.##
 - # These criteria do not apply to tubes which may deform or collapse on the Farley Unit 1 steam generators during a postulated LOCA + SSE Event. These tubes are identified in WCAP-12871, "J. M. Farley Units 1 and 2 SG Tube Plugging Criteria for ODSCC at Tube Support Plates."
 - ## The methodology for calculating expected leak rates from the projected crack distribution must be consistent with WCAP-12871.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

2. A tube can remain in service if the signal amplitude of a crack indication is greater than 3.0 volts but less than or equal to 8.0 volts and the depth of tube wall penetration is less than 50%. Any contribution to the total leakage during a postulated SLB must be considered in combination with 4.4.6.4.a.11.1 above.
3. A tube shall be plugged or repaired if the signal amplitude of the crack indication is greater than 3.0 volts and the depth of wall penetration is greater than or equal to 50% through-wall or the signal amplitude of the crack indication is greater than 8.0 volts, regardless of depth of tube wall penetration.

For all cases above, the tubes will be dispositioned for continued service based upon bobbin voltage and depth measurements. The plant specific guidelines used for all inspections shall be amended as appropriate to accommodate the additional information needed to evaluate tube support plate signals with respect to the above voltage/depth parameters. Pending incorporation of the voltage verification requirement in ASME standard verifications, an ASME standard calibrated against the laboratory standard will be utilized in the Farley steam generator inspections for consistent voltage normalization.

- b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug or repair of all tubes exceeding the plugging or repair limit) required by Table 4.4-2.

4.4.6.5 Reports

- a. Following each inservice inspection of steam generator tubes, the number of tubes plugged or repaired in each steam generator shall be reported to the Commission within 15 days of the completion of the plugging or repair effort.
- b. The complete results of the steam generator tube and sleeve inservice inspection shall be submitted to the Commission in a Special Report pursuant to Specification 6.9.2 within 12 months following the completion of the inspection. This Special Report shall include:

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

1. Number and extent of tubes and sleeves inspected.
 2. Location and percent of wall-thickness penetration for each indication of an imperfection.
 3. Identification of tubes plugged or repaired.
- c. Results of steam generator tube inspections which fall into Category C-3 shall be considered a REPORTABLE EVENT and shall be reported pursuant to 10CFR50.73 prior to resumption of plant operation. The written report shall provide a description of investigations conducted to determine the cause of the tube degradation and corrective measures taken to prevent recurrence.
- d. The results of inspections performed under 4.4.6.2 for all tubes in which the tube support plate elevations alternate plugging limit has been applied shall be reported to the Commission following the inspection and prior to the resumption of plant operation. The report shall include:
1. Listing of applicable tubes.
 2. Location (applicable intersections per tube) and extent of degradation (volume and indicated depth).

REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.7.2 Reactor Coolant System leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE,
- b. 1 GPM UNIDENTIFIED LEAKAGE,
- c. 450 gallons per day total primary-to-secondary leakage through all steam generators and 150 gallons per day through any one steam generator,
- d. 10 GPM IDENTIFIED LEAKAGE from the Reactor Coolant System, and
- e. 31 GPM CONTROLLED LEAKAGE at a Reactor Coolant System pressure of 2235 ± 20 psig.
- f. The maximum allowable leakage of any Reactor Coolant System Pressure Isolation Valve shall be as specified in Table 3.4-1 at a pressure of 2235 ± 20 psig.

APPLICABILITY: MODES 1, 2, 3 and 4

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With any Reactor Coolant System Pressure Isolation Valve leakage greater than the limit specified in Table 3.4-1, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed manual or deactivated automatic valves, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.7.2.1 Reactor Coolant System leakages shall be demonstrated to be within each of the above limits by:

- a. Monitoring the containment atmosphere particulate radioactivity monitor at least once per 12 hours.
- b. Monitoring the containment air cooler condensate level system or containment atmosphere gaseous radioactivity monitor at least once per 12 hours.

REACTOR COOLANT SYSTEM

BASES

3/4.4.6 STEAM GENERATORS

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on a modification of Regulatory Guide 1.83, Revision 1. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or inservice conditions that lead to corrosion. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

The plant is expected to be operated in a manner such that the secondary coolant will be maintained within those chemistry limits found to result in negligible corrosion of the steam generator tubes. If the secondary coolant chemistry is not maintained within these limits, localized corrosion may likely result in stress corrosion cracking. The extent of cracking during plant operation would be limited by the limitation of steam generator tube leakage between the primary coolant system and the secondary coolant system. The allowable primary-to-secondary leak rate is 150 gallons per day per steam generator. Cracks having a primary-to-secondary leakage less than this limit during operation will have an adequate margin of safety to withstand the loads imposed during normal operation and by postulated accidents. Operational leakage of this magnitude can be readily detected by the existing Farley Unit 1 radiation monitors. Leakage in excess of this limit will require plant shutdown and an inspection, during which the leaking tubes will be located and plugged or repaired.

Wastage-type defects are unlikely with proper chemistry treatment of the secondary coolant. However, even if a defect should develop in service, it will be found during scheduled inservice steam generator tube examinations. Plugging or repair will be required for all tubes with imperfections exceeding 40% of the tube nominal wall thickness. If a sleeved tube is found to have through wall penetration of greater than or equal to 31% for the mechanical sleeve and 37% for the laser welded sleeve of sleeve nominal wall thickness in the sleeve, it must be plugged. The 31% and 37% limits are derived from R.G. 1.121 calculations with 20% added for conservatism. The portion of the tube and the sleeve for which indications of wall degradation must be evaluated can be summarized as follows:

BASES

a. Mechanical

1. Indications of degradation in the entire length of the sleeve must be evaluated against the sleeve plugging limit.
2. Indication of tube degradation of any type including a complete guillotine break in the tube between the bottom of the upper joint and the top of the lower roll expansion does not require that the tube be removed from service.
3. The tube plugging limit continues to apply to the portion of the tube in the entire upper joint region and in the lower roll expansion. As noted above the sleeve plugging limit applies to these areas also.
4. The tube plugging limit continues to apply to that portion of the tube above the top of the upper joint.

b. Laser Welded

1. Indications of degradation in the length of the sleeve between the weld joints must be evaluated against the sleeve plugging limit.
2. Indication of tube degradation of any type including a complete break in the tube between the upper weld joint and the lower weld joint does not require that the tube be removed from service.
3. At the weld joint, degradation must be evaluated in both the sleeve and tube.
4. In a joint with more than one weld, the weld closest to the end of sleeve represents the joint to be inspected and the limit of the sleeve inspection.
5. The tube plugging limit continues to apply to the portion of the tube above the upper weld joint and below the lower weld joint.

Steam generator tube inspections of operating plants have demonstrated the capability to reliably detect degradation that has penetrated 20% of the original tube wall thickness.

REACTOR COOLANT SYSTEM

BASES

Tubes experiencing outer diameter stress corrosion cracking within the thickness of the tube support plates are plugged or repaired by the criteria of 4.4.6.4.a.11.

Whenever the results of any steam generator tubing inservice inspection fall into Category C-3, these results will be reported to the Commission pursuant to 10CFR50.73 prior to resumption of plant operation. Such cases will be considered by the Commission on a case-by-case basis and may result in a requirement for analysis, laboratory examinations, tests, additional eddy-current inspection, and revision of the Technical Specifications, if necessary.

3/4.4.7 REACTOR COOLANT SYSTEM LEAKAGE

3/4.4.7.1 LEAKAGE DETECTION SYSTEMS

The RCS leakage detection systems required by this specification are provided to monitor and detect leakage from the Reactor Coolant Pressure Boundary. These detection systems are consistent with the recommendations of Regulatory Guide 1.45, "Reactor Coolant Pressure Boundary Leakage Detection Systems, " May 1973.

3/4.4.7.2 OPERATIONAL LEAKAGE

Industry experience has shown that while a limited amount of leakage is expected from the RCS, the unidentified portion of this leakage can be reduced to a threshold value of less than 1 GPM. This threshold value is sufficiently low to ensure early detection of additional leakage.

The 10 GPM IDENTIFIED LEAKAGE limitation provides allowance for a limited amount of leakage from known sources whose presence will not interfere with the detection of UNIDENTIFIED LEAKAGE by the leakage detection systems.

The CONTROLLED LEAKAGE limitation restricts operation when the total flow supplied to the reactor coolant pump seals exceeds 31 GPM with the modulating valve in the supply line fully open at a nominal RCS pressure of 2235 psig. This limitation ensures that in the event of a LOCA, the safety injection flow will not be less than assumed in the accident analyses.

The surveillance requirements for RCS Pressure Isolation Valves provide added assurance of valve integrity, thereby reducing the probability of gross valve failure and consequent intersystem LOCA. Leakage from the RCS Pressure Isolation valves is IDENTIFIED LEAKAGE and will be considered a portion of the allowed limit.

REACTOR COOLANT SYSTEM

BASES

The 150 GPD tube leakage limit per steam generator (450 GPD total) helps to maintain steam generator tube integrity in the event of a main steam line rupture or under LOCA conditions. By maintaining an operating leakage limit of 150 GPD per steam generator, the leak rate following a steam line rupture is also limited to 55 GPM in the faulted loop and 150 GPD per steam generator in the intact loops, which will limit offsite doses to within 10 percent of the 10 CFR 100 guidelines.

PRESSURE BOUNDARY LEAKAGE of any magnitude is unacceptable since it may be indicative of an impending gross failure of the pressure boundary. Therefore, the presence of any PRESSURE BOUNDARY LEAKAGE requires the unit to be promptly placed in COLD SHUTDOWN.

3.4.4.8 CHEMISTRY

The limitations on Reactor Coolant System chemistry ensure that corrosion of the Reactor Coolant System is minimized and reduces the potential for Reactor Coolant System leakage or failure due to stress corrosion. Maintaining the chemistry within the Steady State Limits provides adequate corrosion protection to ensure the structural integrity of the Reactor Coolant System over the life of the plant. The associated effects of exceeding the oxygen, chloride and fluoride limits are time and temperature dependent. Corrosion studies show that operation may be continued with contaminant concentration levels in excess of the Steady State Limits, up to the Transient Limits, for the specified limited time intervals without having a significant effect on the structural integrity of the Reactor Coolant System. The time interval permitting continued operation within the restrictions of the Transient Limits provides time for taking corrective actions to restore the contaminant concentrations to within the Steady State Limits.

The surveillance requirements provide adequate assurance that concentrations in excess of the limits will be detected in sufficient time to take corrective action.

BASES3/4.4.9 SPECIFIC ACTIVITY

The limitations on the specific activity of the primary coolant ensure that the resulting 2 hour doses at the site boundary will not exceed an appropriately small fraction of Part 100 limits following a steam generator tube rupture accident in conjunction with an assumed steady state primary-to-secondary steam generator leakage rate of 1.0 GPM. The values for the limits on specific activity represent limits based upon a parametric evaluation by the NRC of typical site locations. These values are conservative in that specific site parameters of the Farley site, such as site boundary location and meteorological conditions, were not considered in this evaluation.

The leakage limit is restricted to 450 gpd (0.315 gpm) total leakage (150 gpd per steam generator) to ensure that offsite doses following a main steam line break will be limited to 10 percent of the 10 CFR 100 guideline. This restriction is based on the results of a Farley site specific radiological evaluation that assumes a primary coolant iodine activity level corresponding to 1 percent fuel defects (approximately 4.0 microCurie/gram DOSE EQUIVALENT I-131), rather than a specific activity of 1.0 microCurie/gram DOSE EQUIVALENT I-131, and a post-accident primary-to-secondary leak rate of 55 GPM in the faulted loop.

The ACTION statement permitting POWER OPERATION to continue for limited time periods with the primary coolant's specific activity greater than 1.0 microCuries/gram DOSE EQUIVALENT I-131, but within the allowable limit shown on Figure 3.4-1, accommodates possible iodine spiking phenomenon which may occur following changes in THERMAL POWER.

REACTOR COOLANT SYSTEM

3/4.4.6 STEAM GENERATORS

LIMITING CONDITION FOR OPERATION

3.4.6 Each steam generator shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3 and 4.

ACTION:

With one or more steam generators inoperable, restore the inoperable generator(s) to OPERABLE status prior to increasing T_{avg} above 200°F.

SURVEILLANCE REQUIREMENTS

4.4.6.0 Each steam generator shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

4.4.6.1 Steam Generator Sample Selection and Inspection - Each steam generator shall be determined OPERABLE during shutdown by selecting and inspecting at least the minimum number of steam generators specified in Table 4.4-1.

4.4.6.2.1 Steam Generator Tube# Sample Selection and Inspection - The steam generator tube minimum sample size, inspector result classification, and the corresponding action required shall be as specified in Table 4.4-2. The inservice inspection of steam generator tubes shall be performed at the frequencies specified in Specification 4.4.6.3 and the inspected tubes shall be verified acceptable per the acceptance criteria of Specification 4.4.6.4. The tubes selected for each inservice inspection shall include at least 3% of the total number of tubes in all steam generators. Selection of tubes to be inspected is not affected by the F* designation. Implementation of the steam generator tube/tube support plate elevation alternate plugging criteria requires a 100% bobbin probe inspection. An inspection using the rotating pancake coil (RPC) probe is required for all outer diameter stress corrosion cracking (OD SCC) indications exceeding a 1.5 volt bobbin coil signal amplitude. The RPC results are to be evaluated to establish that the principal indications can be characterized as OD SCC. When applying the exceptions of 4.4.6.2.1.a through 4.4.6.2.1.c, previous defects or imperfections in the area repaired by sleeving are not considered an area requiring reinspection. The tubes selected for these inspections shall be selected on a random basis except:

- a. Where experience in similar plants with similar water chemistry indicates critical areas to be inspected, then at least 50% of the tubes inspected shall be from these critical areas.

#When referring to a steam generator tube, the sleeve shall be considered a part of the tube if the tube has been repaired per Specification

4.4.6.4.a.9.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- b. The first sample of tubes selected for each inservice inspection (subsequent to the preservice inspection) of each steam generator shall include:
 1. All nonplugged tubes that previously had detectable wall penetrations greater than 20%.
 2. Tubes in those areas where experience has indicated potential problems.
 3. At least 3% of the total number of sleeved tubes in all three steam generators or all of the sleeved tubes in the generator chosen for the inspection program, whichever is less. These inspections will include both the tube and the sleeve.
 4. A tube inspection (pursuant to Specification 4.4.6.4.a.8) shall be performed on each selected tube. If any selected tube does not permit the passage of the eddy current probe for a tube or sleeve inspection, this shall be recorded and an adjacent tube shall be selected and subjected to a tube inspection.

- c. The tubes selected as the second and third samples (if required by Table 4.4-2) during each inservice inspection may be subjected to a partial tube inspection provided:
 1. The tubes selected for these samples include the tubes from those areas of the tube sheet array where tubes with imperfections were previously found.
 2. The inspections include those portions of the tubes where imperfections were previously found.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

The results of each sample inspection shall be classified into one of the following three categories:

<u>Category</u>	<u>Inspection Results</u>
C-1	Less than 5% of the total tubes inspected are degraded tubes and none of the inspected tubes are defective.
C-2	One or more tubes, but not more than 1% of the total tubes inspected are defective, or between 5% and 10% of the total tubes inspected are degraded tubes.
C-3	More than 10% of the total tubes inspected are degraded tubes or more than 1% of the inspected tubes are defective.

Note: In all inspections, previously degraded tubes or sleeves must exhibit significant (greater than 10%) further wall penetrations to be included in the above percentage calculations.

4.4.6.2.2 Steam Generator F* Tube Inspection - In addition to the minimum sample size as determined by Specification 4.4.6.2.1, all F* tubes will be inspected within the tubesheet region. The results of this inspection will not be a cause for additional inspections per Table 4.4-2.

4.4.6.3 Inspection Frequencies - The above required inservice inspections of steam generator tubes shall be performed at the following frequencies:

- a. The first inservice inspection shall be performed after 6 Effective Full Power Months but within 24 calendar months of initial criticality. Subsequent inservice inspections shall be performed at intervals of not less than 12 nor more than 24 calendar months after the previous inspection. If two consecutive inspections following service under AVT conditions, not including the preservice inspection, result in all inspection results falling into the C-1 category or if two consecutive inspections demonstrate that previously observed degradation has not continued and no additional degradation has occurred, the inspection interval may be extended to a maximum of once per 40 months.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

- b. If the results of the inservice inspection of a steam generator conducted in accordance with Table 4.4-2 at 40 month intervals fall in Category C-3, the inspection frequency shall be increased to at least once per 20 months. The increase in inspection frequency shall apply until the subsequent inspections satisfy the criteria of Specification 4.4.6.3.a; the interval may then be extended to a maximum of once per 40 months.
- c. Additional, unscheduled inservice inspections shall be performed on each steam generator in accordance with the first sample inspection specified in Table 4.4-2 during the shutdown subsequent to any of the following conditions:
1. Primary-to-secondary tube leaks (not including leaks originating from tube-to-tube sheet welds) in excess of the limits of Specification 3.4.7.2.
 2. A seismic occurrence greater than the Operating Basis Earthquake.
 3. A loss-of-coolant accident requiring actuation of the engineered safeguards.
 4. A main steam line or feedwater line break.
- d. Tubes in which the tube support plate elevation alternate plugging criteria have been applied shall be inspected during all future refueling outages.

4.4.6.4. Acceptance Criteria

- a. As used in this Specification:
1. Imperfection means an exception to the dimensions, finish or contour of a tube or sleeve from that required by fabrication drawings or specifications. Eddy-current testing indications below 20% of the nominal wall thickness, if detectable, may be considered as imperfections.
 2. Degradation means a service-induced cracking, wastage, wear or general corrosion occurring on either inside or outside of a tube or sleeve.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

3. Degraded Tube means a tube, including the sleeve if the tube has been repaired, that contains imperfections greater than or equal to 20% of the nominal wall thickness caused by degradation.
4. % Degradation means the percentage of the tube or sleeve wall thickness affected or removed by degradation.
5. Defect means an imperfection of such severity that it exceeds the plugging or repair limit. A tube or sleeve containing a defect is defective.
6. Plugging or Repair Limit means the imperfection depth at or beyond which the tube shall be repaired (i.e., sleeved) or removed from service by plugging and is greater than or equal to 60% of the nominal tube wall thickness. This definition does not apply to the area of the tubesheet region below the F* distance in F* tubes. For a tube that has been sleeved with a mechanical joint sleeve, through wall penetration of greater than or equal to 31% of sleeve nominal wall thickness in the sleeve requires the tube to be removed from service by plugging. For a tube that has been sleeved with a welded joint sleeve, through wall penetration greater than or equal to 37% of sleeve nominal wall thickness in the sleeve between the weld joints requires the tube to be removed from service by plugging. This definition does not apply for tubes experiencing outer diameter stress corrosion cracking confirmed by bobbin probe inspection to be within the thickness of the tube support plates. See 4.4.6.4.a.14 for the plugging limit to use within the thickness of the tube support plates.
7. Unserviceable describes the condition of a tube or sleeve if it leaks or contains a defect large enough to affect its structural integrity in the event of an Operating Basis Earthquake, a loss-of-coolant accident, or a steam line or feedwater line break as specified in 4.4.6.3.c, above.
8. Tube Inspection means an inspection of the steam generator tube from the point of entry (hot leg side) completely around the U-bend to the top support of the cold leg. For a tube that has been repaired by sleeving, the tube inspection should include the sleeved portion of the tube. For a tube in which the tube support plate elevation alternate plugging criteria have been applied, the inspection will include all the hot leg intersections and all cold leg intersections down to, at least, the level of the last crack indication.

SURVEILLANCE REQUIREMENTS (Continued)

9. Tube Repair refers to mechanical sleeving, as described by Westinghouse report WCAP-11178 Rev. 1, or laser welded sleeving, as described by Westinghouse report WCAP-12672, which is used to maintain a tube in service. This includes the removal of plugs that were installed as a corrective or preventive measure.
10. Preservice Inspection means an inspection of the full length of each tube in each steam generator performed by eddy current techniques prior to service to establish a baseline condition of the tubing. This inspection shall be performed after the field hydrostatic test and prior to initial POWER OPERATION using the equipment and techniques expected to be used during subsequent inservice inspections.
11. F* Distance is the distance of the expanded portion of a tube which provides a sufficient length of undegraded tube expansion to resist pullout of the tube from the tubesheet. The F* distance is equal to 1.79 inches and is measured down from the top of the tubesheet or the bottom of the roll transition, whichever is lower in elevation.
12. F* Tube is a tube:
 - a) with degradation equal to or greater than 40% below the F* distance, and b) which has no indication of imperfections greater than or equal to 20% of nominal wall thickness within the F* distance, and c) that remains in service.
13. Tube Expansion is that portion of a tube which has been increased in diameter by a rolling process such that no crevice exists between the outside diameter of the tube and the hole in the tubesheet.
14. The following bobbin coil signal amplitude and indicated depth criteria (Tube Support Plate Alternate Plugging Criteria) are used for dispositioning a steam generator tube for continued service that is experiencing outer diameter initiated stress corrosion cracking confined within the thickness of the tube support plates#:

These criteria do not apply to tubes which may deform or collapse on the Farley Unit 2 steam generators during a postulated LOCA + SSE Event. These tubes are identified in WCAP-12871 "J. M. Farley Units 1 and 2 SG Tube Plugging Criteria for ODSCC at Tube Support Plates."

SURVEILLANCE REQUIREMENTS (Continued)

1. A tube can remain in service if the signal amplitude of a crack indication is less than or equal to 3.0 volts, regardless of the depth of wall tube penetration, if, as a result, the projected end of cycle distribution of crack indications is verified to result in primary to secondary leakage less than 55.0 gpm in the faulted loop (less than 150 gpd in the intact loops) during a postulated steam line break event. This would ensure acceptable radiological conditions following a SLP.##
2. A tube can remain in service if the signal amplitude of a crack indication is greater than 3.0 volts but less than or equal to 8.0 volts and the depth of tube wall penetration is less than 50%. Any contribution to the total leakage during a postulated SLB must be considered in combination with 4.4.6.4.a.14.1 above.
3. A tube shall be plugged or repaired if the signal amplitude of the crack indication is greater than 3.0 volts and the depth of wall penetration is greater than or equal to 50% through-wall or the signal amplitude of the crack indication is greater than 8.0 volts, regardless of depth of tube wall penetration.

For all cases above, the tubes will be dispositioned for continued service based upon bobbin voltage and depth measurements. The plant specific guidelines used for all inspections shall be amended as appropriate to accommodate the additional information needed to evaluate tube support plate signals with respect to the above voltage/depth parameters. Pending incorporation of the voltage verification requirement in ASME standard verifications, an ASME standard calibrated against the laboratory standard will be utilized in the Farley steam generator inspections for consistent voltage normalization.

- b. The steam generator shall be determined OPERABLE after completing the corresponding actions (plug or repair of all tubes exceeding the plugging or repair limit) required by Table 4.4-2.

The methodology for calculating expected leak rates from the projected crack distribution must be consistent with WCAP-12871.

REACTOR COOLANT SYSTEM

SURVEILLANCE REQUIREMENTS (Continued)

4.4.6.5 Reports

- a. Following each inservice inspection of steam generator tubes, the number of tubes plugged, repaired or designated F* in each steam generator shall be reported to the Commission within 15 days of the completion of the inspection, plugging or repair effort.
- b. The complete results of the steam generator tube and sleeve inservice inspection shall be submitted to the Commission in a Special Report pursuant to Specification 6.9.2 within 12 months following the completion of the inspection. This Special Report shall include:
 1. Number and extent of tubes and sleeves inspected.
 2. Location and percent of wall-thickness penetration for each indication of an imperfection.
 3. Identification of tubes plugged or repaired.
- c. Results of steam generator tube inspections which fall into Category C-3 shall be considered a REPORTABLE EVENT and shall be reported pursuant to 10CFR50.73 prior to resumption of plant operation. The written report shall provide a description of investigations conducted to determine the cause of the tube degradation and corrective measures taken to prevent recurrence.
- d. The results of inspections performed under 4.4.6.2 for all tubes in which the tube support plate elevations alternate plugging limit has been applied shall be reported to the Commission following the inspection and prior to the resumption of plant operation. The report shall include:
 1. Listing of applicable tubes.
 2. Location (applicable intersections per tube) and extent of degradation (voltage and indicated depth).

REACTOR COOLANT SYSTEM

OPERATIONAL LEAKAGE

LIMITING CONDITION FOR OPERATION

3.4.7.2 Reactor Coolant System leakage shall be limited to:

- a. No PRESSURE BOUNDARY LEAKAGE,
- b. 1 GPM UNIDENTIFIED LEAKAGE,
- c. 450 gallons per day total primary-to-secondary leakage through all steam generators and 150 gallons per day through any one steam generator,
- d. 10 GPM IDENTIFIED LEAKAGE from the Reactor Coolant System, and
- e. 31 GPM CONTROLLED LEAKAGE at a Reactor Coolant System pressure of 2235 ± 20 psig.
- f. The maximum allowable leakage of any Reactor Coolant System Pressure Isolation Valve shall be as specified in Table 3.4-1 at a pressure of 2235 ± 20 psig.

APPLICABILITY: MODES 1, 2, 3 and 4

ACTION:

- a. With any PRESSURE BOUNDARY LEAKAGE, be in at least HOT STANDBY within 6 hours and in COLD SHUTDOWN within the following 30 hours.
- b. With any Reactor Coolant System leakage greater than any one of the above limits, excluding PRESSURE BOUNDARY LEAKAGE, reduce the leakage rate to within limits within 4 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.
- c. With any Reactor Coolant System Pressure Isolation Valve leakage greater than the limit specified in Table 3.4-1, isolate the high pressure portion of the affected system from the low pressure portion within 4 hours by use of at least two closed manual or deactivated automatic valves, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.4.7.2.1 Reactor Coolant System leakages shall be demonstrated to be within each of the above limits by:

- a. Monitoring the containment atmosphere particulate radioactivity monitor at least once per 12 hours.
- b. Monitoring the containment air cooler condensate level system or containment atmosphere gaseous radioactivity monitor at least once per 12 hours.

REACTOR COOLANT SYSTEM

BASES

3/4.4.6 STEAM GENERATORS

The Surveillance Requirements for inspection of the steam generator tubes ensure that the structural integrity of this portion of the RCS will be maintained. The program for inservice inspection of steam generator tubes is based on a modification of Regulatory Guide 1.83, Revision 1. Inservice inspection of steam generator tubing is essential in order to maintain surveillance of the conditions of the tubes in the event that there is evidence of mechanical damage or progressive degradation due to design, manufacturing errors, or inservice conditions that lead to corrosion. Inservice inspection of steam generator tubing also provides a means of characterizing the nature and cause of any tube degradation so that corrective measures can be taken.

The plant is expected to be operated in a manner such that the secondary coolant will be maintained within those chemistry limits found to result in negligible corrosion of the steam generator tubes. If the secondary coolant chemistry is not maintained within these limits, localized corrosion may likely result in stress corrosion cracking. The extent of cracking during plant operation would be limited by the limitation of steam generator tube leakage between the primary coolant system and the secondary coolant system. The allowable primary-to-secondary leak rate is 150 gallons per day per steam generator. Cracks having a primary-to-secondary leakage less than this limit during operation will have an adequate margin of safety to withstand the loads imposed during normal operation and by postulated accidents. Operational leakage of this magnitude can be readily detected by the existing Farley Unit 2 radiation monitors. Leakage in excess of this limit will require plant shutdown and an inspection, during which the leaking tubes will be located and plugged or repaired.

Wastage-type defects are unlikely with proper chemistry treatment of the secondary coolant. However, even if a defect should develop in service, it will be found during scheduled inservice steam generator tube examinations. Plugging or repair will be required for all tubes with imperfections exceeding 40% of the tube nominal wall thickness. If a sleeved tube is found to have through wall penetration of greater than or equal to 31% for the mechanical sleeve and 37% for the laser welded sleeve of sleeve nominal wall thickness in the sleeve, it must be plugged. The 31% and 37% limits are derived from R.G. 1.121 calculations with 20% added for conservatism. The portion of the tube and the sleeve for which indications of wall degradation must be evaluated can be summarized as follows:

a. Mechanical

1. Indications of degradation in the entire length of the sleeve must be evaluated against the sleeve plugging limit.

REACTOR COOLANT SYSTEM

BASES

2. Indication of tube degradation of any type including a complete guillotine break in the tube between the bottom of the upper joint and the top of the lower roll expansion does not require that the tube be removed from service.
3. The tube plugging limit continues to apply to the portion of the tube in the entire upper joint region and in the lower roll expansion. As noted above the sleeve plugging limit applies to these areas also.
4. The tube plugging limit continues to apply to that portion of the tube above the top of the upper joint.

b. Laser Welded

1. Indications of degradation in the length of the sleeve between the weld joints must be evaluated against the sleeve plugging limit.
2. Indication of tube degradation of any type including a complete break in the tube between the upper weld joint and the lower weld joint does not require that the tube be removed from service.
3. At the weld joint, degradation must be evaluated in both the sleeve and tube.
4. In a joint with more than one weld, the weld closest to the end of the sleeve represents the joint to be inspected and the limit of the sleeve inspection.
5. The tube plugging limit continues to apply to the portion of the tube above the upper weld joint, and below the lower weld joint.

F* tubes do not have to be plugged or repaired provided the remainder of the tube within the tubesheet that is above the F* distance is not degraded. The F* distance is equal to 1.79 inches and is measured down from the top of the tubesheet or the bottom of the roll transition, whichever is lower in elevation. Included in this distance is an allowance of 0.25 inch for eddy current elevation measurement uncertainty.

Tubes experiencing outer diameter stress corrosion cracking within the thickness of the tube support plates are plugged or repaired by the criteria of 4.4.6.4.a.14.

Steam generator tube inspections of operating plants have demonstrated the capability to reliably detect wastage type degradation that has penetrated 20% of the original tube wall thickness.

REACTOR COOLANT SYSTEM

BASES

Whenever the results of any steam generator tubing insert inspection fall into Category C-3, these results will be reported to the Commission pursuant to 10CFR50.73 prior to resumption of plant operation. Such cases will be considered by the Commission on a case-by-case basis and may result in a requirement for analysis, laboratory examinations, tests, additional eddy-current inspection, and revision of the Technical Specifications, if necessary.

3/4.4.7 REACTOR COOLANT SYSTEM LEAKAGE

3/4.4.7.1 LEAKAGE DETECTION SYSTEMS

The RCS leakage detection systems required by this specification are provided to monitor and detect leakage from the Reactor Coolant Pressure Boundary. These detection systems are consistent with the recommendations of Regulatory Guide 1.45, "Reactor Coolant Pressure Boundary Leakage Detection Systems," May 1973.

3/4.4.7.2 OPERATIONAL LEAKAGE

Industry experience has shown that while a limited amount of leakage is expected from the RCS, the unidentified portion of this leakage can be reduced to a threshold value of less than 1 GPM. This threshold value is sufficiently low to ensure early detection of additional leakage.

The 10 GPM IDENTIFIED LEAKAGE limitation provides allowance for a limited amount of leakage from known sources whose presence will not interfere with the detection of UNIDENTIFIED LEAKAGE by the leakage detection systems.

The CONTROLLED LEAKAGE limitation restricts operation when the total flow supplied to the reactor coolant pump seals exceeds 31 GPM with the modulating valve in the supply line full open at a nominal RCS pressure of 2235 psig. This limitation ensures that in the event of a LOCA, the safety injection flow will not be less than assumed in the accident analyses.

The surveillance requirements for RCS Pressure Isolation Valves provide added assurance of valve integrity, thereby reducing the probability of gross valve failure and consequent intersystem LOCA. Leakage from the RCS Pressure Isolation valves is IDENTIFIED LEAKAGE and will be considered a portion of the allowed limit.

REACTOR COOLANT SYSTEM

BASES

The 150 GPD tube leakage limit per steam generator (450 GPD total) helps to maintain steam generator tube integrity in the event of a main steam line rupture or under LOCA conditions. By maintaining an operating leakage limit of 150 GPD per steam generator, the leak rate following a steam line rupture is also limited to 55 GPM in the faulted loop and 150 GPD per steam generator in the intact loops, which will limit offsite doses to within 10 percent of the 10 CFR 100 guidelines.

PRESSURE BOUNDARY LEAKAGE of any magnitude is unacceptable since it may be indicative of an impending gross failure of the pressure boundary. Therefore, the presence of any PRESSURE BOUNDARY LEAKAGE requires the unit to be promptly placed in COLD SHUTDOWN.

3/4.4.8 CHEMISTRY

The limitations on Reactor Coolant System chemistry ensure that corrosion of the Reactor Coolant System is minimized and reduces the potential for Reactor Coolant System leakage or failure due to stress corrosion. Maintaining the chemistry within the Steady State Limits provides adequate corrosion protection to ensure the structural integrity of the Reactor Coolant System over the life of the plant. The associated effects of exceeding the oxygen, chloride and fluoride limits are time and temperature dependent. Corrosion studies show that operation may be continued with contaminant concentration levels in excess of the Steady State Limits, up to the Transient Limits, for the specified limited time intervals without having a significant effect on the structural integrity of the Reactor Coolant System. The time interval permitting continued operation within the restrictions of the Transient Limits provides time for taking corrective actions to restore the contaminant concentrations to within the Steady State Limits.

The surveillance requirements provide adequate assurance that concentrations in excess of the limits will be detected in sufficient time to take corrective action.

3/4.4.9 SPECIFIC ACTIVITY

The limitations on the specific activity of the primary coolant ensure that the resulting 2 hour doses at the site boundary will not exceed an appropriately small fraction of Part 100 limits following a steam generator tube rupture accident in conjunction with an assumed steady state primary-to-secondary steam generator leakage rate of 1.0 GPM. The values for the limits on specific activity represent limits based upon a parametric evaluation by the NRC of typical site locations. These values are conservative in that specific site parameters of the Farley site, such as site boundary location and meteorological conditions, were not considered in this evaluation.

REACTOR COOLANT SYSTEM

BASES

The leakage limit is restricted to 450 GPD (0.315 GPM) total leakage (150 GPD per steam generator) to ensure that offsite doses following a main steam line break will be limited to 10 percent of the 10 CFR 100 guideline. This restriction is based on the results of a Farley site specific radiological evaluation that assumes a primary coolant iodine activity level corresponding to 1 percent fuel defects (approximately 4.0 microCurie/gram DOSE EQUIVALENT I-131), rather than a specific activity of 1.0 microCurie/gram DOSE EQUIVALENT I-131, and a post-accident primary-to-secondary leak rate of 55 GPM in the faulted loop.

The ACTION statement permitting POWER OPERATION to continue for limited time periods with the primary coolant's specific activity greater than 1.0 microCurie/gram DOSE EQUIVALENT I-131, but within the allowable limit shown on Figure 3.4-1, accommodates possible iodine spiking phenomenon which may occur following changes in THERMAL POWER.

Attachment 2

Significant Hazards Evaluation

FARLEY NUCLEAR POWER PLANT UNIT 1 AND 2

TUBE SUPPORT PLATE EVALUATION SG TUBE PLUGGING CRITERIA

SIGNIFICANT HAZARDS CONSIDERATION ANALYSIS

INTRODUCTION

A license amendment is proposed to preclude unnecessarily plugging tubes due to the occurrence of outer diameter initiated stress corrosion cracking (OD SCC) at the tube support plate elevations in the Farley Unit 1 and 2 steam generators. Using the existing Technical Specification steam generator tube plugging limit of 40% allowable tube wall penetration, many of the tubes with crack indications would needlessly have to be removed from service. The alternate plugging criteria for the tube support plate elevation OD SCC occurring in the Farley Unit 1 and 2 steam generators may result in tubes with both partial and through-wall cracks returning to service. In the limiting case, it is demonstrated that the presence of through wall cracks alone is not reason enough to remove a tube from service. The alternate plugging criteria will not be applied to tubes which may deform or collapse in the Farley Unit 1 and 2 steam generators during a postulated Loss of Coolant Accident plus Safe Shutdown Earthquake (LOCA + SSE) event. This action minimizes the potential for excessive secondary to primary in-leakage due to any potential pre-existing cracks. For all other tubes, secondary to primary in-leakage during a postulated LOCA + SSE event is expected to be negligible.

DESCRIPTION OF THE AMENDMENT REQUEST

As required by 10 CFR 50.91 (a)(1), an analysis is provided to demonstrate that the proposed license amendment to implement alternate steam generator tube plugging criteria for the tube support plate elevations at Farley Unit 1 and 2 involves no significant hazards considerations. The alternate plugging criteria involve a correlation between eddy current bobbin probe signal amplitude (voltage) and indicated depth (phase angle) versus tube burst and leakage capability. The principal parameter is voltage amplitude which is correlated with tube burst capability and leakage potential. Indicated depth is a secondary parameter utilized as a threshold value below which added margins incorporated in the plugging criteria to minimize excessive steam line break leakage are not necessary. The plugging criteria are developed from testing of laboratory induced OD SCC specimens, extensive examination of pulled tubes from operating steam generators, and field experience from leakage due to indications at the tube support plates. Specifically, crack indications with bobbin probe voltages less than or equal to 3.0 volts, regardless of indicated depth, do not require remedial action if postulated steam line break leakage can be shown to be acceptable. Crack indications with bobbin probe voltages exceeding 3.0 volts with depths greater than or equal to 50% will be plugged. For crack indications with voltages greater than 8 volts, the tubes are recommended to be plugged no matter what the indicated depth.

The proposed amendment would modify Technical Specification 3/4.4.6 "Steam Generators", 3/4.4.7, "Reactor Coolant System Leakage", and the associated basis which provides tube inspection requirements and acceptance criteria to determine the level of degradation for which a tube experiencing OD SCC at the tube support plate elevations may remain in service in the Farley Unit 1 and 2 steam generators. The proposed amendment would add definitions required for the alternate plugging criteria and prescribe the portion of the tube subject to the criteria.

EVALUATION

Steam Generator Tube Integrity

In the development of the alternate plugging criteria, Regulatory Guide (RG) 1.121, "Bases for Plugging Degraded PWR Steam Generator Tubes" and RG 1.83 "Inservice Inspection of PWR Steam Generator Tubes" are used as the bases for determining that steam generator integrity considerations are maintained within acceptable limits. RG 1.121 describes a method acceptable to the NRC staff for meeting General Design Criteria 2, 14, 15, 31, and 32 by reducing the probability and consequences of steam generator tube rupture through determining the limiting safe conditions of tube wall degradation beyond which tubes with unacceptable cracking, as established by inservice inspection, should be removed from service by plugging. This regulatory guide uses safety factors on loads for tube burst that are consistent with the requirements of Section III of the ASME Code. For the tube support plate elevation degradation occurring in the Farley steam generators, tube burst criteria are inherently satisfied during normal operating conditions by the presence of the tube support plate. The presence of the tube support plate enhances the integrity of the degraded tubes in that region by precluding tube deformation beyond the diameter of the drilled hole. It is not certain whether the tube support plate would function to provide a similar constraining effect during accident condition loadings in either Farley Unit 1 or 2. Therefore, no credit is taken in the development of the plugging criteria for the presence of the tube support plate during accident condition loadings. Conservatively, based on the existing data base, burst testing shows that the safety requirements for tube burst margins during both normal and accident condition loadings can be satisfied with bobbin coil signal amplitudes less than 10.3 volts, regardless of the depth of tube wall penetration of the cracking. RG 1.83 describes a method acceptable to the NRC staff for implementing GDC 14, 15, 31, and 32 through periodic inservice inspection for detection of significant tube wall degradation.

Upon implementation of the two parameter plugging criteria, tube leakage considerations must also be addressed. It must be determined that the cracks will not leak excessively during all plant conditions. For the alternate tube plugging criteria developed for the Farley Unit 1 and 2 steam generator tubes, no leakage is expected during normal operating conditions even with the presence of through-wall cracks. This is the case as the stress corrosion cracking occurring in the tubes at the support plate elevations in the Farley Unit 1 and 2 steam generators are short, tight, axially oriented macrocracks separated by ligaments of material. No leakage during normal operating conditions has been observed for crack

indications with signal amplitudes less than 6.5 volts. Relative to the expected leakage during accident condition loadings, the limiting event with respect to primary to secondary leakage is a postulated steam line break event. Field and laboratory data show minimal leakage for a criterion of under 10.0 volts.

Additional Considerations

The proposed amendment would preclude occupational radiation exposure that would otherwise be incurred by plant workers involved in tube plugging operations. The proposed amendment would minimize the loss of margin in the reactor coolant flow through the steam generator in LOCA analyses. The proposed amendment would avoid loss of margin in reactor coolant system flow and, therefore, assist in demonstrating that minimum flow rates are maintained in excess of that required for operation at full power. Reduction in the amount of tube plugging required can reduce the length of plant outages and reduce the time that the steam generator is open to the containment environment during an outage.

ANALYSIS (3 FACTOR TEST)

In accordance with the three factor test of 10 CFR 50.92(c), implementation of the proposed license amendment is analyzed using the following standards and found not to: 1) involve a significant increase in the probability or consequences for an accident previously evaluated; or 2) create the possibility of a new or different kind of accident from any accident previously evaluated; or 3) involve a significant reduction in a margin of safety.

Conformance of the proposed amendment to the standards for a determination of no significant hazard as defined in 10 CFR 50.92 (three factor test) is shown in the following:

- 1) Operation of Farley Unit 1 and 2 in accordance with the proposed license amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

Burst testing of pulled tubes and laboratory specimens undergoing OD SCC have resulted in burst pressures at room temperature conditions of over 5900 psi for crack indications in free standing tubes with voltages as high as 10 volts. Correcting for the effects of temperature on material properties and minimum strength levels (as the burst testing was done at room temperature), tube burst capability significantly exceeds the RG 1.121 criterion requiring the maintenance of a margin of three times normal operating pressure differential on tube burst if through-wall cracks are present. Based on the existing data base, this criterion is satisfied with bobbin coil indications with signal amplitudes less than 10.3 volts regardless of the indicated depth measurement. This structural limit is based on a lower bound limit of the data. A 3 volt, greater than 50% indicated depth plugging criterion, compares favorably with the structural limit considering expected growth rates of OD SCC at Farley. This voltage

criterion is smaller than the 2 sigma lower bound limit of the data used to determine the structural limit. Tube support plate intersections with voltages much less than 10 volts (e.g., 3.0 volts, greater than 50% indicated depth) are expected to result in significantly higher tube burst strengths.

Relative to expected leakage during normal operating conditions, upon implementation of the alternate plugging criteria, both field and laboratory data show no leakage is expected from tubes through an indication with a voltage level of under 6.5 volts (as compared to the 3.0 volt, greater than 50 percent through-wall plugging limit proposed alternate tube plugging limit). Field and laboratory data support a "no leakage" criterion during all plant conditions of 1.5 volts, regardless of associated depth of tube wall penetration.

Relative to the expected leakage during accident condition loadings, the accidents that are affected by primary to secondary leakage and steam release to the environment are Loss of External Electrical Load and/or Turbine Trip, Loss of All AC Power to Station Auxiliaries, Major Secondary System Pipe Failure, Steam Generator Tube Rupture, Reactor Coolant Pump Locked Rotor, and Rupture of a Control Rod Drive Mechanism Housing. Of these, the Major Secondary System Pipe Failure is the most limiting for Farley Unit 1 and 2 in considering the potential for off-site doses. Upon implementation of the alternate plugging criteria, it must be verified that the expected distribution of cracking indications at the tube support plate intersections are such that primary to secondary leakage would result in site boundary doses within a small fraction of the 10 CFR 100 guideline during a postulated steam line break event. Application of the proposed plugging criteria require that the current distribution of number of indications versus voltage be obtained during each refueling outage. The current voltage is then combined with the rate of change in voltage measurement to establish an end of cycle voltage distribution and, thus, leak rate during steam line break pressure differential.

- 2) The proposed license amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

Implementation of the proposed alternate tube support plate elevation steam generator tube plugging criteria does not introduce any significant changes to the plant design basis. Use of the criteria does not provide a mechanism which could result in an accident outside of the region of the tube support plate elevations. Neither a single or multiple tube rupture event would be expected in a steam generator in which the plugging criteria has been applied (during all plant conditions). The two parameter plugging criteria are established such that operational leakage or excessive leakage during a postulated steam line break condition is not anticipated.

Alabama Power Company will implement a leakage rate limit of 150 gpd per steam generator (reduced from the current 500 gpd per steam generator limit) to help preclude the potential for excessive leakage during all plant conditions upon application of the plugging criteria. The RG 1.21 criterion for establishing operational leakage rate limits

that require plant shutdown are based upon leak-before-break considerations to detect a free span crack before potential tube rupture. The 150 gpd limit should provide for leakage detection and plant shutdown in the event of the occurrence of an unexpected single crack resulting in leakage that is associated with the longest permissible crack length. The longest permissible through-wall crack length is the length that provides a factor of safety of 3 on tube burst for normal operating pressure differential. For the Farley steam generators, the through-wall crack length resulting in tube burst at three times normal operating pressure differential (4380 psi) is approximately 0.42 inch. The nominal leak rate from this crack length is 0.11 gpm. A 150 gpd (approximately 0.1 gpm) leak rate limit provides for plant shutdown prior to reaching the critical crack length corresponding to three times normal operating pressure differential.

- 3) The proposed license amendment does not involve a significant reduction in margin of safety.

The use of the alternate tube support plate elevation plugging criteria at Farley Unit 1 and 2 is demonstrated to maintain steam generator tube integrity commensurate with the requirements of RG 1.121. RG 1.121 describes a method acceptable to the NRC staff for meeting GDCs 2, 14, 15, 31, and 32 by reducing the probability of the consequences of steam generator tube rupture. This is accomplished by determining the limiting conditions of degradation of steam generator tubing, as established by inservice inspection, for which tubes with unacceptable cracking should be removed from service. Upon implementation of the criteria, even under the worst case conditions, the occurrence of OD SCC at the tube support plate elevations is not expected to lead to a steam generator tube rupture event during normal or faulted plant conditions. The most limiting effect would be a possible increase in leakage during a steam line break event. Excessive leakage during a steam line break event, however, is precluded by verifying that, once the criteria is applied, the expected end of cycle distribution of crack indications at the tube support plate elevations would result in minimal, and acceptable primary to secondary leakage during all plant conditions and, hence, help to demonstrate radiological conditions are less than a small fraction of the 10 CFR 100 guideline.

In addressing the combined effects of LOCA + SSE on the steam generator component, it has been determined that tube collapse may occur in the steam generators at some plants. This is the case as the tube support plates may become deformed as a result of radial loads at the wedge supports at the periphery of the plate due to either the LOCA rarefaction wave and/or SSE loadings. Then, the resulting pressure differential on the deformed tubes may cause some of the tubes to collapse.

There are two issues associated with steam generator tube collapse. First, the collapse of steam generator tubing reduces the RCS flow area through the tubes. The reduction in flow area increases the resistance to flow of steam from the core during a LOCA which, in turn, may potentially increase Peak Clad Temperature (PCT). Second, there is a potential that partial through-wall cracks in tubes could progress to

through-wall cracks during tube deformation or collapse.

Consequently, Alabama Power Company has performed a detailed leak before break analysis and it is concluded that the leak-before-break methodology (as permitted by GDC 4) is applicable to the Farley Unit 1 and 2 reactor coolant system primary loops and, thus, the probability of breaks in the primary loop piping is sufficiently low that they need not be considered in the structural design basis of the plant. Excluding breaks in the RCS primary loops, the LOCA loads from the large branch line breaks were analyzed at Farley Unit 1 and 2 and were found to be of insufficient magnitude to result in steam generator tube collapse.

Regardless of whether or not leak-before-break is applied to the primary loop piping at Farley Unit 1 and 2, any flow area reduction is expected to be minimal (much less than 1%) and PCT margin is available to account for this potential effect. Tube locations which may potentially result in secondary to primary inleakage due to tube collapse as a result of pre-existing crack indications are excluded from the application of this criteria. For all other steam generator tubes, the possibility of secondary to primary leakage in the event of a LOCA + SSE event is not significant. In actuality, the amount of secondary to primary leakage in the event of a LOCA is expected to be less than currently allowed since steam generator tube integrity is expected to improve with the reduction of leakage allowed from 500 gpd to 150 gpd per steam generator. Furthermore, secondary to primary leakage would be less than primary to secondary leakage for the same differential pressure since the cracks would tend to tighten under a secondary to primary differential pressure. Additionally, the presence of the tube support plate is expected to reduce the in-leakage.

Addressing RG 1.83 considerations, implementation of the tube plugging criteria is supplemented by 100% inspection requirements at the tube support plate elevations having OD SCC indications, eddy current inspection guidelines to provide consistency in voltage normalization, and rotating pancake coil inspection requirements for the larger indications left in service to characterize the principal degradation mechanism as OD SCC.

As noted previously, implementation of the tube support plate elevation plugging criteria will decrease the number of tubes which must be taken out of service with tube plugs. The installation of steam generator tube plugs reduces the RCS flow margin, thus implementation of the alternate plugging criteria will maintain the margin of flow that would otherwise be reduced in the event of increased tube plugging.

Based on the above, it is concluded that the proposed change does not result in a significant reduction in margin with respect to plant safety as defined in the Final Safety Analysis Report or any bases of the plant Technical Specification.

CONCLUSION

Based on the preceding analysis, it is concluded that using the bobbin coil probe voltage/depth alternate steam generator tube plugging criteria for removing tubes from service at Farley Unit 1 and 2 is acceptable and the proposed license amendment does not involve a Significant Hazards Consideration Finding as defined in 10 CFR 50.92.

REM:2313

Attachment 3

Technical Justification

Proprietary Information Notice

Transmitted herewith are proprietary and/or non-proprietary versions of documents furnished to the NRC in connection with requests for generic and/or plant-specific review and approval.

In order to conform to the requirements of 10 CFR 2.790 of the Commission's regulations concerning the protection of proprietary information so submitted to the NRC, the information which is proprietary in the proprietary versions is contained within brackets, and where the proprietary information has been deleted in the non-proprietary versions, only the brackets remain (the information that was contained within the brackets in the proprietary versions having been deleted). The justification for claiming the information so designated as proprietary is indicated in both versions by means of lower case letters (a) through (g) contained within parentheses located as a superscript immediately following the brackets enclosing each item of information being identified as proprietary or in the margin opposite such information. These lower case letters refer to the types of information Westinghouse customarily holds in confidence identified in Sections (4)(ii)(a) through (4)(ii)(g) of the affidavit accompanying this transmittal pursuant to 10 CFR 2.790(b)(1).

Copyright Notice

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Westinghouse
Electric Corporation

Energy Systems

Box 355
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February 21, 1991
CAW-91-130

Document Control Desk
US Nuclear Regulatory Commission
Washington, DC 20555

Attention: Dr. Thomas Murley, Director

APPLICATION FOR WITHHOLDING PROPRIETARY
INFORMATION FROM PUBLIC DISCLOSURE

Subject: J. M. Farley Units 1 and 2 SG Tube Plugging Criteria for ODSCC at
Tube Support Plates (WCAP-12871)

Dear Dr. Murley:

The proprietary information for which withholding is being requested in the above referenced letter is further identified in Affidavit CAW-91-130 signed by the owner of the proprietary information, Westinghouse Electric Corporation. The affidavit, which accompanies this letter, sets forth the basis on which the information may be withheld from public disclosure by the Commission. It addresses with specificity the considerations listed in paragraph (b)(4) of 10 CFR Section 2.790 of the Commission's regulations.

Accordingly, this letter authorizes the utilization of the accompanying Affidavit by Alabama Power Company.

Correspondence with respect to the proprietary aspects of the application for withholding or the Westinghouse affidavit should reference this letter, CAW-91-130, and should be addressed to the undersigned.

Very truly yours,

R. P. DiPiazza, Manager
Operating Plant Licensing Support

Enclosures

cc: M. P. Siemien, Esq.
Office of the General Counsel, NRC

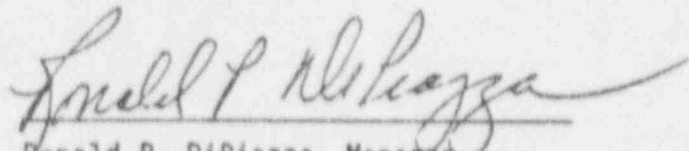
AFFIDAVIT

COMMONWEALTH OF PENNSYLVANIA:

SS

COUNTY OF ALLEGHENY:

Before me, the undersigned authority, personally appeared Ronald P. DiPiazza, who, being by me duly sworn according to law, deposes and says that he is authorized to execute this Affidavit on behalf of Westinghouse Electric Corporation ("Westinghouse") and that the averments of fact set forth in this Affidavit are true and correct to the best of his knowledge, information, and belief:



Ronald P. DiPiazza, Manager
Operating Plant Licensing Support

Sworn to and subscribed
before me this 22nd day
of February, 1991.

Lorraine M. Piplica
Notary Public

NOTARIAL SEAL
LORRAINE M. PIPLICA, NOTARY PUBLIC
MONROEVILLE BORO, ALLEGHENY COUNTY
MY COMMISSION EXPIRES DEC. 14, 1991

Member: Pennsylvania Association of Notaries

- (1) I am Manager, Operating Plant Licensing Support, in the Nuclear and Advanced Technology Division, of the Westinghouse Electric Corporation and as such, I have been specifically delegated the function of reviewing the proprietary information sought to be withheld from public disclosure in connection with nuclear power plant licensing and rulemaking proceedings, and am authorized to apply for its withholding on behalf of the Westinghouse Energy Systems Business Unit.
- (2) I am making this Affidavit in conformance with the provisions of 10CFR Section 2.790 of the Commission's regulations and in conjunction with the Westinghouse application for withholding accompanying this Affidavit.
- (3) I have personal knowledge of the criteria and procedures utilized by the Westinghouse Energy Systems Business Unit in designating information as a trade secret, privileged or as confidential commercial or financial information.
- (4) Pursuant to the provisions of paragraph (b)(4) of Section 2.790 of the Commission's regulations, the following is furnished for consideration by the Commission in determining whether the information sought to be withheld from public disclosure should be withheld.
 - (i) The information sought to be withheld from public disclosure is owned and has been held in confidence by Westinghouse.

(ii) The information is of a type customarily held in confidence by Westinghouse and not customarily disclosed to the public. Westinghouse has a rational basis for determining the types of information customarily held in confidence by it and, in that connection, utilizes a system to determine when and whether to hold certain types of information in confidence. The application of that system and the substance of that system constitutes Westinghouse policy and provides the rational basis required.

Under that system, information is held in confidence if it falls in one or more of several types, the release of which might result in the loss of an existing or potential competitive advantage, as follows:

- (a) The information reveals the distinguishing aspects of a process (or component, structure, tool, method, etc.) where prevention of its use by any of Westinghouse's competitors without license from Westinghouse constitutes a competitive economic advantage over other companies.
- (b) It consists of supporting data, including test data, relative to a process (or component, structure, tool, method, etc.), the application of which data secures a competitive economic advantage, e.g., by optimization or improved marketability.

- (c) Its use by a competitor would reduce his expenditure of resources or improve his competitive position in the design, manufacture, shipment, installation, assurance of quality, or licensing a similar product.
- (d) It reveals cost or price information, production capacities, budget levels, or commercial strategies of Westinghouse, its customers or suppliers.
- (e) It reveals aspects of past, present, or future Westinghouse or customer funded development plans and programs of potential commercial value to Westinghouse.
- (f) It contains patentable ideas, for which patent protection may be desirable.
- (g) It is not the property of Westinghouse, but must be treated as proprietary by Westinghouse according to agreements with the owner.

There are sound policy reasons behind the Westinghouse system which include the following:

- (k) The use of such information by Westinghouse gives Westinghouse a competitive advantage over its competitors. It is, therefore, withheld from disclosure to protect the Westinghouse competitive position.

- (b) It is information which is marketable in many ways. The extent to which such information is available to competitors diminishes the Westinghouse ability to sell products and services involving the use of the information.
- (c) Use by our competitor would put Westinghouse at a competitive disadvantage by reducing his expenditure of resources at our expense.
- (d) Each component of proprietary information pertinent to a particular competitive advantage is potentially as valuable as the total competitive advantage. If competitors acquire components of proprietary information, any one component may be the key to the entire puzzle, thereby depriving Westinghouse of a competitive advantage.
- (e) Unrestricted disclosure would jeopardize the position of prominence of Westinghouse in the world market, and thereby give a market advantage to the competition of those countries.
- (f) The Westinghouse capacity to invest corporate assets in research and development depends upon the success in obtaining and maintaining a competitive advantage.

- (iii) The information is being transmitted to the Commission in confidence and, under the provisions of 10CFR Section 2.790, it is to be received in confidence by the Commission.
- (iv) The information sought to be protected is not available in public sources or available information has not been previously employed in the same original manner or method to the best of our knowledge and belief.
- (v) The proprietary information sought to be withheld in this submittal is that which is appropriately marked in "J. M. Farley Units 1 and 2 SG Tube Plugging Criteria for ODSCC at Tube Support Plates", WCAP-12871, (Proprietary) for J. M. Farley Units 1 and 2, being transmitted by the Alabama Power Company (APCo) letter and Application for Withholding Proprietary Information from Public Disclosure, Mr. W. G. Hairston III, APCo, to Document Control Desk, to the Attention Dr. Thomas Murley, February, 1991. The proprietary information as submitted for use by Alabama Power Company for the J. M. Farley Units 1 and 2 is expected to be applicable in other licensee submittals in response to certain NRC requirements for steam generator tube plugging criteria for ODSCC at tube support plates.

This information is part of that which will enable Westinghouse to:

- (a) Provide documentation of the analyses, methods, and testing for reaching a conclusion relative to steam generator tube plugging criteria for ODSCC at the tube support plates.
- (b) Establish acceptable steam generator tube plugging criteria based on (a) above.
- (c) Establish the available margins for the revised plugging criteria.
- (d) Assist the customer to obtain NRC approval for steam generator tube plugging criteria for ODSCC at tube support plates.

Further this information has substantial commercial value as follows:

- (a) Westinghouse plans to sell the use of similar information to its customers for purposes of satisfying NRC requirements for licensing documentation.

(b) Westinghouse can sell support and defense of the Technical Justification technology to its customers in the licensing process.

Public disclosure of this proprietary information is likely to cause substantial harm to the competitive position of Westinghouse because it would enhance the ability of competitors to provide similar analytical documentation and licensing defense services for commercial power reactors without commensurate expenses. Also, public disclosure of the information would enable others to use the information to meet NRC requirements for licensing documentation without purchasing the right to use the information.

The development of the technology described in part by the information is the result of applying the results of many years of experience in an intensive Westinghouse effort and the expenditure of a considerable sum of money.

In order for competitors of Westinghouse to duplicate this information, similar technical programs would have to be performed and a significant manpower effort, having the requisite talent and experience, would have to be expended for developing testing and analytical methods and performing tests.

Further the deponent sayeth not.

NEC PROPRIETARY CLASS 3



Westinghouse Energy Systems




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WCAP-12872
Revision 2

J. M. Farley Units 1 and 2
SG Tube Plugging Criteria for
ODSCC at Tube Support Plates

February 1992

Approved by:



M. J. Wootten, Manager
Steam Generator Technology & Engineering

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1.0 INTRODUCTION

This report provides the technical basis for tube plugging criteria for outside diameter stress corrosion cracking (ODSCC) at tube support plate (TSP) intersections in the Farley Units 1 and 2 steam generators (SGs). The recommended plugging criteria are based upon bobbin coil inspection voltage amplitude which is correlated with tube burst capability and leakage potential. The recommended criteria are demonstrated to meet the guidelines of Regulatory Guide 1.121.

The tube plugging criteria are based upon the conservative assumptions that the tube to TSP crevices are open (negligible crevice deposits or TSP corrosion) and that the TSPs are displaced under accident conditions. The ODSCC generated within the TSPs is thus assumed to be free span degradation under accident conditions and the principal requirement for tube plugging considerations is to provide margins against tube burst per R.G. 1.121. The open crevice assumption leads to maximum leak rates compared to packed crevices and also maximizes the potential for TSP displacements under accident conditions. Tests performed with incipient denting or dented tube intersection show no or very small leakage such that leakage even under SLB conditions would be negligible. It is demonstrated, using Farley Unit-1 as an example, that if the crevices are packed as a consequence of TSP corrosion or if small tube to TSP gaps are present, TSP displacements under accident conditions are minimal such that tube burst would be prevented by the presence of the TSPs. TSP displacement analyses under SLB loads were also performed for the open crevice assumption and the further conservative assumption of zero friction at the tube to TSP intersections and also for the TSP wedge to wrapper interaction. The wedges are installed in the TSP to wrapper gaps to align the TSPs for tubing of the SGs. While the TSP wedges are pressed into the gap during manufacturing, the forces are not known and thus the preload or friction force at the TSP to wrapper interface is not known. It is reasonable to expect that the friction forces at the TSP to wrapper interface would significantly reduce TSP displacements under accident conditions. However, the analytical results based upon the open crevice/zero friction assumptions indicate the potential for TSP displacements under SLB conditions such that prevention of tube rupture cannot be assured for the 51 Series SGs with the applied analytical assumptions. Thus the requirements for tube burst margins assuming free span degradation have been applied to develop the tube plugging criteria.

The plugging criteria were developed from testing of laboratory induced ODSCC specimens, extensive examination of pulled tubes from operating SGs and field experience for leakage due to indications at TSPs. The recommended criteria represent conservative criteria based upon EPRI and industry supported development programs that are continuing toward further refinement of the plugging criteria. Revision 2 of this report significantly increases the pulled tube data base for the voltage versus burst pressure correlation compared to the initial release of this report. The increased data base provides additional support for utilization of burst pressures at the lower 95% confidence level as the basis for the tube plugging limits.

Implementation of the tube plugging criteria is supplemented by 100% inspection requirements at TSP elevations having ODSCC indications, reduced operating leakage requirements, inspection guidelines to provide consistency in the voltage normalization and RPC inspection requirements for the larger indications left in service to

characterize the principal degradation mechanism as ODSCC. In addition, it is required that potential SLB leakage be calculated for tubes left in service to demonstrate that the cumulative leakage is less than allowable limits.

Two tubes were pulled from Farley Unit-2 in November, 1990 to provide direct support for these criteria. Testing on these pulled tubes included leak rate tests, burst pressure tests and destructive examinations to establish crack morphology. In addition, results of prior pulled tube examinations from Farley Units-1 and 2 and other plants have been used to support the tube plugging criteria.

To provide the technical bases for tube plugging due to ODSCC at TSPs, the following activities have been performed as documented in this report:

- o Summary of Regulatory Requirements against which the recommended plugging criteria are evaluated - Section 3
- o Review of Farley-1,2 and other plant pulled tube examinations - Section 4
- o Review of Farley-1,2 eddy current inspection results including historical growth rate data - Section 5
- o Review of field experience from pulled tube data and plant leakage occurrences to define the field data base which is supplemented by laboratory tests to develop the plugging criteria - Section 6
- o Preparation of cracked test specimens for NDE and leak testing in a model boiler or doped steam environment for inducing ODSCC cracks, or by cyclic fatigue to produce cracks in the test samples - Section 7
- o NDE analysis guidelines, measurement uncertainties, and NDE inspection results for the test specimens based upon defined procedure and data analysis guidelines and including sensitivity to: probe manufacturer, open or packed crevices, probe wear, etc. - Section 8
- o Burst and leak testing to relate the NDE parameters to burst pressure and SLB leak rates - Section 9
- o Results of test specimen destructive examinations to assess prototypicality of sample tube crack morphology and to characterize test specimen crack sizes and depths - Section 10
- o SLB evaluations to assess TSP displacements under SLB loads, plant requirements on SLB leakage limits and a description of the SLB leakage model - Section 11
- o Integration of the inspection and burst test results to develop the tube plugging criteria - Section 12
- o Description of eddy current data analysis guidelines for application of the plugging criteria - Appendix A

o Evaluation of the pulled tube data from Plant L - Appendix B

The overall summary and conclusions for this report are described in Section 2.

2.0 SUMMARY AND CONCLUSIONS

The report describes the technical support for tube plugging criteria for ODSCC at TSPs applicable to the Farley Units 1 and 2 SGs. The plugging criteria are based upon use of bobbin coil inspection voltage amplitudes. These eddy current measurements are directly correlated to tube integrity issues including tube burst margins and the potential for tube leakage under postulated accident conditions. Eddy current and leak rate measurements from pulled tubes and laboratory cracked specimens as well as field data have been used to correlate voltage plugging limits to leakage potential and tube burst capability. Regulatory Guide 1.121 for tube integrity guidelines as well as the General Design Criteria are shown to be satisfied by the tube plugging criteria of this report.

This section summarizes the tube plugging criteria under Overall Conclusions (Section 2.1) and the key results of the development program under Summary (Section 2.2).

2.1 Overall Conclusions

The general approach taken to develop the tube plugging criteria for ODSCC at TSPs includes:

- 1) Specifying conservative burst correlations based on free (uncovered) span ODSCC under accident conditions to demonstrate structural integrity.
- 2) Conservatively assuming open crevice conditions to maximize leakage potential.
- 3) Satisfying the R.G. 1.121 structural guidelines for tube burst margins by establishing a conservative structural limit on voltage amplitude that provides three times normal operating pressure differential for tube burst capability.
- 4) Satisfying the FSAR requirements for allowable leakage under accident conditions by demonstrating that the dose rate associated with potential leakage from tubes remaining in service is a small fraction of 10 CFR 100 limits.
- 5) Including considerations for crack growth and NDE uncertainties in both the structural assessment and SLB leakage analysis.
- 6) Specifying a requirement to perform 100% BC inspection for all hot leg TSP intersections and all cold leg intersections down to the lowest cold leg TSP where ODSCC indications have been identified.

The resulting tube plugging criteria for ODSCC at TSPs in Farley Units 1 and 2 can be summarized as follows:

Tube Plugging Criterion

Tubes with bobbin coil indications exceeding 3.6 volts will be plugged or repaired.

SLB Leakage Criterion

Predicted SLB leak rates from ODSCC at TSPs for the tubes left in service must be less than 55 gpm for each SG, including considerations for NDE uncertainties and ODSCC growth rates.

Inspection Requirements

A 100% bobbin coil inspection shall be performed for all hot leg TSP intersections and all cold leg intersections down to the lowest cold leg TSP with ODSCC indications.

All tubes with bobbin coil indications >1.5 volts at TSP intersections shall be inspected using RPC probes. The RPC results shall be evaluated to support ODSCC as the dominant degradation mechanism. Indications at TSPs confirmed to be ODSCC shall be reinspected by RPC during alternate refueling outages for reconfirmation as ODSCC.

Operating Leakage Limits

Plant shutdown will be implemented if normal operating leakage exceeds 150 gpd per SG.

Exclusions from Tube Plugging Criterion

Tubes with RPC indications not attributable to ODSCC or with circumferential indications shall be evaluated for tube plugging based on a 40% eddy current indicated depth limit.

Although the tube plugging guidelines of R.G. 1.121 are used to establish tube plugging limits, the potential for tube burst at SLB conditions is shown to be negligible based on both deterministic crack length considerations and probability estimates. The burst pressure versus crack length correlation utilizing the Belgian burst data (EPRI NP-6864-L) developed under prototypic flow conditions shows that a through wall crack length of 0.84 inch is required for tube burst at SLB pressure differentials. This crack length exceeds the 0.75 inch TSP thickness which bounds the potential crack lengths for ODSCC at TSPs. Consequently, tube burst for ODSCC is essentially precluded by the crack length limit. More over, an alternate assessment was performed by Monte Carlo analyses considering the probabilities associated with a limiting EOC (end of cycle) voltage including allowances for NDE uncertainties and growth. This analysis shows that an indication left in service at the tube plugging limit would have a conditional probability of burst at SLB conditions of $\sim 3 \times 10^{-5}$ per cycle. This value does not include the probability of an SLB event occurring; hence, the actual burst probability (combined probability of SLB and burst) would be further reduced. In addition, it is shown that even the small degree of tube denting in Farley-1 together with EC detected TSP corrosion provide tube to TSP contact pressures that prevent TSP displacement under accident conditions. A partial assessment of Farley-2 TSP corrosion indicates the likelihood that an evaluation, if completed for Farley-2, would also demonstrate no TSP displacement. Thus tube burst is not a significant concern for application of the plugging limits for ODSCC at TSPs. The use of free span burst pressure criteria to establish tube plugging limits thus leads to very conservative plugging limits.

2.2 Summary

A summary of the results of this report is provided below:

Regulatory Requirements

- o Tube integrity acceptance criteria for SG tubes are defined in Regulatory Guide 1.121 and General Design Criteria. For tubes with through wall cracks, these criteria establish guidelines for tube burst margins and operating leakage limits. The tube plugging limits of this report are developed to demonstrate that these guidelines are met.
- o FSAR accident analyses include tube leak rates utilized to show acceptable radiological consequences. A limiting accident condition leak rate of 55 gpm per SG in an SLB event was developed to meet site boundary dosage limits. At each outage, projected potential SLB leak is determined for tubes left in service to demonstrate satisfaction of the 55 gpm leakage limit.

Pulled Tubes from Farley SGs

- o Two tubes pulled from Farley-2 in 1990 and one tube pulled in 1986 (examined in 1991) have been leak and burst tested to support the tube plugging criteria. Prior Farley pulled tubes with significant cracks include one additional tube from Unit 2 and one from Unit 1. These tubes had crack depths exceeding 60% and voltages ranging from 0.3 to 9.9 volts. Three tubes having voltages ranging from 2.8 to 9.9 volts had short through wall indications. None of these tubes were identifiable as leakers in service. Nine additional Farley-2 TSP intersections from 3 tubes were destructively examined and found to have insignificant (<22% depth) degradation.
- o Two of the five Farley tubes with significant indications had bobbin coil amplitudes of 2.8 and 9.9 volts, indicated depths of 82% and 86% and through wall crack lengths of 0.18 and 0.15 inch, respectively. Laboratory tests showed no significant leakage (a few drops indistinguishable from test system leakage) at normal operating conditions. These tubes had very low SLB leak rates of <0.2 l/hr (~1 gpd).
- o Two of the five Farley tubes at 1.44 and 7.2 volts with bobbin coil indicated depths of 68% and 83% had actual crack depths of 78% and 100% (0.02" through wall crack length), respectively. The tube with the 1.44 volt indication was leak tested with no leakage at normal operating and SLB conditions. It can be inferred from the crack morphology that the tube with the 7.2 volt indication would not have measurable leakage even at SLB conditions.
- o Free span burst pressures for the Farley pulled tubes exceeded 5900 psi and thus exceed Reg. Guide 1.121 guidelines for 3 times normal operating pressure differentials (4380 psi) on a temperature and minimum property adjusted basis.
- o A total of 14 TSP intersections from 8 tubes have been examined for TSP degradation. The tube pulls occurred between 1985 and 1990. None of the tube

exam results show significant IGA involvement with ODS/CC being the dominant degradation mechanism. These results show that IGA has not been an active corrosion mechanism at TSPs in the Farley SGs and the six year trend indicates that significant IGA is not expected in future operation.

Pulled Tubes from Other SGs

- o In addition to the 8 tubes pulled from Farley, the overall pulled tube data base includes 21 pulled tubes with 57 tube to TSP intersections that have both NDE and destructive examination results. The bobbin coil indications for these tubes range from 0.1 to 3.8 volts with destructive examination depths up to 98%. One tube with a 1.9 volt indication had a through wall crack, 0.01" long. None of these tubes would be expected to leak even at SLB conditions and all would have free span burst pressures that satisfy Reg. Guide 1.121 acceptance criteria.
- o These pulled tubes support no leakage at normal operating or SLB conditions at voltages up to 3.8 volts, independent of depth, while the Farley tubes indicate no measurable operating leakage up to about 10 volts and very low leakage at SLB conditions above 2.8 volts.
- o Overall, the pulled tube data show multiple, segmented axial cracks with short lengths for the deepest penetrations. Through wall cracks have been identified in 4 tubes but the associated crack lengths are short (<0.18") and have no measurable leakage at normal operating conditions.
- o Pulled tube examination results have been reviewed from 5 plants with more significant IGA involvement than found at the Farley SGs. These results indicate that the degradation develops as IGA + SCC particularly when maximum IGA depths greater than about 25% are found. A large number (>100) of axial cracks around the tube circumference are commonly found in these tubes. The maximum depth of IGA is typically 1/3 to 1/2 of the SCC depth. Patches of cellular IGA/SCC formed by combined axial and circumferential orientation of microcracks are frequently found in pulled tube examinations.
- o Comparisons of corrosion morphology between tubes have been made semi-quantitatively using comparisons of cracking density, extent of IGA associated with the major cracks and extent (depth, width) of IGA involvement.

Operating Plant Experience

- o To date, only 3 tubes in operating SGs have been identified as probable tube leakers attributable to ODS/CC at TSPs. No leakers for ODS/CC at TSPs have been identified in domestic plants. The occurrences were in European plants. These leaking tubes had bobbin coil indicated depths exceeding 75% and voltage amplitudes of 7.7, 13 and 39 volts. Plant leak rates associated with these tubes cannot be quantified as other tubes with PWSCC contributed to the total plant leak rates of 63 and 140 gpd associated with ODS/CC leakers.

- o Bobbin coil inspection results from domestic and European units for tubes with no identifiable leakage have been collected to support selection of plugging limits for no expected operating leakage. These data include indications exceeding 10 volts amplitude and generally support negligible leakage for ODSCC at TSPs.
- o Based on voltage versus actual depth trends from pulled tubes, indications with IGA and IGA/SCC show comparable or higher voltage levels than obtained for SCC with minor IGA. These data support adequate detectability for IGA and IGA/SCC degradation using bobbin coil inspections.
- o Inspection results from French units provide growth trends at BOC (beginning of cycle) amplitudes higher than those obtained from domestic units. The French data indicate that percent voltage growth is essentially independent of BOC voltage amplitude. Farley data, which are at lower BOC amplitudes, show a trend toward decreasing percent voltage growth with increasing amplitude. For conservatism, percent voltage growth is assumed to be independent of amplitude to develop the tube plugging limits.

Farley Operating Experience for ODSCC at TSPs

- o Results from prior inspections at Farley Units 1 and 2 were evaluated to develop growth rates for both voltage amplitudes and indicated depths. This assessment indicates that the ODSCC initiated prior to 1986. Improvements to secondary chemistry and sludge management have led to progressively decreasing growth rates since 1987.
- o Average growth rates over the last operating cycle in Farley 1 were 0.23 volt and 6% depth. For Farley 2, growth rates were 0.1 volt and <3% depth. Conservative voltage growth rates over the last operating cycles were 37% for Farley-1 and 29% for Farley-2.
- o The eddy current review shows low levels of tube denting in Unit 1 and negligible tube denting in Unit 2. Denting progression has been negligible in both units for at least the last three operating cycles. TSP corrosion, indicating hard magnetite in the crevices, was found by eddy current examination at most tube to TSP intersections in Unit-1 and to a lesser extent in Unit-2.

TSP Displacement Under SLB Loads

- o The potential for TSP displacement under SLB loading conditions has been evaluated for open crevices, for small gaps and for corroded TSP conditions of incipient denting which leads to contact forces between the tube and the hard magnetite in the crevices. These evaluations were performed to assess the potential for uncovering of the ODSCC under SLB conditions.
- o With the corroded TSP conditions of the Farley Unit-1 SGs, the maximum SLB loads on the TSPs are less than the forces resulting from tube to TSP contact pressures. To support this conclusion, pull tests were performed to determine the force required to pull the tube from incipient denting and dented crevice conditions.

These pull forces per TSP intersection ranged from [] pounds for incipient denting up to [] pounds for less than 1 mil dents. These results show that the ODSCC at TSPs in Farley Unit-1 SGs will continue to be enveloped by the TSPs even under accident conditions and thus preclude the potential for tube rupture. A comparable evaluation was not performed for Farley Unit-2 SGs as the plugging criteria are based upon free span ODSCC under accident conditions.

- o Analyses for TSP displacements with crevice gaps in an SLB event were performed using finite element, dynamic time history analysis methods. Conservative analysis assumptions, such as no friction which ignores the TSP to wrapper contact forces, lead to overestimates of the TSP displacements. Given these assumptions, analyses for open, as manufactured crevices, indicate potential displacements yielding plastic deformation of some TSPs. For these results, it could not be assured that the TSPs would envelope the ODSCC at the times of increasing primary to secondary pressure differentials in an SLB event. Rather than pursuing analytical refinements of these models, the analyses were performed to reflect the crevice conditions of TSP corrosion as found by eddy current inspection. As noted above, the incipient denting and dented conditions at TSP intersections prevent TSP displacement under SLB conditions. TSP displacement analyses for varying crevice gaps show that even if the TSP corrosion resulted in up to [] mil gaps at the most limiting plate, the TSP displacements would not uncover the ODSCC.

Preparation of Cracked Specimens

- o Samples cracked due to ODSCC in model boilers with simulated TSP intersections have been found to produce crack morphologies and leak rates typical of field experience. This method of sample preparation is used for development of the tube plugging limits.
- o Samples prepared in doped steam were found to yield relatively open cracks, with less prototypic crack morphology and voltage amplitudes as well as non-prototypically high leak rates due to the high hoop stresses applied to crack these specimens within reasonable test periods. These samples have only been applied for tests with dented TSP intersections to demonstrate the influence of dented crevices on leak rates.
- o Fatigue induced crack specimens have also been used to evaluate the influence of dented crevices on leak rates. Fatigue cracks were used based upon the capability to closely control through wall crack lengths and the reasonably predictable and relatively high leak rates associated with fatigue cracks.

Non-Destructive Examinations (NDE)

- o Bobbin coil measurements of laboratory prepared uniform IGA specimens show voltage amplitudes exceeding 1-2 volts for ~30% IGA depth. These results support the field data trends indicating IGA and IGA/SCC detectability at comparable voltage levels to that found for SCC.

- o The NDE program applied to laboratory specimens included sensitivity comparisons for probes manufactured by two vendors. The probe vendor has been found to have no impact on the NDE results based on using 400 kHz for voltage calibration and a 400/100 kHz mix for analysis. However, each of the probes can have different frequency response characteristics. This effect can be minimized by calibrating each of the frequencies individually or calibrating the planned mix channel. The latter has been recommended for implementation in the Farley SG inspections. Probe centering uncertainty has been found to be <5% for "new", unworn probes based on results of EC tests performed in a horizontal position (orientation) of a specimen with EDM notches (notches at the bottom or top).
- o The bobbin coil voltage response for magnetite packed crevices is essentially the same as found for open crevices. The results show a 5% increase in response with the magnetite present and a scatter of about 10%. Thus the presence of magnetite does not significantly influence the bobbin coil voltage measurements.
- o An example typical of field experience for which the environment (residual TSP, small dent responses) can mask or distort the indication response was found in one test specimen. When the amplitude response grew from 0.3 to 0.7 volts after leak testing and handling, the indication could be readily detected. These small responses, which are masked by environmental factors in operating SGs (as shown by pulled tube results) do not impact the serviceability of the SG, as the indications are small compared to the plugging limits of this report.
- o Voltage calibrations for different ASME standards were compared against the laboratory standard used in this program. Variations up to 18% were identified. Pending incorporation of a voltage verification requirement in ASME standard certifications, an ASME standard calibrated against the laboratory standard will be utilized in Farley SG inspections for consistent voltage normalization.
- o Bobbin coil probe wear sensitivity tests were performed by varying the diameter of the probe centering devices to determine changes in voltage and depth responses against a 4 hole standard. The test results indicate that probe wear typical of field inspections leads to voltage variations of []^a between the 4 holes staggered around the tube circumference. To limit uncertainties associated with probe wear, a four staggered hole standard will be implemented in Farley SG inspections. Pending additional field experience with the probe wear standard, the NDE uncertainty for probe wear has been increased to 15% for the data acquisition guidelines. If voltage amplitudes between a new probe and later measurements for any of the 4 holes differ by more than []^a, the probe will be replaced.
- o Eddy current analyst variability for bobbin coil voltage measurements was evaluated by comparing Farley-1 and Farley-2 measurements between two analysts. The voltage difference between analysts was found to be about 10% at 90% cumulative probability. When this uncertainty is combined with the probe wear uncertainty, an NDE uncertainty of 16% at 90% cumulative probability is obtained for application to establishing the tube plugging voltage limit.

- o The variables affecting the voltage/burst correlation can be split into NDE uncertainties and burst correlation uncertainties. The NDE uncertainty represents the repeatability of a voltage measurement and is dominated by probe centering variations as a result of probe wear. Minimizing the uncertainty on repeatability of voltage measurements reduces the spread or uncertainty in the burst correlation. The remaining voltage measurement uncertainties contribute to the burst pressure correlation uncertainty and influence plugging limits through use of the lower 95% confidence level on the voltage/burst correlation to establish the voltage plugging limits.

Leak Rate and Tube Burst Testing

- o Leak rates at normal operating and SLB conditions have been measured for 33 model boiler specimens with voltage levels as high as 137 volts. Overall, the model boiler data indicate that []^o volt indications are required for significant (>10 gpd) operating leakage.
- o Burst pressure tests were performed for 31 model boiler specimens and 29 intersections from 13 pulled tubes. The model boiler and pulled tube burst pressure measurements were combined to develop a correlation of burst pressure versus bobbin coil voltage. This correlation was reduced for operating temperatures and minimum material properties to determine the voltage amplitude that satisfies the R.G. 1.121 structural guidelines for burst capability of 3 times normal operating pressure differentials. The results evaluated at the lower 95% prediction interval define a []^a volt amplitude for the structural limit on tube burst margins. The burst correlation indicates that a []^o volt ODSCC crack would meet an SLB burst pressure requirement of 2650 psi at the lower 95% confidence level.
- o Available burst data for laboratory uniform IGA specimens and pulled tubes with IGA/SCC are enveloped by the voltage/burst correlation dominated by ODSCC data.
- o The model boiler and pulled tube leak rate measurements have been applied to define a correlation of SLB leak rate to bobbin coil voltage. This correlation including defined uncertainty levels is used to calculate potential SLB leakage for tubes returned to service following an outage.
- o Leak rates for the 3 model boiler tubes tested with magnetite packed crevices that showed some leakage had increased leak rates after the magnetite was removed from the crevice.
- o Leak rates were also measured for 11 tubes with incipient denting and dented conditions, average dent sizes less than 1 mil and including through wall fatigue cracks up to 0.7 inch long. Only []^o of these dented tubes leaked at normal operating or SLB conditions even though open crevice leakage for the 0.7 inch fatigue cracks would exceed 1000 gpd. The []^o at normal operating and SLB conditions, respectively, had a 0.5 inch long fatigue crack and an average dent size of <0.2 mils.

- o Pull tests were performed to determine the force required to pull tubes from TSPs with incipient denting and dented crevices. The forces required to initiate tube displacement exceeded []^g pounds for tubes with minor denting (> 0.3 mil, 6 volt dent signals). For the tubes with less denting, pull forces of []^g pounds were required.

Specimen Destructive Examinations

- o Destructive examinations of the model boiler specimen show crack morphologies typical of the pulled tube experience. Destructive examinations were performed on model boiler specimens following burst testing to characterize the cracks associated with typical voltage levels and leak rates.
- o Destructive examination of a laboratory induced dent specimen has shown corrosion product layers that are relatively dense and leakage paths that are highly tortuous which is consistent with the negligible leakage found for dented tube conditions.

SLB Tube Burst Probability

- o The Monte Carlo model used to calculate distributions of EOC voltages for the SLB leakage analysis is also used to calculate the probability of tube burst at SLB conditions. Applying the Monte Carlo model to the last Farley-2 inspection results given in Figure 5-5 yields a SLB tube burst probability of 3×10^{-5} based on a 3.6 volt tube repair limit.

Tube Plugging Criteria

- o The pulled tube and model boiler leak rate and burst data together with field leakage experience and field inspection results have been used to relate bobbin coil voltage to tube integrity to define tube plugging limits.
- o The burst pressure versus voltage correlation defines a voltage of []^a volts for the structural limit to meet R.G. 1.121 burst margins. The voltage structural limit is reduced by conservative allowances of 20% for NDE uncertainties and 50% for crack growth to obtain a tube plugging limit of 3.6 volts.
- o An SLB leakage model is defined for demonstrating that projected SLB leakage from tubes left in service is less than the allowable 55 gpm per steam generator. The SLB leakage model is a probabilistic model that combines an inspection determined distribution for voltage indications, voltage growth rate distributions, eddy current uncertainties and a leak rate versus voltage formulation to obtain the projected cumulative EOC SLB leak rate for all indications left in service. If the plugging criteria and SLB leakage model are applied to the last Farley-2 inspection results, the projected SLB leakage for the end of the next cycle would be a maximum of 0.4 gpm per steam generator (at the 90% cumulative probability level).
- o RPC inspection for indications above 1.5 volts is required to establish that the more significant indications are ODSCC. The 1.5 volt threshold for RPC inspection

provides a threshold value below which non-ODSCC indications would be acceptable for continued operation and SLB leakage is expected to be negligible.

- o An operating leakage limit of 150 gpd has been established to provide for detection of a rogue crack which could leak at much higher SLB leak rates than used in the criteria limits. The 150 gpd limit permits detection of a through wall crack of about []^a inch for nominal leak rates and about []^a inch for lower 95% confidence level leak rates.
- o To enhance consistency of the field EC inspection guidelines with the data base used to develop the plugging limits, the Farley inspections will include: use of an ASME standard calibrated against the laboratory standard; use of a staggered 4-hole standard to assess probe wear effects and normalization of voltages to 6.4 volts for 400/100 kHz (support plate -- Mix.1) on the four 100% ASME holes.

3.0 REGULATORY REQUIREMENTS

3.1 General Design Criteria

The two parameter eddy current criteria, which establishes a basis for removing tubes from service experiencing outside diameter stress corrosion cracking (SCC) occurring at tube support plate elevation intersections in the Farley (Units 1 and 2) steam generators, have been developed to ensure compliance with the applicable General Design Criteria of Appendix A of Part 50 of Title 10 of the Code of Federal Regulations (10CFR50). The GDCs considered are: 2, 4, 14, 15, 31, and 32 and are summarized below.

GDC 2. Design Basis for Protection Against Natural Phenomena, requires that structures, systems and components important to safety be designed to withstand the effects of earthquakes in combinations with the effects of design basis loadings without loss of safety function.

GDC 4. Environmental and Missile Design Bases, requires that structures, systems, and components important to safety are designed to accommodate the effects of and to be compatible with the environmental conditions associated with normal operation, maintenance, testing, and postulated accident condition loadings, including loss-of-coolant accidents.

GDC 14. Reactor Coolant Pressure Boundary, requires the reactor coolant pressure boundary to be designed, fabricated, erected, and tested so as to have an extremely low probability of abnormal leakage, of rapidly propagating to failure, and of gross rupture.

GDC 15. Reactor Coolant System Design, requires the reactor coolant system and associated auxiliary, control, and protection systems to be designed with sufficient margin to assure the design margins of the reactor coolant pressure boundary are not exceeded during any condition of normal operation, including anticipated operating occurrences.

GDC 31. Fracture Prevention of the Reactor Coolant Pressure Boundary, requires that the reactor coolant pressure boundary shall be designed with sufficient margin to ensure that when stressed under operating, maintenance, testing, and postulated accident condition loadings, the boundary behaves in a nonbrittle manner and the probability of a rapidly propagating fracture is minimized.

GDC 32. Inspection of the Reactor Coolant Pressure Boundary, requires that components that are part of the reactor coolant pressure boundary be designed to permit periodic inspection and testing of critical areas to assess their structural and leaktight integrity.

General Design Criteria 2 and 4 are considered in Section 3.3 below where the potential for steam generator tube collapse during the combined effects of LOCA + SSE loadings are addressed for the Farley steam generators.

3.2 Regulatory Guide 1.121

Background

R.G. 1.121, "Bases for Plugging Degraded PWR Steam Generator Tubes" issued for comment in August of 1976, describes a method acceptable to the NRC staff for meeting GDCs 14, 15, 31, and 32 by reducing the probability and consequences of steam generator tube rupture through determining the limiting safe conditions of degradation of steam generator tubing, beyond which tubes with unacceptable cracking, as established by inservice inspection, should be removed from service by plugging. The recommended plugging criteria for the tube support plate elevation OD SCC occurring in the Farley steam generators may result in tubes with both partial through-wall and through-wall (non-leaking) cracks being returned to service. In the limiting case, the presence of a through-wall crack alone is not reason enough to remove a tube from service. The regulatory basis for leaving through-wall cracks in service in the Farley (Units 1 and 2) steam generators is provided below.

Steam generator "tube failure" is defined by the NRC within RG 1.83 as the full penetration of the primary pressure boundary with subsequent leakage. Consistent with this definition, upon the implementation of the tube plugging criteria of this report, known leaking tubes will be removed from service from the Farley steam generators. Steam generator tube bundle leak tightness will be re-established by conducting 100% inspection of the S/G tubes. The tube plugging criteria of this report are established such that operational leakage is not anticipated.

The NRC defines steam generator tube rupture within RG 1.121 as any perforation of the tube pressure boundary accompanied by a flow of fluid either from the primary to secondary side of the steam generator or vice versa, depending on the differential pressure condition. As stated within the regulatory guide, the rupture of a number of single tube wall barriers between primary and secondary fluid has safety consequences only if the resulting fluid flow exceeds an acceptable amount and rate.

Consistent with the philosophy of the NRC's definition of tube rupture, during testimony by the NRC staff (on March 24, 1976) to provide information to the ASLB on the plans for measures to reasonably assure steam generator tube integrity under operating conditions including off-nominal and accident condition loadings at the Farley plants, the following definition of loss of steam generator tube integrity was provided. Loss of steam generator tube integrity means loss of "leakage integrity". Loss of "leakage integrity" is defined as the degree of degradation by a through-wall crack penetration of a tube wall membrane that can adversely affect the margin of safety leading to "tube failure", burst, or collapse during normal operation and in the event of postulated accidents. Acceptable service in terms of tube integrity limits the allowable primary to secondary leakage rate during normal operating conditions, and assures that the consequences of postulated accidents would be well within the guidelines of 10CFR100. In order to assure steam generator tube integrity is not reduced below a level acceptable for adequate margins of safety, the NRC staff position focused on specific criteria for limiting conditions of operation. These include:

1. Secondary Water Monitoring
2. Primary-to-Secondary tube leakage

3. Steam Generator Tube Surveillance
4. Steam Generator Tube Plugging Criteria

Tubes with through-wall cracks will maintain "leakage integrity" and are acceptable for continued operation if the extent of cracking can be shown to meet the following RG 1.121 criteria:

1. Tubes are demonstrated to maintain a factor of safety of 3 against failure for bursting under normal operating pressure differential.
2. Tubes are demonstrated to maintain adequate margin against tube failure under postulated accident condition loadings (combined with the effects of SSE loadings) and the loadings required to initiate propagation of the largest longitudinal crack resulting in tube rupture. All hydrodynamic and flow induced forces are to be considered in the analysis to determine acceptable tube wall penetration of cracking.
3. A primary-to-secondary leakage limit under normal operating conditions is set in the plant technical specifications which is less than the leakage rate determined theoretically or experimentally from the largest single permissible longitudinal crack. This action would ensure orderly plant shutdown and allow sufficient time for remedial action(s) if the crack size increases beyond the permissible limit during service.

The two parameter support plate elevation plugging criteria discussed in this report are shown to meet all of the necessary acceptance criteria.

3.3 Steam Generator Tube Deformation Discussion

In addressing the combined effects of the LOCA and SSE loadings (as required by GDC 2) on the steam generator component, [

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This issue has been addressed for the Farley (Units 1 and 2) steam generators through the application of leak-before-break principles to the primary loop piping. Alabama Power Company has performed a detailed leak before break analysis for Farley Units 1 and 2. Based on the results, it is concluded that the leak-before-break methodology (as permitted by GDC 4) is applicable to the Farley reactor coolant system primary loops and, thus, the probability of breaks in the primary loop piping is sufficiently low that they need not be considered in the structural design basis of the plant. Excluding breaks

in the RCS primary loops, the LOCA loads from the large branch lines breaks were also assessed and found to be of insufficient magnitude to result in steam generator tube collapse. Utilizing results from recent tests and analysis programs (discussed more fully in section 11.2), it has also been shown that there will not be any tubes that undergo permanent deformation where the change in diameter exceeds 0.025 inch. Although specific leakage data is not available, it is judged that deformations of this magnitude will not lead to significant tube leakage. On this basis there will not be any tubes that need to be excluded from the APC for reasons of deformation resulting from combined LOCA + SSE loading.

4.0 PULLED TUBE EXAMINATIONS

4.1 Introduction

The following provides summary information regarding OD originated corrosion at support plate crevice regions of Alloy 600 tubing pulled from steam generators at Farley and other plants. The data is presented in support of the development of alternate plugging criteria. First, pulled tube data from the Farley power plants are reviewed, followed by data from other plants.

The type of intergranular corrosion with regard to crack morphology and density (number, length, depth) of cracks can influence the structural integrity of the tube and the eddy current response of the indications. For support of tube plugging criteria, the emphasis for destructive examination is placed upon characterizing the morphology (SCC, IGA involvement), the number of cracks, and characterization of the largest crack networks with regard to length, depth and remaining ligaments between cracks. These crack details support interpretation of structural parameters such as leak rates and burst pressure, and of eddy current parameters such as measured voltage, depth and crack lengths with the goal of improving structural and eddy current evaluations of tube degradation. In selective cases, such as the 1990 Farley-2 pulled tubes the pulled tube evaluations included leak rate and burst pressure measurements for further support of the integrity and plugging limit evaluations.

4.1.1 Definitions of OD Corrosion Degradation Observed at Support Plate Crevice Locations

Before the support plate region corrosion degradation can be adequately described, some key corrosion morphology terms need to be defined. Intergranular corrosion morphology can vary from IGA to SCC to combinations of the two. IGA (Intergranular Attack) is defined as a three dimensional corrosion degradation which occurs along grain boundaries. The radial dimension has a relatively constant value when viewed from different axial and circumferential coordinates. IGA can occur in isolated patches or as extensive networks which may encompass the entire circumferential dimension within the concentrating crevice. Figure 4-1 provides a sketch of these IGA morphologies. As defined by Westinghouse, the width of the corrosion should be equal to or greater than the depth of the corrosion for the degradation to be classified as IGA. The growth of IGA is relatively stress independent. IGSCC (Intergranular Stress Corrosion Cracking) is defined as a two-dimensional corrosion degradation of grain boundaries that is strongly stress dependent. IGSCC is typically observed in the axial-radial plane in steam generator tubing, but can occur in the circumferential-radial plane or in combinations of the two planes. The IGSCC can occur as a single two dimensional crack, or it can occur with branches coming off the main plane. Figure 4-2 provides a sketch of these IGSCC morphologies. Both of the IGSCC variations can occur with minor to major components of IGA. The IGA component can occur simply as an IGA base with SCC protruding through the IGA base or the SCC plane may have a semi-three dimensional characteristic. Figure 4-3 provides a sketch of some of the morphologies possible with combinations of IGSCC and IGA. Based on laboratory corrosion tests, it is believed that the later, SCC protrusions with significant IGA aspects, grow at rates similar to that of SCC, as opposed to the slower rates usually associated with IGA. When IGSCC and IGA are both present, the IGSCC will penetrate throughwall first and provide the leak path.

To provide a semi-quantitative way of characterizing the amount of IGA associated with a given crack, the depth of the crack is divided by the width of the IGA as measured at the mid-depth of the crack, creating a ratio D/W. Three arbitrary D/W categories were created: minor (D/W greater than 20) (all or most PWSCC would be included in this category if it were being considered in this analysis); moderate (D/W between 3 and 20); and significant (D/W less than 3) where for a given crack with a D/W of 1 or less, the morphology is that of patch IGA.

The density of cracking can vary from one single large crack (usually a macrocrack composed of many microcracks which nucleated along a line that has only a very small width and which then grew together by intergranular corrosion) to hundreds of very short microcracks that may have partially linked together to form dozens of larger macrocracks. Note that in cases where a very high density of cracks are present (usually axial cracks) and where these cracks also have significant IGA components, then the outer surface of the tube (crack origin surface) can form regions with effective three dimensional IGA. Axial deformations of the tube may then cause circumferential openings on the outer surface of the tube within the three dimensional network of IGA; these networks are sometimes mistakenly referred to as circumferential cracks. The axial cracks, however, will still be the deeper and the dominant degradation, as compared to IGA.

Recognizing all of the gradations between IGA and IGSCC can be difficult. In addition to observing patch IGA, cellular IGA/SCC has recently been recognized. In cellular IGA/SCC, the cell walls have IGSCC to IGA characteristics while the interiors of the cells have nondegraded metal. The cells are usually equiaxial and are typically 25 to 50 mils in diameter. The cell walls (with intergranular corrosion) are typically 3 to 10 grains (1 to 4 mils) thick. The thickness and shape of the cell walls do not change substantially with radial depth. Visual examinations or limited combinations of axial and transverse metallography will not readily distinguish cellular IGA/SCC from extensive and closely spaced axial IGSCC with circumferential ledges linking axial microcracks, especially if moderate to significant IGA components exist in association with the cracking. Radial metallography is required to definitively recognize cellular IGA/SCC. Cellular IGA/SCC can cover relatively large regions of a support plate crevice (a large fraction of a tube quadrant within the crevice region). Figure 4-4 shows an example of cellular IGA/SCC from Plant L.

A given support plate region can have intergranular corrosion that ranges from IGA through individual IGSCC without IGA components.

As described in Sections 4.2 and 4.3, the Farley SG tubes examined show dominantly SCC crack networks with IGA components that vary from minor to, in one case, significant. The larger eddy current indications for the 1990 pulled tubes are principally related to the single, large macrocrack found for these tubes.

4.1.2 Through Wall SCC at Support Plate Crevice Locations

The following presents OD intergranular corrosion data at support plate intersection locations on steam generator tubes which have experienced through wall corrosion. The latest results for the 1990 examination of through wall cracks at the support plate locations at Farley Unit 2, however, are presented in Section 4.2.

Farley Unit 2, Steam Generator C, Hot Leg Tube R31-C46, Support Plate 1

OD origin, axially oriented, intergranular stress corrosion cracks were observed confined entirely within the first support crevice region on hot leg tube R31-C46, removed in 1986 from Steam Generator C of Farley Unit 2. Destructive examination was confined to half of the tube circumference (the half with a single axial NDE indication). The main crack, composed of at least four microcracks which grew together, was 0.52 inch long and was through wall for approximately 0.02 inch. The morphology of the individual microcracks was branched SCC with moderate IGA components to the SCC. Figure 4-5 shows a sketch of SEM fractographic results of the main crack (only the mid to upper portion of the crack was examined) and a sketch of the overall crack distribution observed within the support plate crevice region. Figure 4-6 shows a sketch of the crack distribution (a composite of two transverse metallographic sections) and depth as viewed by metallography as well as a micrograph of a crack showing the crack morphology. In addition to the main macrocrack, (which included a 46% deep axial crack next to the main crack), two smaller axial cracks were observed at other circumferential positions on the half-circumference section examined.

Field eddy current probe inspection (in April 1986, just prior to the tube pull) of the first support plate crevice region produced a 6.2 volt, 81% deep indication in the 400/100 kHz differential mix data. Voltage renormalization to the calibration standards of this report yields 7.2 volts for this indication.

Plant B-1, Steam Generator C, Hot Leg Tube R4-C61, Support Plate 5

OD origin, axially orientated, intergranular stress corrosion cracks were observed confined entirely within the fifth support plate crevice region on the hot leg side of tube R4-C61 from Steam Generator C of Plant B-1. Six axial macrocracks were observed around the circumference. The largest of these was examined by SEM fractography without any metallography. The macrocrack was 0.4 inch long and through wall for 0.01 inch. However, the crack was nearly (effectively) through wall for 0.1 inch. The macrocrack was composed of seven individual microcracks that had mostly grown together by intergranular corrosion (the separating ledges had intergranular features that ranged from 40 to 90% of the length of the ledges). Since no metallography was performed on the axial cracks, it is not possible to definitively describe the axial crack morphology at this location. At the eighth support plate region of the same tube, metallography showed that the morphology was that of SCC with a crack depth to IGA width ratio (D/W) of 15. Figure 4-7 summarizes the crack distribution and morphology data for the fifth support plate crevice region.

In addition to the OD origin axial macrocracks observed at the fifth support plate region, five local areas had circumferential intergranular corrosion. The maximum penetration observed for the circumferential cracking was 46% through wall. The morphology of the circumferential cracking was more that of IGA patches than of SCC. Figure 4-8 provides micrographs of relevant cracks showing the morphology of axial and circumferential cracks. As stated above, the axial cracks had a morphology of IGSCC with a moderate D/W ratio of 15 while the circumferential cracking had a morphology more like that of IGA, with a D/W ratio of 1.

Field eddy current bobbin probe inspection (in June 1989, just prior to the tube pull) of the fifth support plate crevice region produced a 1.9 volt, 74% deep indication in the 550/100 kHz differential mix.

4.2 Farley Unit 2 1990 Tube Pull Results at TSP Locations

Hot leg tubes R4-C73 and R21-C22 were pulled from Steam Generator B and hot leg tube R38-C46 was pulled from Steam Generator C. The sections pulled included the first support plate region from each tube. Laboratory NDE, leak and burst testing, and destructive examinations were performed. The following summarizes the data obtained at the first support plate region of each tube.

NDE Testing

Laboratory eddy current testing was performed using 0.720 inch diameter bobbin and RPC probes. RPC results showed a main axial indication within the support plate crevice region of tube R4-C73. The length of the signal was 0.44 inch and the depth was estimated as 77 to 82% deep based on an ASME drilled hole standard. In addition to the main signal, a less intense RPC signal was observed parallel to the main axial signal approximately 20 to 30° away. Tube R21-C22 produced a single axial indication within its first support plate crevice region. The 0.5 inch long RPC signal was estimated to be 76 to 81% deep. Tube R38-C46 had a 90% deep RPC signal that was 0.4 inches long.

Note that this tube was elongated during the tube pull. As a consequence of the reduced OD dimension, a 0.69 inch diameter RPC probe was used.

Laboratory bobbin probe examination of tubes R4-C73 and R21-C22 was performed using two 0.720 inch diameter bobbin probes. One was a brand new Echoram probe with very stiff spacers (it was difficult to insert the probe into the tube). The other was a slightly used (in terms of length of tubing previously examined) SFRM Zetec probe in which the spacers were less stiff (probe insertion into the tubes was easy). The probes were pulled mechanically through the tubes at speeds similar to those used in the field; however, unlike the field situation, the tubes were examined with the tubes positioned horizontally. In addition, multiple passes were made with each probe with the specimen being rotated between each pass to vary the position of the crack indication. Table 4.1 presents the results. An indication was observed within both support plate crevice locations. Depth estimates were similar for both of the support plate crevice regions and for both of the probes. A range of 86 to 91% deep covered all depth estimates. The voltage varied noticeably depending on the probe used and the orientation of the specimen, with the stiffer Echoram probe producing the smaller voltage variation. For tube R4-C73, the Echoram probe voltage variation ranged from 3.6 to 4.3 volts. For tube R21-C22, the Echoram probe voltage variation ranged from 9.6 to 11.6 volts.

While tube R38-C46 was reduced in diameter during the tube pull, the Zetec 0.72 inch diameter bobbin probe could still be used for the laboratory examination. However, it passed through the deformed tube with difficulty. Consequently, the estimates of depth and voltage are not judged to be reliable. The field bobbin test produced a 1.4 volt signal with an indicated depth of 68%. This is considered more reliable than the laboratory

result of 4.8 volts and 90% depth. Since the tube pull opened crack networks which were readily visible, the larger laboratory voltage is not surprising.

Double wall radiography was performed using four rotations. No crack indications were observed on tube R21-C22 but a single axial crack-like indication was observed within the first support plate region of tube R4-C73. The faint indication, located near 0°, was 0.3 inch long and was located entirely within the support plate crevice region. These results imply that any deep corrosion cracks present on the two pulled tubes from S/G-B are very tight. OD measurements of the pulled tubes from S/G-B showed that the average diameter of both tubes was 0.8755 inch with a 0.001 inch ovality. No noticeable tube deformation occurred during tube pulling to open any corrosion cracks. Tube R38-C46 from S/G-C was noticeably reduced in diameter. The tube had an average OD of 0.830 inch within the TSP crevice region. Radiography of tube R38-C46 showed three main locations of crack networks within the TSP crevice region. The crack networks contain both axial and circumferential indications. The circumferential cracks were wide, as would be expected for a tube which experienced elongation during the tube pull.

Ultrasonic testing was also performed on the support plate specimens. A 0.4 inch long axial indication was observed near 330° on tube R21-C22 within the support plate crevice region. On tube R4-C73 a 0.25 inch axial indication was observed near 30° within the support plate crevice region. A shorter and fainter axial indication was also observed near 70°. On tube R38-C46 three patches of indications were noted, one near 20°, one near 230°, and one near 340°. All were located within the TSP crevice region.

Leak and Burst Testing

Following NDE characterization, the three first support plate tube sections were leak and burst tested. The leak tests were performed in two parts. The specimens were first tested under simulated normal operating conditions. At a test temperature of 610° F, the primary side of the specimen was connected by insulated pressure tubing to an autoclave maintained at a pressure of 2250 psi by bottled nitrogen gas. The specimen was located in a second autoclave maintained at 616° F and a pressure of 750 psi, resulting in a differential pressure of 1500 psi. The 750 psi pressure in the second autoclave was maintained by a back pressure regulator (BPR) connected to the autoclave by pressure tubing. Any water vapor passing through the BPR was then passed through cooling coils immersed in ice water. The amount of condensed water was measured as a function of time. Following the initial leak testing, a simulated steam line break (SLB) leak test was performed using the same system. For the SLB test, the primary pressure was increased to 3050 psi and the secondary side pressure was decreased to 400 psi for a pressure differential of 2650 psi.

Leak test data are presented in Table 4.2. Results from the SLB test are considered reliable. The measured SLB leak rates were [redacted] and no leakage for tube R38-C46. These values are considerably below the maximum leak rate capability of the system, estimated to be approximately 2 l/hr based on a test with the test specimen removed. Results for the normal operating conditions are considered to be less accurate. The observed leak rate for tube R4-C73 was [redacted] for tube R21-C22. No leakage was observed for tube R38-C46. These

rates include any overflows from the BPR. Leakage through the BPR was encountered, especially with the testing of tube R21-C22. The BPR may have contributed to the entire amount of leakage observed* for the normal operating condition test. While this amount of overflow from the BPR is small in comparison to the SLB test leak rates, it is very large in comparison to potential leak rates from the normal operating conditions test. Consequently, the normal operating condition leak rate at the lower end of potential leak rates for these specimens should be considered zero. The upper value presented, at least for tube R21-C22, probably includes significant contributions from the BPR.

Room temperature burst tests were performed on the two specimens following leak testing. The specimens were pressurized with water at a pressurization rate of approximately 1000 psi/sec. Tygon tubing internal bladders were inserted into the specimens to permit testing with their through wall corrosion cracks. No support plate restraints were placed on the specimens. Consequently, the burst pressures measured may be somewhat lower than would be observed with the presence of a support plate.

The first support plate region of tube R4-C73 burst at []^g psi, the first support plate region of tube R21-C22 burst at []^g psi, and the first support plate region of tube R38-C46 burst at []^g psi. Table 4.2 presents this data as well as other burst data characterizing the specimens.

Characterization of the Corrosion Cracks

Figure 4-9 shows a sketch of the SEM fractographic observations on the burst fracture face of the first support plate region of tube R4-C73. Within the center of the burst opening, a 0.42 inch long OD origin macrocrack was observed. The macrocrack was located at an orientation** of 20° and was entirely confined to within the support plate crevice region. It had only intergranular corrosion features. The macrocrack was composed of four microcracks, all of which had joined together by intergranular corrosion. The crack was through wall for 0.18 inch. A parallel axial macrocrack was observed near 40°. It was 0.46 inch long and up to 69% through wall. In addition, numerous short axial cracks were observed at various locations within the crevice region. The depth of these short cracks ranged from minor penetrations to 34% deep cracks. Figure 4-9 also provides a sketch estimating the crack distribution within the support plate crevice region as well as a description of the crack morphology of the main

* Prior to initiation of the leak tests, the specimen fittings were tested to verify that they were leak tight. The fittings were tested by pressurizing the specimens with 500 to 600 psi air and holding the specimens and their fittings under water. No fitting leaks were observed. The R4-C73 specimen was observed to leak air bubbles at the location of the support plate at a pressure of 500 psi air. The R21-C22 specimen did not leak air bubbles at a pressure of 600 psi. Consequently, it is believed reasonable that the normal operating leak rate for tube R21-C22 should be lower than that for tube R4-C73. This would also be consistent with the SLB leak results.

** In the orientation system used, 0° faces the steam generator divider plate and 90° is clockwise of 0° when looking in the primary flow (up) direction.

macrocrack. Figure 4-10 provides a sketch of the crack distribution and depth within the center of the support plate crevice region as determined by metallography. The main crack morphology was that of SCC with moderate IGA components ($D/W = 4$). The width of IGA surrounding the SCC is estimated to be approximately 0.012 inch, except at the OD surface where the width was larger. Other cracks tended to have less IGA components. Figure 4-11 provides micrographs showing both the main crack morphology as well as the crack morphology of one of the lesser cracks. The morphology of the latter crack, which has been opened wide by tube deformation, is more that of IGSCC ($D/W = 12$).

Figure 4-12 shows a sketch of the SEM fractographic observations on the burst fracture face of the first support plate region of tube R21-C22. Within the center of the burst opening, a 0.50 inch long OD origin macrocrack was observed. The macrocrack was located at an orientation of 330° and was entirely confined to within the support plate crevice region. The corrosion crack had only intergranular features. The macrocrack was composed of four microcracks. Three of the microcracks were joined by intergranular corrosion, while the top most microcrack was still separated from the others by metal. The macrocrack was through wall for approximately 0.15 inch. Figure 4-12 also provides a description of the crack morphology. The crack morphology was that of SCC with significant IGA components. The width of IGA surrounding the SCC is estimated to be approximately 0.030 inch ($D/W = 1.7$). One additional crack was later observed on the specimen by metallographic examination. Figure 4-13 provides a sketch of the crack distribution and depth observed by metallography. Figure 4-14 provides micrographs of the cracks. As can be observed, the secondary crack morphology had lesser IGA components ($D/W = 19$ to 37).

Figure 4-15 shows a sketch of the SEM fractographic observations on the burst fracture face of the first support plate region of tube R38-C46. A 0.37 inch long, OD origin, axial macrocrack was observed. The intergranular crack was up to 78% through wall and was contained within the support plate crevice region. The macrocrack was composed of numerous microcracks which had an unusual spatial distribution. They had orientations which ranged from axial to circumferential generating a spider-like crack distribution. It is believed that this local network had a cellular IGA/SCC morphology similar to that on tube R16-C74 from Plant L presented in Figure 4-4. Three other crevice locations had less deep but significant intergranular crack distributions. Their locations are also shown in Figure 4-15. Figure 4-16 shows the crack distribution and depth as determined by transverse metallographic examinations. Figure 4-17 shows photomicrographs of cracks in transverse sections obtained from within the crevice region. The cracks are opened wide by tube deformation. The morphology of the cracks is that of IGSCC with minor to moderate IGA components ($D/W = 14$ to 29).

4.3 Other Farley Pulled Tube Examination Results at TSP Locations

Prior to 1990, Westinghouse examined a total of 10 hot leg support plate intersection locations on steam generator tubing removed from Farley Unit 2. In 1985 the first three hot leg support plate regions of tube R34-C44 from Steam Generator A were destructively examined. In 1986 the first support plate region of tube R31-C46 from the hot leg side of Steam Generator C was destructively examined as described in Section 4.1. In 1989 the first three support plate regions of tubes R16-C50 and R16-C53

from the hot leg side of Steam Generator C were destructively examined. Of these 10 support plate locations, 6 were found to have OD origin intergranular corrosion. In addition, Westinghouse examined in 1990 a support plate region on a tube removed from Farley Unit 1. This support plate region also had OD origin intergranular corrosion. The following describes the extent and morphology of the degradation found.

Farley Unit 2, 1985 Examination

The first three support plate regions of hot leg tube R34-C44 from Steam Generator A were destructively examined to determine the origin of residual eddy current signals left at the location of the support plates after frequency mixing to eliminate the support plate signal. No corrosion degradation was found by destructive examination at the three support plate locations.

Farley Unit 2, 1986 Examination

The first support plate region of hot leg tube R31-C46 from Steam Generator C was destructively examined. A 6.16 volt, 81% deep eddy current signal was detected in the field bobbin probe examination using a 400/100 kHz frequency mix. Renormalization to the standard used in this report yielded 7.2 volts. Destructive examination found a 0.5 inch long macrocrack that extended from 0.1 inch above the support plate bottom edge location to 0.2 inch below the top edge location. The crack averaged 80 to 90% through wall with local area penetrating 100% through wall, with a length of 0.02 inch. The macrocrack was composed of a number of axially orientated microcracks which had grown together by corrosion. The intergranular cracking was of OD origin and a number of shallow cracks existed parallel and nearby to the major macrocrack. The morphology of the cracking was predominately SCC, but moderate IGA components (D/W = 3.2) were also present.

Farley Unit 2, 1989 Examination

Two hot leg steam generator tubes from Farley Unit 2, Steam Generator C were examined to determine the origin of residual eddy current signals at support plate locations. The destructive examination included the support plate crevice regions 1-3 from tubes R16-C50 and R16-C53.

All six support plate intersections had residual type eddy current signals. The second support plate region of both tubes was chosen for more detailed examination. Following removal of both ID and OD deposits by honing, abrasion, and later by chemical cleaning, the eddy current examination was repeated. No significant change was observed in the eddy current signals indicating that the residuals were not related to surface deposits.

Destructive examination of tube R16-C50 found OD origin intergranular corrosion within the first and second support plate regions. No corrosion degradation was found within the third support plate crevice region. The first support plate region had only an isolated region of minor OD origin, intergranular, axial SCC. The maximum depth of SCC was 0.007 inch. The second support plate region from tube R16-C50 had experienced some negligible (no wall thickness change measurable) OD general corrosion with some intergranular penetrations. While most of the tube OD cracking within the TSP crevice regions had these features, at one location the intergranular corrosion was somewhat

deeper though still regarded as minor. At this location, 0.2 inch below the support plate top edge, the penetrations formed two short parallel axial cracks, 0.06 inch long and up to 0.0015 inch deep. Consequently, all three support plate regions of tube R16-C50 had no, or only very minor, IGSCC degradation. Figure 4-18 shows a micrograph of the largest crack found, that within the first support plate crevice region. The morphology is that of IGSCC with only minor to moderate IGA components.

Destructive examination of tube R16-C53 found OD origin intergranular corrosion within all three support plate regions. The first support plate region of tube R16-C53 had numerous OD origin, intergranular, axial stress corrosion cracks, but the depth of cracking was shallow (0.0055 inch maximum depth). At the second support plate of tube R16-C53, axial intergranular stress corrosion cracking was found on the tube OD concentrated near the support plate top edge and to a lesser extent near the support plate bottom edge. There were dozens of very tight stress corrosion cracks located discontinuously around the circumference, but located within all four quadrants of the tube. The maximum depth of penetration was 0.011 inch (22%). The third support plate region also had numerous but relatively shallow OD origin, intergranular, axial SCC. The maximum depth of degradation was 0.0065 inch. Consequently, the only support plate region of tube R16-C53 with corrosion degradation of any potentially noticeable (by eddy current) depth was the second support plate region where the maximum depth was 0.011 inch (22% through wall). The morphology of these cracks ranged from that of IGSCC (Figure 4-19) to that of IGSCC with significant IGA components (Figure 4-20).

The 1989 pulled tubes were removed primarily to determine the cause of eddy current support plate residual signals. Laboratory eddy current testing showed that the residual eddy current signals were not caused by surface deposits. Destructive examination also showed that the residual signals were not caused by corrosion degradation, even though minor OD origin SCC was present at five of the six support plate locations. For tube R16-C50 the deepest support plate region SCC was 0.007 inch while for tube R16-C53 the deepest crack was 0.011 inch. For the other support plate locations with cracks, the deepest cracks were 0.0015, 0.0055, and 0.0065 inch.

Farley Unit 1, 1989 Pulled Tube Examination

The first support plate crevice region of hot leg tube R20-C26 from Steam Generator C of Farley Unit 1 was destructively examined. Dozens of short, OD origin, intergranular, axial stress corrosion cracks existed within the crevice region and just above the crevice region. Most of these cracks were found within two 30° wide axial bands on opposite sides of the tube. The band located at 255 to 285° extended from the support plate bottom edge to just above the support plate top edge. The deepest crack in this band penetrated 62% through wall and was located approximately 0.2 inch below the top edge. The second band occurred between 75 and 105° with the cracking extending from the bottom edge to approximately 0.275 inch above the support plate top edge. Within the crevice region, the deepest crack in the second band of cracks occurred near the support plate top edge. This crack was 42% through wall. Above the top edge, the depth of cracking decreased rapidly. At 0.13 inch above the top edge, the deepest crack was 10% through wall. With respect to the length of individual cracks, they were typically much less than 0.1 inch long. Where individual cracks had grown together, cracks up to 0.13 inch long were found. Figure 4-21 sketches the crack distribution found within the

first support plate crevice region and also provides crack depth data. Figures 4-22 and 4-23 provide photomicrographs of typical cracks as observed in transverse metallographic sections that have been deformed to open cracks. The morphology is that of IGSCC with minor to moderate IGA components (D/W = 17 to 28). Figure 4-24 shows similar transverse micrographs, but ones in which not all cracks were opened during the tube deformation.

Field eddy current inspection (bobbin probe) of the first support plate region revealed (by initial interpretation) no corrosion degradation. Later analysis suggested a very low voltage (0.2 volts) indication signal, partially hidden between larger voltage dent signals. Laboratory bobbin probe inspection produced similar results, with an indication voltage of 0.4 volts within the overall 7 volt dent signal. The phase angle of the indication component, within the overall dent signal, suggested a 61% deep indication. RPC testing revealed many indications confined to within the crevice region.

4.4 IGA and Corrosion Morphology at Support Plate Crevice Locations

A review of available Westinghouse tube pull data was made for the purpose of comparing corrosion morphology in various plants and for determining the extent of IGA (in contrast to the IGSCC previously discussed in Sections 4.1.2, 4.2, and 4.3) present at support plate crevice locations on steam generator tubing. This review also included recent data from Plant L tube R12-C8 from Steam Generator D. Due to its special interest, the Plant L tube will be discussed first in detail.

4.4.1 IGA and Corrosion Morphology at Support Plate Crevice Locations at Plant L

Corrosion Degradation at the First Support Plate Region of Tube R12-C8

A summary of corrosion observations at the first support plate region of plugged tube R12-C8 is as follows. Within the first support plate crevice region of tube R12-C8, very high densities of axially oriented IGSCC microcracks were observed. Corrosion was not observed outside of the crevice region. These microcracks had moderate IGA components associated with them. A good description of the microcracks would be that of IGA fingers, with the depth of the cracks typically being 6 to 18 times the width of the IGA associated with the crack. The microcracks were less than 0.05 inch long, in axial extent. The density of support plate region cracking was significantly higher than that observed for most other domestic power plants. For a given elevation, crack densities of three to five hundred individual microcracks could be extrapolated to exist around the circumference based on metallographic and SEM fractographic data if the maximum local crack density observed extended completely around the circumference. (For tubes examined by Westinghouse at support plate regions, crack densities of 1 to 24 are most typically observed. The highest, support plate region, crack density previously observed in tube examinations by Westinghouse was 20 to 100 at Plant D-2. It has been reported that high crack densities of approximately 300 cracks over the circumference of a support plate region have also been observed in some EdF steam generators.) Because of the very high densities of cracks and the IGA associated with the cracks, local regions sometimes formed effective patches of IGA. (Alternatively, the IGA patches may have formed independently of the IGSCC.) The depth of these IGA patches was typically half that of the the maximum depth of cracking penetrating through the IGA patch. The

largest circumferential length of continuous IGA observed by metallography was 0.05 inch, or approximately 7 degrees, with a maximum depth of 33%. The maximum depth of IGSCC in the same general region was approximately 85%. Figure 4-25 provides supporting metallographic data.

Another aspect of the very high density of axially orientated microcracks, was the formation of larger axial macrocracks. (Before this aspect can be considered, further details regarding the destructive examination need to be mentioned. The first support plate region was initially separated circumferentially near the center of the crevice region by applying a tensile force axially to the tube. The fracture would have occurred where the volumetric corrosion degradation was deepest. SEM fractography of the separation showed intergranular corrosion greater than 10% deep over 310 degrees of the circumference. Table 4.3 presents complete depth data for the corrosion front. It is believed that the corrosion front was composed of a large number of axially orientated cracks that frequently had interconnecting IGA components.* The deepest region of corrosion was 80 to 92% deep, via IGSCC, over approximately 20 degrees of the circumference.) Above this local region with the deepest corrosion, the tubing was deformed to open any axial crack networks. Many were revealed. One of the deeper looking ones was broken apart and SEM fractography was performed. A fairly uniform crack front was observed from the support plate crevice center to the support plate crevice top. The front ranged from 41 to 55% through wall, with an average depth of 48%. Several ledges were observed in the fractograph where it is believed that individual, axially oriented, microcracks had joined together to form the macrocrack. Below the support plate center, only metallography was performed. Transverse metallography (Figure 4-25) revealed the morphology of the axial cracking and IGA in the form of IGSCC with moderate IGA components and IGA patches at the bases of the IGSCC. Axial metallography was performed from the bottom edge of the crevice to the center of the crevice region, through a region with corrosion. A fairly uniform corrosion front, approximately 50% through wall, was observed that is similar to that revealed by SEM fractography above the center of the crevice. From this data it is concluded that axial macrocrack networks existed from the bottom edge of the support plate crevice region to the top edge, with the crack fronts having a fairly uniform depth.

Corrosion Degradation at the Second Support Plate Region of Tube R12-C8

Metallographic data available from the second support plate crevice region of tube R12-C8 indicated the presence of approximately 50 axial penetrations around the circumference. The morphology of the penetrations was that of narrow IGA fingers. The maximum depth of cracking observed was approximately 48% throughwall. In addition to cracking, patch IGA was also present. ABB (CE) conservatively calculated that the maximum depth of the intermittently distributed, patch type IGA was 27% throughwall. Their conservative definition of IGA (corrosion greater than 5 grains wide on a given crack) produced results that were judged not to be directly relevant to a structural integrity analysis. Their definition of patch-type IGA would include corrosion that

* Alternatively, cellular IGA/SCC may have been present in this region. It could have produced similar SEM fractographic and metallographic (both axial and transverse) results.

would behave as axial cracks rather than as IGA patches that would behave as tubing with either a thinned wall or localized pitting. Consequently, the data was reexamined using a definition of IGA judged more relevant to a structural integrity analysis. IGA was identified where corrosion associated with two or more separate cracks intersected or, alternatively, where the corrosion associated with one crack or area (if no cracking was present) had a D/W (depth-to-width ratio) of one or less.

Using this definition, patch IGA was identified at two locations around the circumference at one elevation within the crevice region and at four locations at a second crevice elevation. All patch IGA, that was 10% or more deep, was associated with isolated regions that were very small in circumferential extent. The maximum depths and their corresponding circumferential extents were 11% deep and 1 degree in width for the first elevation and 12% deep and 1 degree in width for the second elevation. The widest IGA patch was 21 degrees (6% of the circumference) with a maximum depth of 6%. For the examined elevations, the largest total circumferential involvement (summation of the widths of the IGA patches) was 22 degrees (6% of the circumference).

From a structural integrity viewpoint, since the IGA patches in the crevice region were isolated from each other and few in number, it is believed that the IGA patches act more like a limited number of pits rather than tubing which has experienced general thinning.

Corrosion Degradation at the Third Support Plate Region of Tube B12-C8

After burst testing of the third support plate crevice region (burst occurred at 10,500 psi), visual examination revealed numerous, axially oriented, corrosion openings adjacent to the main burst opening. Most of the corrosion appeared to be shallow. SEM fractography performed on the burst opening showed intergranular corrosion existing from the bottom to the top edge of the crevice region. Large ledges were frequently observed between axially orientated microcracks.

Transverse metallography showed approximately 85 axial cracks around the circumference with a morphology of axial IGSCC with moderate IGA aspects. The maximum depth of corrosion was 55%. In addition to cracking, patch IGA was also present. ABB (CE) conservatively calculated that the maximum depth of the intermittently distributed, patch type IGA was 33% throughwall. Again, the data was reexamined using the definition of IGA judged to be more relevant to a structural integrity analysis. Patch IGA was identified at seven locations around the circumference at a mid-support plate crevice region elevation. All patch IGA, that was 10% or more deep, was associated with isolated regions that were very small in circumferential extent. The maximum depths and their corresponding circumferential extents were 21% deep and 0.5 degree in width, 10% deep and 0.3 degree wide, 14% deep and 0.3 degree wide, 14% deep and 0.8 degree wide and 17% and 6 degrees wide. In addition, a number of metallographic grinds were made at the location of the 17% deep and 6 degree wide IGA patch. Patch IGA was found in two of the three grinds at this location. These IGA patches were 10% deep and 2 degrees wide and 8% deep and 1.5 degrees wide. For the mid-crevice region elevation, the total circumferential involvement (summation of the widths of the IGA patches) was 10 degrees (3% of the circumference).

From a structural integrity viewpoint, since the IGA patches in the crevice region were isolated from each other and few in number, it is believed that the IGA patches act more like a limited number of pits rather than tubing which has experienced general thinning.

Corrosion Degradation of Other Tubes from the Second Tube Pull Campaign at Plant L

Preliminary destructive examination of the first, second, and third support plate regions of tube R29-C70 has produced the following results. All three regions had similar corrosion degradation. Axially oriented IGSCC with only minor to moderate IGA components was present without effective surface IGA (intermittent minor surface IGA, 1 to 2 grains deep, was occasionally present). The absence of the effective surface IGA is in contrast to the results for tube R12-C8. At a mid-support plate elevation, 2 to 3, 5, and 4 cracks were found distributed around the circumference for the first, second and third support plate regions, respectively. The presence of such a small number of cracks is typical of support plate cracking at many power plants and is in great contrast to the results for tube R12-C8 from Plant L. The burst strengths for the three regions were 10,400 psi, 9000 psi, and 10,400 psi, respectively. SEM fractography of the burst faces showed IGSCC macrocracks, confined to the crevice regions, that were 0.29, 0.62 and 0.45 inch long, respectively. These macrocracks were composed of microcracks that were separated by ligaments with dimple rupture fractures. The numbers of such microcracks were 1, 12, and 6, respectively, for the first, second and third support plate crevice regions.* The maximum spacing between microcrack ledges with tensile overload features was 0.29, 0.20 and 0.14 inch, respectively. The maximum depth of IGSCC observed was 74%, 74%, and 70%, respectively.

Preliminary destructive examination of the first, second, and third support plate regions of tube R30-C64 has produced the following results. All three regions had similar corrosion degradation. Axially oriented IGSCC with only minor to moderate IGA components was present without effective surface IGA (intermittent minor surface IGA, 1 to 2 grains deep, was occasionally present). The absence of the effective surface IGA is again in contrast to the results for tube R12-C8 and is similar to the results for tube R29-C70. At a mid-support plate elevation, 29, 85, and 30 cracks were found distributed around the circumference for the first, second and third support plate regions, respectively. The presence of this moderate number of cracks is also typical of support plate cracking at many power plants and is in contrast to the results for tube R12-C8 from Plant L, at least for the first support plate region. The burst strengths for the three regions were 10,500 psi, 8800 psi, and 10,200 psi, respectively. SEM fractography of the burst faces showed IGSCC macrocracks, confined to the crevice regions, that were 0.74 (0.53)**, 0.61 and 0.45 inch long, respectively. These

* Other ligaments or ledges with intergranular features were also present. In the case of the first support plate region, even though only predominantly intergranular ligaments were observed (i.e., one microcrack = one macrocrack), three microcracks were effectively present due to the profile of the scallop-shaped crack front.

** ABB reported the crack length greatly increased during the burst test during the burst test and that a corrected length would be 0.53 inch.

macrocracks were composed of microcracks that were separated by ligaments with dimple rupture fractures. The numbers of such microcracks were 12, 6, and 6 respectively, for the first, second and third support plate crevice regions.* The maximum spacing between microcrack ledges with tensile overload features was 0.175, 0.28 and 0.12 inch, respectively. The maximum depth of IGSCC observed was 55%, 62%, and 49%, respectively.

Corrosion Degradation on Tubes R16-C74, R20-C66 and R8-C66 from S/G-D and Tubes R8-C69 and R12-C70 from S/G-C, from the Third Tube Pull Campaign at Plant L

The first, second and third support plate regions of each of these tubes (except for the second support plate region of tube R12-C70) have been burst tested and SEM fractography has been performed on the axially oriented burst fracture faces. All crevice regions had predominantly axial IGSCC. In addition, nine of the fifteen support plate crevice regions may have had local areas within the crevice region with some cellular IGA/SCC. The third and fourth pages of Table 4.4 (which includes summary corrosion morphology data from many plants which will be discussed in more detail in Section 4.4.3) provide further details. The probability of cellular IGA/SCC existing is indicated by the choice of the adjectives definitely, probably, and possibly. The single definite observation is based on radial metallography. The other observations are based on visual observations and standard transverse and longitudinal metallography. Pages 3 and 4 of Table 4.4 also provide details of the crack densities of these 15 crevice regions from Plant L (all had moderate to low crack densities), as well as details of the extent of IGA associated with the major cracks (most had only minor IGA components) and the extent of OD IGA (only three of the fifteen crevices had IGA, all in the form of a few isolated IGA patches). The lengths and depths of the burst fracture faces and the burst pressures are provided in Table 4.5.

Further, details are presented for the first support plate crevice region on tube R16-C74 where cellular IGA/SCC was confirmed. SEM fractography on the burst opening found numerous axially oriented, OD region, intergranular microcracks, up to 69% deep. Ignoring shallow and isolated cracks near the support plate edges, the main macrocrack was 0.56 inches long and averaged 48% deep. Transverse metallography through the center of the crevice revealed 44 axially oriented intergranular cracks, with minor IGA components. The maximum local depth of cracking was approximately 40%. Negligible uniform IGA (typically 1 to 2 grains deep) was found on most of the OD (and ID) surface. Several small patches of IGA (maximum depth of 19%) were also observed.

Axial metallography revealed what looked like patch IGA that was up to 52% deep, that was confined to the lower central region of the support plate crevice. (This corrosion was later shown to be cellular IGA/SCC.) In the region near this zone where visual

* Other ligaments or ledges with intergranular features were also present. In the case of the first support plate region, even though only predominantly intergranular ligaments were observed (i.e., one microcrack = one macrocrack), three microcracks were effectively present due to the profile of the scallop-shaped crack front.

observations revealed a complex mixture of axial and circumferential cracking. In one quadrant of the crevice, radial metallography was performed on a 0.5 x 0.5 inch section of tubing that had been flattened. Cellular IGA/SCC was found on one third of the section. Figures 4-26 and 4-27 show the radial metallography obtained at various radial depths. Later grinding operations performed on this radial section showed that only axial cracks existed at a depth of approximately 69%. At present, it is believed that the cellular IGA/SCC was mostly contained in this region and probably was not the dominant morphology at the burst fracture. This data is highlighted since similar corrosion morphologies may exist at the first support plate region of tube R12-C8.

Summary of Plant L Degradation

All three support plate regions of Plant L plugged tube R12-C8 had multiple axial IGSCC macrocrack networks from the bottom to the top edge of the crevice. The first support plate region had the deepest cracking, 92% through wall. For the second and third support plate regions, the maximum crack depths were 48 and 55%, respectively. In addition, effective IGA patches were observed. In the case of the first support plate crevice location, the IGA patches occurred in regions with the highest crack densities. The depths of the IGA patches were typically half that of the associated axial cracking. For the second and third support plate regions, limited data was directly available regarding the IGA patches, but it was reported by CE that the maximum depths of IGA for these two support plate regions were 27% and 33%, respectively. The twenty-one support plate regions from the other seven pulled Plant L tubes had corrosion more typical of other plants: a small to moderate number of axial IGSCC, minor to moderate IGA components to the cracking, and little or no separate IGA (patch IGA). While the IGSCC on these tubes had IGA components, the appearance was more that of stress corrosion cracking than that of IGA fingers as was observed at the first support plate region of tube R12-C8. Finally, cellular IGA/SCC was locally observed at the first support plate crevice region of tube R16-C74. Cellular IGA/SCC may have also been present, and even played a major role, in the corrosion degradation at the first support plate region of tube R12-C8. Non-confirmed cellular IGA/SCC was also suspected in local areas of the crevice regions of another eight of the total 24 crevice regions examined.

4.4.2 Cellular IGA/SCC at Plant E-4

EdF steam generator tubes at support plate crevice regions at Plant E-4 have developed cellular IGA/SCC. The cellular IGA/SCC is localized in the crevice region such that most of the crevice region is free of corrosion. The second page of Table 4.4 presents summary corrosion morphology data available from five crevice regions. The crevice regions had moderate crack densities, moderate IGA components associated with individual major cracks, and no significant IGA independent cracking. Burst tests conducted produced the expected axial opening through complex mixtures of axial, circumferential and oblique cracks. For the more strongly affected areas, while the cracking remained multi-directional, there was a predominance of axial cracking. Figures 4-28 and 4-29 provide radial section photomicrographs through two of the more strongly affected areas showing cellular IGA/SCC at Plant E-4.

4.4.3 Comparison of Plant L Support Plate Corrosion to Support Plate Corrosion at Other Plants

It is difficult to compare corrosion morphology from one plant to another since the visual, metallographic and SEM fractographic data are frequently not comparable and seldom provide a complete description. It is especially difficult if the person doing the comparison has not directly worked with the raw data since it will not be known if the reported data represents typical or the more spectacular and extreme data. With these caveats, the following presents a corrosion comparison in which the raw data, not just the reported data, were all studied by the same person.

To compare support plate corrosion morphology, three ways of data characterization were utilized. All three need to be considered to characterize the corrosion. The first measures cracking density. Since most cracking within support plates is axial in nature, cracking density is usually measured from a transverse metallographic section. If a complete section is available, the cracking density at the given elevation can be directly measured. If only a partial section is available, an estimate by extrapolation can be made. Cracking densities were arbitrarily divided into three density categories: low (1 to 24 cracks); moderate (25 to 100 cracks); and high (greater than 100 cracks). Note that since most axial cracking is composed of short microcracks, usually less than 0.05 inches long, a cracking density of say 25 at a given elevation would correspond to several hundred microcracks within a support plate region. The second way of characterizing the data involved measuring the amount of IGA associated with a given crack. To do this the depth of the crack was divided by the width of the IGA as measured at the mid-depth of the crack, creating a ratio D/W. Again, three arbitrary D/W categories were created: minor ($D/W > 20$) (all or most PWSCC would be included in this category if it were being considered in this analysis); moderate ($D/W 3$ to 20); and significant ($D/W < 3$) where for a given crack with a D/W of 1 or less, the morphology approaches that of patch IGA. The third way of characterizing the data involved considering the extent of IGA present on the tie tube, but only the IGA not obviously associated with a single crack was considered. Consequently, IGA independent of cracking is measured and IGA associated with the interaction of more than one crack is measured. The measurement of IGA arbitrarily divided the circumferential extent of IGA into three categories: negligible (IGA < 5% deep); moderate (IGA 5 to 10% deep); and significant (IGA greater than 10% deep).

Table 4.4 presents a corrosion morphology comparison of Plant L support plate region data, similar data from other plants examined by Westinghouse, and data from laboratory corrosion tests conducted in model brackets. With respect to cracking density at support plate locations, it is obvious that the cracking densities at Plant L for the first support plate region of tube R12-C8 are considerably higher than experienced at other plants examined by Westinghouse. However, similar cracking densities of several hundred cracks at a given support plate elevation are believed to exist in some EdF plants in Europe and in some plants in Japan. The cracking densities for the other support plate regions of tubes from Plant L are more typical of other power plants with low to moderate crack densities. With respect to the amount of IGA associated with the axial IGSCC, the Plant L data are similar to those at most other plants; moderate IGA components are found in association with the axial IGSCC. With respect to IGA that is present to a significant extent (i.e., excluding isolated IGA patches), only Plant L (tube R12-C8 only) and Plant M-2, among the plants examined by Westinghouse, were found

to have significant IGA (greater than 10% through wall). It is believed in the case of Plant L that the formation of IGA in the form of IGA patches is a result of the high cracking densities and IGA aspects associated with the individual cracks. Where the cracks are particularly close together, IGA patches form at the base of the cracks where the width of the IGA is greatest. In the case of Plant M-2, the typical IGA morphology was that of uniform IGA as is shown in the lower two photomicrographs in Figure 4-30. The top photomicrograph in Figure 4-30 also shows uniform IGA but with some axial IGSCC appearing through the uniform IGA.

While not examined by Westinghouse, the following presents data regarding Plant J-1 and Plant N-1. Figure 4-31 and 4-32 show photomicrographs from the first and second support plate regions of tube R8-C74 of steam generator 2 of European Plant J-1. The intergranular corrosion appears to be very similar to that at Plant L. Table 4.4 presents qualitative morphological data. While there is a slightly lower crack density, the extent of IGA associated with individual cracks is similar (moderate D/W ratios), the extent and depth of IGA is similar and the origin of the IGA also appears to be that of closely spaced axial IGSCC interacting near the surface to form local IGA patches. It is also interesting that the maximum depth of IGA compared to the depth of IGSCC is similar, typically one-third to one-half of the IGSCC depth. The data from the support plate regions at Plant N-1 was not in a form where firm conclusions regarding corrosion morphology could be made. Table 4.4 also presents an attempt to force conclusions from the data available to Westinghouse. Averaging the data from the three support plate regions, it is concluded that a corrosion morphology similar to Plant L (tube R12-C8) and Plant J-1 exists at Plant N-1.

Table 4.1

Laboratory Eddy Current Data for Tubes Removed from Farley Unit 2

Results at Bottom TSP Location for Ali Tubes

<u>Examination</u>	<u>Tube R4-C73</u>	<u>Tube R21-C22</u>	<u>Tube R38-C46</u>
RPC Exam	Axial indication with faint parallel indication 20 to 30 degrees away; 0.44 inch long, 77-82% deep.	Axial indication; 0.5 inch long, 76-81% deep.	Axial indication 0.4 inch long, 90% deep.
Bobbin Exam			
Echoram Probe	Indication 86-88% deep; voltage ranged from 3.6 to 4.3 volts depending on specimen orientation.	Indication 86-87% deep; voltage ranged from 9.6 to 11.6 volts depending on specimen orientation.	Use field data only: 1.4 volts, 68% deep.
Zetec Probe	Indication 86-91% deep; voltage ranged from 2.6 to 5.0 volts depending on specimen orientation.	Indication 86-90% deep; voltage ranged from 7.7 to 14.2 volts depending on specimen orientation.	

Table 4.2

Leak and Burst Data for Tubes Removed from Farley Unit 2

Results at Bottom TSP Location for All Tubes

<u>Test</u>	<u>Tube R4-C73</u>	<u>Tube R21-C22</u>	<u>Tube R38-C46</u>
Leak Test			
Operating Leak Rate (delta P = 1500 psi)	0 - 0.3 ml/hr	0 - <<7 ml/hr *	No Leak
Steam Line Break Rate (delta P = 2650 psi)	174 ml/hr	108 ml/hr	No Leak
Burst Test			
Burst Pressure (psig)	[]9
Burst Ductility (% delta D)	5.6	6.8	7.6
Burst Opening Length (Inches)	0.459	0.784	0.881
Burst Opening Width (Inches)	0.135 (OD), 0.100 (ID)	0.210 (OD), 0.148 (ID)	0.167

* Problems with back pressure regulator increased the measured leak rate.

Table 4.3

Depth of Corrosion Observed on Circumferential Fracture Face
from Center of the First Support Plate Crevice Region for
Plant L Tube R12-C8

<u>Circumferential Location</u> (degrees)	<u>Maximum Depth of Penetration</u> (%)
0	62
10	90
20	92
30	78
40	52
50	60
60	52
70	40
80	18
90	60
100	48
110	48
120	56
130	60
140	58
150	60
160	56
170	44
180	56
190	2
200	14
210	10
220	18
230	8
240	14
250	14
260	16
270	14
280	44
290	40
300	44
310	18
320	18
330	0
340	16
350	0

TABLE 4.4

Comparison of Intergranular Corrosion Morphology at Support Plate Regions on S/G Tubing and Laboratory Specimens

Data Source	Cracking Density (as Measured or Estimated for One Plane)*			Extent of IGA Associated with the Major Cracks			Extent of OD IGA (Not Obviously Associated with a Single Crack)		
	Low (1-24 Cracks)	Moderate (25 - 100 Cracks)	High (Greater than 100)	Minor (D/W** >20)	Moderate (D/W 3 to 20)	Significant (D/W <3)	Negligible (<5% Deep)	Moderate (5 to 10% Deep)	Significant (>10% Deep)
Farley 2									
R31-C46 SP1	-6				3.2		0		
R4-C73 SP1	-8				4.2		0		
R21-C22 SP1	-2					1.7	0		
R38-C46 SP1	-10			14-28			0		(Possible cellular IGA/SCC in a local area)
Plant B-1									
R4-C61 SP1 (2H)	-4			25			0		
R4-C61 SP2 (5H)	-15 (for axial) -5 Circ. Cracks)				15 (for axial), -1 (for Circ. Cracks)		2% Deep Intermittently around circumference		
R4-C61-SP5 (12H)	-6 (Circ. Cracks)					1-2	0		(Possible cellular IGA/SCC in a local area)
Plant B-2									
R6-C67 SP1 (2H)	16				18		0		
R6-C67 SP3 (8H)		28			15		4% deep intermittently around circ.		
R6-C67 SP5 (11H)	6				No Data		No Data		

* Since most support plate cracking is composed of short axial microcracks, typically 0.02 to 0.05 inch long for a 50% deep crack, a microcracking density of 25 could be associated with more than several hundred individual microcracks within a support plate crevice region.

** D=Depth of SCC as measured from the OD surface exclusive of any surface IGA. W=Width of IGA component to the SCC as measured at the mid-point of the SCC.

TABLE 4.4 (Continuation)

Data Source	Cracking Density (as Measured or Estimated for One Plane)*			Extent of IGA Associated with the Major Cracks			Extent of OD IGA (Not Obviously Associated with a Single Crack)		
	Low	Moderate	High	Minor	Moderate	Significant	Negligible	Moderate	Significant
	(1-24 Cracks)	(25 - 100 Cracks)	(Greater than 100)	(D/W** >20)	(D/W 3 to 20)	(D/W <3)	(<5% Deep)	(5 to 10% Deep)	(>10% Deep)
Plant D-1									
R21-C31 SP1	3				5			Possible small IGA patches?	
R18-C33 SP1	None?						4% max. for 1% of circ.		
R17-C33 SP1	0 to 5				Inadequate Data		0		
Plant D-2									
R7-C38 SP1		90			4-8		0		
R7-C38 SP2		42			15		0		
R7-C38 SP3		23		24			1% deep intermittently around circumference		
R11-C25 SP3		-50			14		0		
Plant E-4									
R19-C35 SP2			-106		9.5		-0		Cellular IGA/ SCC present
R19-C35 SP3		-79			No Data		-0		Cellular IGA/ SCC present
R19-C35 SP4			-103		No Data		-0		Cellular IGA/ SCC present
R42-C49 SP2		-92			3 to 12		-0		Cellular IGA/ SCC present
R8-C47 SP3		-63			4			7% deep IGA, Intermittently around circ.	Cellular IGA/ SCC present

* Since most support plate cracking is composed of short axial microcracks, typically 0.02 to 0.05 inch long for a 50% deep crack, a microcracking density of 25 could be associated with more than several hundred individual microcracks within a support plate crevice region.

** D=Depth of SCC as measured from the OD surface exclusive of any surface IGA. W=Width of IGA component to the SCC as measured at the mid-point of the SCC.

TABLE 4.4 (Continuation)

Data Source	Cracking Density (as Measured or Estimated for One Plane)*			Extent of IGA Associated with the Major Cracks			Extent of OD IGA (Not Obviously Associated with a Single Crack)		
	Low	Moderate	High	Minor (D/W** >20)	Moderate (D/W 3 to 20)	Significant (D/W <3)	Negligible (.5% Deep)	Moderate (5 to 10% Deep)	Significant (>10% Deep)
	(1-24 Cracks)	(25 - 100 Cracks)	(Greater than 100)						
Plant L									
R12-CB SP1			400 (272-504)		6-18 (Possible cellular IGA/ SCC)				210° at SP Center, less above and below Center (see Table 4.2), max depth 43% by Metallography
R12-CB SP2		50			Moderate (no quantitative data available)				Intermittently distributed, patch IGA (6% of circ.), max. depth 12%.
R12-CB SP3		85			Moderate (no quantitative data available)				Intermittently distributed, p-icr IGA (3% of circ.), max. depth 21%.
R29-C70 SP1	2-3			35			-0		
R29-C70 SP2	5				17		-0		
R29-C70 SP3	4			24 to 50			-0		

* Since most support plate cracking is composed of short axial microcracks, typically 0.02 to 0.05 inch long for a 50% deep crack, a microcracking density of 25 could be associated with more than several hundred individual microcracks within a support plate crevice region.

** D=Depth of SCC as measured from the OD surface exclusive of any surface IGA. W=Width of IGA component to the SCC as measured at the mid-point of the SCC.

TABLE 4.4 (Continuation)

Data Source	Cracking Density (as Measured or Estimated for One Plane)*			Extent of IGA Associated with the Major Cracks			Extent of OD IGA (Not Obviously Associated with a Single Crack)		
	Low	Moderate	High	Minor (D/W** >20)	Moderate (D/W 3 to 20)	Significant (D/W <3)	Negligible (<5% Deep)	Moderate (5 to 10% Deep)	Significant (>10% Deep)
	(1-24 Cracks)	(25 - 100 Cracks)	(Greater than 100)						
Plant 1 (Cont.)									
R30-C64 SP1		29			16		-0		Cellular IGA/ SCC possibly present locally
R30-C64 SP2		85			9		-0		
R30-C64 SP3		30		11			-0		
R16-C74 SP1		31		32					A few patches of IGA present, max. depth 19%; Cellular IGA/ SCC definitely present locally
R16-C74 SP2		62		20					A few patches of IGA, 17% max. depth.
R16-C74 SP3		43		21			-0		
R20-C66 SP1		60			4 to 28		-0		Cellular IGA/ SCC probably present locally
R20-C66 SP2		27		21			-0		
R20-C66 SP3		34		36			-0		Cellular IGA/ SCC possibly present locally

* Since most support plate cracking is composed of short axial microcracks, typically 0.02 to 0.05 inch long for a 50% deep crack, a microcracking density of 25 could be associated with more than several hundred individual microcracks within a support plate crevice region.

** D-Depth of SCC as measured from the OD surface exclusive of any surface IGA. W-Width of IGA component to the SCC as measured at the mid-point of the SCC.

TABLE 4.4 (Continuation)

Data Source	Cracking Density (as Measured or Estimated for \perp -Plane)*			Extent of IGA Associated with the Major Cracks			Extent of OD IGA (Not Obviously Associated with a Single Crack)		
	Low (1-24 Cracks)	Moderate (25 - 100 Cracks)	High (Greater than 100)	Minor (D/W** >20)	Moderate (D/W 5 to 20)	Significant (D/W <3)	Negligible (<5% Deep)	Moderate (5 to 10% Deep)	Significant (>10% Deep)
Plant L (Cont.)									
RB-C66 SP1		41		29			-0		Cellular IGA/ SCC probably present locally
RB-C66 SP2	8				8		-0		Cellular IGA/ SCC probably present locally
RB-C66 SP3	2				16		-0		
RB-C69 SP1		46		60			-0		Cellular IGA/ SCC possibly present locally
RB-C69 SP2		47		32				Intermittent patch IGA present, <10% max. depth	Cellular IGA/ SCC probably present locally
RB-C69 SP3		60		29			-0		Cellular IGA/ SCC probably present locally
R12-C70 SP1		44		22			-0		
R12-C70 SP2	24			32			-0		
R12-C70 SP3		28		33			-0		Cellular IGA/ SCC possibly present locally

* Since most support plate cracking is composed of short axial microcracks, typically 0.02 to 0.05 inch long for a 50% deep crack, a microcracking density of 25 could be associated with more than several hundred individual microcracks within a support plate crevice region.

** D=Depth of SCC as measured from the OD surface exclusive of any surface IGA. W=Width of IGA component to the SCC as measured at the mid-point of the SCC.

TABLE 4.4 (Continuation)

Data Source	Cracking Density (as Measured or Estimated for One Plane)*			Extent of IGA Associated with the Major Cracks			Extent of OD IGA (Not Obviously Associated with a Single Crack)		
	Low (1-24 Cracks)	Moderate (25 - 100 Cracks)	High (Greater than 100)	Minor (D/W** >20)	Moderate (D/W \leq to 20)	Significant (D/W <3)	Negligible ($<$ 5% Deep)	Moderate (5 to 10% Deep)	Significant ($>$ 10% Deep)
Plant W-2 Q29-C467L SP1	0	\leq 50 if IGA fronts are defined as cracks with L/W \leq 11				\leq 1 if crack is defined as being present			$>$ 300° uniform IGA; max. depth 26%
Plant C-2 R26-C56 SP1 (2R)		36				2-10			2°, max. depth 8%
Plant P-1 R16-C60 SP1		82		Minor to moderate, 6 to 25 D/W			0		
		58		11 to 17					A few IGA patches at OD crack locations max. depth 31%
		39		10 to 20			-0		
		56		11					IGA and IGA patches, max. depth 9%, less than 20% of circumference

* Since most support plate cracking is composed of short axial microcracks, typically 0.02 to 0.05 inch long for a 50% deep crack, a microcracking density of 25 could be associated with more than several hundred individual microcracks within a support plate crevice region.

** D=Depth of SCC as measured from the OD surface exclusive of any surface IGA. W=Width of IGA component to the SCC as measured at the mid-point of the SCC.

TABLE 4.4 (Continuation)

Data Source	Cracking Density (as Measured or Estimated for One Plane)*			Extent of IGA Associated with the Major Cracks			Extent of OD IGA (Not Obviously Associated with a Single Crack)		
	Low (1-24 Cracks)	Moderate (25 - 100 Cracks)	High (Greater than 100)	Minor (D/W** >20)	Moderate (D/W 3 to 20)	Significant (D/W <3)	Negligible (<5% Deep)	Moderate (5 to 10% Deep)	Significant >10% Deep)
Plant J-1									
LB-C74 SP1			<141		14-20				-200°, max. depth -40%.
LB-C74 SP2			<176		10-15				-80°, max. depth -20%.
Plant N									
L59-C95 SP1			<340		4 (misleading ratio for these cracks, should be larger value)				45°, max. depth 20%.
L59-C95 SP2	0 (no cracks just IGA)								45°, max. depth 13%.
L120-C12 SP3	3			34 (not accurate since obtained from unetched specimen)			1% deep intermit- tently around cir- cumference		
Laboratory Tests									
S71-1	1			-50			0		
S43-4	5			40			0		
S36-1	2			60			0		
S43-4	13				14		0		
S25-1	4				14		0		
S33-3	10			11-40			0		
SL-FH-11	10				11		0		

* Since most support plate cracking is composed of short axial microcracks, typically 0.02 to 0.05 inch long for a 50% deep crack, a microcracking density of 25 could be associated with more than several hundred individual microcracks within a support plate crevice region.

** D-Depth of SCC as measured from the OD surface exclusive of any surface IGA. W-Width of IGA component to the SCC as measured at the mid-point of the SCC.

Table 4.5

Summary Data on Tubes Burst from
Third Tube Pull Campaign at Plant L

<u>Tube/TSP</u>	<u>Burst Pressure</u> (psi)	<u>Intergranular Macrocrack Length On Burst Face</u> (inch)	<u>Maximum Depth</u> (%)	<u>Average Depth</u> (%)
R8-C66 TSP 1	7500	0.83	98	58
R8-C66 TSP 2	8750	0.88	75	44
R8-C66 TSP 3	10600	0.50	53	32
R8-C69 TSP 1	5900	0.92	98	72
R8-C69 TSP 2	7700	1.00*	88	57
R8-C69 TSP 3	9700	0.49	60	51
R12-C70 TSP 1	7100	0.92	98	68
R12-C70 TSP 2	9500	0.85*	63	43
R16-C74 TSP 1	8600	0.56	70	58
R16-C74 TSP 2	9500	0.30	62	38
R16-C74 TSP 3	10400	0.25	51	33
J-C-1 TSP 1	8150	0.60	80	45
R20-C66 TSP 2	8750	0.68	58	47
R20-C66 TSP 3	9300	0.60	58	44

* Direct measurement shows that the pre-burst length does not exceed 0.75 inch.

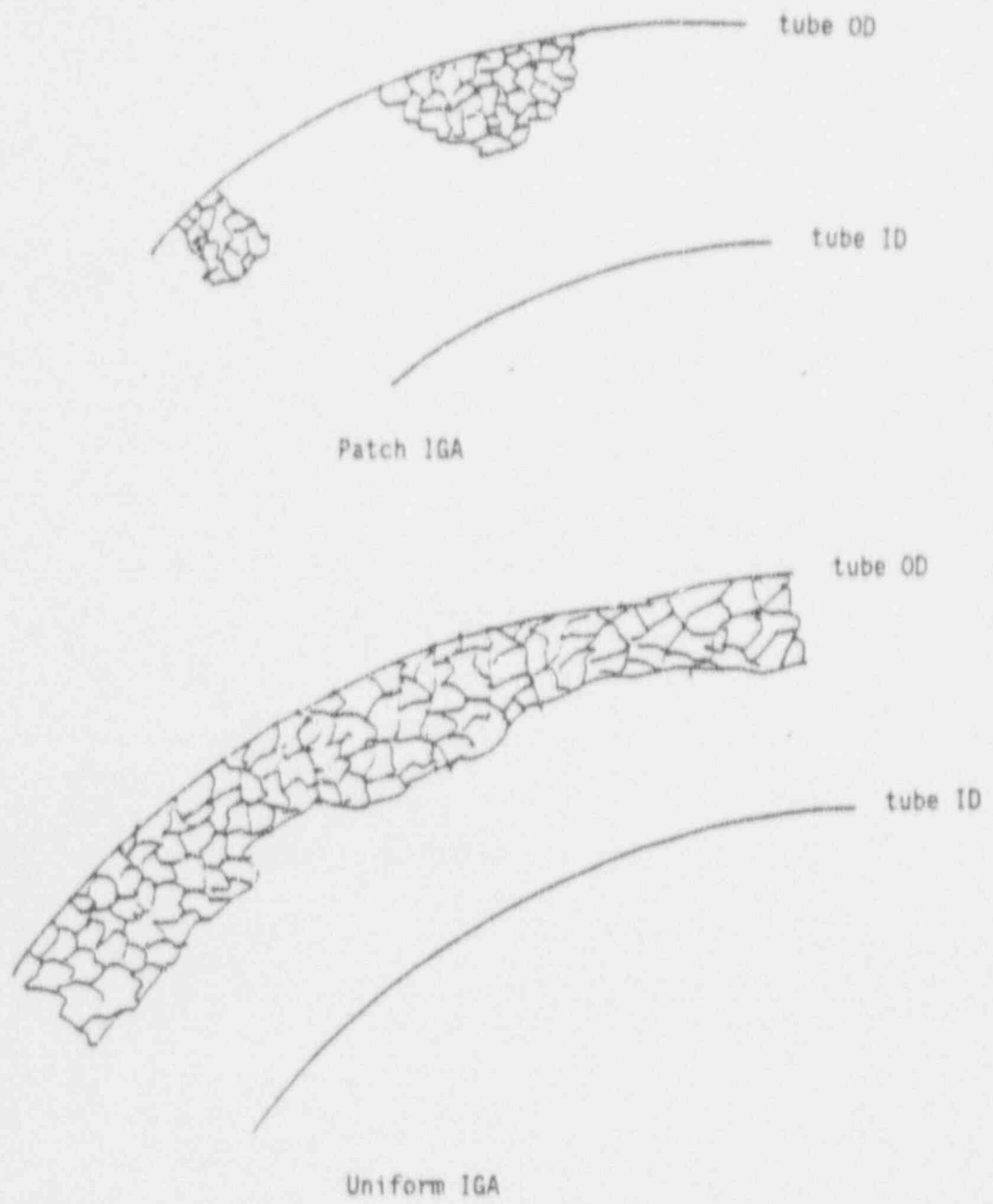
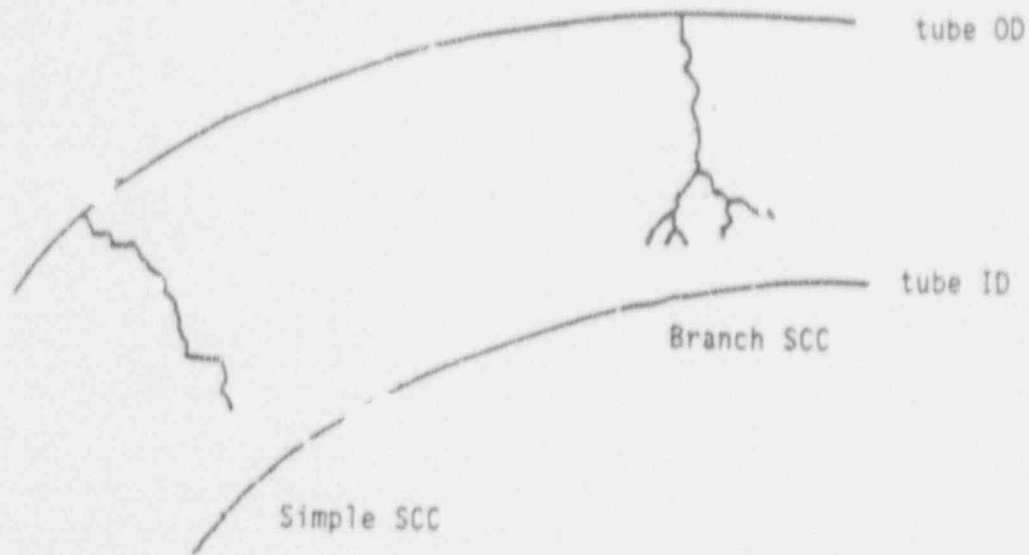
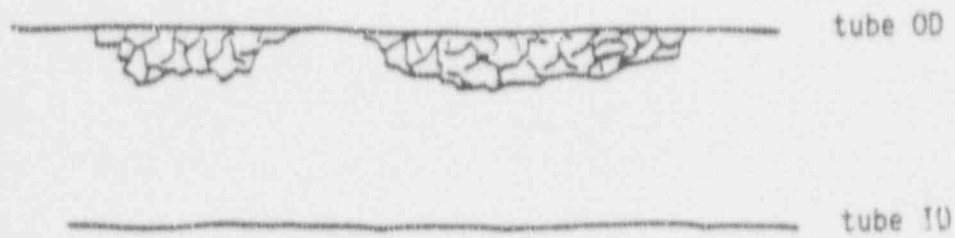


Figure 4-1 Patch and uniform IGA morphology as observed in a transverse tube section. (A similar observation would be made from a longitudinal section.)



transverse section
schematic



longitudinal section
schematic

Figure 4-2 Schematic of simple IGSCC and branch IGSCC. Note that branch and simple IGSCC are not distinguishable from a longitudinal metallographic section. From a longitudinal section, they also look similar to IGA (See Figure 4.3).

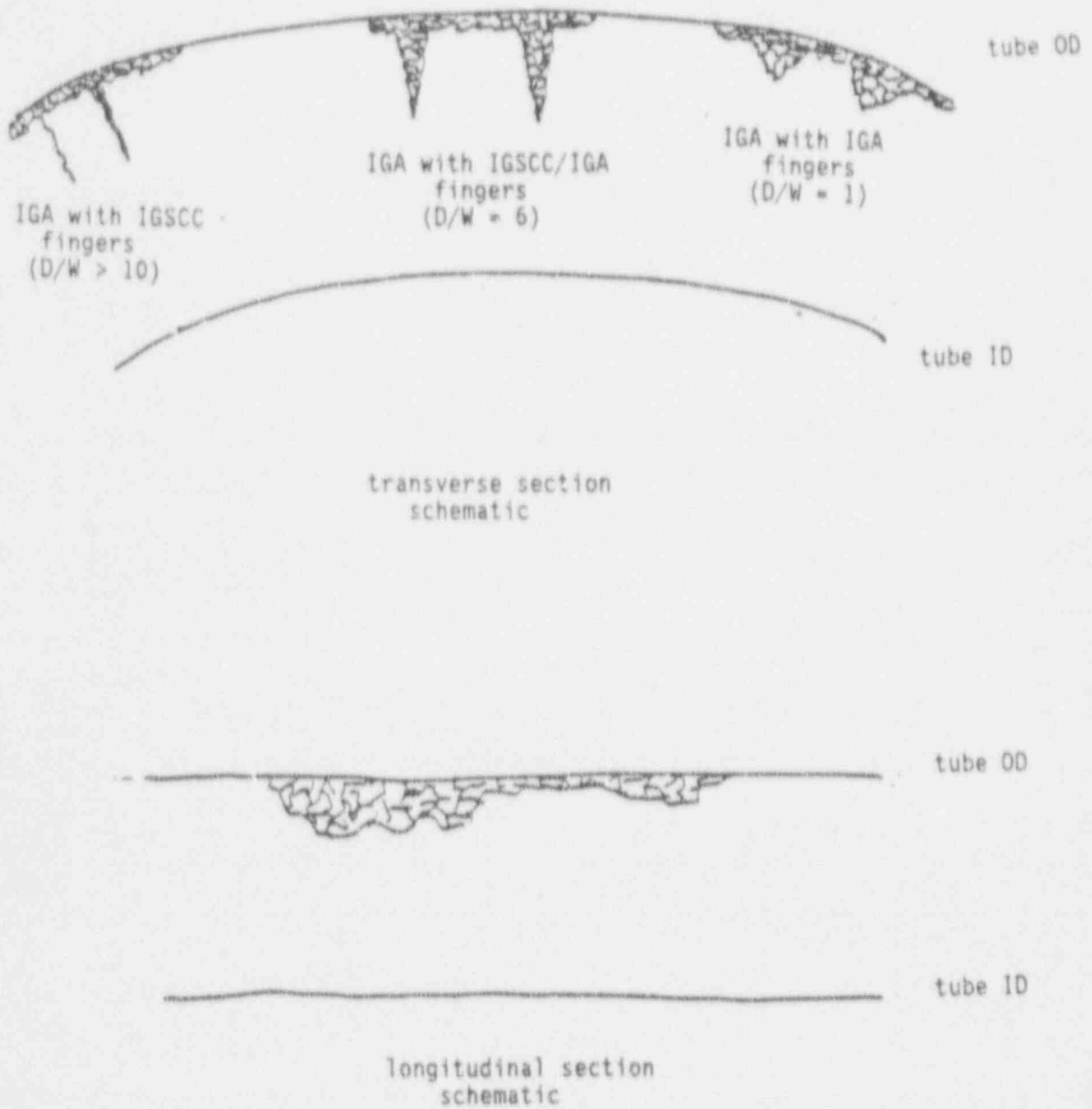


Figure 4-3 Schematic of IGA with IGSCC fingers and IGA with IGA fingers. Note that neither of the above variations can be distinguished from a longitudinal section.

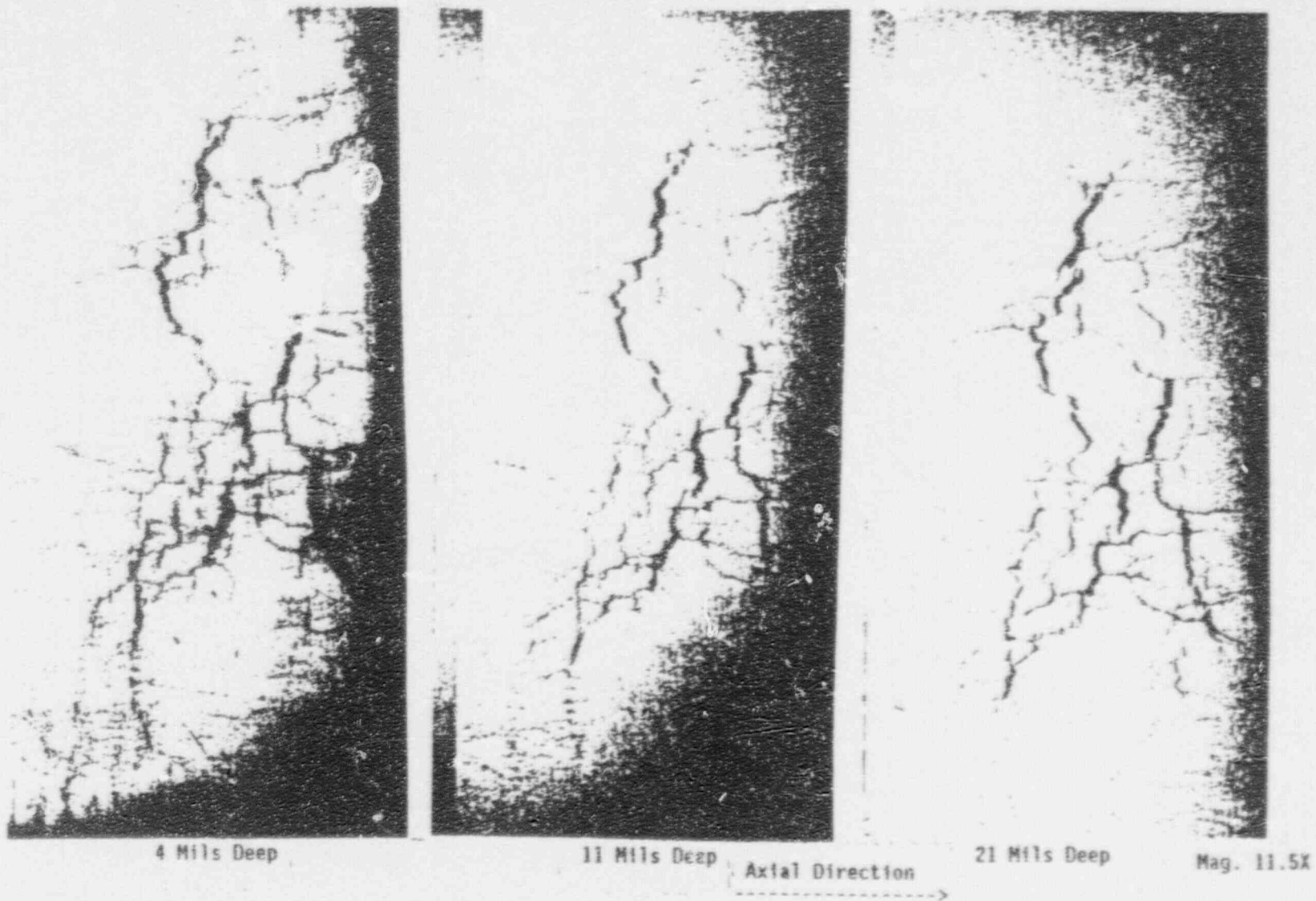
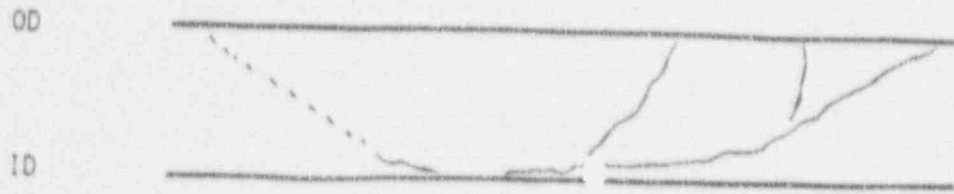


Figure 4-4 Photomicrographs of radial metallography performed on a region with axial and circumferential degradation on tube R16-C74, support plate 1. Cellular IGA was found with little change in the cell shape and cell wall thickness at depths of 4, 11 and 21 mils below the OD surface. Note that the cut section was flattened, preferentially opening the circumferential wall of the cells.



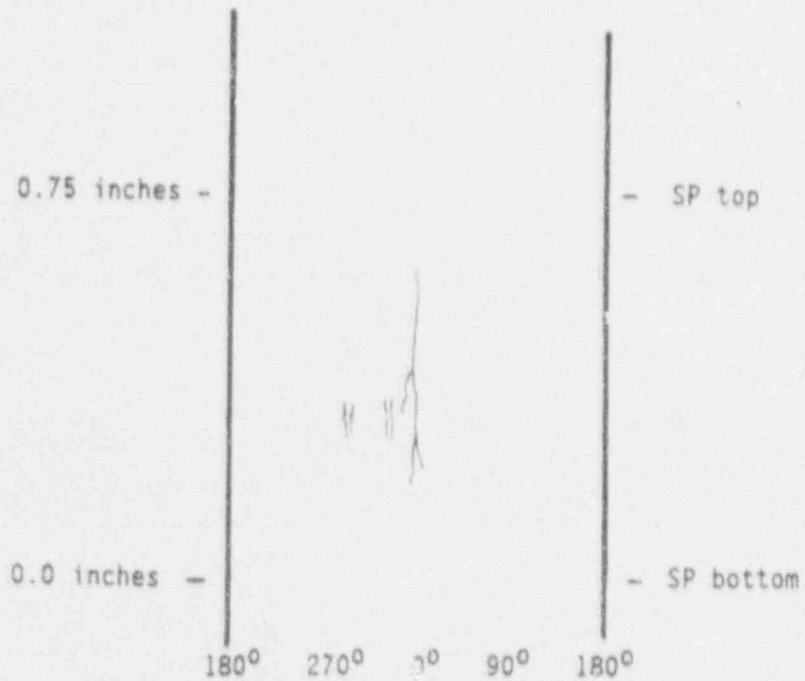
Sketch of Burst Crack

Macrocrack Length = 0.52 inch

Throughwall Length = 0.02 inch

Number of Microcracks = at least 3

Morphology = IGSCC with moderate IGA components



Sketch of Crack Distribution

Figure 4 - 5. Summary of crack distribution and morphology observed on the first support plate crevice region of tube R31-C46, Farley Unit 2.

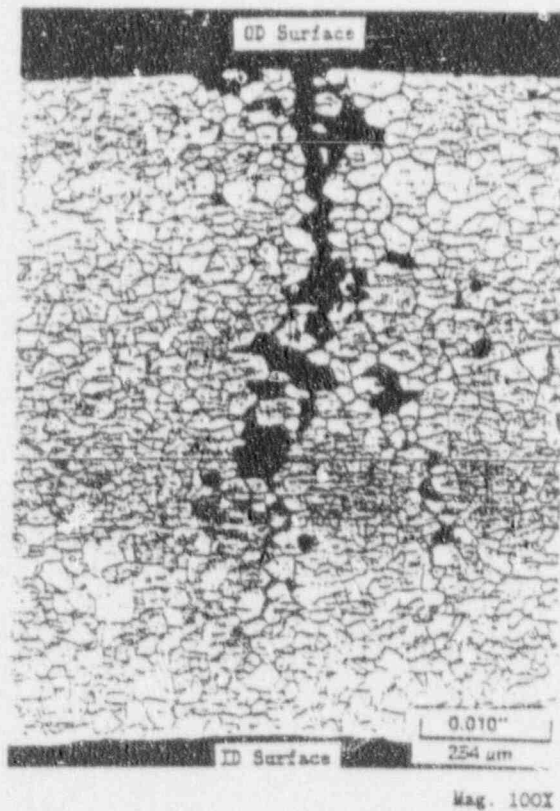
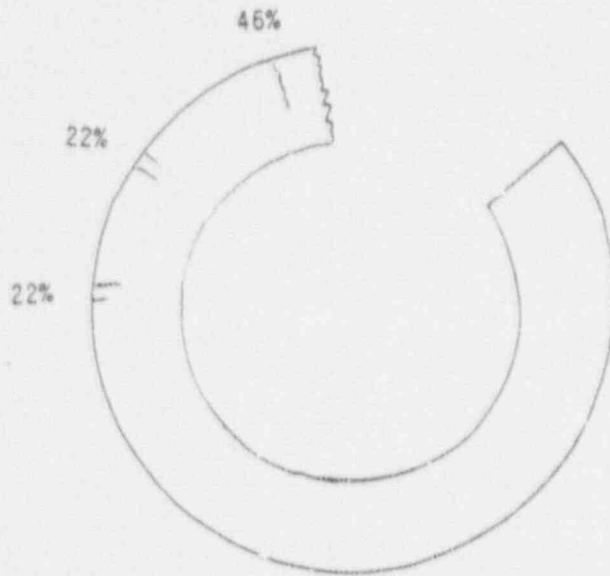
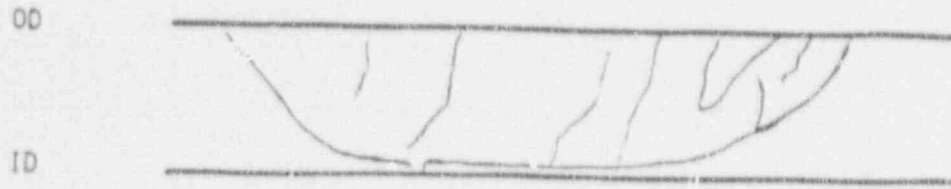


Figure 4 - 6. Secondary crack distribution and a photomicrograph of one of the cracks in a transverse metallographic section of the first support plate crevice region of tube R31-C46. The crack morphology is that of IGSCC with moderate IGA components.



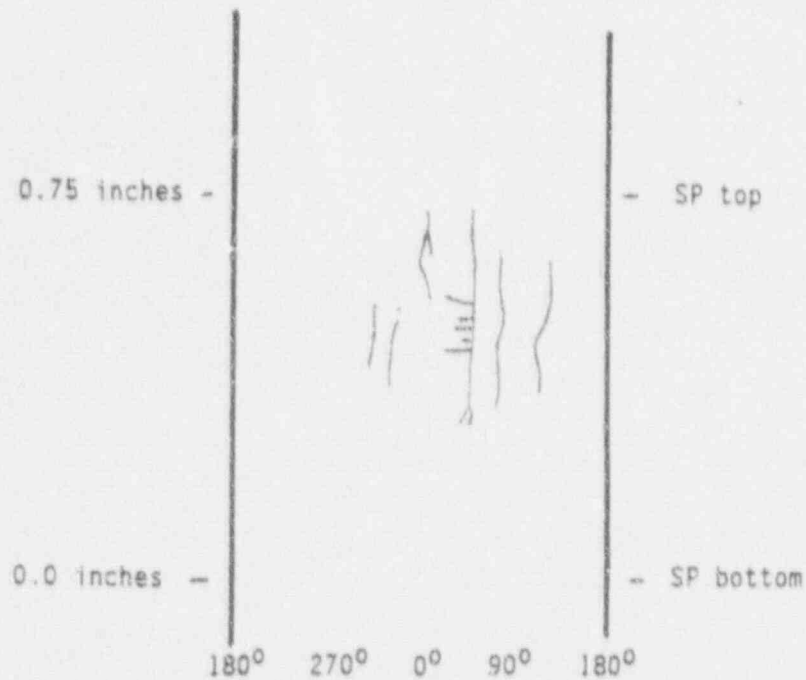
Sketch of Burst Crack

Macrocrack Length = 0.4 inch

Throughwall Length = 0.01 inch

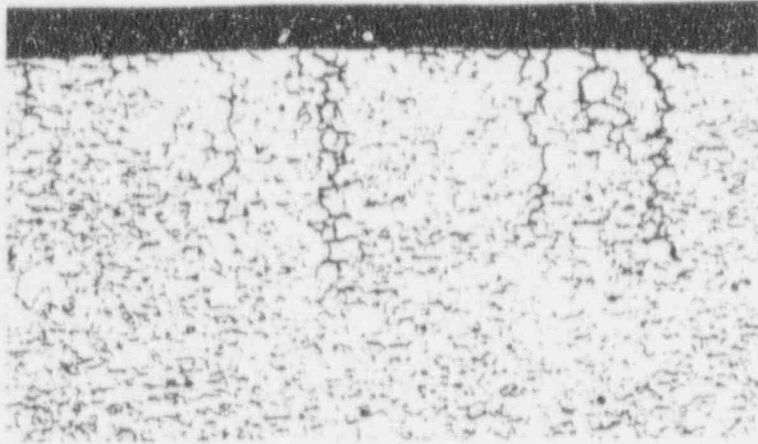
Number of Microcracks = 7 (all ligaments have predominantly intergranular features)

Morphology = IGSCC with some IGA aspects (circumferential cracking has more IGA characteristics)

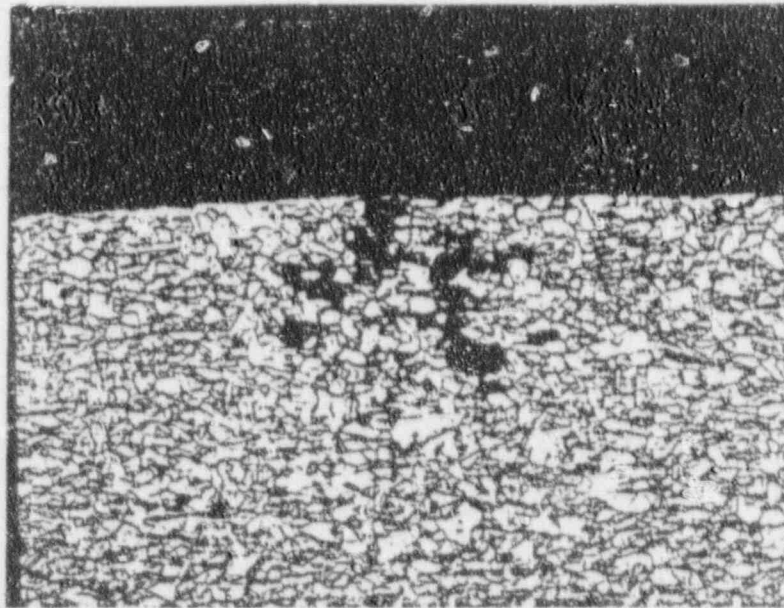


Sketch of Crack Distribution

Figure 4 - 7. Description of OD origin corrosion at the fifth support plate crevice region of tube R4-CE1, Plant B-1



Mag. 100X



Mag. 100X

Figure 4 - 8. Photomicrographs of tube R4-C61 corrosion degradation. Top photo shows axial crack morphology (transverse section) at the eighth support plate location (no transverse metallography was performed at the fifth support plate region). Bottom photo shows circumferential crack morphology (axial section) at fifth support plate region.



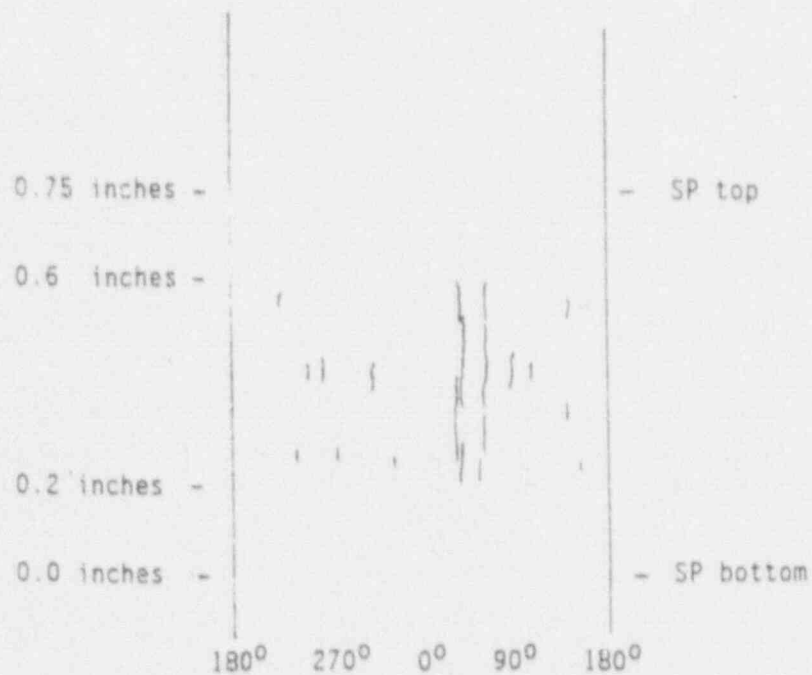
Sketch of Burst Crack

Macrocrack Length = 0.42 inches

Throughwall Length = 0.18 inches

Number of Microcracks = 4 (all ligaments with intergranular features)

Morphology = Intergranular SUC with some IGA characteristics (width of IGA 0.012 inches)



Sketch of Crack Distribution

Figure 4 - 9. Description of OD origin corrosion at the first support plate crevice region of tube R4-C7².

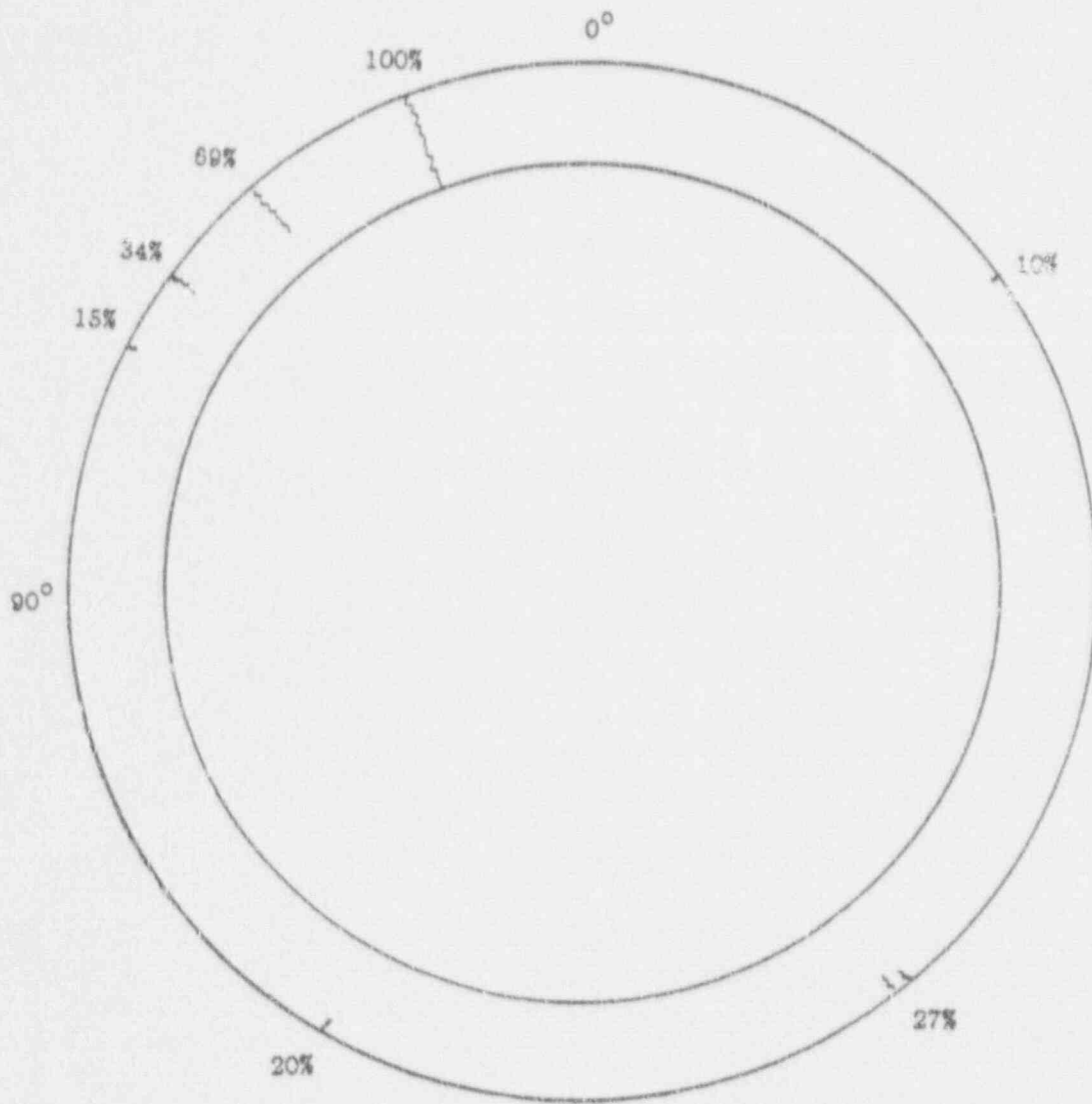
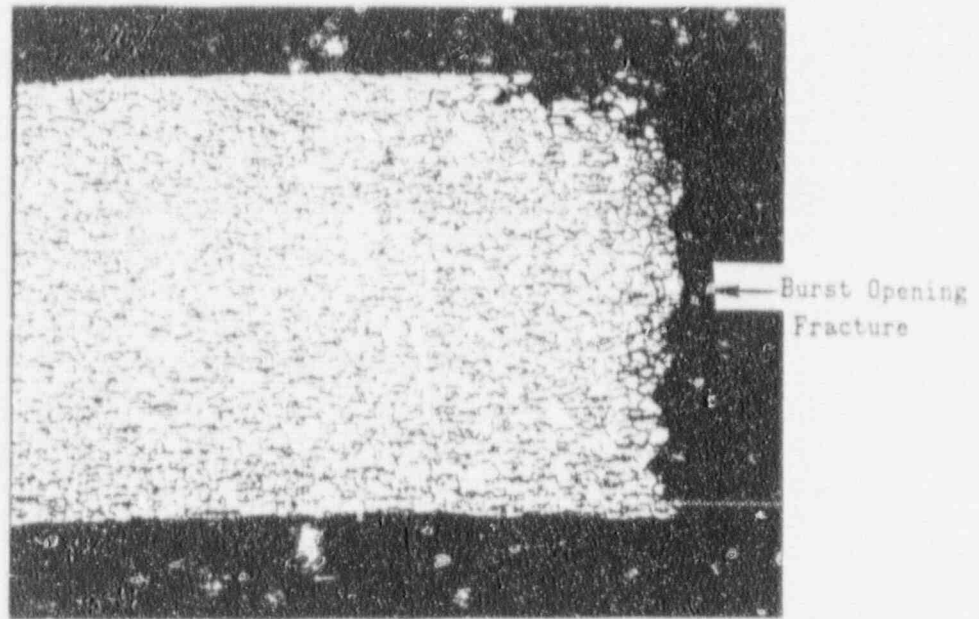
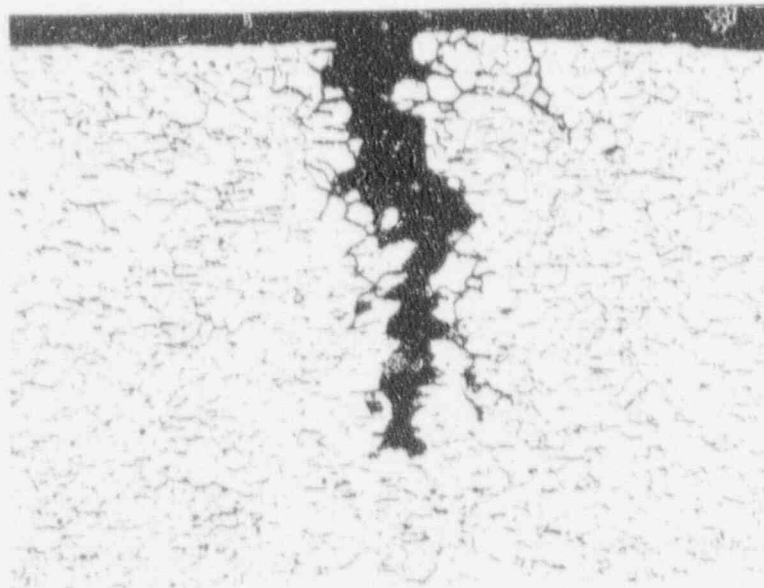


Figure 4 - 10. Sketch of crack distribution and depth within the center of the first support plate intersection in tube R4-C73.



Mag. 50X



Mag. 100X

Figure 4 - 11. Top photomicrograph is from a transverse section through one half of the main burst crack. The crack morphology is that of IGSCC with some IGA characteristics (width of IGA is 0.006 inch on one side of the crack). Bottom micrograph is from a transverse section through a typical crack located near the burst crack. The morphology is that of IGSCC with only minor IGA characteristics. (Note: crack is opened wide by tube deformation).



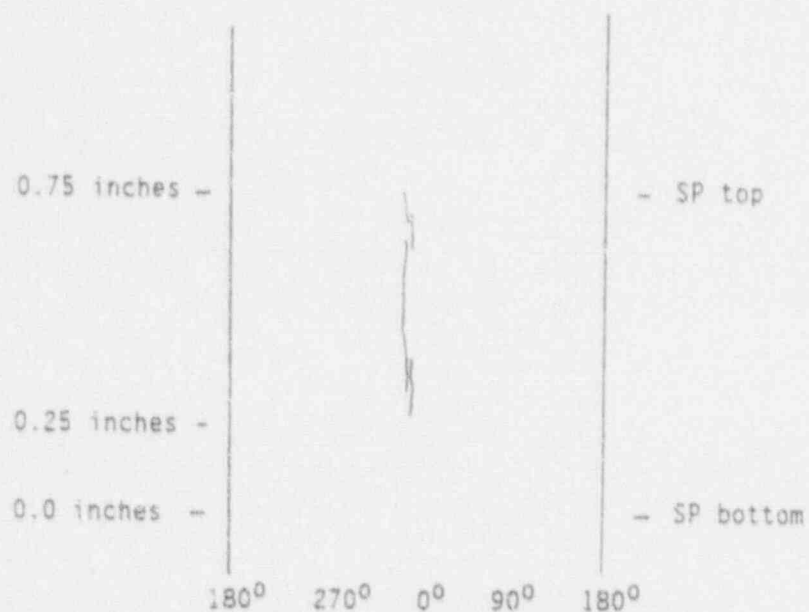
Sketch of Burst Crack

Macrocrack Length = 0.50 inches

Throughwall Length = 0.15 inches

Number of Microcracks = 4 (two ligaments with intergranular features, one with ductile overload features)

Morphology = Intergranular SCC with significant IGA characteristics (width of IGA 0.030 inches)



Sketch of Crack Distribution

Figure 4 - 12. Description of OD origin corrosion at the first support plate crevice region of tube R21-C22.

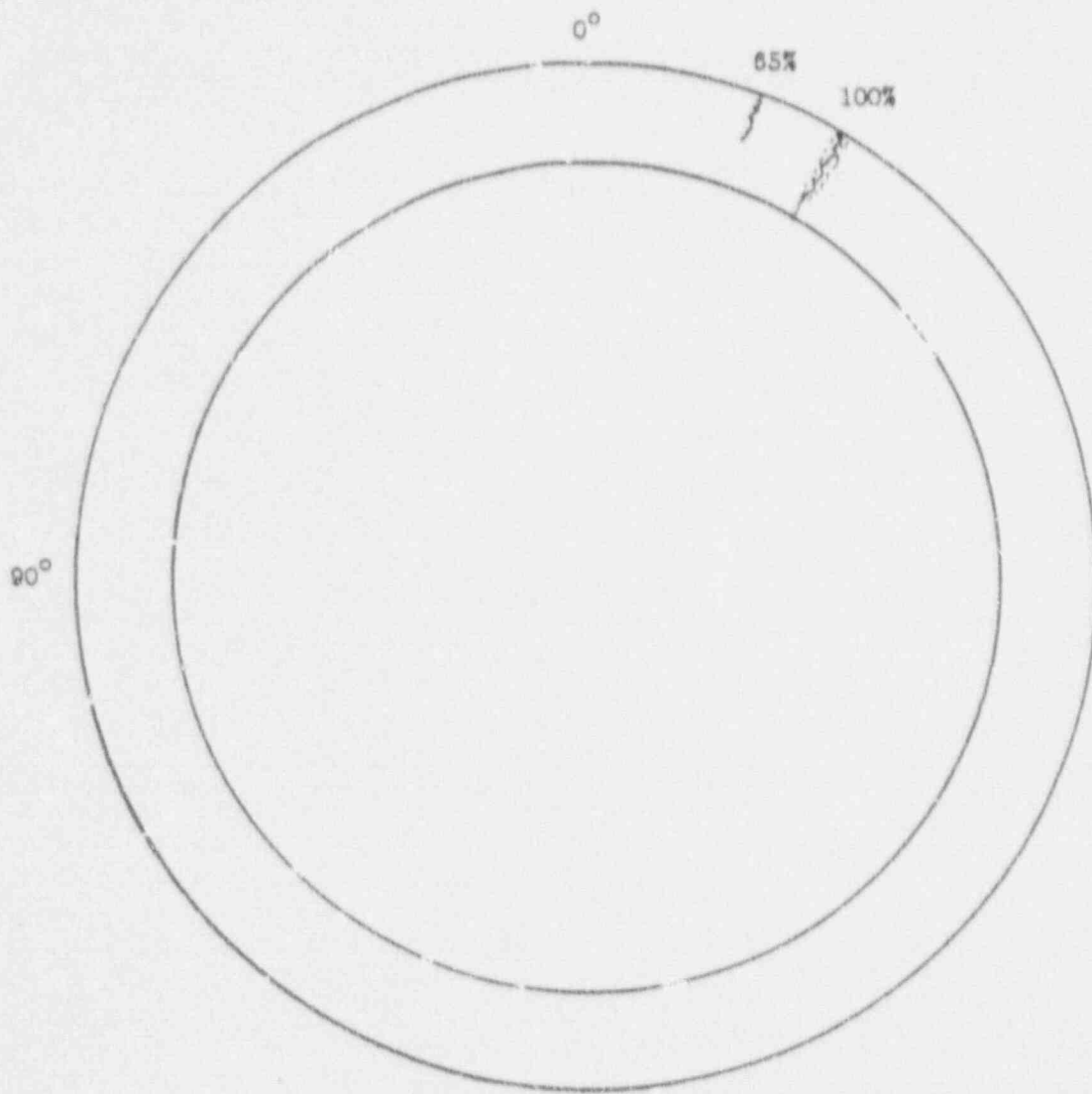
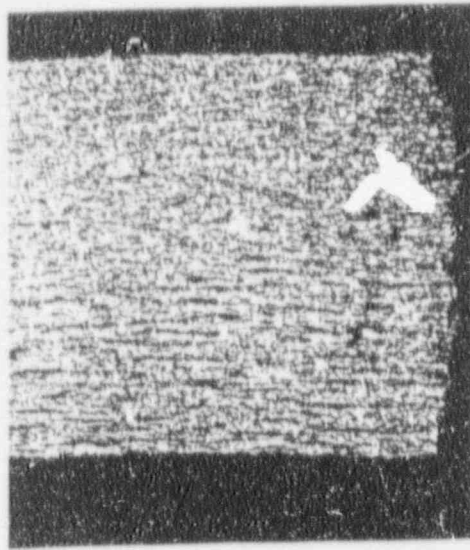
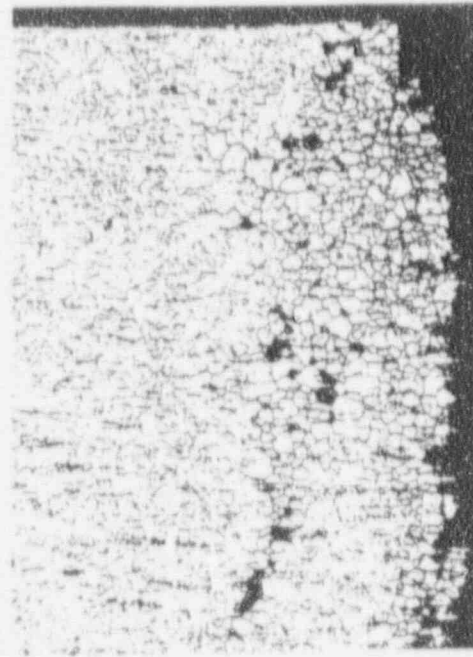


Figure 4 - 13. Sketch of crack distribution and depth within the first support plate crevice region in tube R21-C22.



Mag. 50X

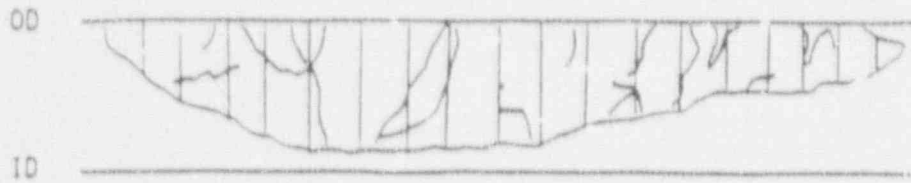


Mag. 100X



Mag. 100X

Figure 4 - 14. Top micrographs are from a transverse section through one half of the main burst crack. The morphology is that of IGSCC with significant IGA characteristics (width of IGA is 0.015 inch on one side of the crack). Bottom micrograph is from a transverse section through the only other crack found in the crevice region. Its morphology is more that of IGSCC. (Note: crack has been opened wide by tube deformation).



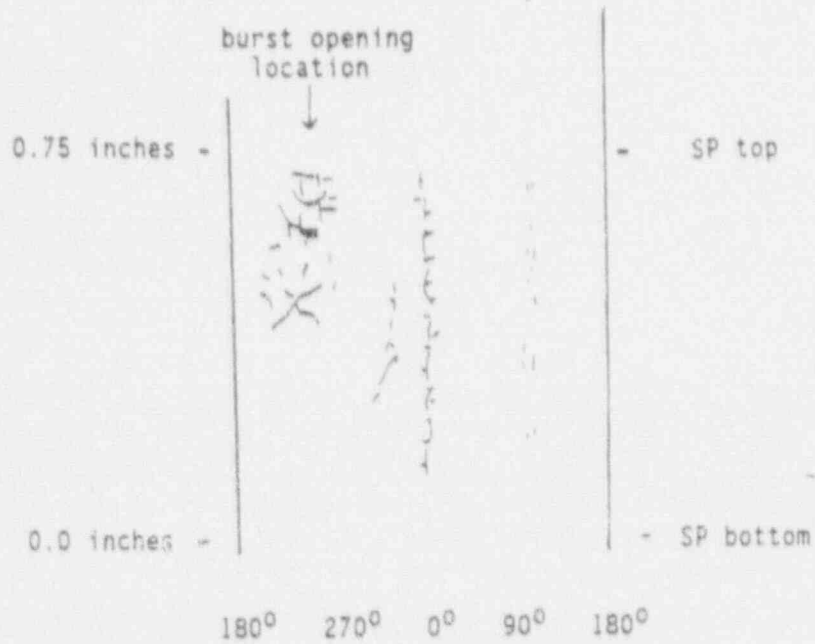
Sketch of Burst Crack

Macrocrack Length = 0.37 inches

Throughwall Length = 0 (78% throughwall)

Number of Microcracks = numerous (ligaments have intergranular features)

Morphology = Intergranular SCC with minor IGA features
(Unusual spider-shaped crack distribution)



Sketch of Crack Distribution

Figure 4 - 15. Description of OD origin corrosion at the first support plate crevice region of tube R38-C46.

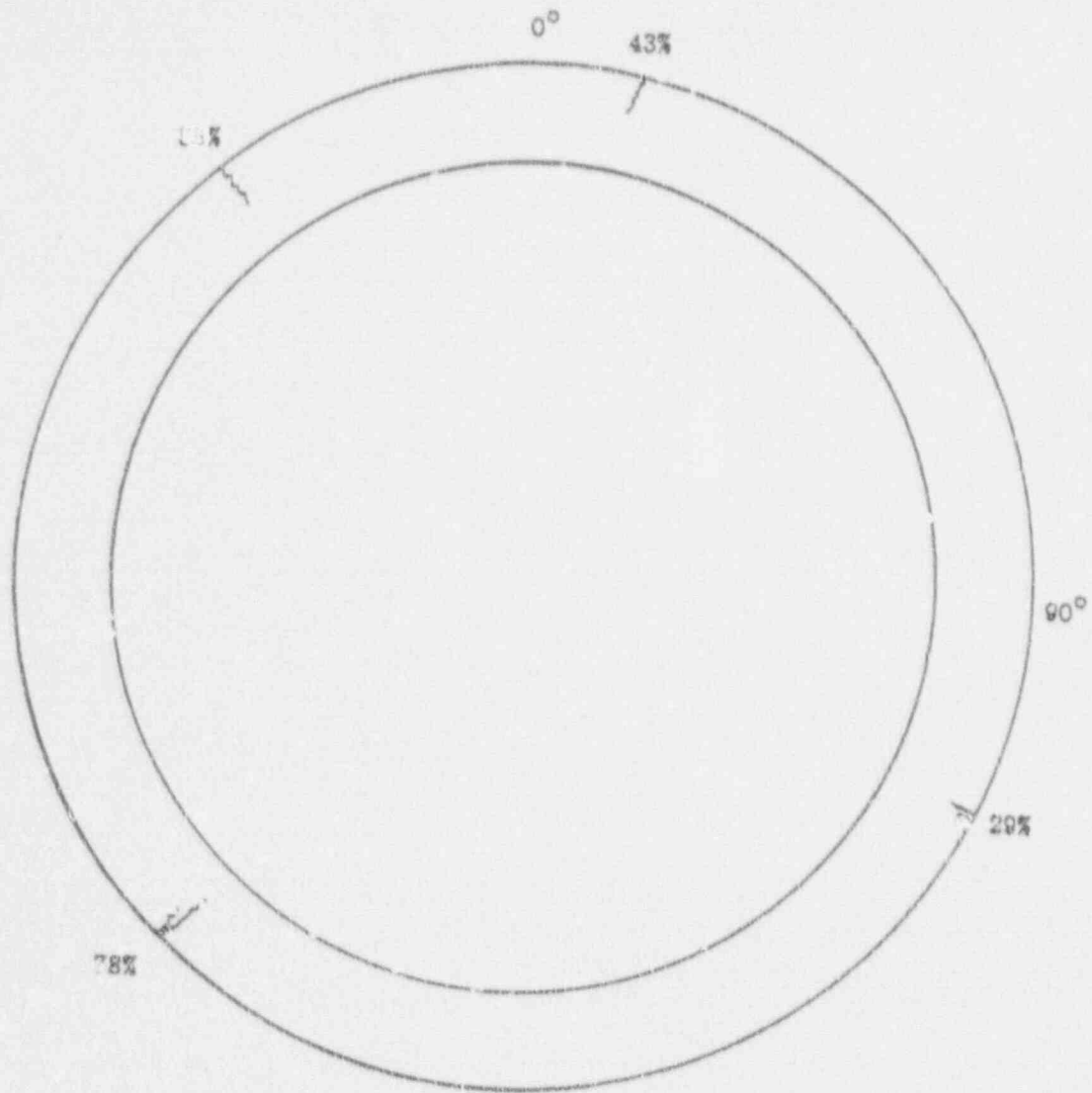


Figure 4 - 16. Summary of distribution and maximum depth of cracks found within the first support plate crevice region of tube R38-C46.



Area B



Area C

Figure 4 - 17. Photomicrographs of a transverse metallographic section through the first support plate crevice region of tube R38-C46. The crack morphology is that of IGSCC with minor IGA characteristics. Mag. 100X.

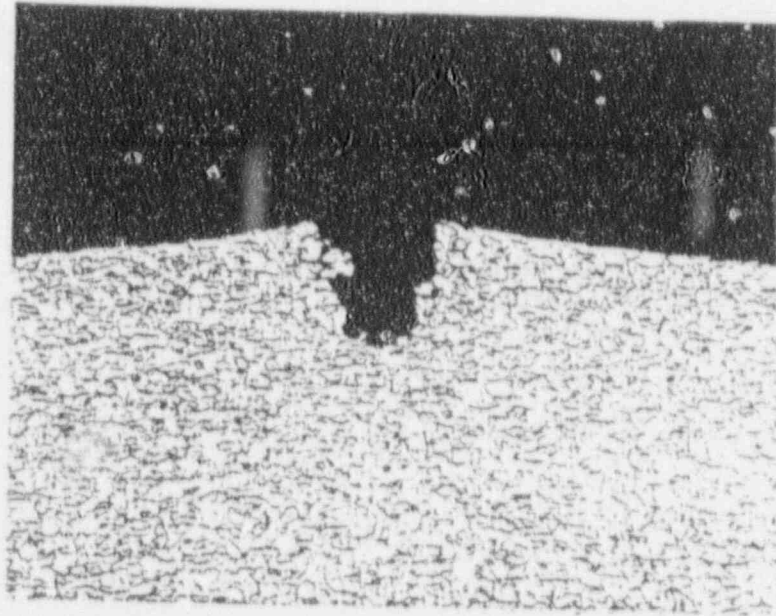
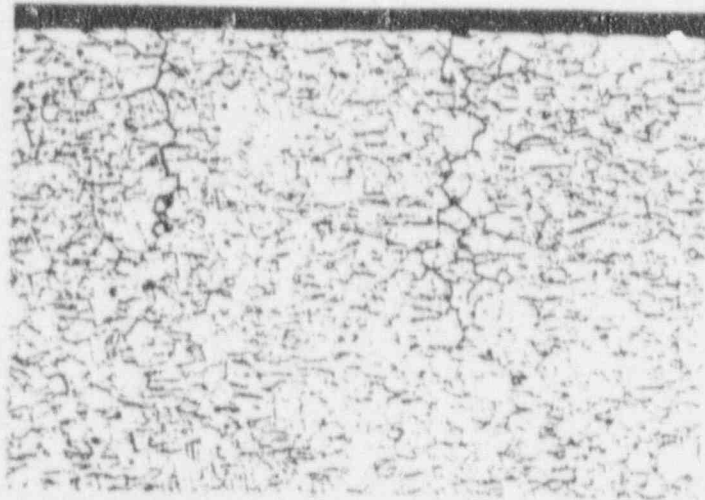
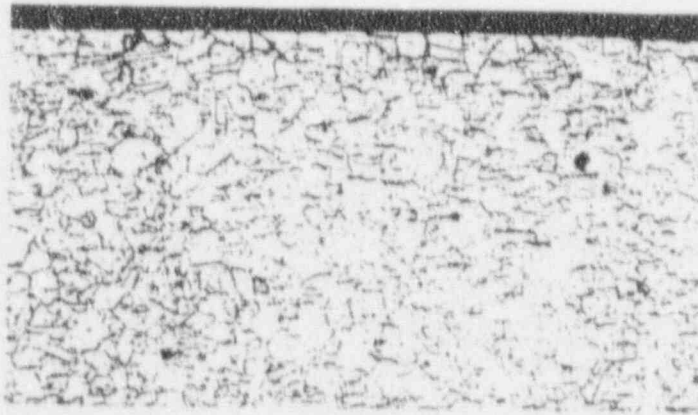


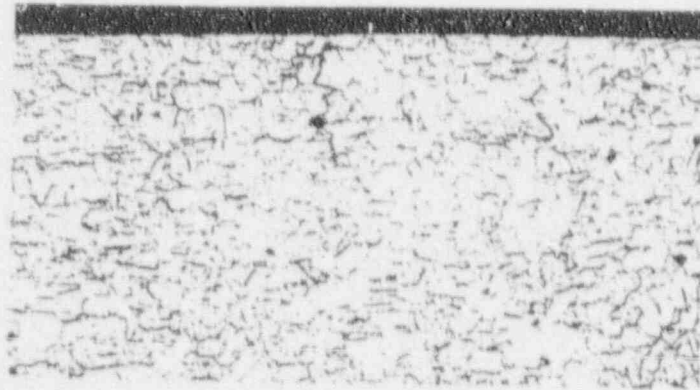
Figure 4 - 18. Cracks at the OD surface of Farley-2 tube R16-C50 at the first tube support plate crevice. Mag. 100X.



H

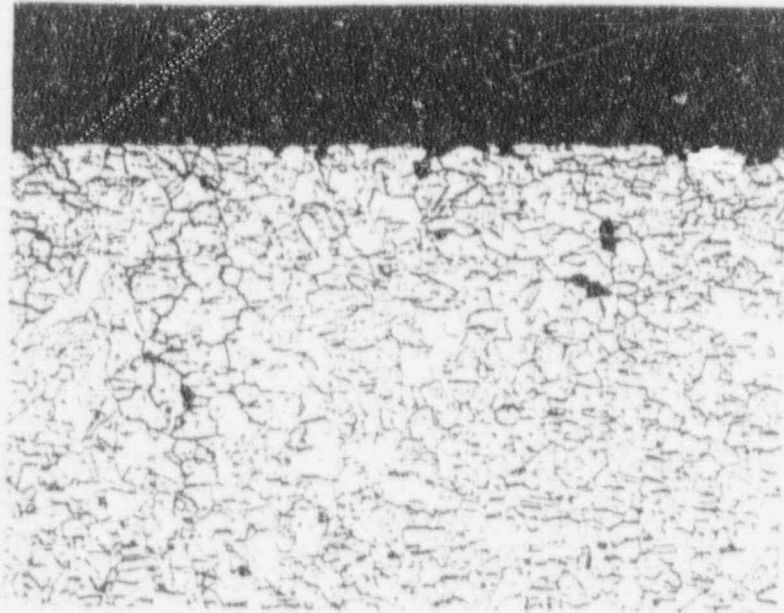


I

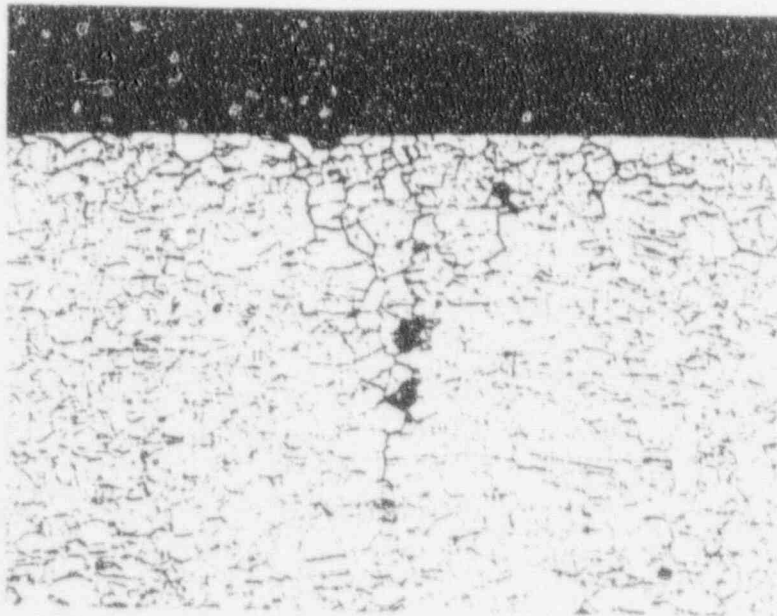


J

Figure 4 - 19. Photomicrographs of a transverse section from the first support plate crevice region of tube R16-C53. Mag. 100X.



f



g

Figure 4 - 20. Additional micrographs from the same transverse section shown in Figure 4-15, tube R16-C53. Mag. 100X.

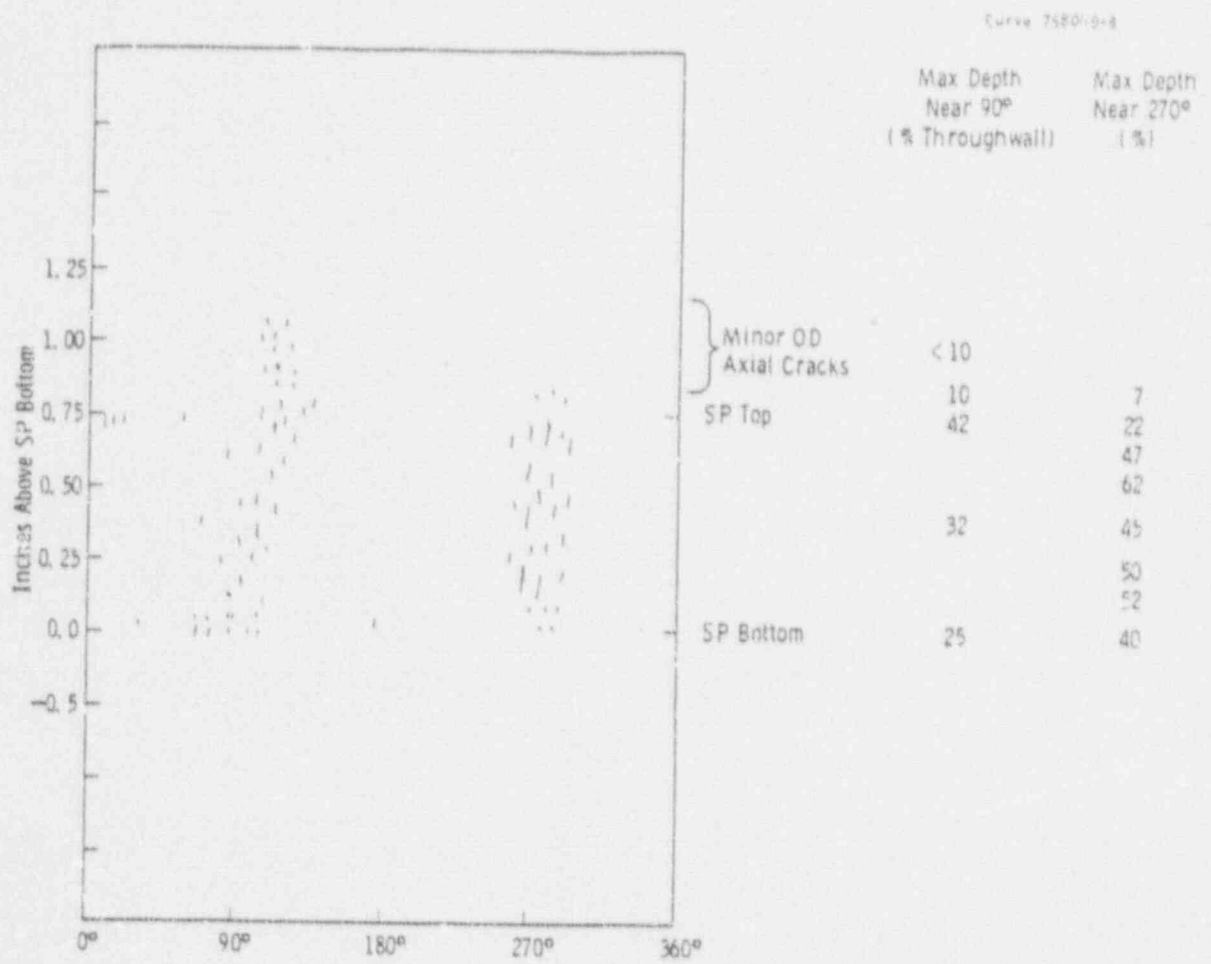


Figure 4 - 21. Crack network location at first support plate region on tube R20-C26 HL.

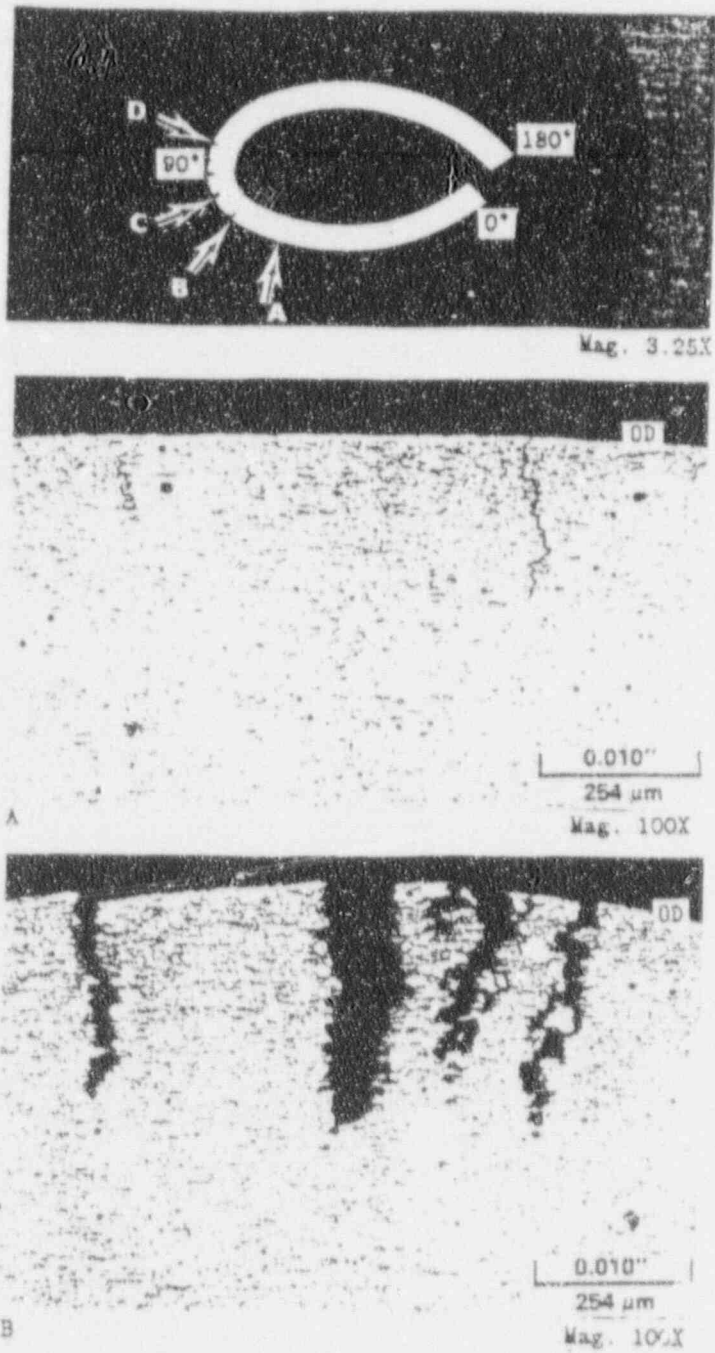
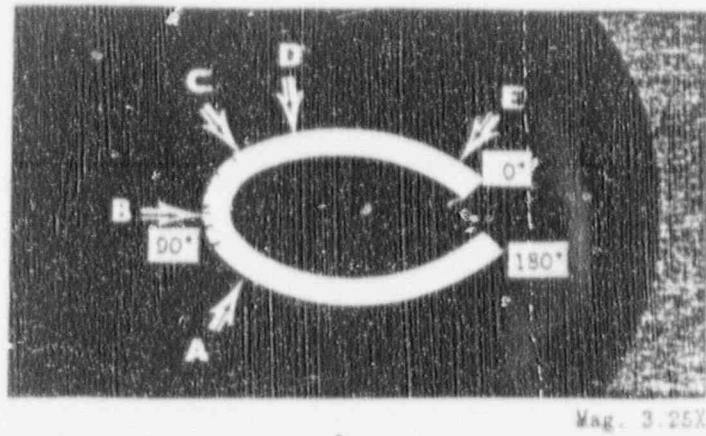
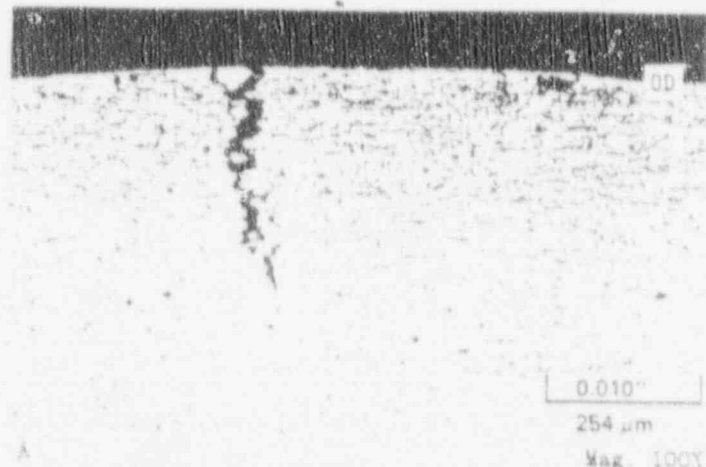


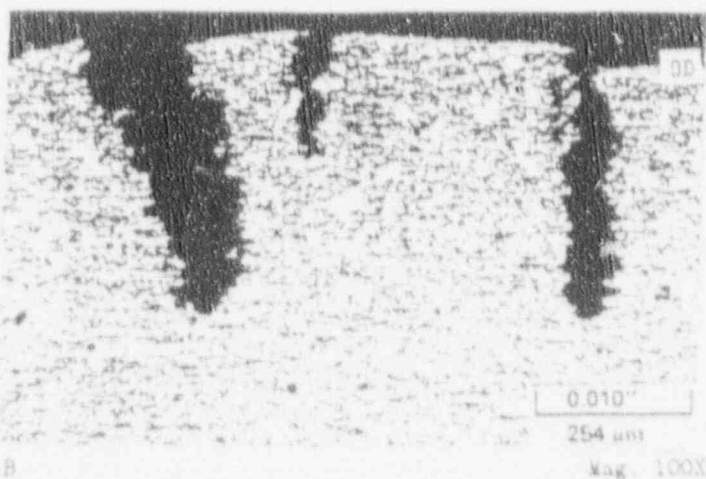
Figure 4 - 22. Transverse metallographic section through tube R20-C26 HL at the mid-point of the first support plate crevice region (90° deformed half) with crack details in Areas A and B.



Mag. 3.25X



0.010"
254 μ m
Mag. 100X



0.010"
254 μ m
Mag. 100X

Figure 4 - 23. Transverse metallographic section through Farley-1 tube R20-C26 HL at the first support plate crevice region (90° deformed half) below the support plate crevice top with crack details in Areas A and B.

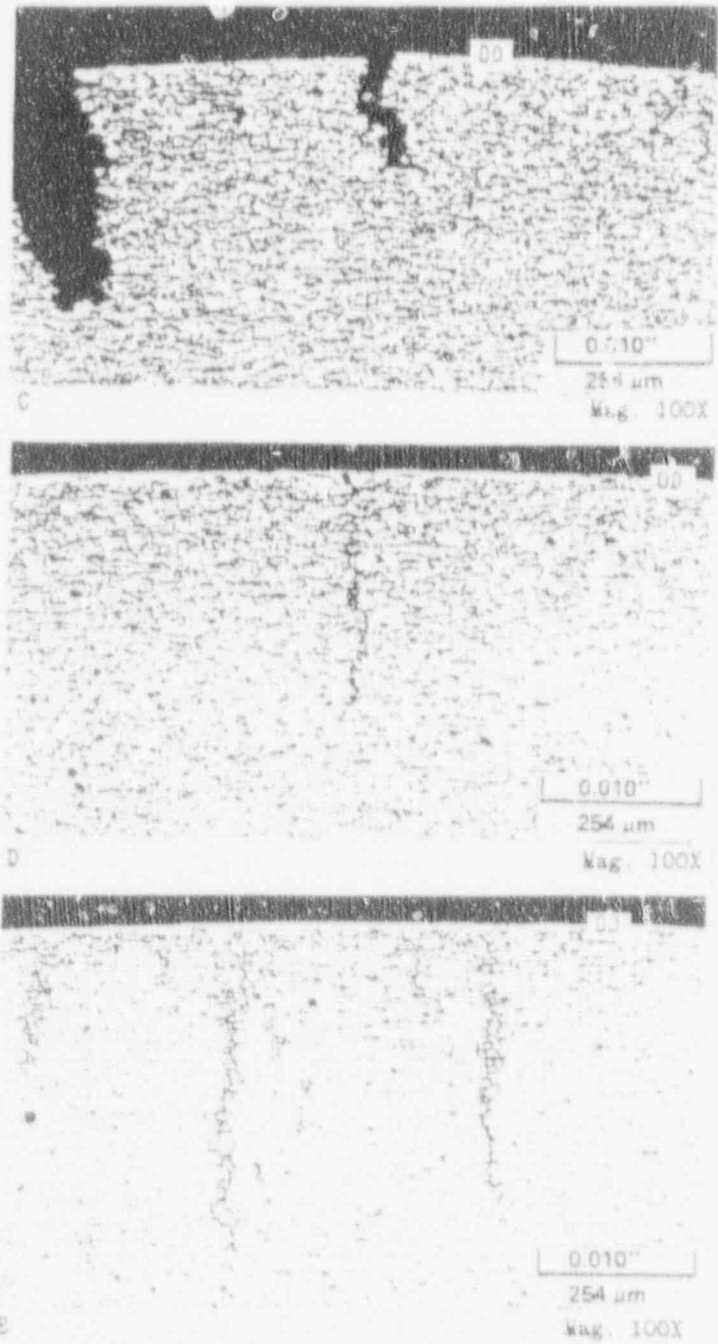


Figure 4 - 24. Crack details in Areas C, D, and E of metallographic cross section shown in previous figure, tube R20-C26 HL, Farley-1.

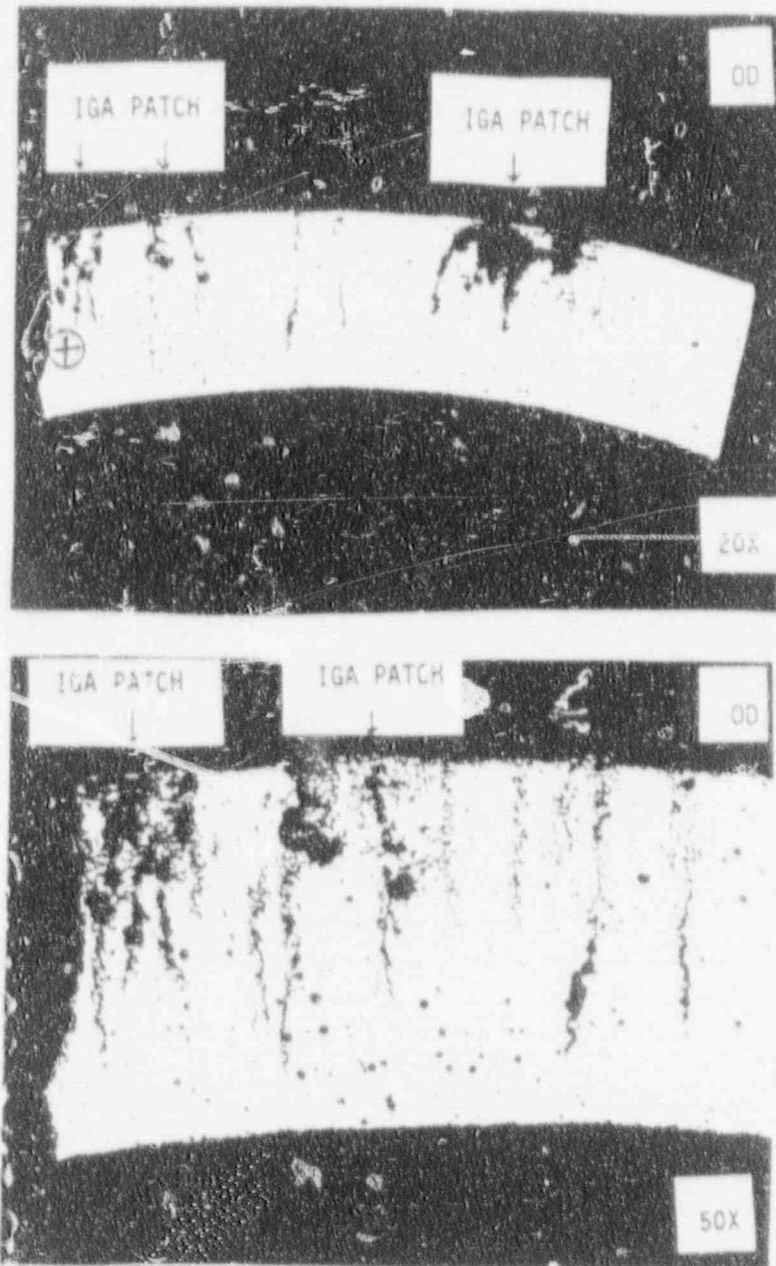


Figure 4 - 25. Transverse optical micrographs obtained just below the circumferential fracture at the center of the support plate. The circumferential location is that where the deepest corrosion was found. The deepest axial IGSCC is 85% through wall and three IGA patches are observed: one 43% through wall and 0.015 inch long, one 33% through wall and 0.05 inch long, and one 28% through wall and 0.015 inch long. The axial IGSCC had IGA aspects to individual cracks. These aspects can be characterized by ratios comparing the crack length (depth from OD surface) to IGA width at the mid-crack location. L/W ratios vary from 6 to 18. Plant L, R12C8

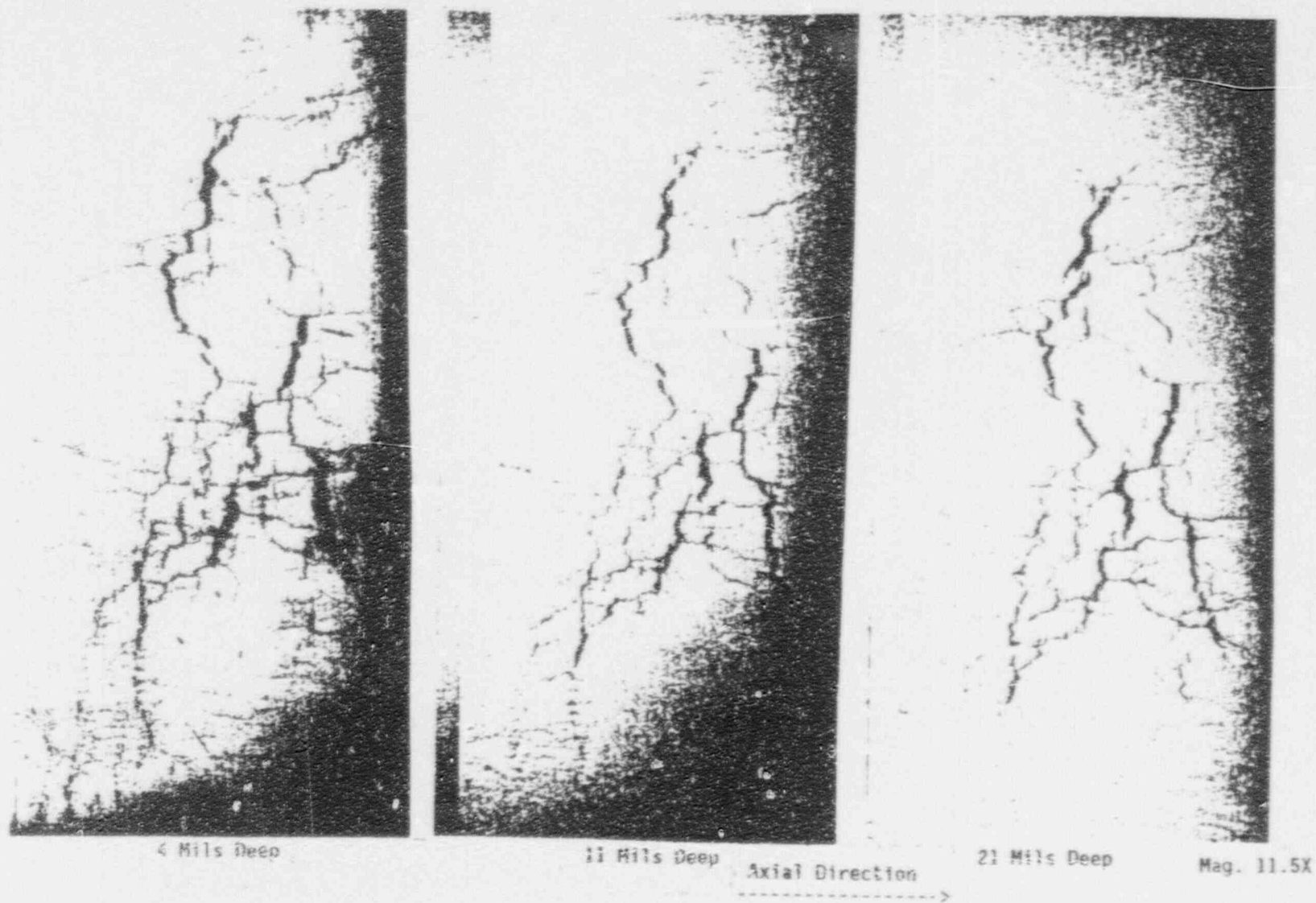


Figure 4 - 26. Photomicrographs of radial metallography performed on a region with axial and circumferential degradation on tube R16-C74, support plate 1. Cellular IGA was found with little change in the cell shape and cell wall thickness at depths of 4, 11 and 21 mils below the GD surface. Note that the cut section was flattened, preferentially opening the circumferential wall of the cells.

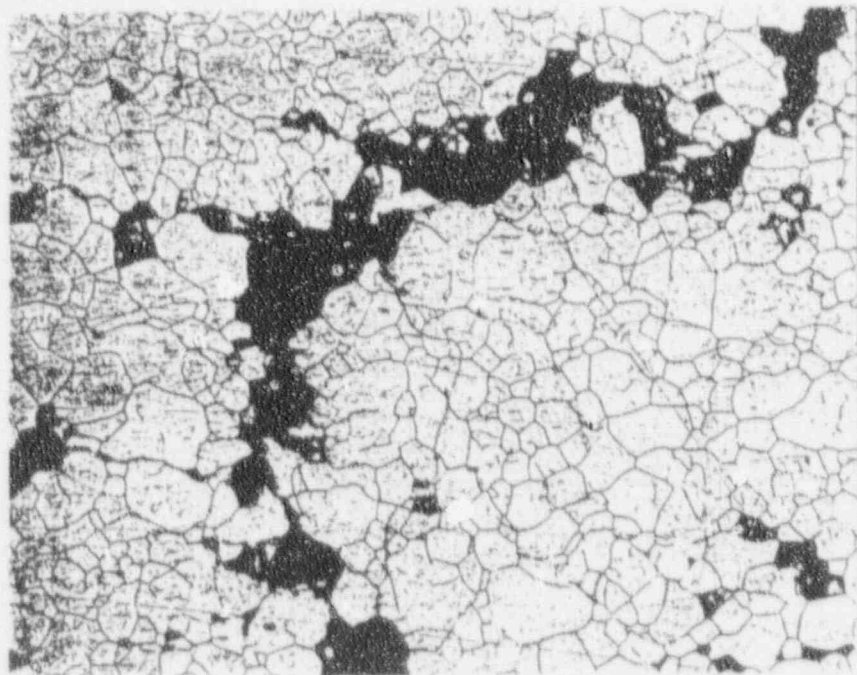
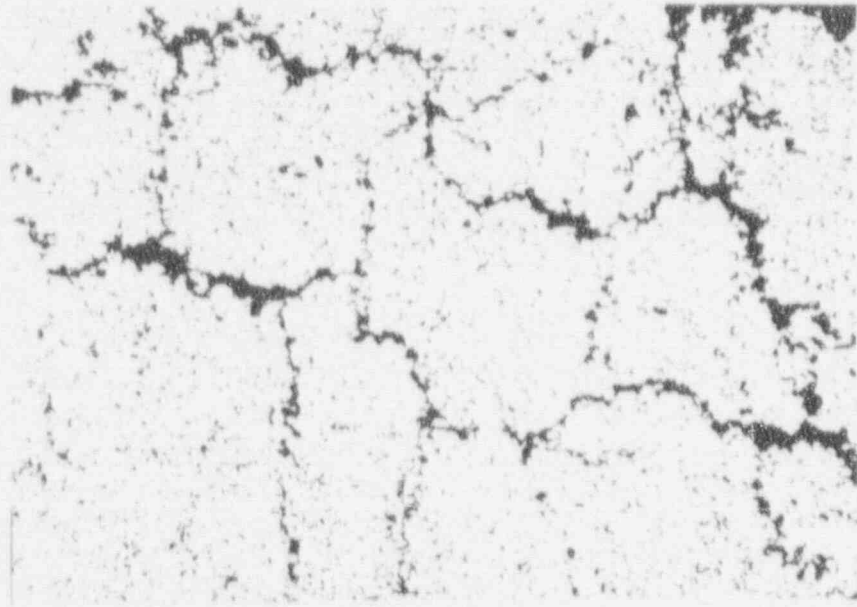
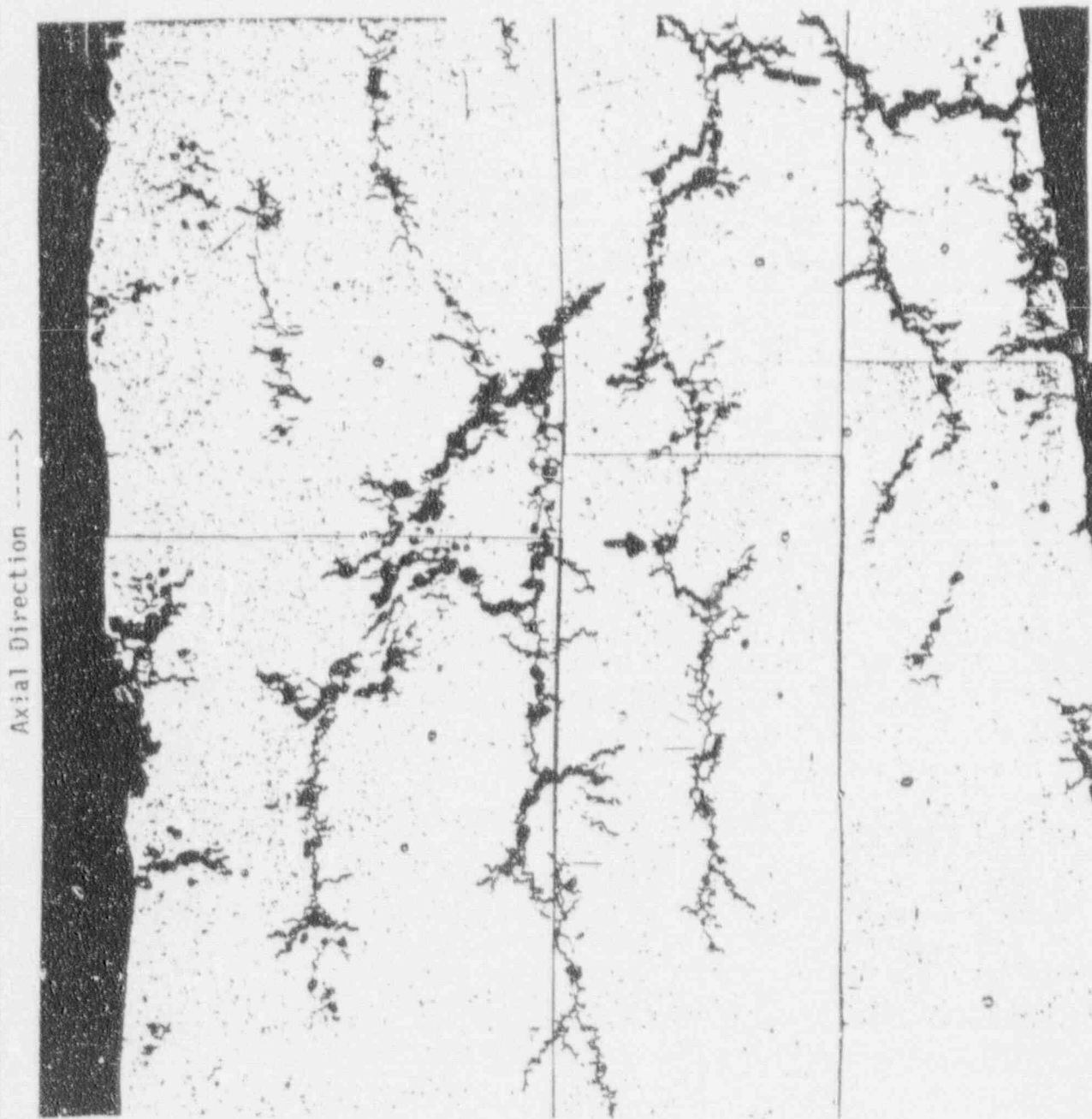
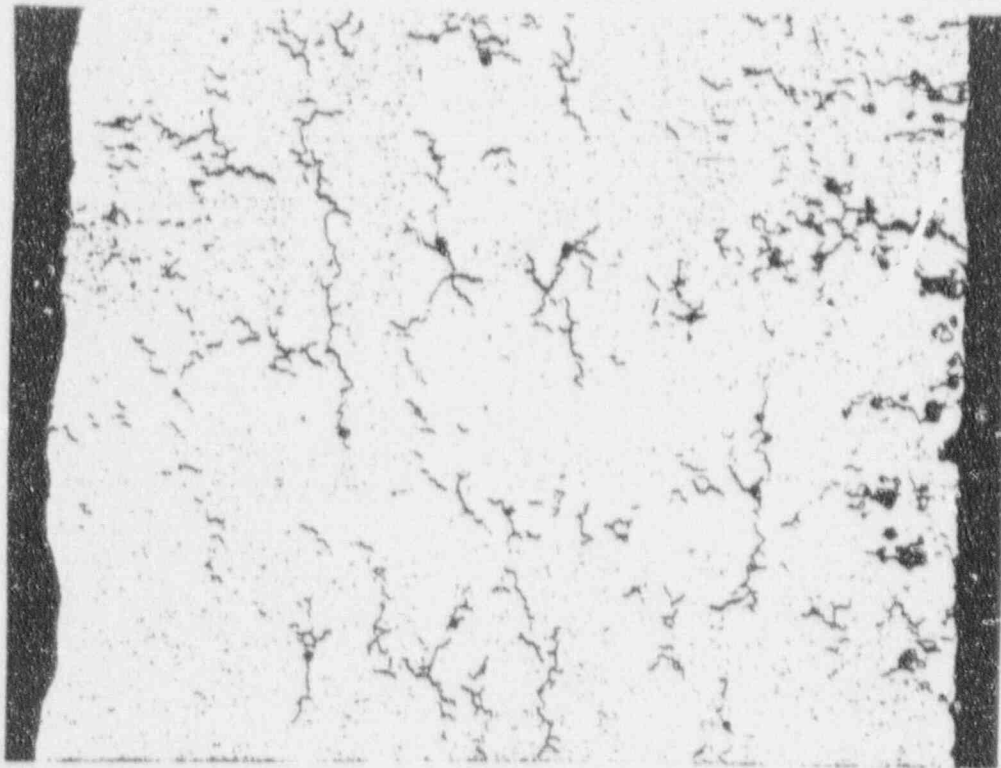


Figure 4 - 27. Higher magnification photomicrographs of the radial section shown in the previous figure. Top photo 50X, bottom 200X



50X

Figure 4-28 Radial metallographic section through a portion of the third support plate crevice region of tube R19-C35 from Plant E-4. A cellular IGA/SCC structure is observed. The depth of the section was not specified.



Axial Direction ----->

50X

Figure 4-29 Radial metallographic section through a portion of the fourth support plate crevice region of tube R19-C35 from Plant E-4. A cellular IGA/SCC structure is observed. The depth of the section was not specified.

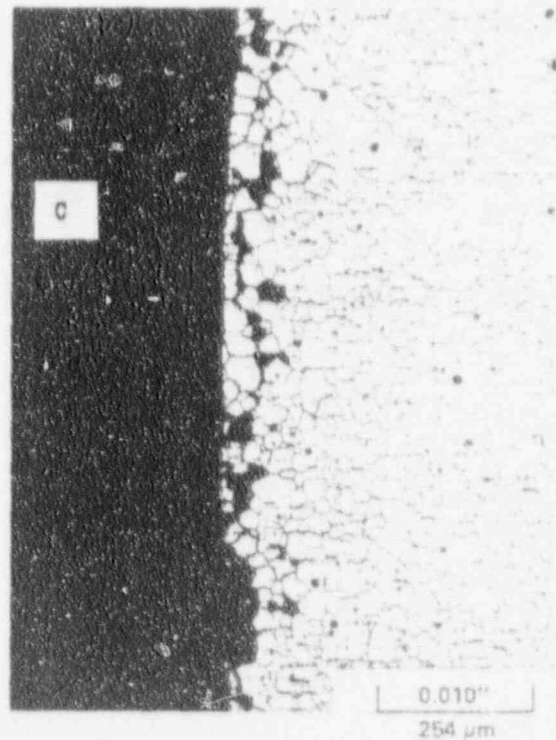
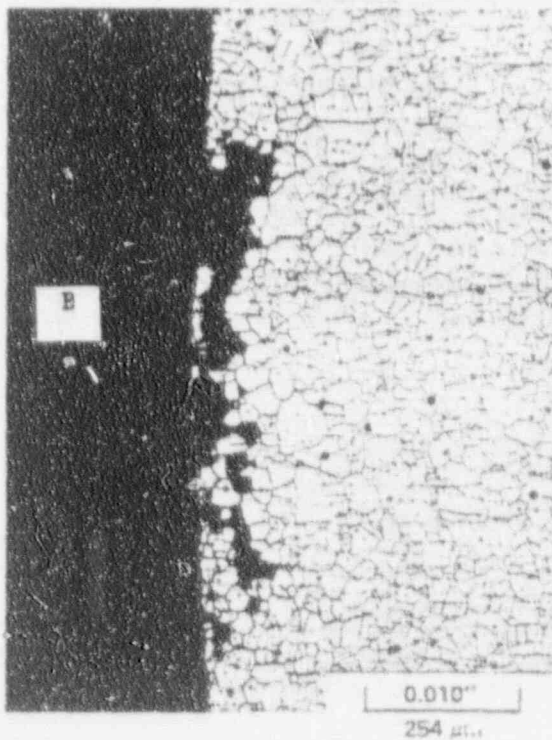
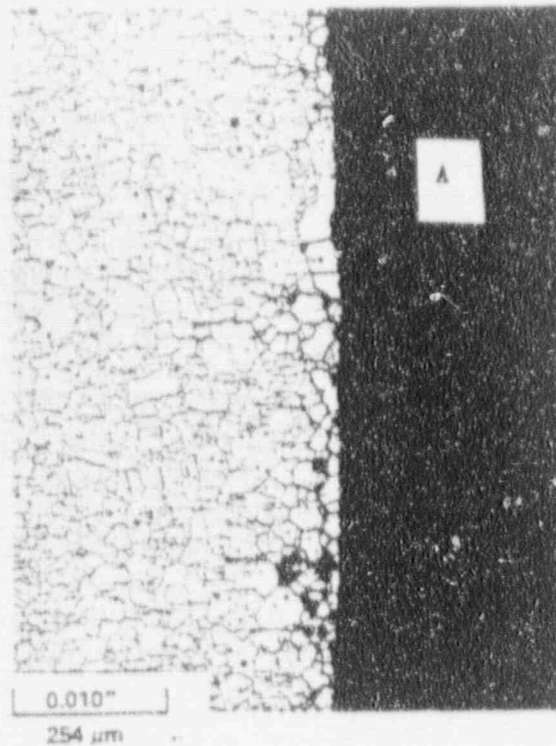
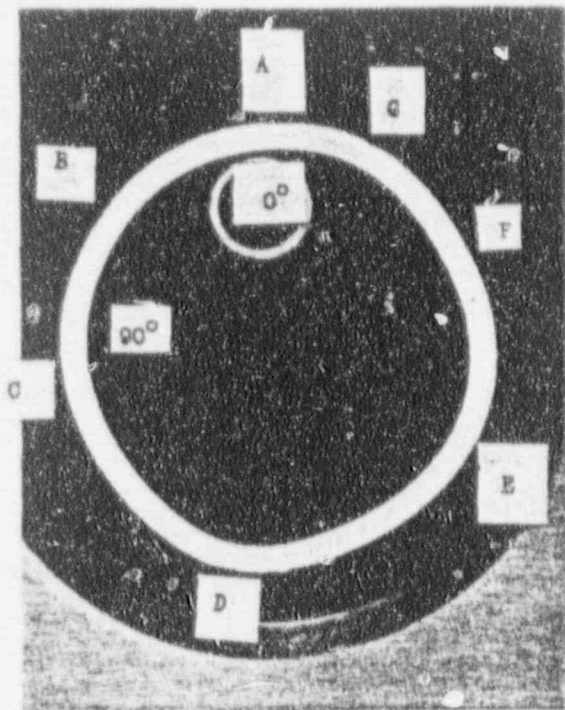
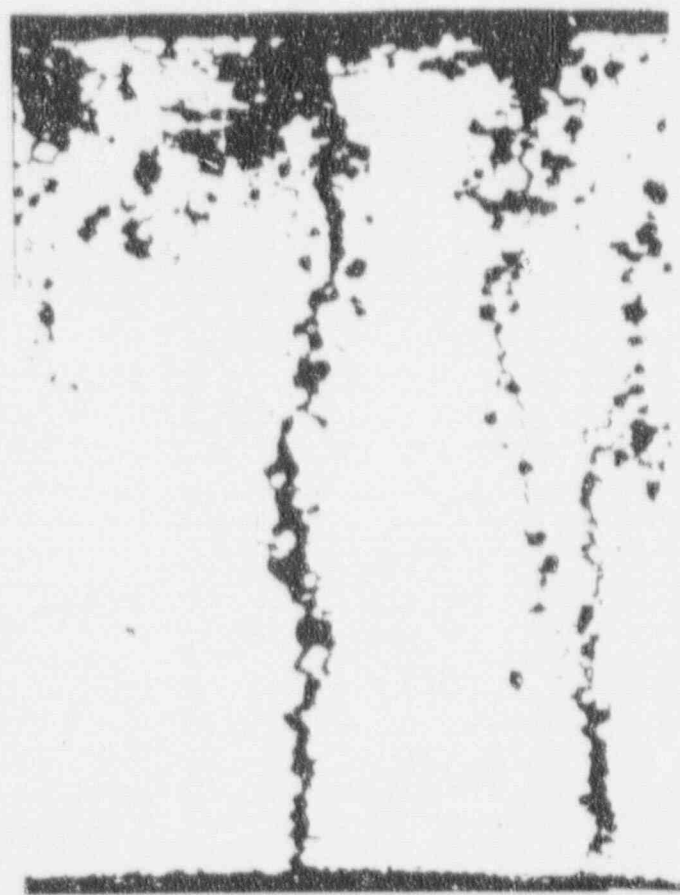


Figure 4 - 30. Transverse metallography at 0.2 inches above first support plate bottom edge showing almost continuous OD IGA around circumference. Maximum depth is 24%.

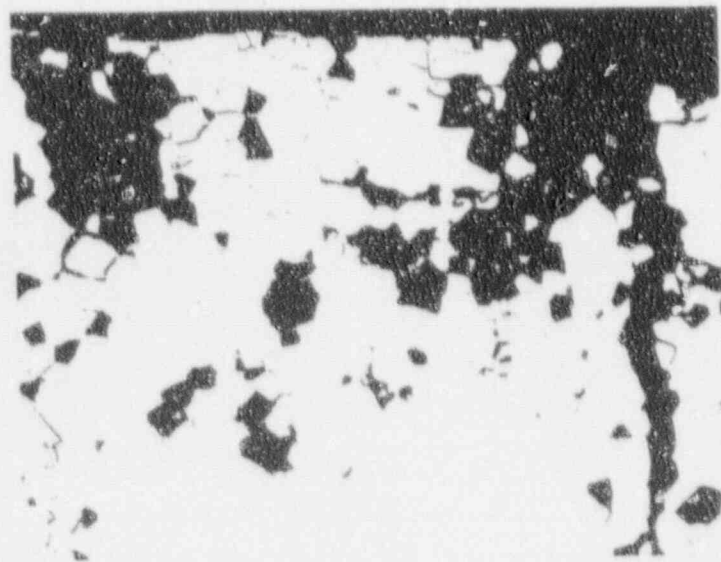
Tube R29-C46 CL, Plant M-2



17X



85X



170X

Figure 4 - 31. Transverse photomicrographs of intergranular corrosion at the first support plate region of tube L8-C74 from Plant J-1

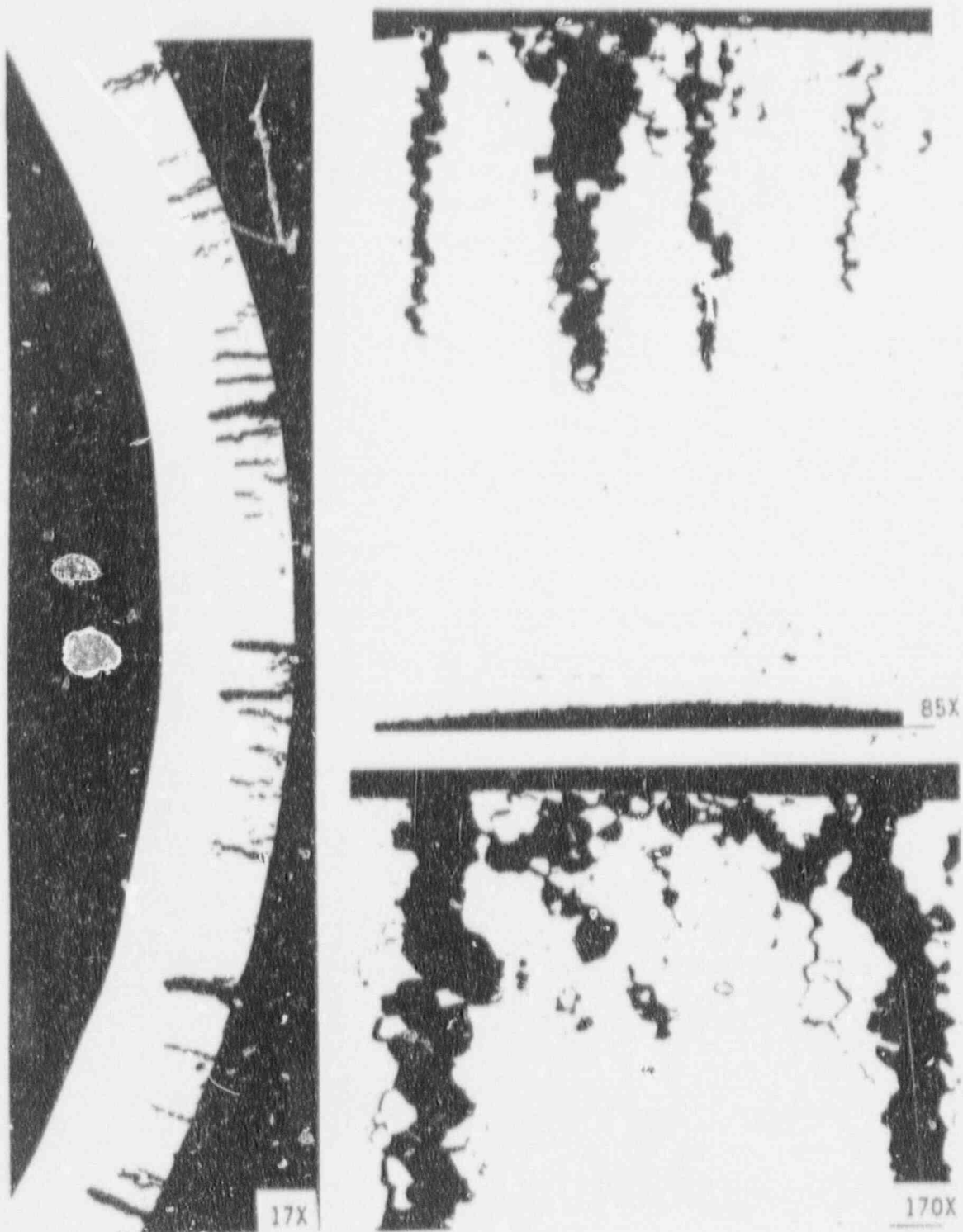


Figure 4 - 32. Transverse photomicrographs of intergranular corrosion at the second support plate region of tube LB-C74 from Plant J-1

5.0 FARLEY EDDY CURRENT INSPECTION RESULTS

5.1 Farley Unit-1, March 1991 Inspection

The scheduled program of EC inspections performed at Farley Unit-1 in the 10th refueling outage included full length bobbin probing of 100% of the available tubes. Supplemental inspection was performed with rotating pancake coil (RPC) probes to characterize distorted indications identified at the TSP elevations. A summary of the inspection results is given in Table 5.1.

The TSP bobbin EC indications were observed randomly across the tube bundle with SG-A having 194, SG-B 135 and SG-C 219. The bobbin indications were further characterized using RPC probes; if the data revealed crack tube behavior, then the tube was removed from service by plugging.

The total tube plugging during the outage was 265, with 97 being plugged for ODSCC indications at the support plates, 43 for indications at the top of the tube sheet, 76 for PWSCC in the WEXTX region, and 49 for U-bend PWSCC.

In a manner similar to Unit-2 (see Section 5.2 below), prior eddy current data (1988 and 1989 inspections) were reviewed to assess the progression of ODSCC indications at the TSPs. The results of this re-evaluation show that the rate of progression of ODSCC at the support plates is very slow. During this re-evaluation, percent through wall estimates were made for the distorted indications. Over the last two operating cycles, the average growths in indicated through wall penetration were 5% in 1988-89 (with a standard deviation of 16%) and 6% in 1989-91 (with a standard deviation of 24%); see Figure 5-3.

Histograms depicting the distribution of signal amplitudes for distorted indications for each SG are given in Figure 5-1; Figure 5-2 illustrates the axial distribution of the TSP indications. The dependence of voltage change on observed amplitude is displayed in Figure 5-12 for the data recorded in both the 1989 and 1991 inspections. Voltage growth rate data are discussed in section 5.3.

It may be noted that the growth rates during the last two cycles have been very small, both in depth of wall penetration and in bobbin coil amplitude. This is attributable to the improved secondary system chemistry operation during the recent cycles. It is believed that ODSCC initiated as a consequence of the chemistry excursions which occurred during the 1985-86 cycle. Inhibiting effects of boric acid treatment and the improved secondary side chemistry control have resulted in low nucleation rate and reduced progression of ODSCC. This is reflected in the very small growth rate of ODSCC observed at the TSP intersections.

5.2 Farley Unit-2, October 1990 Inspection

The Cycle 7 refueling outage provided the last Farley Unit-2 steam generator inspection in October, 1990. Bobbin probe EC testing was performed full length on 100% of the tubes in service; all TSP indications greater than 40% in estimated depth of wall penetration as well as all distorted indications of any amplitude were subjected to MRPC

testing to assist in characterizing the extent and nature of the degradation. Tubes found to exhibit degradation suggestive of ODSCC were plugged; this resulted in plugging 244 tubes, 30 in SG-A, 64 in SG-B and 150 in SG-C. Estimation of the growth in tube wall penetrations during Cycle 7 yielded 0.6% for SG-A, 2.3% for SG-B and 3.6% for SG-C. The composite changes for the three SG's combined was -3.3° in phase angle and -2.8% depth of tube wall increase for the last cycle, based on evaluation of 327 signals. Only 14 of the 327 indications reported for Farley-2 exceeded 1.75 volts in amplitude.

Along with the Farley-1 data, Table 5.2 displays a breakdown of the EC indications reported for the Farley-2 SG TSPs in the October 1990 inspection. It is noted that all indications $\geq 40\%$ as well as almost all the distorted indications (208 of 210) were reported on the hot leg side of the tube bundle. Though, 248 of the 308 hot leg indications reprobbed with the MRFC were verified to exhibit signs of degradation, the bobbin amplitude of these signals was below 1.75 volts for at least 94% of them. Figure 5-4 summarizes the axial distribution of the TSP indications.

Figure 5-5 displays the voltage levels recorded for all TSP indications in Farley-2. A discussion of voltage growth is given in Section 3.3.

In order to provide a definitive understanding of the Farley-2 historical context concerning TSP ODSCC, a re-evaluation of the prior EC tape records has been performed beginning with each indication reported in the 10/90 inspection, working back in time for these indications while adding in the previously plugged tubes and their prior tapes; all these records were analyzed using the EC interpretation guidelines employed in the field in 10/90, so that a normalized or rationalized data base could be constructed. Growth rates, more correctly change rates, were developed over four operating cycles from 1985 to 1990. The phase angle changes for the four cycles are displayed in Figure 5-6, from which it can be seen that only a slight negative shift in average phase angle has occurred since 1985. For conservatism, summing only the negative average phase shifts since 1985 yields -7.4° over four cycles, or approximately 6% total growth in equivalent depth of tube wall penetration over that four cycle, 5 year time period.

It must be recognized that these changes reflect the behavior of tubes which in large part were affected by the inhibiting effects of boric acid treatment after 1986. The tubes plugged in 1986 were those that exhibited an enhanced sensitivity to caustic attack, resulting in EC indications greater than the plugging limit. However, a second population of tubes was also affected. The data obtained during the reevaluation demonstrate that the rate of progression of the EC signals is very slow. Further, the statistics suggest that the nucleation rate in the presence of boric acid is very small. In short, the tube plugging observed can be attributed to a fixed population.

5.3 Voltage Growth Rates for Farley SGs

This section summarizes the evaluation of voltage growth rates, including historical trends, for Farley Units 1 and 2. The growth rate data are utilized in Section 1.2 to develop the tube plugging limits.

Table 5.4 shows the historical data on voltage growth per cycle for both Farley units. Percent growth rates are given as the increase in voltage amplitude over an operating

cycle relative to the beginning of cycle amplitude. Average growth is given as the direct average of all data and as the conservative average obtained by setting measured negative growth changes to zero in defining the average. Also shown are the number of indications contributing to the averages and the average voltage amplitudes for all indications. The standard deviations are also given as an indicator of voltage variability. However, as noted below, the growth distributions are not representative of a normal distribution so that cumulative probability distributions are provided for analyses such as SLB leakage projections that incorporate growth rate uncertainties.

Figures 5-10 and 5-11 show scatter plots of percent growth versus the BOC (beginning of cycle) amplitudes for the last two operating cycles. Both units show a decrease in percent growth with increasing amplitude with the weighted averages, shown as solid lines, tending toward negative values at BOC amplitudes above about 0.75 volt. The negative growth values typically result from changes in calibration standards at the analysis guidelines recognizing that no major effort was applied to achieve consistency in voltage evaluations. Negative growth can be assumed to represent zero growth for determining average growth. This is conservative since the random, negative fluctuations are ignored while positive fluctuations are retained in computing averages. The scatter in growth values is much larger at low amplitudes as expected since these amplitudes are near the detection threshold where measurement accuracy is less reliable than for larger indications.

The data also show a few growth values considerably above the overall trend. Most of these occur at low BOC amplitudes where modest growth can significantly increase amplitudes and enhance detectability. Since a relatively large number of indications are included in the database, the frequency of the outliers in growth can be considered to be representative of the population and conservative when extrapolated to growth trends at higher voltage levels such as for statistical SLB leak rate analyses. Figures 5-12 and 5-13 show histograms and cumulative probability curves developed from the data of Figures 5-10 and 5-11. Figures 5-14 and 5-15 show the cumulative growth probability expressed as growth per EFPY and as growth per cycle, respectively. Figures 5-14 and 5-15 are appropriate for SLB leakage and burst analyses as discussed in Section 12.4. Growth per EFPY is appropriate if cycle lengths change, while growth per cycle can be applied if cycle lengths are not varied as is common at Farley. As can be seen in Figure 5-15, the growth trends do not follow a normal distribution at the large ΔV 's. For this reason, cumulative distributions such as Figure 5-15 are used for the SLB analyses required by the Section 12 repair criteria. Use of standard deviations to project growth to EOC conditions would be non-conservative compared to the actual cumulative probability distributions. For example, more than four standard deviations are required to achieve the large ΔV tails of the actual distributions of Figure 5-15.

The larger growths of Figure 5-15 tend to occur for BOC voltages less than about 1.3 volts. The trend for larger growth at low voltages can be seen in Figures 5-11 and 5-12. Upon implementation of the repair criteria of this report including voltage calibration standards, the voltage growth rates for SLB analyses would be based upon the larger population of the largest 200 BOC voltages or the BOC voltages within about 1.5 volts of the tube plugging voltage limit.

Figure 5-16 shows the historical average growth rate trends for the Farley units. Shown are the overall average of all data as well as averages for < 0.75 and > 0.75 volt

BOC amplitudes. The data show the strong trend for reduced percent growth at higher amplitudes. For conservatism in establishing the plugging criteria, the overall average growth rates are used to project growth at higher amplitudes in Section 12 to develop the voltage-based tube plugging limits. Section 6.7 compares voltage growth trends from other available plant data with the Farley data.

5.4 Denting and TSP Corrosion Review

The condition of the Farley steam generator tube support plates with respect to the incidence of denting has previously been regarded as in the minor, nearly negligible, category. However, during investigations of the condition of the R20-C26 tube at the first hot leg TSP (tube sample from Farley-1 taken in October 1989), the difficulty in identifying the presence of the 62% ODSCC found in the laboratory from the field EC inspection was attributed in part to interference from dents. Since the influence of TSP corrosion and denting in limiting displacement of the TSP's during accident conditions may be substantial, a review of selected portions of the Farley steam generator tube bundles was undertaken.

Hot leg support plate elevations in all SG's exhibit only small numbers of tube deformation (dent) signals, but examination of the TSP signals at low frequency (10 kHz) shows positive effects of corrosion product buildup in the TSP annuli; this effect is attributed to magnetite formation which accompanies the incipient stage of denting. Dent-related corrosion as defined in NRC Bulletin 88-02 was evaluated for tubes in regions not close to the stay rods. The cold leg portions of the tube bundle show less influence of the presence of TSP corrosion, due to the reduced corrosion rates associated with lower tube temperatures. For example, Figure 5-7 summarizes the results of the EC review for Farley-1 in the cold legs of the longest tubes of each steam generator. Table 5.3 displays the data for all regions of SG-A.

Thus the existence of TSP corrosion in the Farley-1 steam generators is confirmed. An evaluation of TSP corrosion was also performed for the Farley-2 SG-A as given in Table 5.4. TSP corrosion has been confirmed for this SG and it can reasonably be assumed that the other SGs in Unit-2 also have similar TSP corrosion.

5.5 RPC Data

Although TSP/ODSCC indications detected by the bobbin coil tend to exhibit complex lissajous patterns in the Farley steam generators, characterization of the degradation at these locations with the rotating pancake coil probe (RPC) often clarifies the dimensions and distribution of the major crack features. A selection of four (4) such RPC characterizations from each unit are provided in Figures 5-8 and 5-9. For Farley-1 the figure exhibits the data for the pulled tube (SG-C: R20-C26) and the tubes plugged for TSP/ODSCC in November 1989, SG-B: R12-C3 and SG-B: R31-C50; similarly the Farley-2 examples include two tubes pulled in November 1990 SG-B: R4-C73 and SG-B: R21-C22 as well as two with large bobbin signal voltages plugged during the same outage (SG-C: R38-C65 and SG-C: R40-C43).

Table 5.1

Summary of EC Indications in Last Inspection of Farley SGs

	SG-A		SG-B		SG-C	
	HL	CL	HL	CL	HL	CL
UNIT 1 (MARCH 1991)						
Bobbin Signals						
<20% Depth	0	0	0	1	0	0
20-29%	0	0	0	2	0	1
30-39%	0	0	0	0	0	1
40-49%	0	0	0	0	0	0
50-59%	2	0	2	0	0	0
60-69%	2	0	1	0	1	0
70-79%	2	0	2	0	1	0
80-89%	4	0	0	0	0	0
90-100%	0	0	0	0	0	0
Distorted	19	0	126	0	208	0
RPC Results						
Degradation Verified	0	0	24	0	20	0
Tubes Plugged for ODSCC Indication	56		24		18	
UNIT 2 (OCTOBER 1990)						
Bobbin Signals						
<20% Depth	5	2	2	4	1	0
20-29%	7	2	0	2	1	1
30-39%	3	2	5	0	4	1
40-49%	1	0	1	0	6	0
50-59%	1	0	4	0	11	0
60-69%	1	0	8	0	17	0
70-79%	4	0	9	0	23	0
80-89%	1	0	4	0	8	0
90-100%	0	0	0	0	0	0
Distorted	40	0	54	2	114	0
RPC Results						
Indications Probed	68	0	81	2	179	0
Degradation Verified	31	0	66	0	151	0
Tubes Plugged for ODSCC Indication	29		64		147	

Table 5.2

TSP Corrosion Evaluation
Farley-1 Steam Generator A
October 1989

Region	H/C		ISP1	ISP2	ISP3	ISP4	ISP5	ISP6	ISP7
	Leg								
Periphery Perpendicular to Tubelane	Hot	Tubes Examined	49	49	49	49	49	49	59
	Leg	% Corroded Crevice	100	100	100	100	100	100	100
	Cold	Tubes Examined	98	70	70	70	70	70	70
	Leg	% Corroded Crevice	10.2	24.3	11.4	40.0	81.4	97.1	91.4
Periphery at High Columns	Hot	Tubes Examined	63	63	63	63	63	63	63
	Leg	% Corroded Crevice	77.8	83.3	97.2	100	100	100	100
	Cold	Tubes Examined	50	50	50	50	50	50	50
	Leg	% Corroded Crevice	77.8	86.1	100	100	100	100	100
Periphery at Low Columns	Hot	Tubes Examined	36	36	36	36	36	36	36
	Leg	% Corroded Crevice	80.9	82.5	92.1	100	100	100	100
	Cold	Tubes Examined	36	36	36	36	36	36	36
	Leg	% Corroded Crevice	28.0	66	94	100	100	100	100

Table 5.3

TSP Corrosion Evaluation
 Farley-2 Steam Generator A
 October 1990

Region	H/C		ISP1	ISP2	ISP3	ISP4	ISP5	ISP6	ISP7
	Leg								
Periphery Perpendicular to Tubelane	Hot	Tubes Examined	20	20	20	37	60	102	150
	Leg	% Corroded Crevice	90	95	95	100	95	88.2	94
	Cold	Tubes Examined	20	20	20	37	82	90	150
	Leg	% Corroded Crevice	75	95	95	86.5	72	81.8	90.7
Periphery at High Columns Parallel to Tubelane	Both	Tubes Examined	34	35	36	151	151	163	163
	Legs	% Corroded Crevice	79.4	88.6	88.9	96.7	98.7	99.4	89.6
Periphery at Low Columns Parallel to Tubelane	Both	Tubes Examined	37	37	37	66	139	138	143
	Legs	% Corroded Crevice	70.3	62.2	59.5	82.5	94.6	93.9	97.3

Table 5.4

Voltage Growth Per Cycle for Farley Units 1 and 2 (1)

Unit / Cycle	Number of Indications	Average Voltage	% Growth/Cycle		% ≥ 0 Growth/Cycle	
			Average	Std. Dev.	Average	Std. Dev.
Farley-1 Cycles						
1985 to 1986	123	0.45	45%	72%	50%	67%
1986 to 1988	274	0.48	59%	82%	64%	77%
1988 to 1989	431	0.62	36%	68%	43%	61%
1989 to 1991						
Entire voltage range	499	0.70	33%	51%	37%	46%
V _{BOC} < 0.75 volt	306	0.51	48%	54%	50%	51%
V _{BOC} \geq 0.75 volt	193	1.01	8%	33%	17%	24%
Average over last 3 cycles			40%	66%	46%	61%
Farley-2 Cycles						
1986 to 1987	291	0.55	24%	72%	38%	60%
1987 to 1989	316	0.59	34%	79%	45%	70%
1989 to 1991						
Entire voltage range	326	0.71	15%	68%	29%	58%
V _{BOC} < 0.75 volt	207	0.52	30%	74%	40%	66%
V _{BOC} \geq 0.75 volt	119	1.04	-13%	45%	10%	32%
Average over last 3 cycles			24%	74%	37%	63%

Notes

1. Voltage growth per cycle determined as $(V_{EOC} - V_{BOC}) / V_{BOC}$
2. Growth per cycle obtained by conservatively setting voltage change $(V_{EOC} - V_{BOC})$ to zero if measured change is negative.

Figure 5-1

Distorted Indication Signal Amplitudes in Farley-1 S/Gs (March 1991)

J. M. FARLEY UNIT 1 4/91 INSPECTION
DISTORTED INDICATION VOLTAGE DISTRIBUTIONS

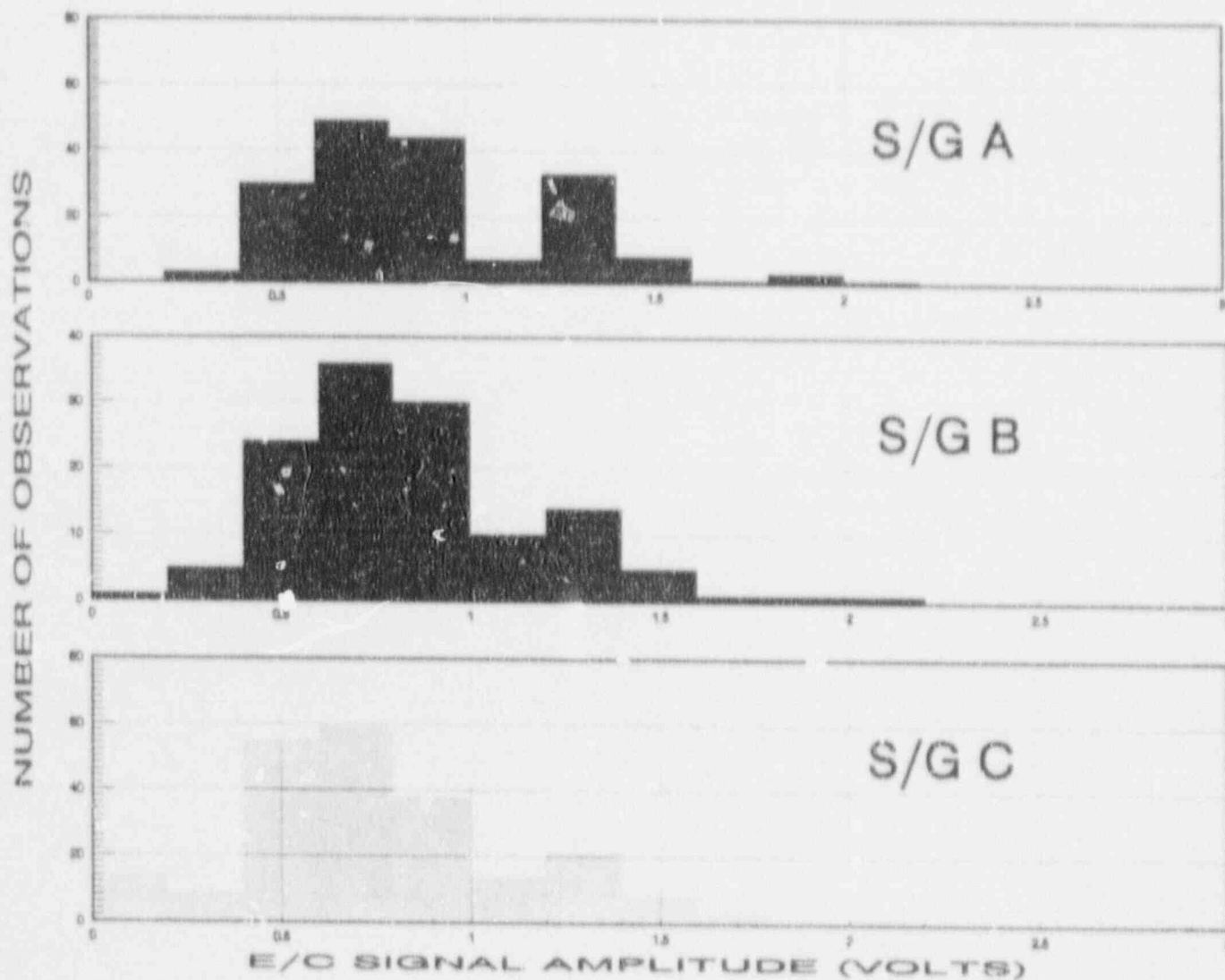


Figure 5-2

Axial Distribution of Distorted Indication Signals in Farley-1 S/Gs (March 1991)

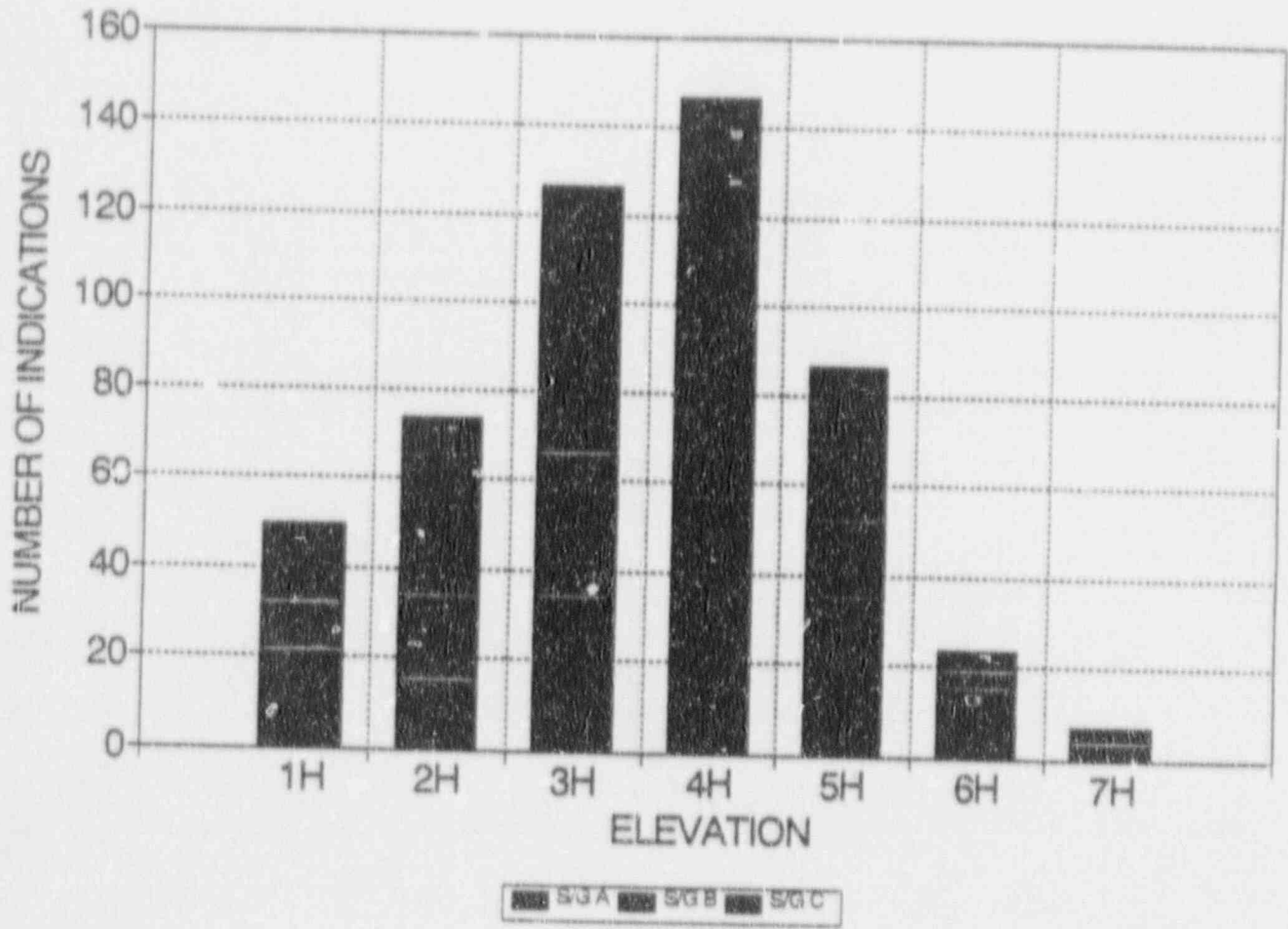


Figure 5-3

Average Growth in Depth for Farley-1 S/Gs Over Last 2 Cycles

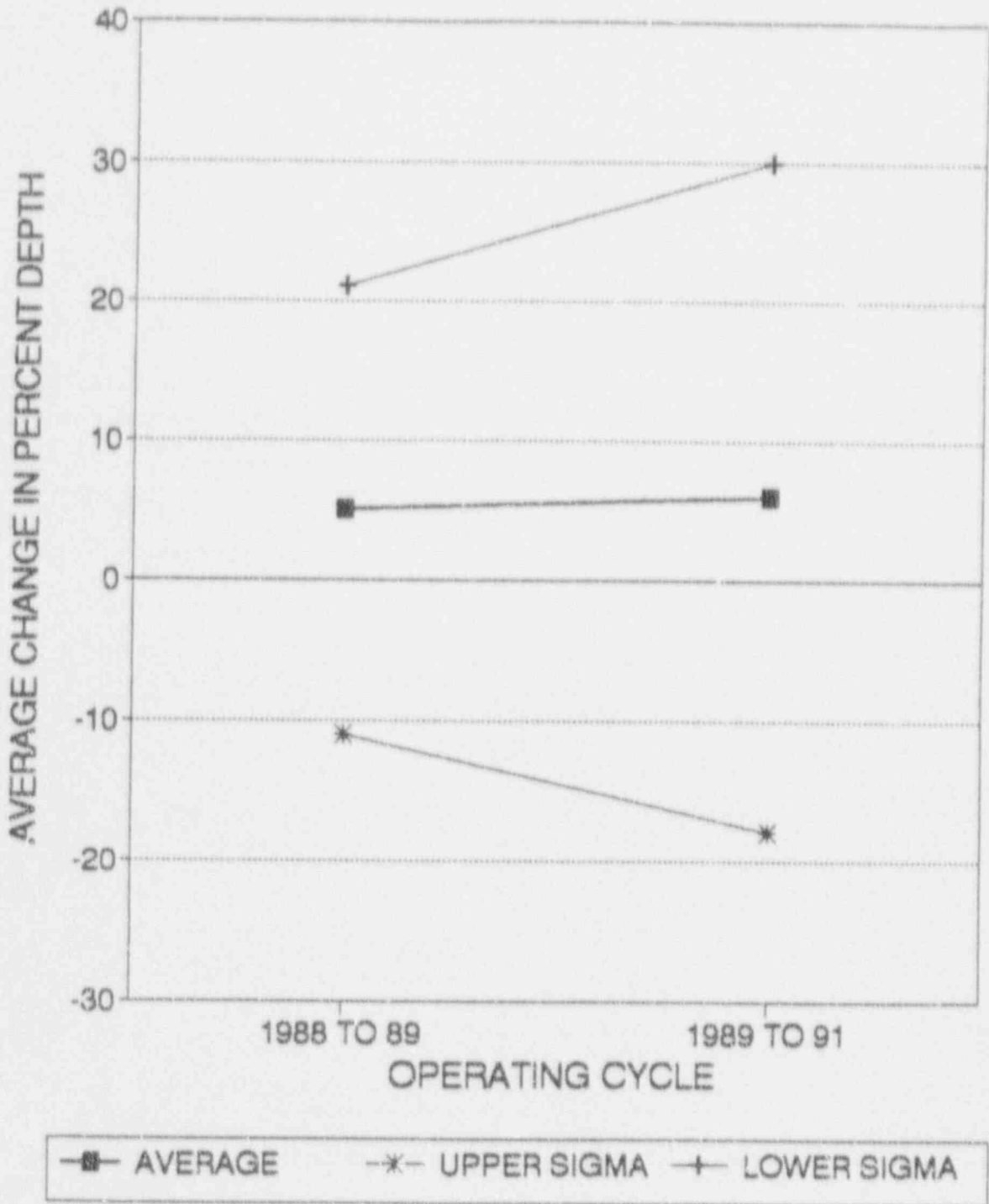


Figure 5-4

Axial Distribution of TSP Indications in Farley-2 S/Gs (October 1990)

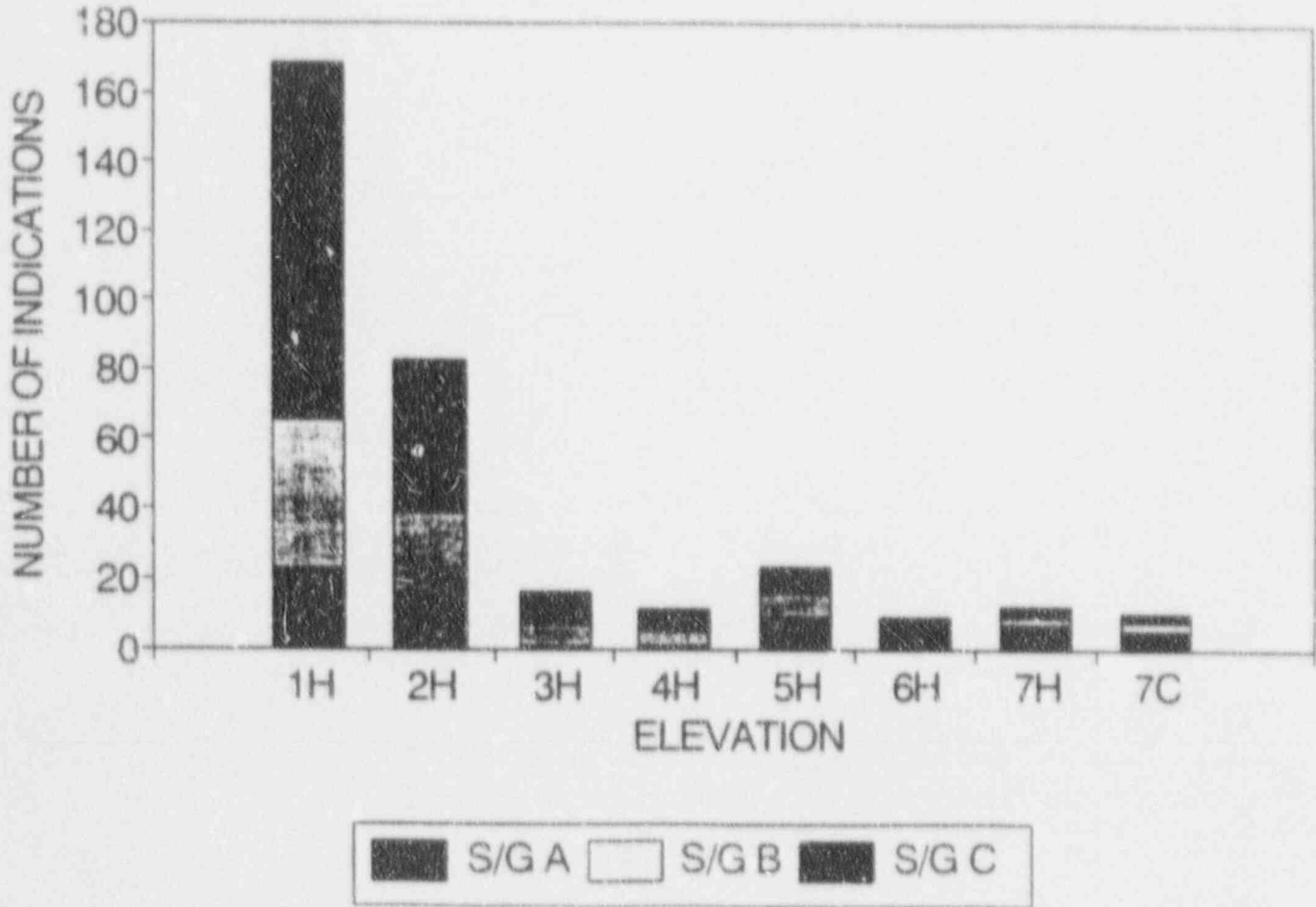


Figure 5-5

Distribution of TSP Indication Amplitudes in Farley-2 S/Gs (October 1990)

J. M. FARLEY UNIT 2 10/90 INSPECTION
DISTRIBUTION OF TSP INDICATION AMPLITUDES

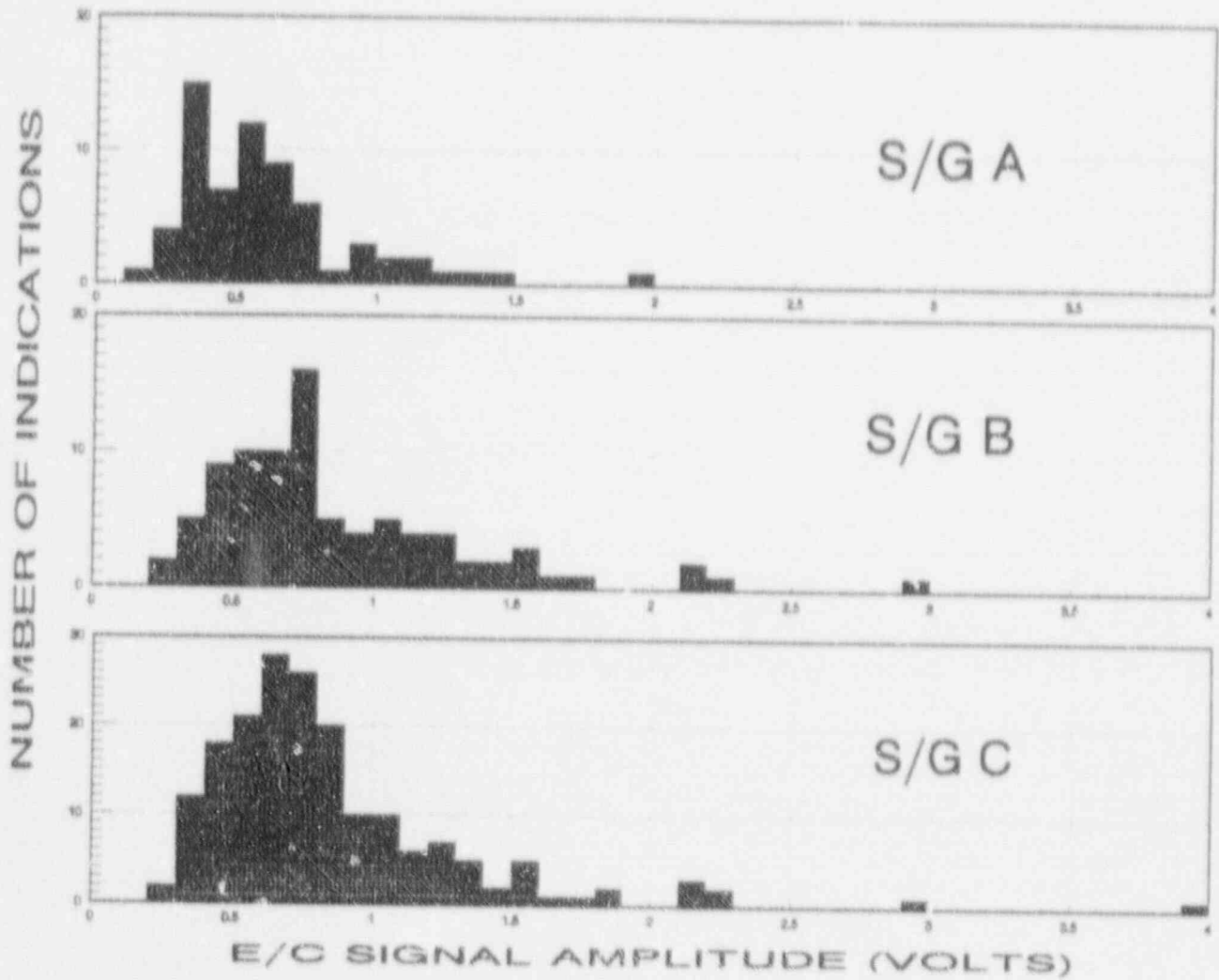


Figure 5-6

Support Plate Indication Progression in Farley-2 SGs

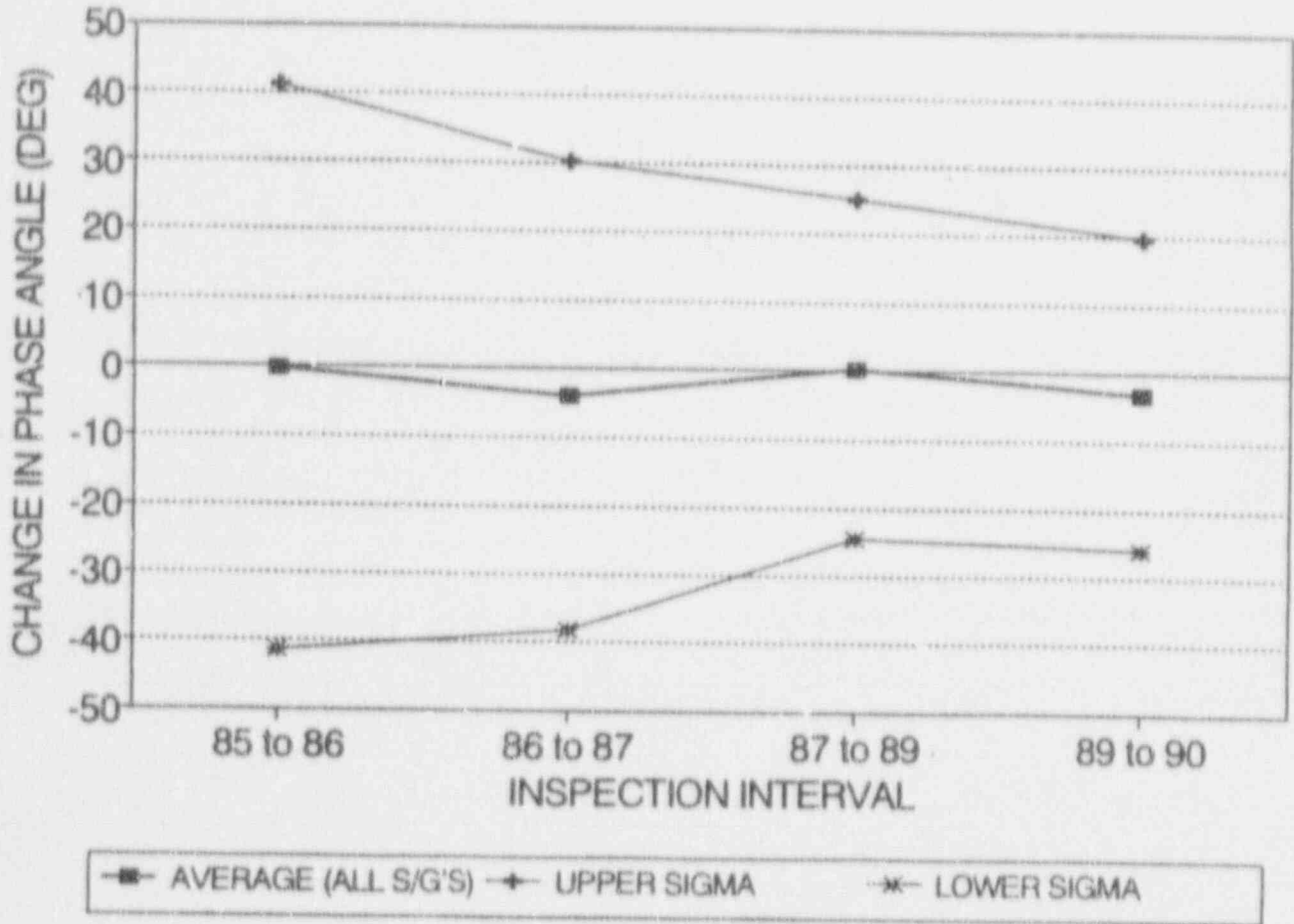


Figure 5-7

Cold Leg TSP Corrosion Assessment in Farley-1

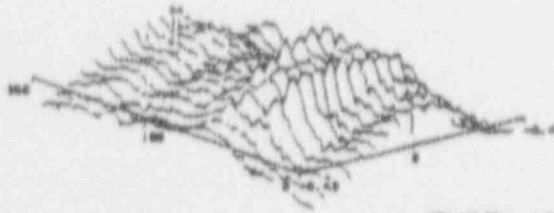


Figure 5-8

Farley-1 RPC Characterization (November 1989)

R12C3 2H S/G 21

CHANN --- 1
 PRES --- 480 kHz
 SPAN --- 25
 ROTATION --- 26.5 DEG



MAX VOLTS 1.7
 AXIAL PITCH 30
 CIRC PITCH 30
 Z ROTATION 211
 X ROTATION 70
 Y OF SCENE 17
 ROTATE SCENE 30

R12C3 5H S/G 21

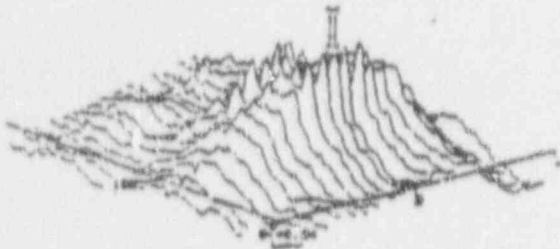
CHANN --- 1
 PRES --- 480 kHz
 SPAN --- 25
 ROTATION --- 26.5 DEG



MAX VOLTS 1.7
 AXIAL PITCH 30
 CIRC PITCH 30
 Z ROTATION 211
 X ROTATION 70
 Y OF SCENE 17
 ROTATE SCENE 30

R31C50 2H S/G 21

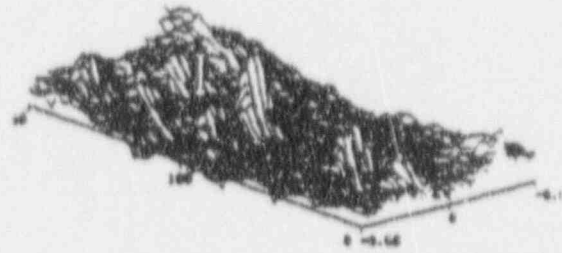
CHANN --- 1
 PRES --- 480 kHz
 SPAN --- 25
 ROTATION --- 26.5 DEG



MAX VOLTS 1.9
 AXIAL PITCH 49
 CIRC PITCH 70
 Z ROTATION 216
 X ROTATION 70
 Y OF SCENE 17

R20C26 1H

CHANN --- 1
 PRES --- 480 kHz
 SPAN --- 21
 ROTATION --- 26.1 DEG

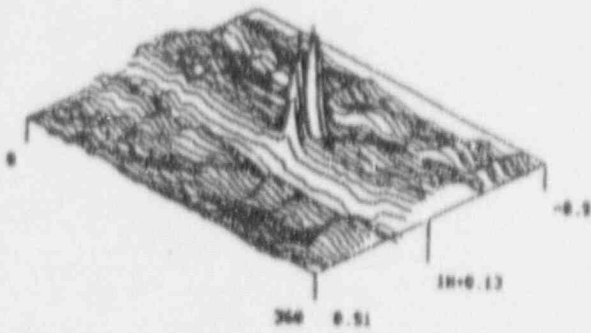


MAX VOLTS 6.1
 AXIAL PITCH 1
 CIRC PITCH 4
 Z ROTATION 21
 X ROTATION 7
 Y OF SCENE 9

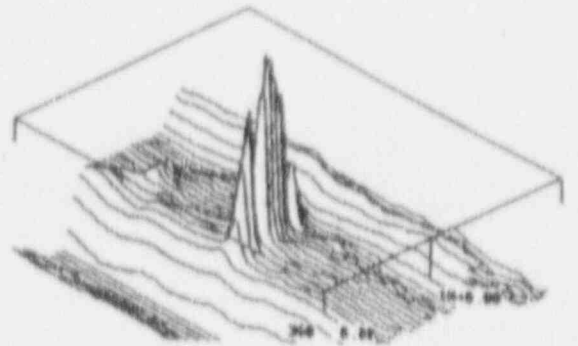
Figure 5-9

Farley-2 RPC Characterization (November 1990)

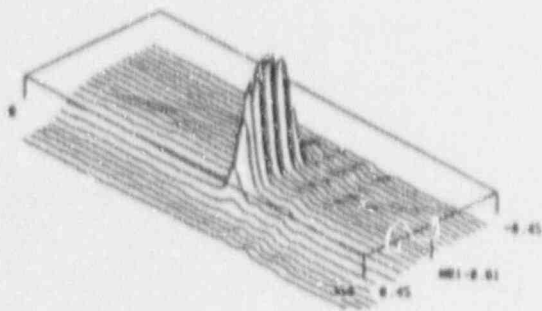
R40C43 S/G 31



R38C65 S/G 31



R21C22 S/G 21



R4C73 S/G 21

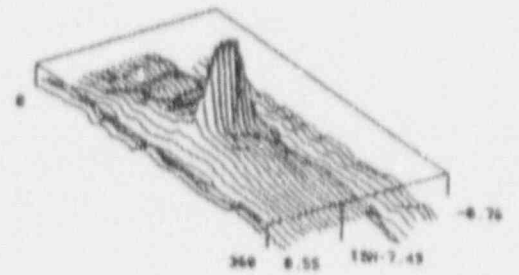


Figure 5-10

Scatter Plot of Voltage Growth in Farley-1 for Last Two Cycles

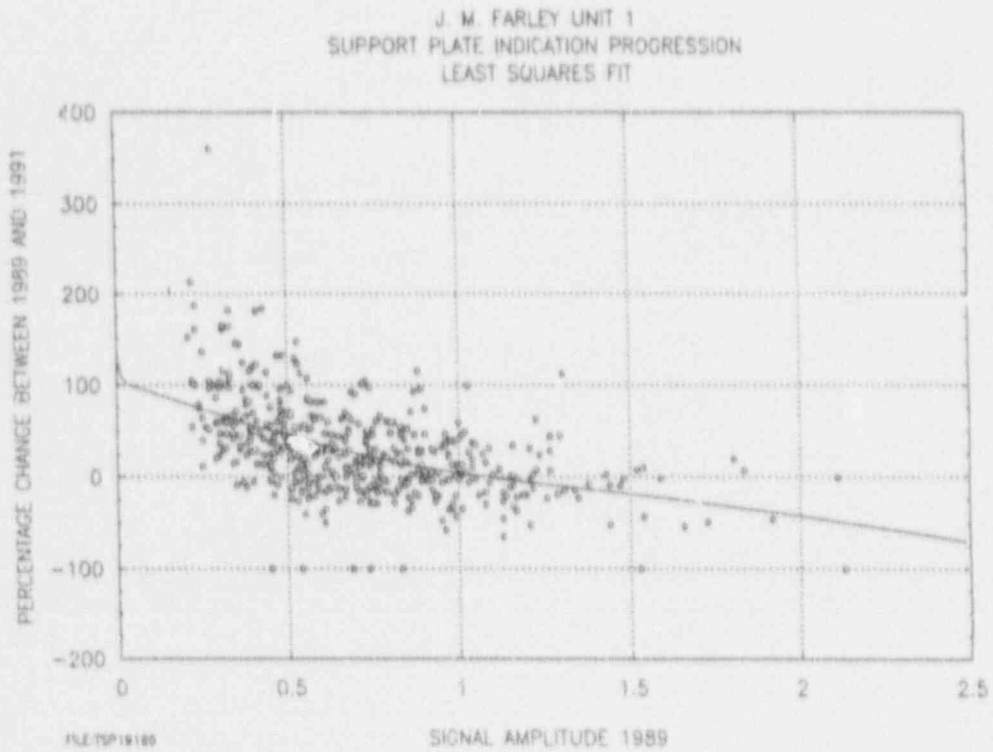
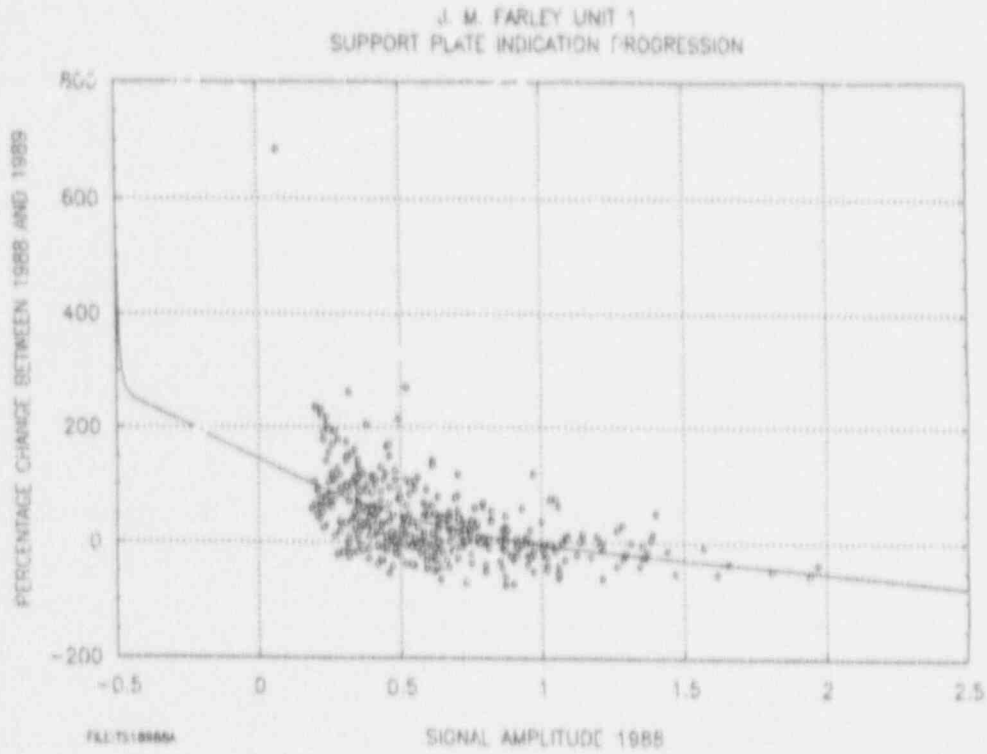


Figure 5-11

Scatter Plot of Voltage Growth in Farley-2 for Last Two Cycles

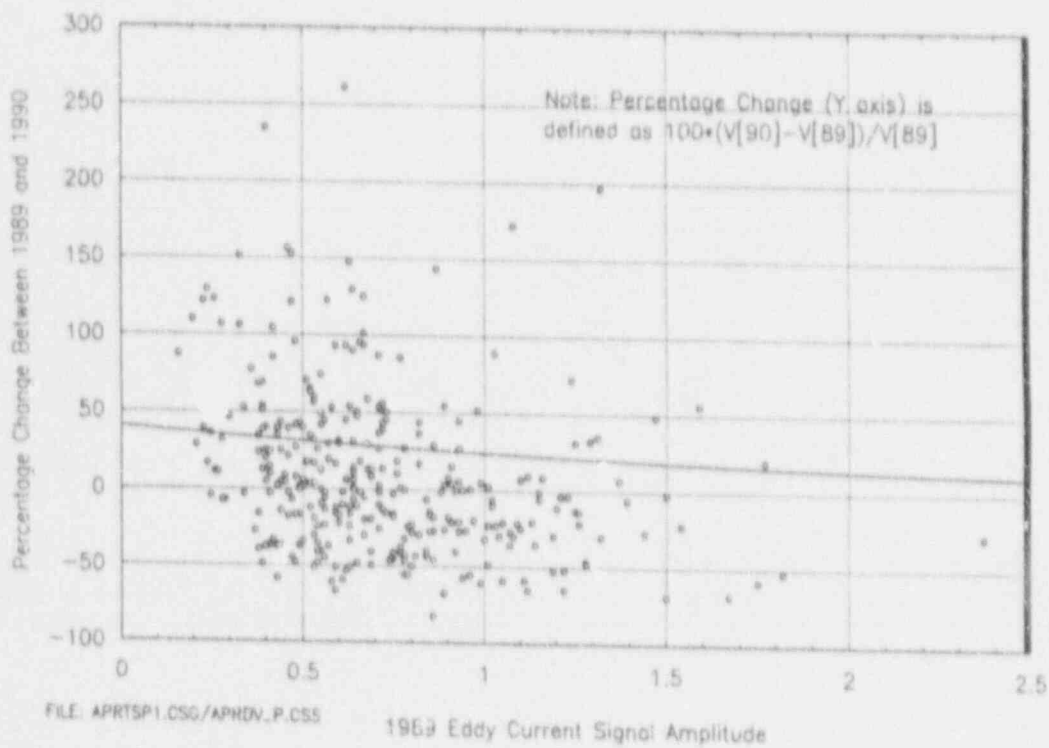
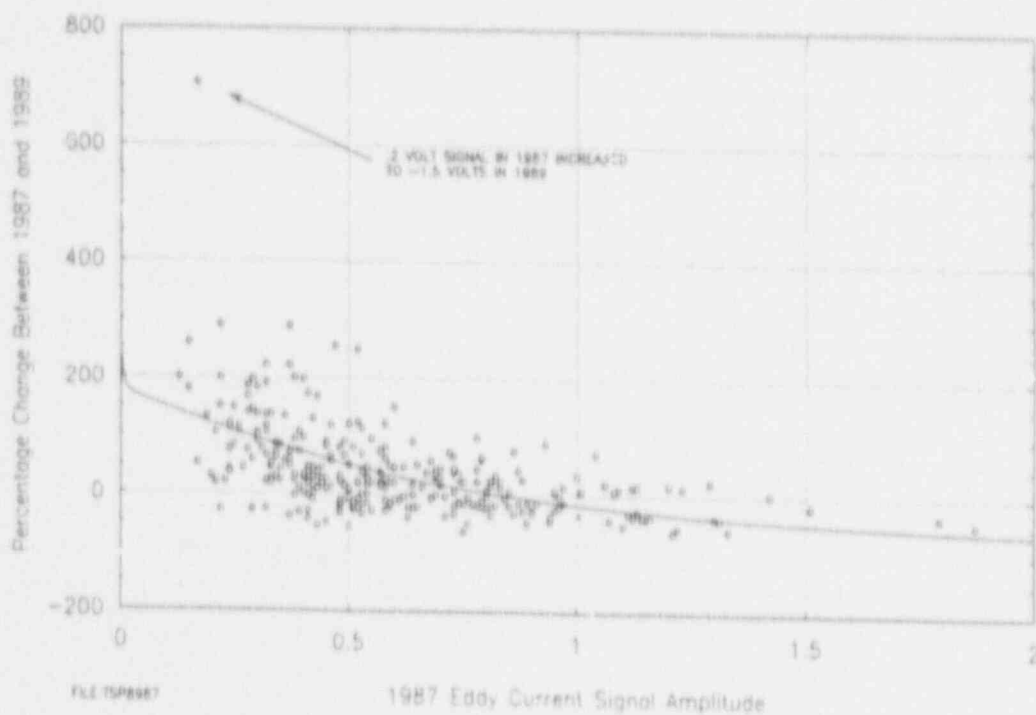


Figure 5-12

Histogram and Cumulative Probability of Voltage Growth in Farley-1 for Last Two Cycles

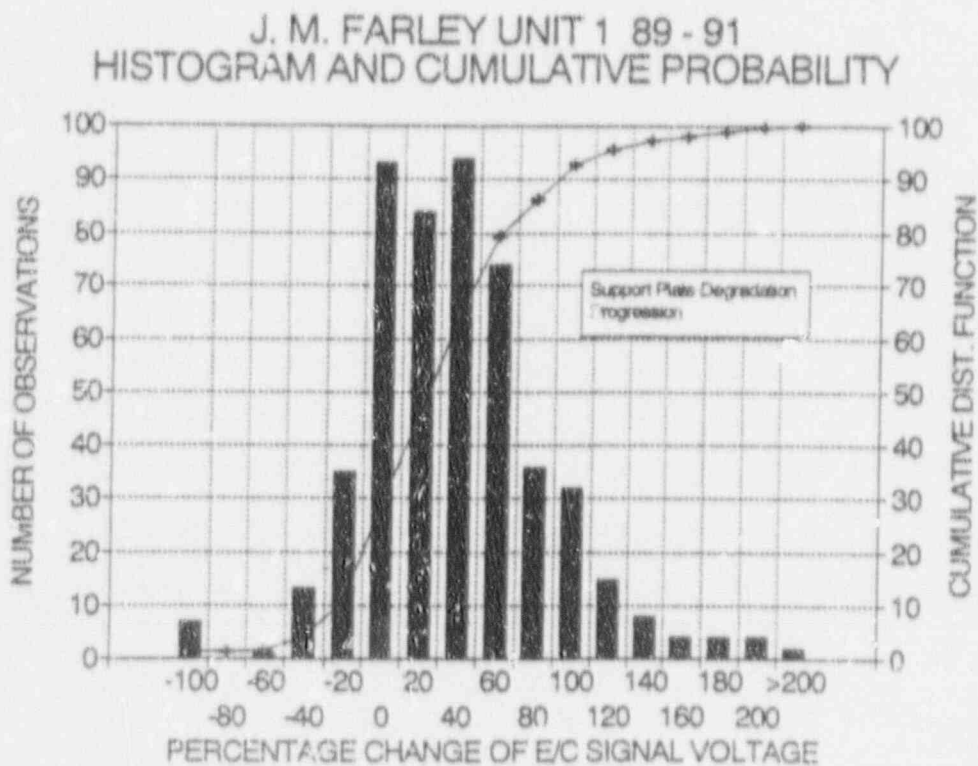
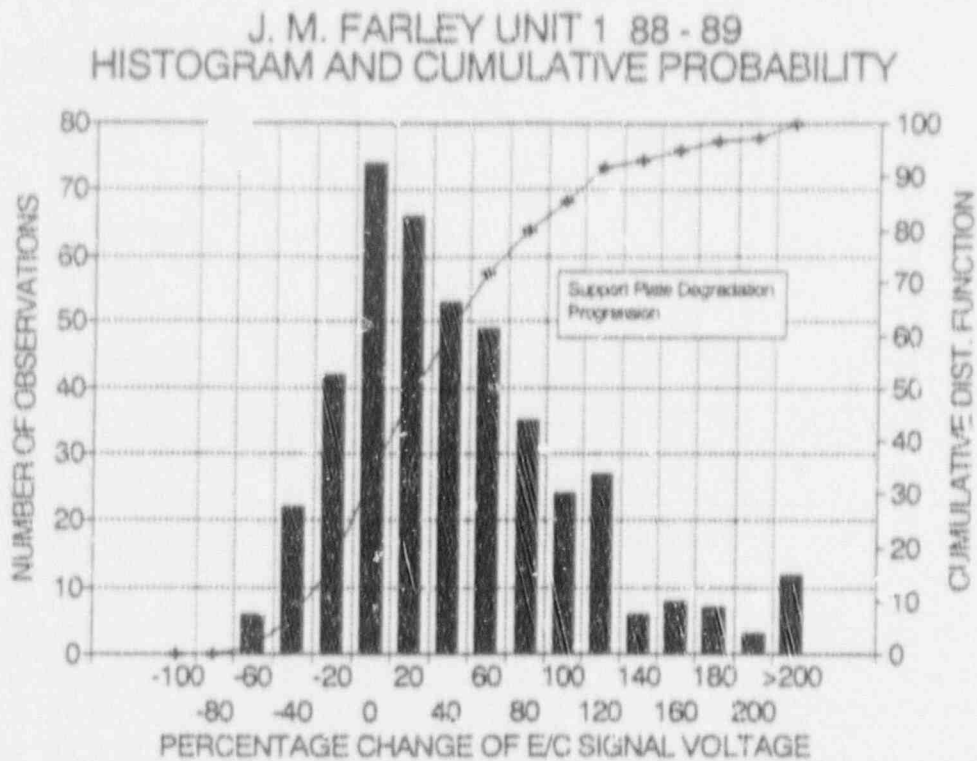


Figure 5-13

Histogram and Cumulative Probability of Voltage Growth in Farley-2 for Last Two Cycles

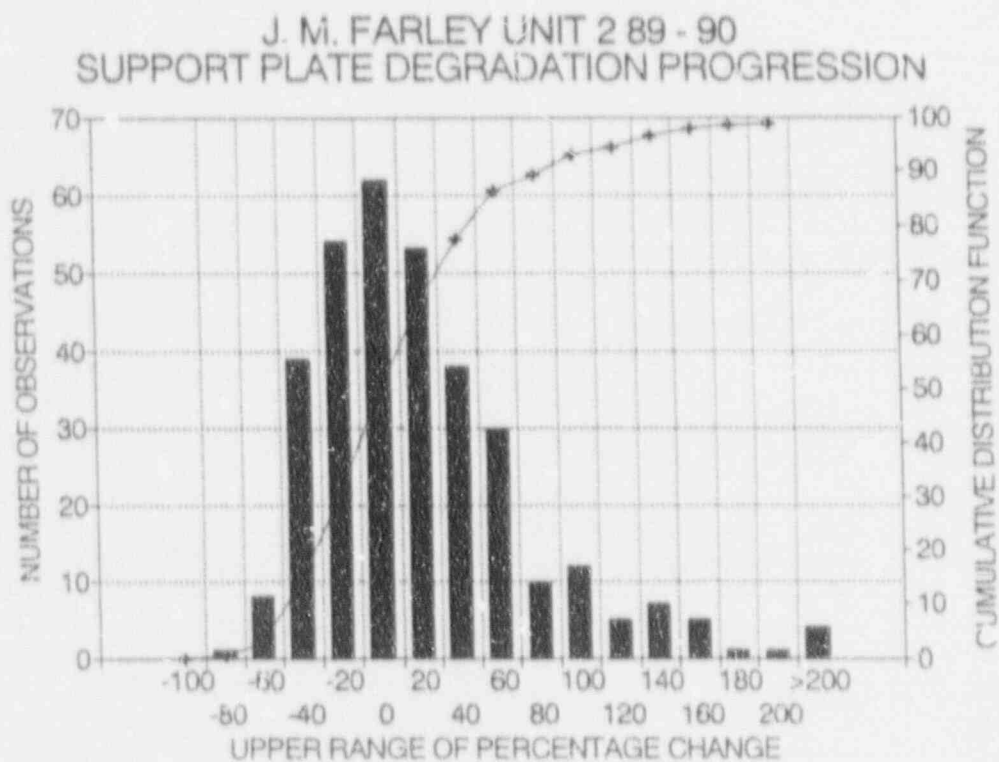
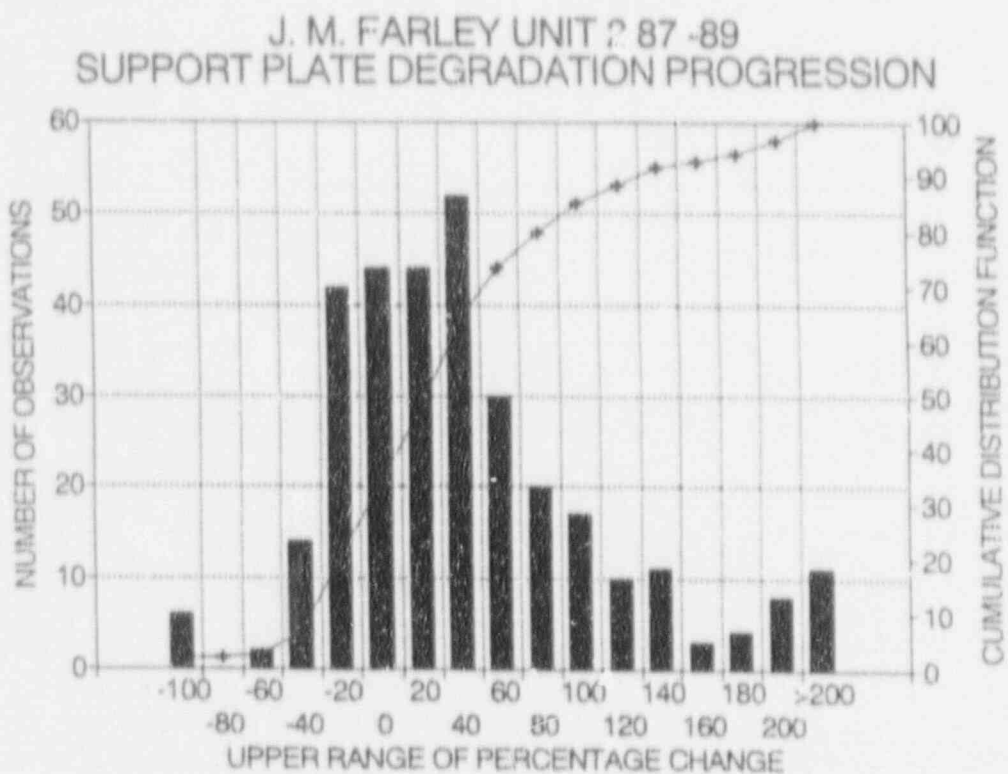


Figure 5-14

Cumulative Probability of Voltage Growth per EFPY for Farley Units 1 and 2

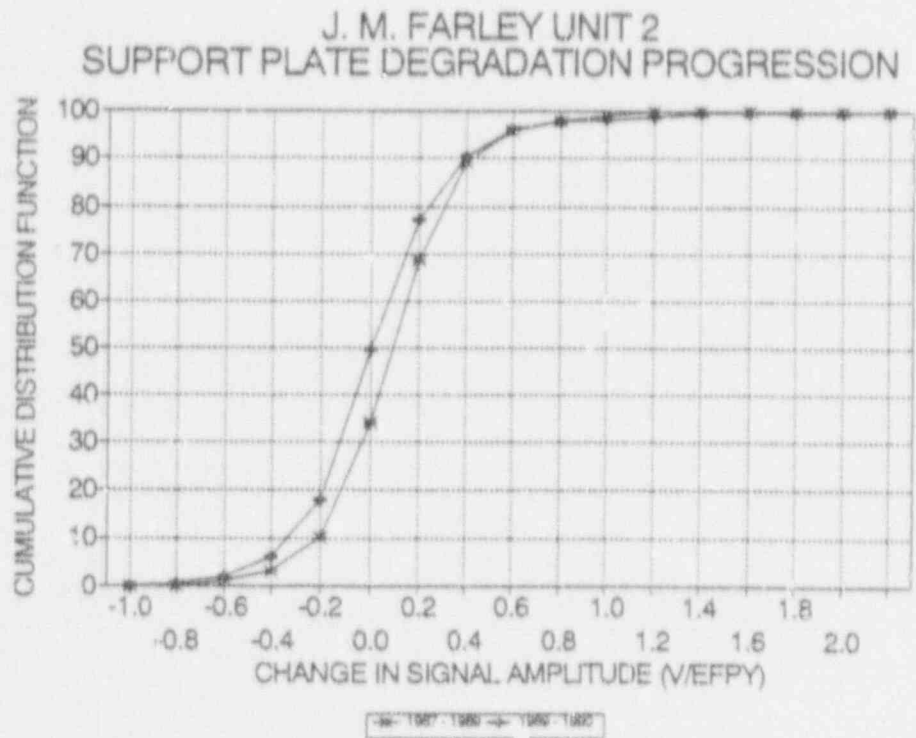
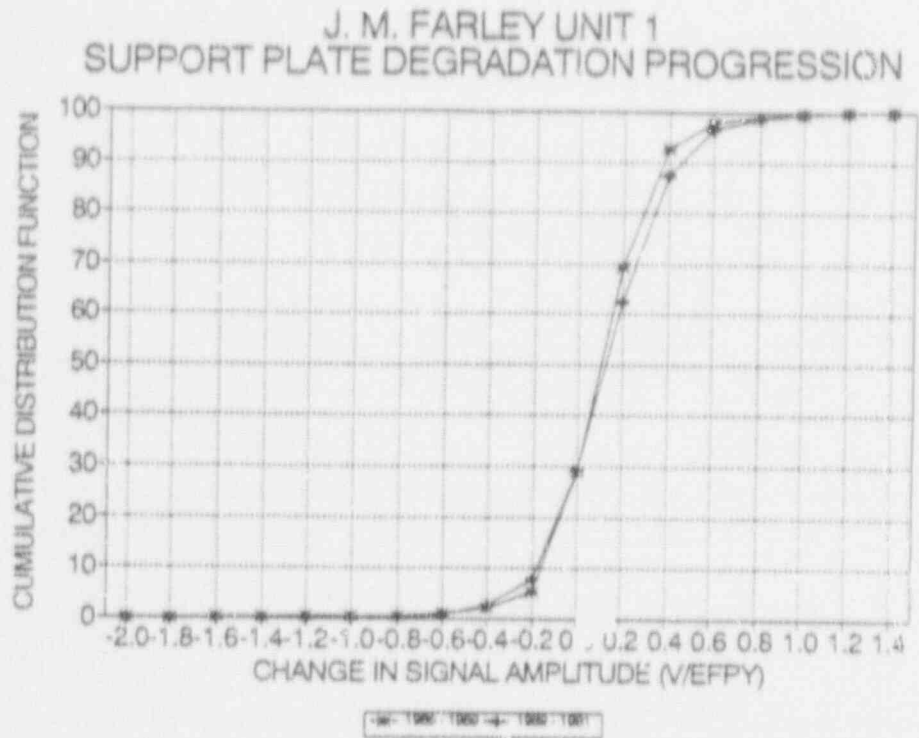
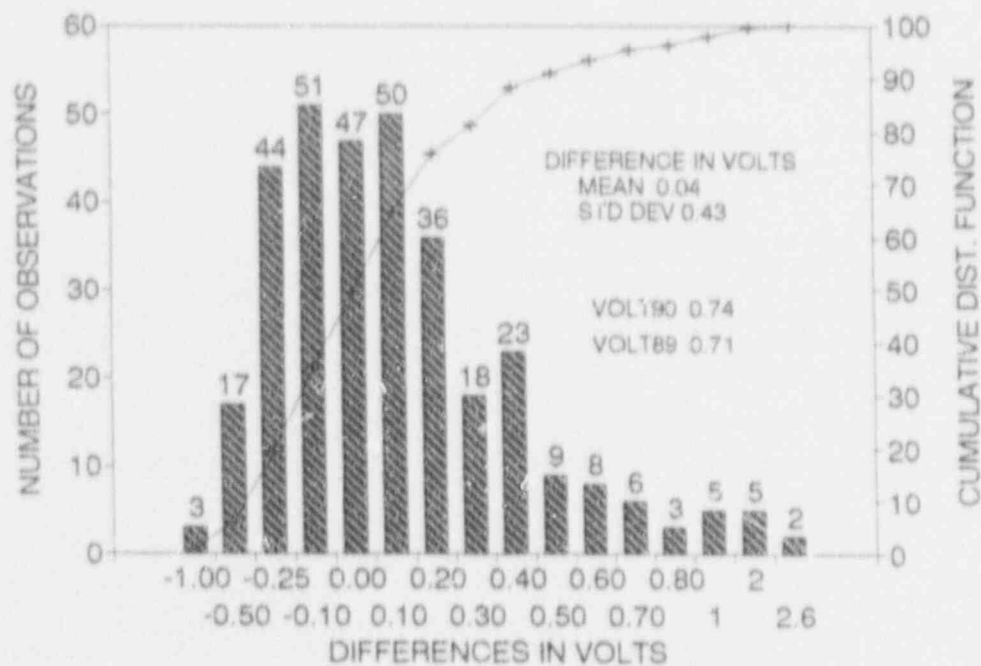


Figure 5-15

Cumulative Probability of Voltage Growth per Cycle for Farley Units 1 and 2

J. M. FARLEY UNIT 2 90 - 89
100% POPULATION, 327 READINGS



J. M. FARLEY UNIT 1 91 - 89
100% POPULATION, 499 READINGS

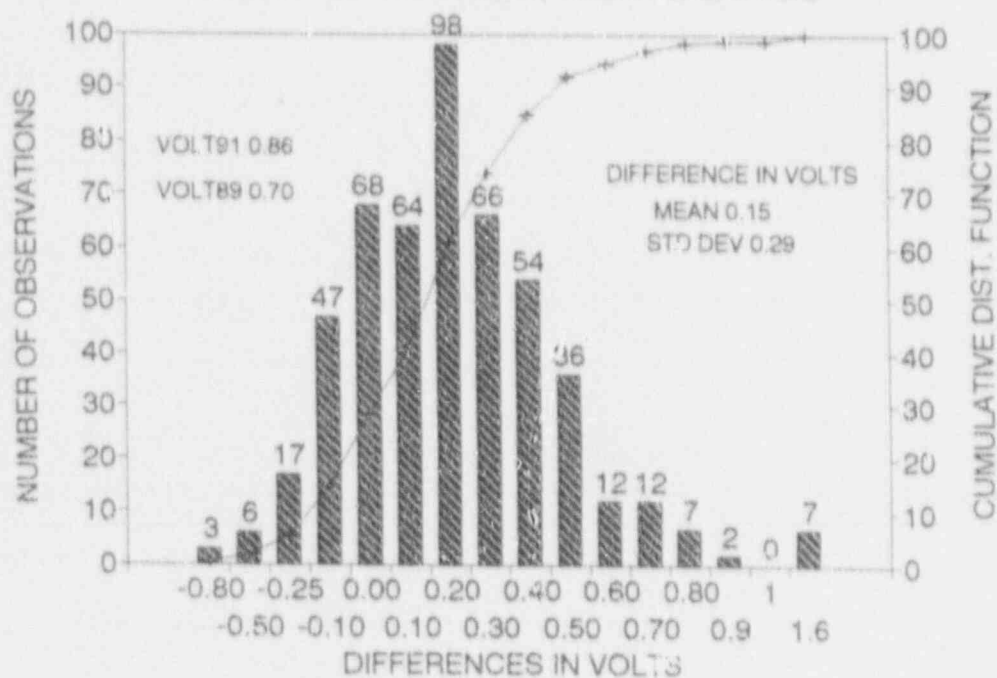
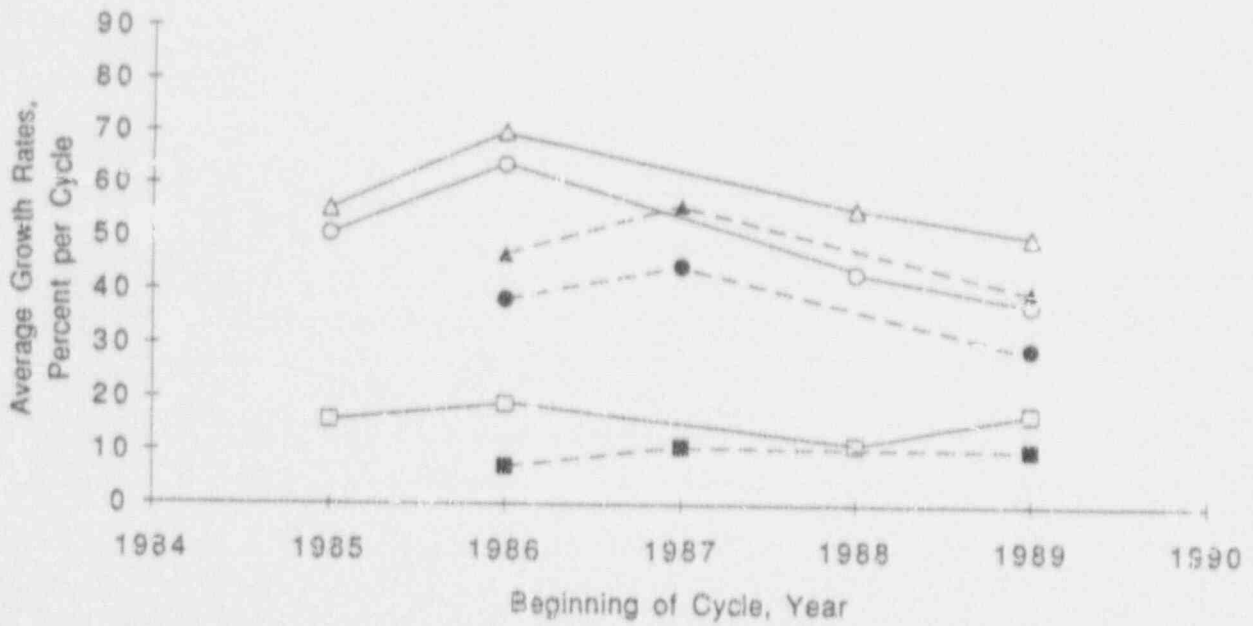


Figure 5-16

Historical Average Voltage Growth Trends in Farley SGs



- Average Growth Over Total BOC Voltage Range: Farley-1
- △— Average Growth For BOC Volts < 0.75 : Farley-1
- Average Growth For BOC Volts > 0.75 : Farley-1
- - ● - - Average Growth Over Total BOC Voltage Range: Farley-2
- - ★ - - Average Growth For BOC Volts < 0.75 : Farley-2
- - ■ - - Average Growth For BOC Volts > 0.75 : Farley-2

6.0 FIELD EXPERIENCE SUMMARY: Pulled Tube, Plant Leakage and Inspection Data

This section identifies the field experience data from operating SGs that are utilized in the development of tube plugging criteria for ODSCC at TSPs. The field data utilized include pulled tube examination results, occurrences of tube leakage for OF/SCC indications at support plates and field inspection results for relatively large OF/SCC indications with no identifiable leakage.

6.1 Utilization of Field Data in Tube Plugging Criteria

Operating SG experience represents the preferred source of data for the plugging criteria. Where the available operating data are insufficient to fully define plugging criteria, data developed from laboratory induced ODSCC specimens were used to supplement the field data base. Table 6.1 summarizes the utilization of field and laboratory data to develop the tube plugging criteria. The field data utilized for the plugging criteria are identified in this report section. Sections 7 to 11 describe the development of the laboratory data. The field and laboratory data are combined in Sections 9 and 12 to develop the tube plugging limits.

The overall approach to the tube plugging criteria is based upon establishing that R.G. 1.121 guidelines are satisfied. It is conservatively assumed that the tube to TSP crevices are open and that the TSPs are displaced under accident conditions such that the ODSCC generated within the TSPs becomes free span degradation under accident conditions. Under these assumptions, preventing excessive leakage and tube burst under SLB conditions is required for plant safety. Tube rupture under normal operating conditions is prevented by the constraint provided by the drilled hole TSPs with small tube to TSP clearances (typically ~ 16 mil diametral clearance for open crevices). For the plugging criteria, however, the R.G. 1.121 criteria for burst margins of 3 times normal operating pressure differentials are applied to define the structural requirements against tube rupture.

In addition to providing margins against tube burst, it is necessary to limit SLB leakage to acceptable levels based on FSAR evaluations for radiological consequences under accident conditions. Thus SLB leakage models are required for the plugging criteria in addition to tube burst data.

Based on the above considerations and the plugging criteria objective of relating tube integrity to NDE measurements, the primary data requirements for the plugging criteria are the correlation of burst pressure capability and SLB leak rates with bobbin coil voltage. For plant operational considerations, it is desirable to minimize the potential for operating leakage to avoid forced outages. Thus an additional objective is to relate bobbin coil voltage to operating leakage. The field data of this section indicate very low leakage potential for ODSCC at TSPs even at voltage amplitudes much higher than the plugging limits.

Within the above overall approach, field data are utilized as follows:

A. Pulled Tube Data

The test results for pulled tubes having had leak rate and burst tests performed, such as the three Farley-2 tubes, are used directly in supporting the plugging limits. If metallographic data on the crack morphology is available but leak/burst tests were not performed, the crack depths and lengths were evaluated to assess the potential for leakage and to estimate burst pressure margins. However, only tubes with measured burst pressure are used in the voltage/burst pressure correlation. The pulled tube data base is discussed in Section 6.2.

B. Plant Leakage Experience for ODSCC at TSPs

Domestic and international data for operating leakage within Westinghouse plants was reviewed for identification of leakers attributable to ODSCC at TSPs. No occurrences of identifiable leakage due to ODSCC at TSPs were found in domestic units. Three occurrences were identified in European units. The latter data (See 6.3 below) together with field inspection data are used to assess the potential for operating leakage at plugging limits that meet tube burst and SLB leakage requirements.

C. Field Inspection Data for Tubes With No Identified Leakage

It is shown in Section 9 that tubes with voltage levels up to about 6 - 7 volts meet the requirements for burst pressure margins under free span burst conditions. Therefore field inspection data for indications above and below this voltage level, and with no identified operating leakage, can be used to assess the potential for significant operating leakage.

The field data base for items A, B and C are described in the following sections.

6.2 Pulled Tube Data Base

The available pulled tube data base for ODSCC at TSPs in Westinghouse SGs includes 29 pulled tubes for which 62 tube to TSP intersections have both NDE and destructive examination results. This group includes four tubes from Farley-2 and one tube from Farley-1 with one intersection destructively examined for each pulled tube. None of the 29 pulled tubes have been reported as leakers during plant operation. The crack morphologies were reviewed for 14 tubes with TSP intersections having no leakage or burst test measurements. This review indicates that no leakage would be expected for these tubes even under SLB conditions and that the burst pressures would exceed 3 times normal operating conditions. The field eddy current data for all pulled tubes were reviewed for voltage normalization consistent with the standard adopted (see Sections 5.6, 8) for the plugging criteria development. Indications for 3/4 inch diameter tubing were normalized to 4.0 volts in the 550 kHz channel and evaluated for the 550/100 kHz mix.

Table 6.2 provides the leak rate and burst pressure data for the 14 pulled tubes (30 intersections) for which these tests were performed. The extensive pulled tube data from Plant L were reevaluated to the eddy current data analysis guidelines of Appendix A of this report. This reevaluation is described in Appendix B. The leak rate and burst

tests were conservatively performed as free span (without collars) tests. Also shown in the table are the estimated leak rates and burst pressure based upon the actual crack morphology for Farley-2 tube R31C46 with the 7.2 volt indication. The 1990 Farley-2 tube leakage tests showed [] of leakage at normal operating conditions but these low values cannot be clearly separated from test system leakage. The measured SLB leak rates are []. The pulled tubes in Table 6.2, with up to 10 volt indications, all show burst pressures for the test conditions (room temperature, as-built material properties) in excess of 4380 psi, 3 times the normal operating pressure differential (adjusted for temperature, the 3 times normal operating pressure differential equivalent is 5250 psi).

The pulled tube NDE data are shown in Figure 6-1 as bobbin coil voltage versus indicated depth. All pulled tube results at normal operating pressure differential represent no leakage conditions while two small leakers at SLB conditions were found from Farley-2. Figure 6-2 shows the same data plotted as voltage versus actual depth from destructive examination.

Correlations of the bobbin coil phase angle based depth estimates with the maximum depths from destructive examinations have shown an uncertainty of 15% for the depth indications. The pulled tubes have typically shown one dominant axial crack network with multiple, small cracks around the tube circumference. With multiple, large axial crack networks around the tube circumference, the bobbin coil depth uncertainty can be larger than 15%.

Figure 6-2 shows that below 2.8 volts the maximum depths are dominantly less than 80% with a few indications up to 98% depth. An occasional, very short, 100% through wall indication, such as the 1.9 volt indication, could potentially occur at these low voltage levels although the pulled tube examination results indicate that the associated crack length can be expected to be too short for any measurable leakage at normal operating or SLB conditions. The smallest voltage found for a through wall crack in the current pulled tube data base is 1.9 volts for 3/4 inch diameter tube R4C61 as noted in Table 6.2. Figure 6-3 shows the field bobbin and RPC data for this indication.

Between 3 and 10 volts, the limited pulled tube data indicate the potential for through wall cracks with negligible leakage at normal operating conditions and very small leaks at SLB conditions.

6.3 Operating Plant Leakage Data for ODSCC at TSPs

Table 6.3 summarizes the available information on three suspected tube leaks attributable to ODSCC at TSPs in operating SGs. These leakers occurred in European plants with two of the suspected leakers occurring at one plant in the same operating cycle. In the latter case, five tubes including the two with indications at TSPs were suspected of contributing to the operating leakage. Leakage for the two indications at TSPs was obtained by fluoresceine leak test as no dripping was detected at 500 psi secondary side pressure.

For the Plant B-1 leakage indication, other tubes also contributed to the approximately 63 gpd total leak rate. Helium leak tests identified other tubes leaking due to PWSCC

indications. Using relative helium leak rates as a guide, it was judged that the leak rate for the ODSCC indication was less than 10 gpx.

These leakage events indicate that limited leakage can occur for indications above about 7.7 volts. No leakage at Farley 1 or 2 has been found that could be attributable to ODSCC at TSP's.

6.4 Plant Inspection Data for Tubes with No Identified Leakage

Additional guidance on the voltage levels at which significant operating leakage might be expected can be obtained from plant inspection results for tubes with large indications but no identified tube leakage. Inspection results from 8 units were reviewed to identify indications above about 1 volt with no suspected leakage. Data from this review are shown in Figure 6-4. These data show a large number of indications below 6 volts and a few larger indications that can be associated with no leakage conditions. This is consistent with the pulled tube results of Figure 6-1. The overall Farley data support no operating leakage below about 10 volts even though the Farley pulled tubes indicate short, through wall crack penetrations.

6.5 RPC Data Considerations

Examples of HPC indications for the Farley SGs were given in Section 5. Although the pulled tube data of Section 4 show multiple small cracks in addition to the dominant one or two cracks, the smaller cracks are typically too short and shallow to be detected. The higher voltage cracks found in Farley SGs tend to be single, dominant cracks although a second crack network can sometimes be found.

RPC inspections are required above 1.5 volts (See Section 12) to support the continued presence of ODSCC as the dominant degradation mechanism. In addition, the data obtained would support further development of SLB leakage models which may utilize leakage correlated with RPC parameters. The RPC inspection results can be optionally applied to verify the presence of the bobbin coil indication.

6.6 Voltage Renormalization for Alternate Calibrations

To increase the supporting data base, it is desirable to be able to renormalize available data to the calibration values used in this report. When 400/100 kHz mix or 400 kHz data normalized to an ASME standard are available, the renormalization is a straight forward ratio of the calibration voltage values. However, when different frequencies are used, the normalization ratio is phase angle or depth dependent and this normalization has been evaluated as described below.

For data on 3/4 inch diameter tubing, voltage renormalization has been obtained by applying a normalization of 4.0 volts for the ASME 20% holes in the 550 kHz channel and evaluated using 550/100 kHz mix. Westinghouse, under EPRI sponsorship, is further evaluating alternate voltage normalizations for 3/4 inch tubing to compare with the normalization adopted in this report for 7/8 inch tubing. Comparisons of responses

to drilled holes and EDM slots as well as burst correlations will be applied to adopt a 3/4 inch tubing voltage normalization for use in alternate plugging criteria. Until this study is complete, 3/4 inch tubing tests are not used in the voltage/burst correlation for 7/8 inch tubing applied for Farley.

The voltage normalizations applicable to the calibrations used in this report are:

ASME 4-hole, 100% deep, 0.033 ± 0.001 inch dia. = 6.4 volts at 400/100 kHz

ASME 4-hole, 20% deep, 0.187 ± 0.003 inch dia. = 2.75 volts at 400/100 kHz

ASME 4-hole, 20% deep, 0.187 ± 0.003 inch dia. = 4.00 volts at 400 kHz

The through wall hole with tighter than ASME hole tolerances has been identified as the primary voltage normalization for application of the voltage plugging limits. The through wall holes result in lower influence of manufacturing tolerances on the voltage calibration than partial depth holes. Calibration at the 400/100 kHz mix used for data evaluation is recommended to minimize potential uncertainties from normalization at other than the evaluation frequency. The above calibrations can be applied to normalize most of the domestic inspection results for 7/8 inch tubing.

In France and Belgium, a 240 kHz differential inspection is most commonly applied. Voltage renormalization was evaluated by fabricating the French and Belgian standards and comparing their procedure with that of this report. Results of this study are given in Table 6.4. The U.S. to French voltage ratio was further evaluated using an Intercontrole probe commonly used by EdF and applying this probe as well as a domestic probe to the calibration standard and to several model boiler specimens with ODSCC. The results of this evaluation are given in Figure 6-5. These results show a consistent ratio (within ~10%) for both probes and between calibration standards (solid symbols) and model boiler specimens with ODSCC. In Figure 6-5, phase angles of 30° and 100° correspond to 100% and 20% ASME hole depths, respectively.

Given Table 6.4 and Figure 6-5, most bobbin coil voltage measurements can be renormalized to the calibration applied in this report.

6.7 Comparisons With European Plant Inspection Results

The pulled tube data described in Section 6.2 and the field inspection results of Section 6.4 were obtained from domestic and European plants that apply essentially the same voltage calibration standards and comparable frequency mixes for indications at TSPs. The operating experience data base can be increased substantially by including plant data from French and Belgian plants. However, these plants utilize different voltage calibrations and frequencies for TSP indications. To compare the domestic plant data with these European data, the voltage ratios of Figure 6-5 (as a function of phase angle) have been applied. However, any conversion factor involves some uncertainties as indicated in Figure 6-5 because it depends on the varying crack responses to different frequencies as well as procedural/environmental conditions. Recognizing uncertainties in the voltage conversion factors, comparisons with the European data are particularly valuable for the following comparisons:

- o Trends in indications and growth with time for equivalent voltages higher than available in domestic plants which have applied 40% depth criteria for tube plugging. None of the European data at higher equivalent voltages have had identifiable operating leakage so these data substantially increase the high voltage data base supporting negligible leakage for ODSCC at TSPs.
- o Percentage growth in voltage from European plants can be used to assess growth rate trends for equivalent voltages much higher than that available in domestic plants.

For these comparisons between domestic and European experience, the domestic data for Farley-2 and Plant F are compared with French and Belgian data.

Distribution of Indications

Data for ODSCC at TSPs for French Unit H-1, SG-1 are available for four successive inspections with no tube plugging as shown in Figure 6-6. The upper figure shows the number of indications versus voltage amplitude while the lower figure shows the percentage distribution of indications within each outage. Without tube plugging to eliminate the larger indications, the distribution becomes more heavily weighted at the larger indications with increasing operating time.

European data is currently available for French Units H-1 and J-1 and Belgian Unit K-1. It is useful to compare the percentage voltage distributions for these units with the Farley-2 data and another domestic unit, Plant F. This is shown in Figure 6-7. The lower figure is scaled to include all available data while the reduced scale of the upper figure is included to emphasize the small voltage range of the U.S. data compared to the European data. Plant J-1 was excluded from the upper figure since breakdown of the data into small ranges was not available. It may be noted that over 97% of the domestic data falls below 0.5 volts when normalized to the French procedure and none of this domestic plant data falls above 1 volt. The French and Belgian data, on the other hand, extends above 3 volts.

Overall, it is seen that the U.S. plants with 40% depth plugging limits are operating with voltage amplitudes notably lower than that in European units.

The European units of Figure 6-7 with higher equivalent voltages have operated with no identifiable operating leakage. This result indicates that operating leakage due to ODSCC at TSPs is expected to be insignificant. This is supported by the fact that only 3 cases of small operating leakage, as shown in Table 6.3, have been identified to date.

French Pulled Tube Data

Fourteen tubes have been removed from French units with destructive examination results currently available. Figure 6-8 shows the crack morphology for a tube with a 0.7 volt (~3.7 U.S. volts) indication. The tube exams indicate many axial cracks of comparable depth around the tube circumference. The cracks are dominantly ODSCC with somewhat more IGA participation than seen in most domestic pulled tube data such as the Farley data. The multiple crack networks would be expected to increase bobbin coil voltage compared to one or only a few deep cracks as typical of the domestic data.

6.8 Comparisons of Voltage Response for ODSCC and IGA/SCC

Figure 6-9 shows the available pulled tube data plotted as bobbin coil voltage versus maximum depth from destructive examination. The French and Belgian (Plant E) data are included in the figure as well as the Farley data. Solid symbols represent tubes with crack morphologies reported to include IGA/SCC. Open symbols are reported to be ODSCC with minor IGA. The Plant L data include both IGA/SCC and ODSCC crack morphologies as shown by the solid and open symbols. The separation of ODSCC and IGA/SCC is judgemental for many Plant L tubes as minor IGA/SCC is present in a number of indications. The Plant L indications identified as including IGA/SCC have voltage amplitudes toward the higher range of the data compared to ODSCC indications.

The French and Belgian data with IGA/SCC and multiple cracks show the trend toward higher voltage amplitudes associated with these crack morphologies. The pulled tube from plant M had IGA up to 26% deep with an amplitude of 1.8 volts which is high compared to indications principally ODSCC at comparable depths. Three pulled tubes from Plant N with egg crate supports are also shown in Figure 6-6. The two indications with IGA also support IGA response at voltage levels comparable to or higher than that for equivalent depth ODSCC with minor IGA.

The ODSCC crack morphologies often show short microcracks separated by uncorroded ligaments. The maximum depths of a short microcrack in the overall macrocrack or crack network are commonly deeper than the average crack depths which tend to dominate tube burst capability. This combination of deep microcracks in shallower average macrocrack depths leads to a variation in maximum depth at a given voltage amplitude. Voltage amplitudes therefore are not a function of maximum crack depths but rather an indicator of crack face area (length, average depth, ligaments).

Overall, the available pulled tube results show that IGA/SCC crack morphologies tend to have as high or higher voltage amplitudes than associated with ODSCC morphologies at equivalent depths. Detectability of IGA/SCC at TSP intersections can be expected to be enhanced compared to ODSCC at equivalent depths. This trend is supported by the domestic data as well as the European data which show greater IGA/SCC involvement than the domestic data.

6.9 Growth Rate Trends

Of particular interest to establishing the plugging limits of this report is voltage growth rate as a function of the voltage amplitude. Current domestic plugging limits result in little data on growth rates in the range of voltage amplitudes being evaluated for the plugging limits of this report. The larger voltage amplitudes of the European data provide guidance on growth rate progression. Figure 6-10 shows growth rate data for Plant H-1 both as voltage amplitude and percentage growth as a function of voltage amplitude. The data of Figure 6-10 tend to indicate percentage growth rates are not a strong function of absolute voltage amplitude. As generally expected, the spread in the data at low amplitudes is greater than for larger voltages due to the greater influence of voltage uncertainties and measurement repeatability at low amplitudes.

Figures 6-11 and 6-12 compare the percentage growth rates per cycle between domestic plants Farley-1, Farley-2 and Plant F with that for Plant H-1. Figure 6-11 shows the individual data points while Figure 6-12 compares average growth rates and standard deviations. The averages are displayed for different ranges of the initial amplitude. The first range is 0 to 0.75 volts, the second range is 0.75 to 2.5 volts and the third is for initial amplitudes greater than 2.5 volts. In the case of the U.S. plants there is very little data above 2.5 volt amplitude; hence such data is included in the second range. The French data (Plant H-1) indicate percent growth rate nearly independent of initial amplitude whereas the domestic units display percent growth rates decreasing with increase in initial amplitude.

The domestic plants dominate the growth rate data of Figures 6-11 and 6-12 for low amplitudes with the French data extending to larger amplitudes. The results indicate that percentage growth rates are roughly comparable between domestic and European plants. In the calculation of average growth rates, the negative growth rates (see Figure 6-11) were conservatively treated as zero growth rates. Ignoring the negative growth rates biases the average growth by including all positive but not all negative random fluctuations. Part of the apparently larger number of negative voltage growth rates for the domestic data may result from variations in calibration standards for the 20% depth normalization which may be more sensitive to fabrication tolerances than the 100% depth normalization standards employed for the European data.

6.10 Field Data Conclusions

The following conclusions can be drawn from the field data described above:

1. Burst tests performed on pulled tubes, which to date include signal levels up to 10 volts, show burst pressures exceeding 3 times normal operating pressure differential, adjusted for operating temperature effect on material properties.
2. The pulled tube, leak rate test results indicate the potential for low []9.
3. Pulled tube examination results indicate that through wall cracks can potentially occur below 10 volts but that the associated crack lengths are short with no measurable leakage at operating conditions.
4. The smallest voltage identified for a through wall crack is 1.9 volts.
5. Leakage at operating conditions has not been identified for bobbin coil voltage below []9 volts with only 1 indication of leakage below 13 volts.
6. Negligible leakage is expected from ODSCC at TSPs based on domestic experience as well as European experience with voltage amplitudes higher than the domestic operating experience.

7. Percent growth in voltage amplitude tends to be approximately independent of voltage amplitude for the available French data while decreasing with amplitude for the domestic plants, including Farley. Assuming growth in voltage is independent of amplitude appears to be very conservative for the Farley SGs.

To supplement the above field data to define tube plugging limits, laboratory tests were performed with the following areas of emphasis:

- o Improved definition of tube burst capability as a function of bobbin coil voltage to better define voltage levels that meet Reg. Guide 1.121 guidelines for burst pressures of 3 times normal operating pressure differentials.
- o Improved resolution of leak rate potential for normal operating and SLB conditions above about 2 volt signal amplitudes.
- o Determining NDE uncertainties associated with application of voltage plugging limits.

Table 6.1

Field and Laboratory Data Utilized for Tube Plugging Criteria Development

Tube Burst Capability: Burst Pressure vs Voltage Correlation

- o Pulled Tube Data
- o Model Boiler Specimens

SLB Leakage Model

- o Pulled Tube Data
- o Model Boiler Specimens
- o Plant Inspection Results
 - ODS/CC Indication Distributions
 - Growth Rates

Operating Leakage Assessment

- o Pulled Tube Data
- o Operating SG Leakage Occurrences
- o Field Inspection Data for Tubes Without Identified Leakage
 - Larger (>1 volt) Indications
- o Model Boiler Specimens

NDE Evaluation: Specimen Characterization, Inspection Sensitivity/Uncertainties

- o Model Boiler Specimens
- o Pulled Tube Data

Influence of Tube Denting on Leakage

- o Fatigue Specimens
- o Doped Steam Specimens

Effects of SLB Loads on TSP Displacement

- o Pull Tests on Laboratory Dented Specimens
- o Thermal-Hydraulic and Structural Analyses

Table 6.3

Field Experience: Suspected Tube Leakage for ODSCC AT TSPs⁽¹⁾

Plant	Inspection	Bobbin Coil		Comments
		Volts	Depth	

Notes:

- 1 Field experience noted is for nominal 0.750" OD tubing with 0.043" wall thickness. No data are known to be available for tubes with 0.875" OD.

- 2 Reported voltages were adjusted (values given in parentheses) to the normalization in this report of 2.75 volts for 20% ASME flaw and 400/100 kHz mix. The adjustment factor was developed based on voltage ratios measured between a metric calibration standard as used to obtain the original data and the reference ASME standard of this report. This adjustment provides an order of magnitude conversion to make these data roughly comparable to other data in this report. However, any conversion factor is disputable because it depends on the procedural/environmental conditions and thus may vary from case to case.

Table 6.4

Comparisons of Voltage Amplitudes Between U.S.-ASME and European Standards

Channel	U. S. - ASME Standard						French	Belgian	U.S.
	20%	40%	60%	80%	100%	Support Plate	4-hole, 1 mm dia. holes 100%	4-hole, 1.25 mm dia. holes 100%	4-hole 33 mil dia. holes 100%
U.S. Calibration Procedure □									
400/100 mix	2.75	2.8	5.3	5.6	8.7	<0.6	10.7	18.96	6.4
400 kHz	4.0	3.5	5.5	5.5	7.8	8.2	9.8	17.19	5.4
240 kHz	6.3	5.4	7.9	7.3	9.5	17.4	12.4*	21.15**	7.6
200 kHz	5.9	4.9	7.1	6.3	8.0	17.5	10.9	18.08	-
100 kHz	5.9	2.8	3.6	3.1	3.8	14.5	5.4	8.5	5.2
French Calibration Procedure									
240 kHz	0.66	0.56	0.82	0.76	0.99	1.8	1.3*		
200 kHz	0.69	0.58	0.84	0.74	0.95	2.09	1.3		
400/100 mix	0.33	0.34	0.64	0.67	1.04	~0.1	1.3		
Belgian Calibration Procedure									
240 kHz	0.59	0.51	0.74	0.68	0.90	1.64		2.0**	
200 kHz	0.62	0.53	0.76	0.67	0.85	1.87		2.0	
400/100 mix	0.29	0.29	0.55	0.59	0.91	~0.1		2.0	
400 kHz	0.46	0.41	0.63	0.63	0.91	0.95		2.0	

□ U.S. procedure involves setting up the signal for 20% ASME holes at 4 volts for 400 kHz differential channel or 2.75 volts for 400/100 kHz differential mix and then using the "Save/Store" functions of the Zetec DDA-4 software for carrying over the calibration to all other channels.

* When using the U.S. calibration procedures, the French 4-hole standard gives 12.4 volts at 240 kHz and 10.7 volts with the 400/100 kHz mix. It is 1.3 volts for the French calibration. Thus U.S. values at 240 kHz/French values at 240 kHz equals ~9.5. U.S. values at 400/100 mix/French values at 240 kHz equals ~8.2.

** When using the U.S. calibration procedures, the Belgian 4-hole standard gives 21.15 volts at 240 kHz and 18.96 volts with the 400/100 kHz mix. It is 2.0 volts for the Belgian calibration. Thus U.S. values at 240 kHz/Belgian values at 240 kHz equals ~10.75. U.S. values at 400/100 kHz mix/Belgian values at 240 kHz equals ~9.5. For general data comparisons, Belgian and French data can be reasonably compared without adjustments or by multiplying the Belgian data by ~0.9 to obtain French volts.

Figure 6-1

Pulled Tube Data: Bobbin Coil Voltage and Indicated Depth



Chart Fig 6-1 2/10/92

Figure 6-2

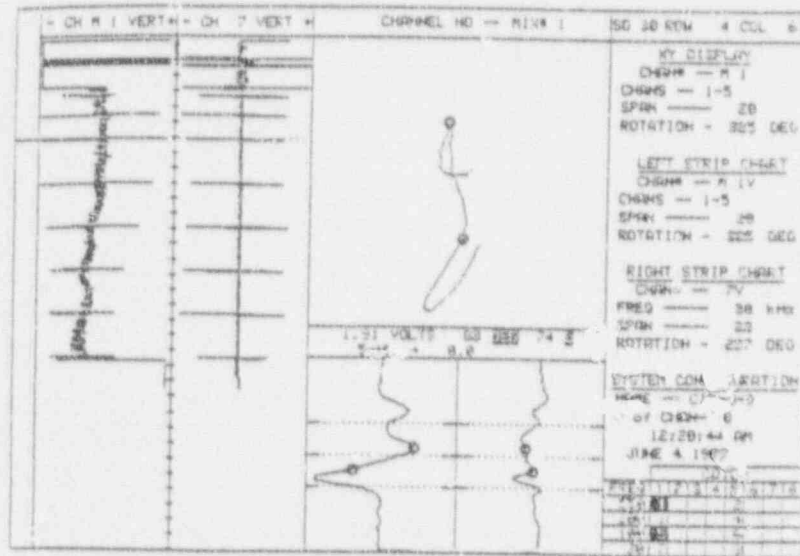
Pulled Tube Data: Bobbin Coil Voltage and Depth from Destructive Exam



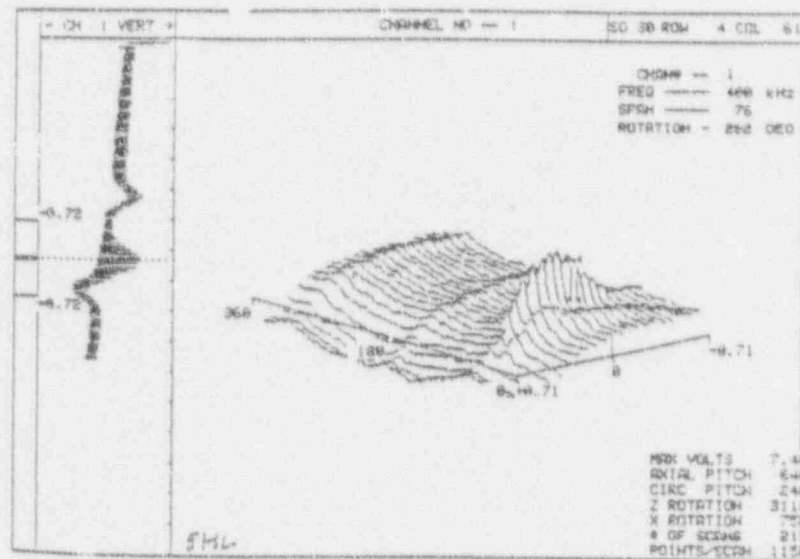
Chart Fig 6-2 2/10/92

Figure 6-3

Field Bobbin Coil and RPC Traces for 3/4 inch Tube



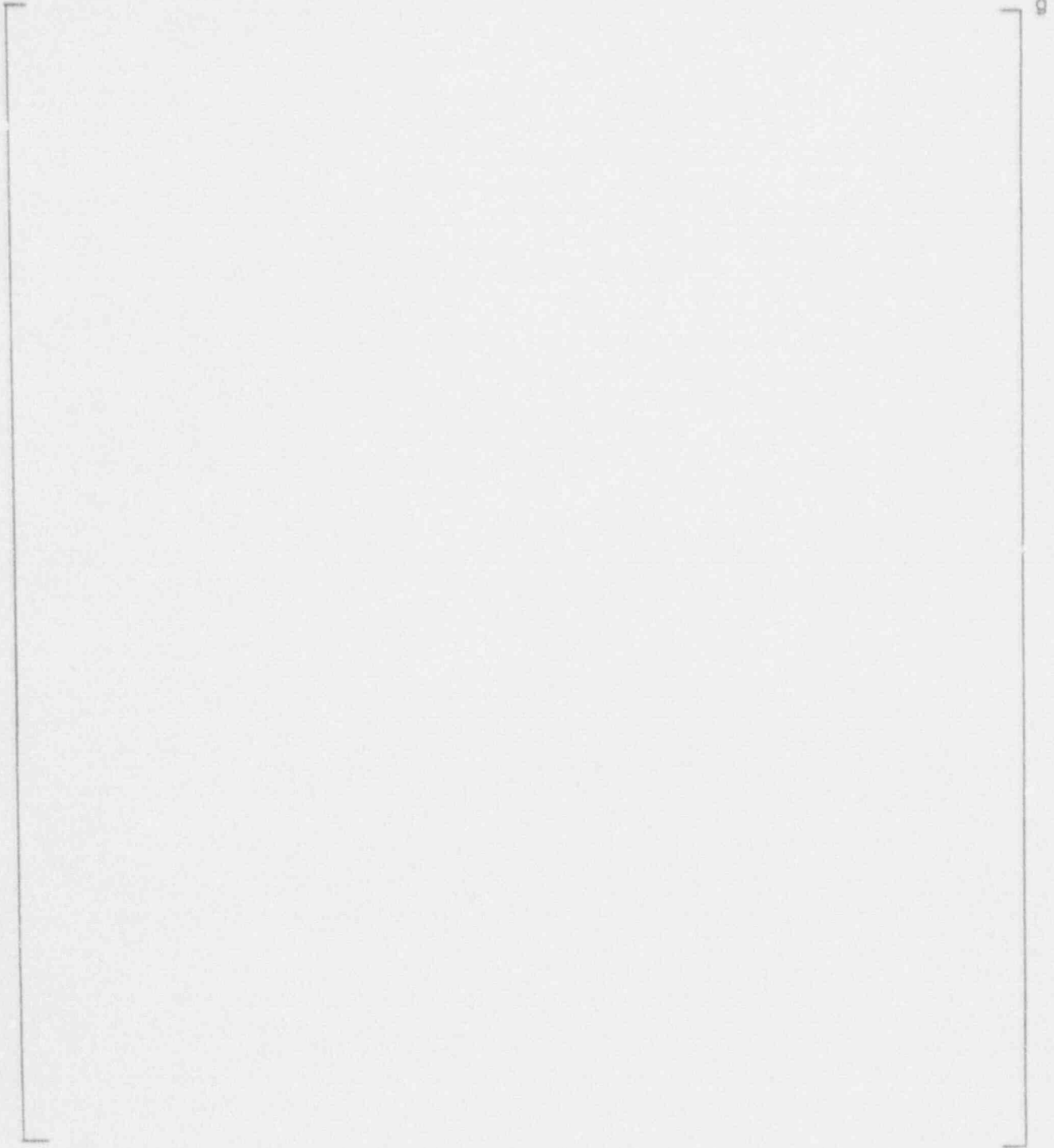
Field bobbin probe data for Plant B-1, tube R4-C61 at TSP 5, using 550/100 kHz differential mod- mix. A 1.91 volt indication with 74% depth is observed.



Field RPC data for Plant B-1, tube R4-C61 at TSP 5. A 0.4 inch long axial indication is observed.

Figure 6-4

Field Inspection Data for Tubes Without Operating Leakage



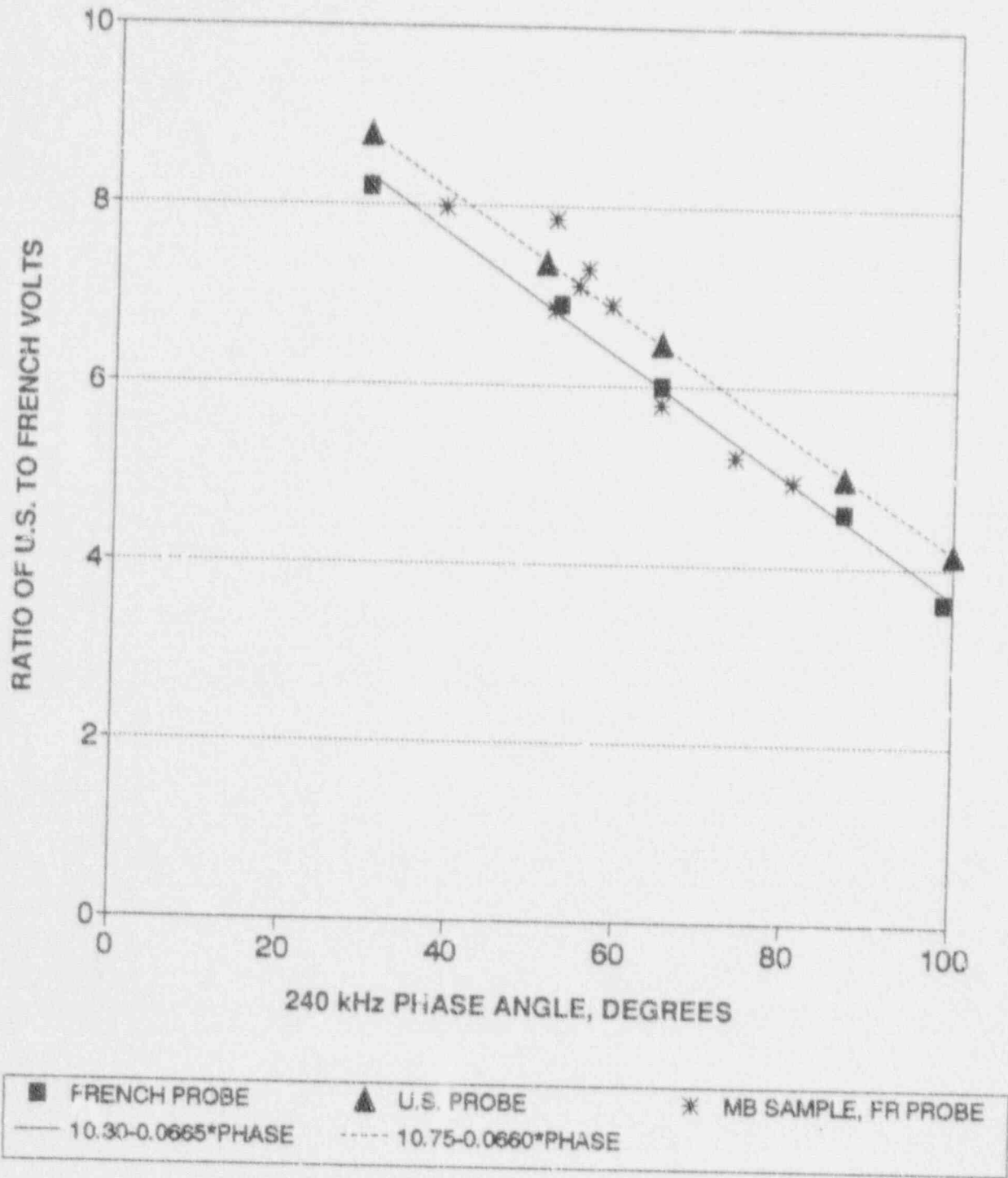


Figure 6-5. Ratio of U.S. to French Volts

Figure 6-6

Distribution of TSP Indications for Plant H-1 (1986 to 1990)

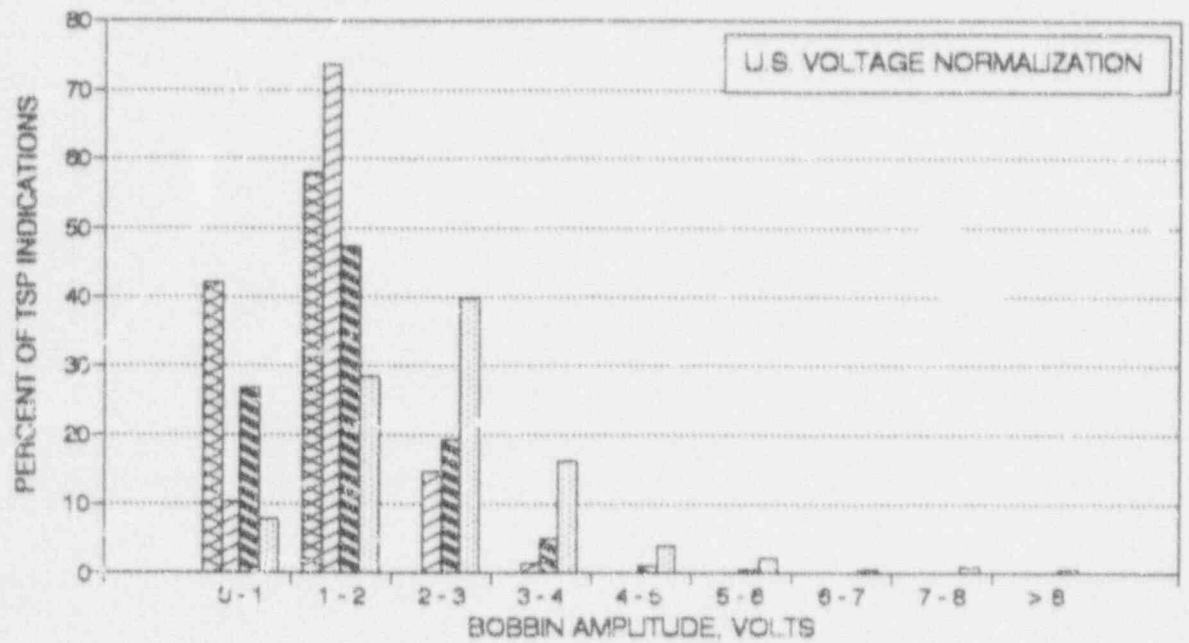
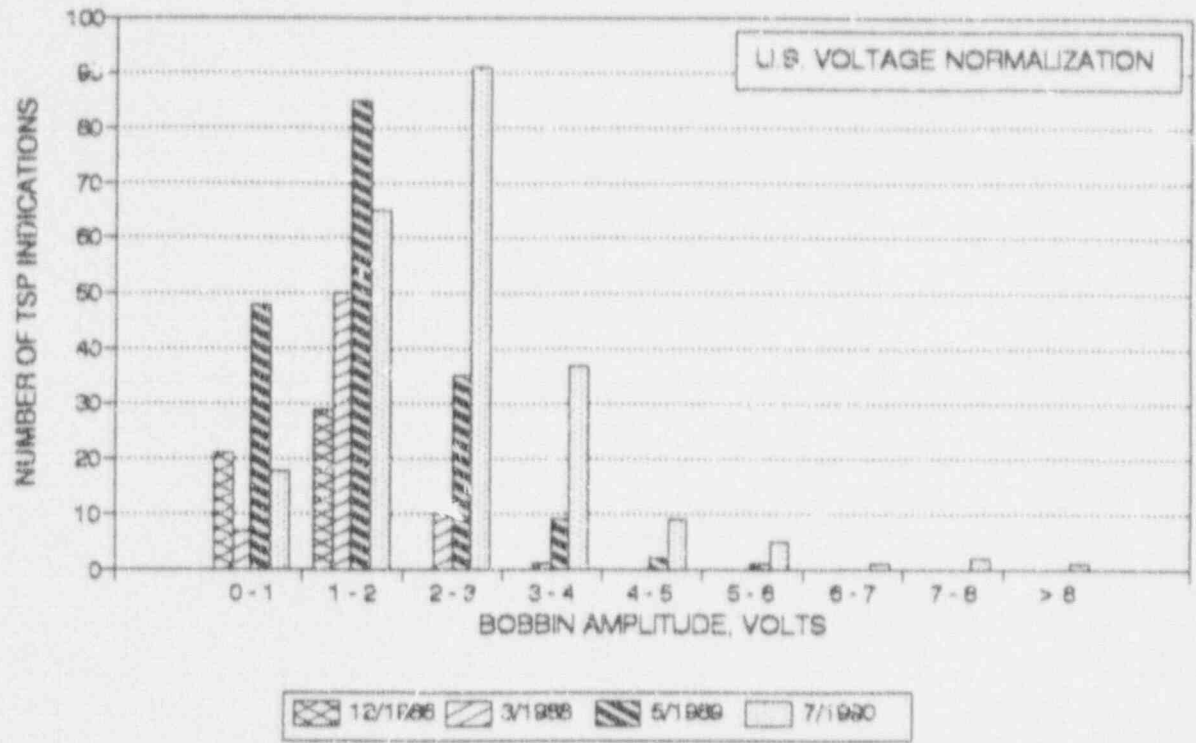


Figure 6-7

Comparison of Voltage Indications at TSPs Between U.S. and European Plants

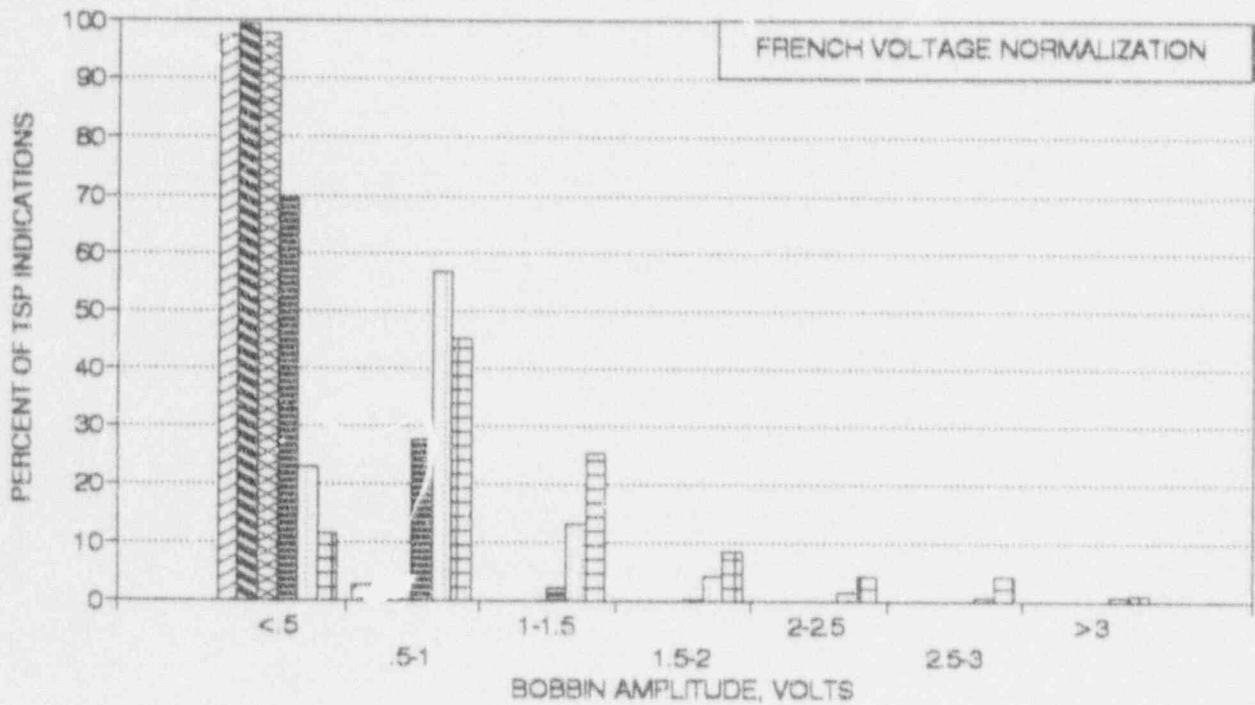
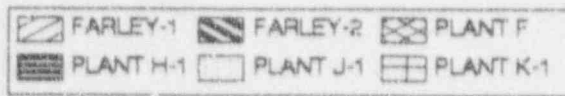
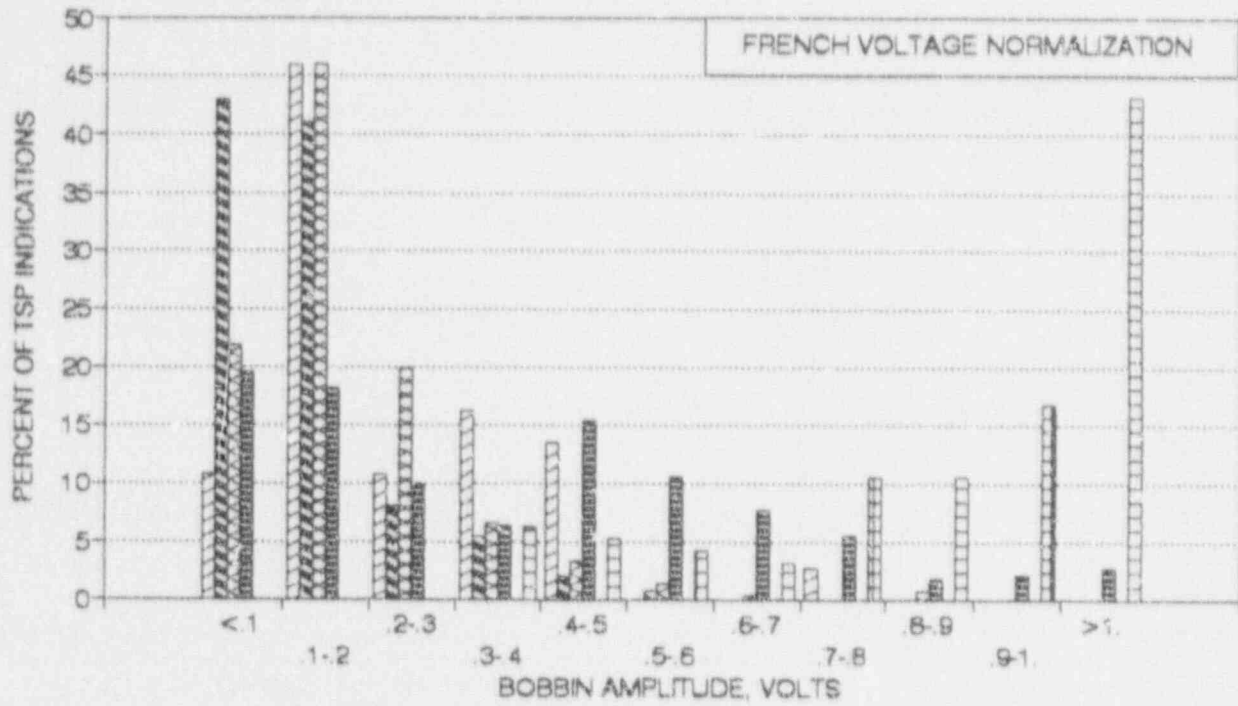
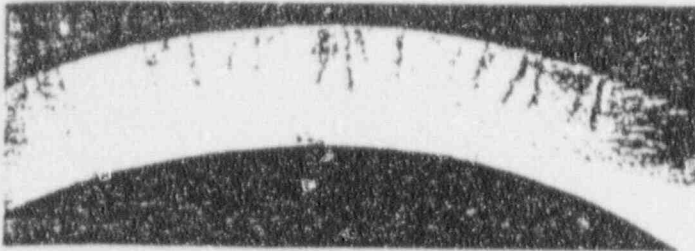


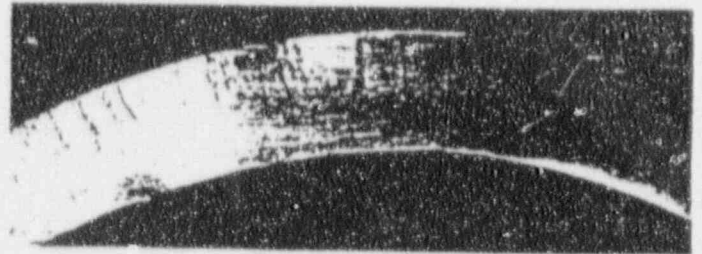
Figure 6-8

Crack Morphology for Pulled Tube from French E/C

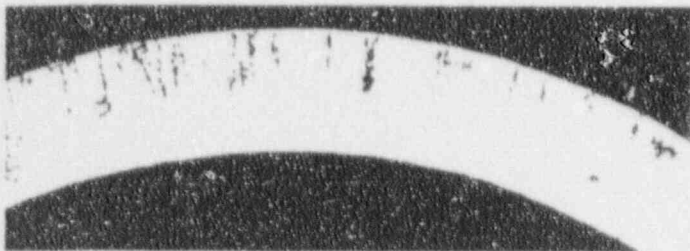
Coupe transversale 2 (cf. annexe 8)
Aspect de la fissuration sur différents plans de polissage



13 X 12,5



14 X 12,5



15 X 12,5



16 X 12,5

Figure 6-9

Pulled Tube Destructive Exam Data Including French and Belgian Data



Chart Fig 6-9 2/10/92

Figure 6-10

TSP Indication Voltage Growth Rates for Plant H-1

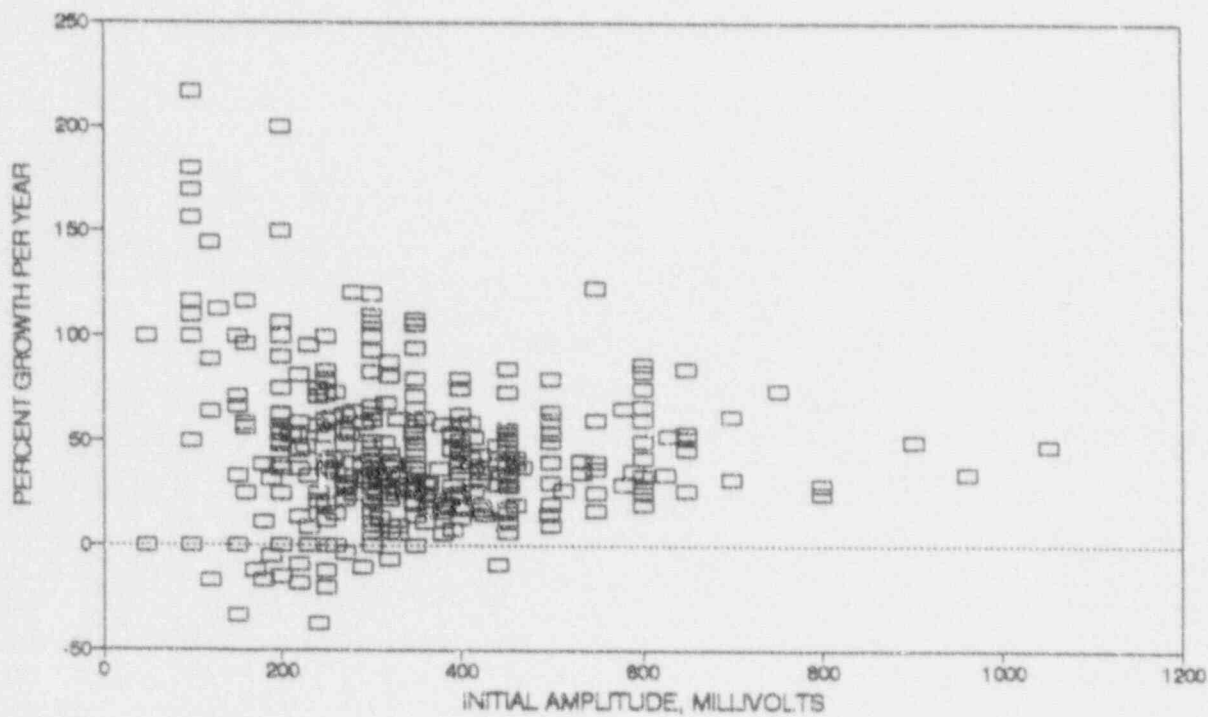
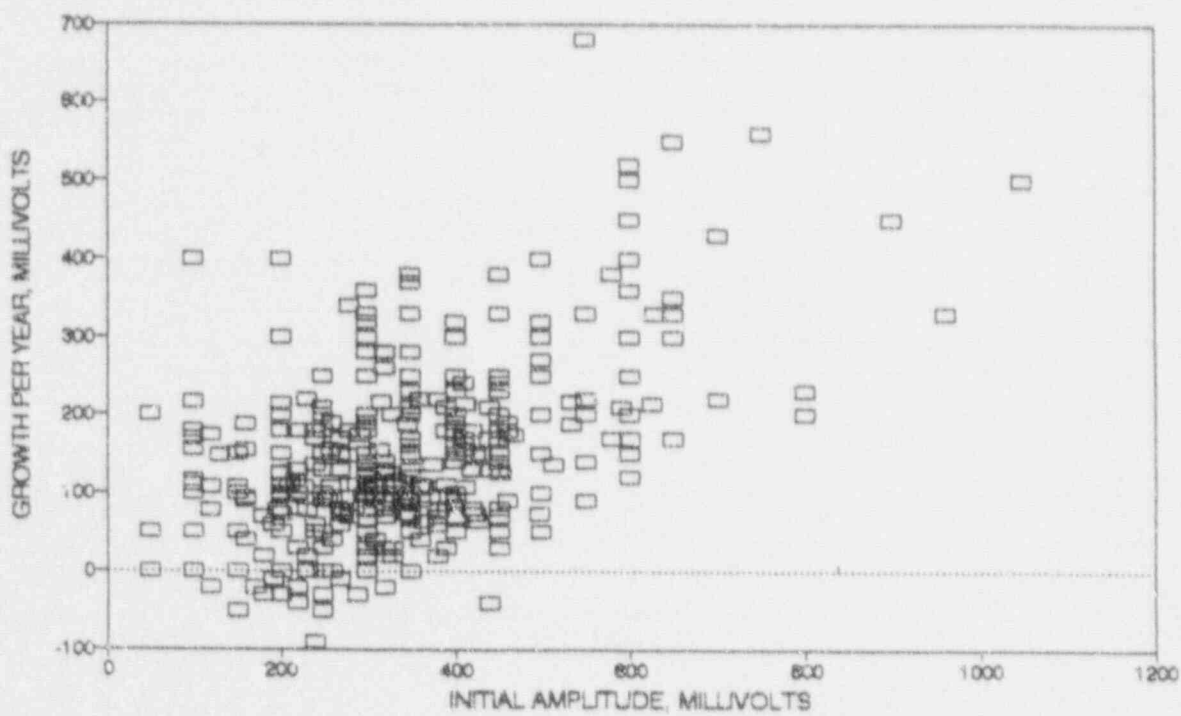
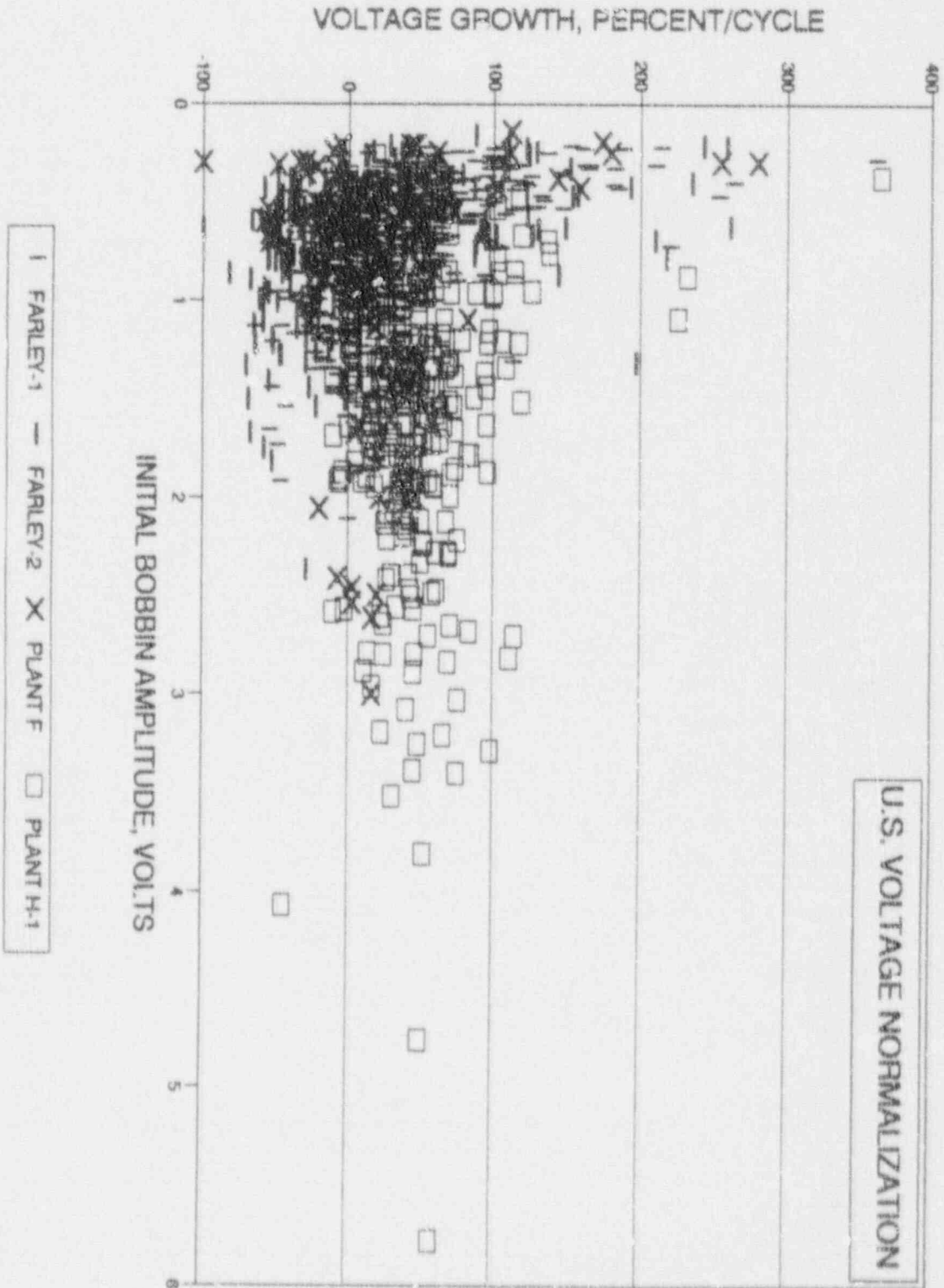


Figure 6-11. Growth Rate Data for Farley, Plant F and Plant H-1



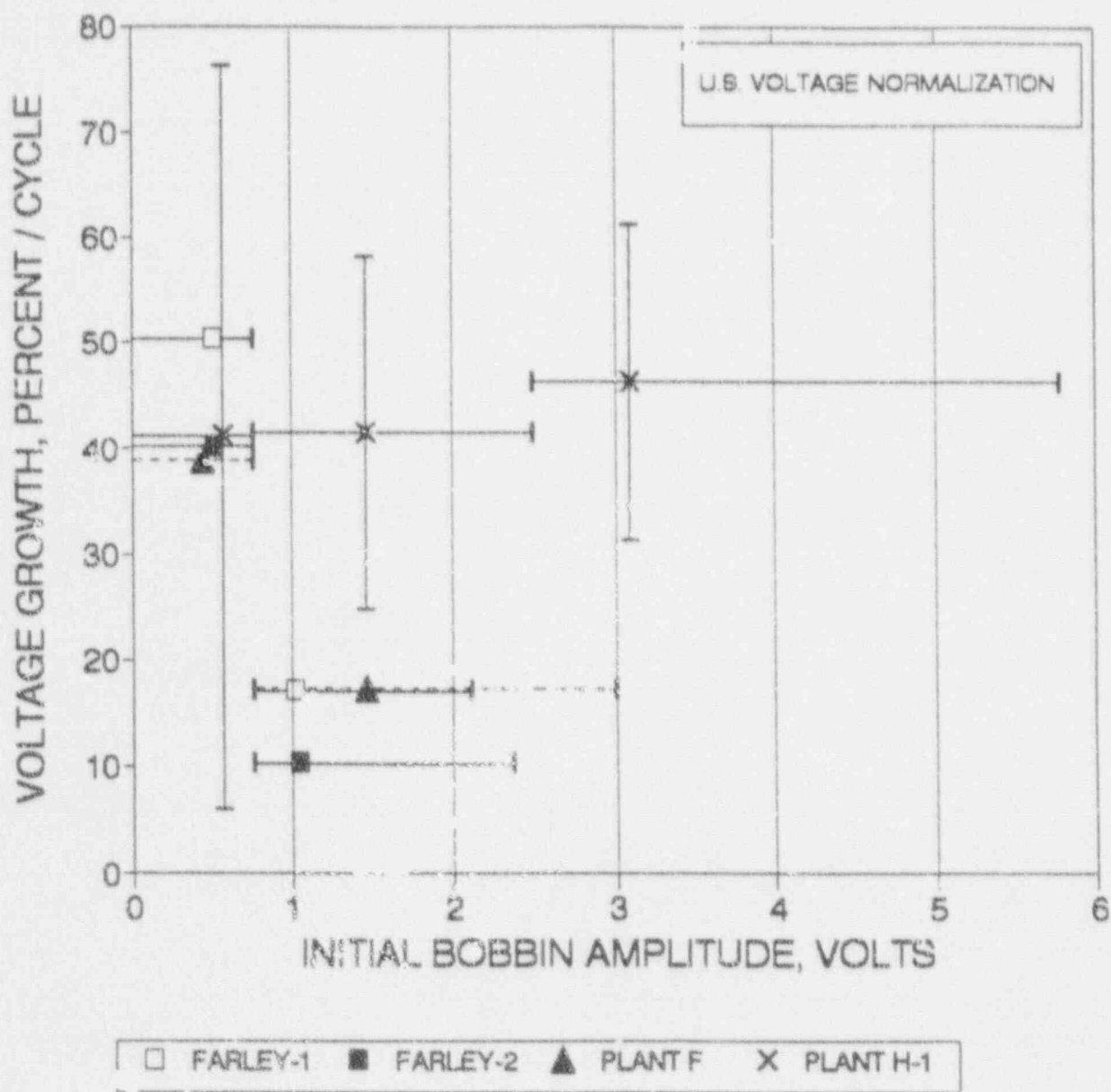


Figure 6-12. Average Percent Voltage Growth Rates for Farley, Plant F and Plant H-1

7.0 LABORATORY SPECIMEN PREPARATION

7.1 Model Boiler Specimens

The Forest Hills Single Tube Model Boiler test facility consists of thirteen pressure vessels in which a forced flow primary system transfers heat to a natural circulation secondary system. Test specimens are placed around the heat transfer tube to simulate steam generator tube support plates. One to six tube support plate crevice assemblies are typically included in a given test. A schematic of the test facility is presented in Figure 7-1, and typical thermal and hydraulic specifications are presented in Table 7.1. As indicated in the table, these specifications are representative of those in a pressurized water reactor steam generator.

Four series of Single Tube Model Boiler tests have been performed to provide test pieces having through wall cracks for subsequent nondestructive examination, leak rate measurements, and destructive examination. The first series consisted of 15 archive crevice assemblies which were produced in previous Westinghouse-funded testing; the second and third series each consisted of 30 crevice assemblies, while the fourth series consisted of 45 crevice assemblies. The crevices in the third and fourth series were specified so as to produce crack networks having lower bobbin probe voltages.

Series 1 Tests: A summary of the archive test pieces is presented in Table 7.2. The test pieces are listed by their tube designation and their location on the tube. The sludge type refers to the manner in which sludge was placed in the tube support plate crevices. Chemically consolidated sludge is formed by baking a mixture of sodium hydroxide, sodium silicate and sodium phosphate with the sludge; mechanically consolidated sludge is formed by hydraulically pressing the sludge into the tube support plate, drilling a hole in the sludge, and sliding the tube through the hole; the fritted design uses an Inconel sinter at each end of the crevice to hold the sludge in place. The test containing an eccentrically mounted tube support plate in which the sludge was removed from the crevice was used to simulate chemical cleaning. All tests utilized simulated plant sludge, consisting of approximately 60% magnetite, 32% copper, 5% copper oxide, 2% nickel oxide, and 1% chromium oxide.

The cracks were produced in what is termed the reference cracking chemistry, consisting of either a 600 ppb (1X) or a 6 ppm (10X) sodium carbonate solution in the makeup tank. Because of hideout in the crevices, the boiler concentration is typically about 75% of the makeup tank concentration. Hydrazine and ammonia are also added to the makeup tank for oxygen and pH control, respectively.

The tubing used for the tests was taken from Heat 2675. This heat of mill annealed Alloy 600 was fabricated by Westinghouse and has been used extensively in other stress corrosion cracking programs. The tubing has a 0.875 inch outside diameter.

Series 2 Tests: The initial program test pieces consisted of eight crevice simulants which were mounted on two tubes. These tests were specified to produce rapid through wall cracking, so that the 10X reference cracking chemistry was utilized. Heat 2675 was also used for these tests. One test utilized chemically consolidated simulated plant sludge (tube 543), while the other used mechanically consolidated sludge (tube 542). The sludge filled the entire crevice volume.

Primary to secondary leakage was noted in the chemically consolidated test after 10 days of boiler operation, and in the mechanically consolidated test after 24 days of operation. During the subsequent nondestructive examination (NDE), indications were identified adjacent to the teflon collars used to support the tube support plates, as well as within the support plates. The bobbin probe voltages were found to be higher than those typically encountered in plant eddy current examinations. As a consequence, subsequent tests were designed to produce smaller cracks.

Series 3 Tests: Since the NDE of the archive and series 2 test pieces produced higher voltages than are measured in steam generator tube support plate crevices, the sludge configuration of the series 3 tests was modified to produce shorter cracks. Two tests were specified, with each containing four test specimens. Both tests utilized chemically consolidated sludge, with the sludge occupying a 60° arc and half of the 0.75 inch height of the tube support plate crevice. As in the series 1 tests, Westinghouse heat 2675 was used for the tubing.

Tube 557 utilized simulated plant sludge and was treated with the 10X reference cracking chemistry, in order to produce accelerated SCC. Tube 558 utilized chromium oxide for crevice packing and was treated with the 1X cracking chemistry. Previous testing has found that using chromium oxide rather than simulated plant sludge promotes IGA rather than SCC. It was believed that field experience with IGA produces lower bobbin probe voltages than does SCC. The 1X chemistry was specified because it was hypothesized that it would produce less grain boundary corrosion, and therefore lower voltages.

Through wall cracking of tube 557 was produced after 16 days of operation. NDE of the tube indicated the voltages to be generally lower than in the previous tests, but still well above what is typically found in the field in TSP crevices.

Operation of tube 558 continued with the 1X chemistry for 56 days, at which time the specification was changed to the 10X chemistry in order to accelerate the corrosion rate. The test was then operated for an additional 52 days, at which time a primary to secondary leak occurred. Eddy current inspection identified a through wall crack at the bottom tube support plate elevation, with the crack producing a 6.5 volt signal and a 69% indicated depth. The voltage is typical of field indications of through wall cracks, while the indicated depth is shallower than what is typically found in the field. Following inspection, the remaining three tube support assemblies were returned to test. Testing continued for an additional 50 days, extending beyond the scheduled program completion date.

Series 4 Tests: This test series was undertaken after it was found that the test specimens produced in doped steam environments exhibited high leak rates compared to those found in tests of tubes pulled from the field. The high leak rates of the specimens produced in doped steam were attributed to the plastic deformation of the tubing required to obtain accelerated corrosion. Consequently, the series 4 tests were intended to produce both bobbin probe voltages and leak rates representative of those expected to be found in the field.

The series 4 tests contained 45 specimens, mounted on eight tubes. 23 of the specimens were fabricated from Teflon, while the remaining 22 were fabricated from carbon steel.

Teflon collars were utilized because the cracks located beneath the teflon collars in the series 2 tests typically produced lower bobbin probe voltages than did the cracks located adjacent to the collars.

Unlike the series 2 and 3 tests, the series 4 tests used tubes supplied to Westinghouse by the EPRI NDE Center. Tubing Heat 96834, lot 8, was originally fabricated by another NSSS vendor, and is the same as was used in the doped steam testing (see Section 7.2). The heat of material was changed because the doped steam testing found that this heat produced more accelerated corrosion than did the Westinghouse heat.

The design of the test specimens was also modified in order to reduce the magnitude of the eddy current voltages. These modifications are outlined in Table 7.3. The specimens in tests 1 were configured to fit snugly around the tube (as did the teflon collars in the series 1 tests), but the height of the collars was varied between 0.25 and 0.7 inch. The expectation is that the shorter collars should limit the length of the cracks which can be produced.

The inside face of the specimens in test 2 were machined to produce a grid pattern to limit the crevice area between the tube and the collar. Six holes were first drilled around the periphery of the inside face of the specimen, so that the crevice would have an approximate 40° arc width. Two rings were milled on the face of three of the specimens. The rings had a width of 0.125 inch, so that the face was divided into three contact regions, with the outer regions having a 0.125 inch width and the central region having a 0.25 inch width. A helical pattern was machined into the face of the remaining three specimens, with the width of the helix being 0.125 inch and the pitch of the helix being 0.25 inch.

The specimens in test 3 all had a 0.75 inch height, but the diametral gap width was varied between a snug fit (as in test 1), a 10-mil gap, and a 20-mil gap. The gap width was varied because previous testing has found that the gap width affects the rate and location of corrosion.

Tests 4 and 5 both utilized chemically consolidated simulated plant sludge located adjacent to carbon steel tube support specimens. Two sets of specimens having thicknesses of 0.25, 0.50, and 0.75 inch were used in each test. The sludge was consolidated over a 40° arc width within the crevice. The 0.25 inch specimens contained one sludge region, which occupied essentially the full thickness of a specimen; the 0.5 inch specimens contained two sludge regions, separated by a 100-mil wide band at the center; the 0.75 inch specimens contained three sludge regions, separated by two bands.

Test 6 contained carbon steel specimens having the same range of thicknesses as in tests 4 and 5. Instead of using simulated plant sludge, however, the crevices remained empty, with the specimens being held in place with porous Inconel sinters located at each end of the crevice. This design has been used to produce accelerated intergranular corrosion in previous tests.

The 10X reference cracking chemistry was specified for use in tests 1 through 6. Because of concern that this specification may produce excessive grain boundary corrosion and therefore high bobbin probe voltages, two additional tests (7 and 8) were specified to utilize the 1X chemistry. Test 7 utilized five Teflon specimens, while test 8

utilized four carbon steel specimens. The specimens utilize a range of designs selected from tests 1 through 6.

The eight tests accumulated between 96 and 328 days of operation, depending upon the test. Through wall cracks had been produced in five test pieces, with a few additional test pieces containing partial through wall cracks. The designs of the test specimens having through wall cracks are presented in Table 7.4. As indicated in the table, three of the through wall cracks were produced adjacent to teflon simulants in test 3, while the other two through wall cracks were produced adjacent to carbon steel simulants. The only common feature of the crevice configurations which produced the cracks is that the crevice length was always 3/4 inch, perhaps suggesting that the shorter crevices could not produce sufficient superheat to promote accelerated corrosion.

Series 5 Tests Two additional tests were conducted when it became apparent that accelerated corrosion was not being produced in Series 4 tests. Both tests utilized tubing from heat 2675, the same heat used in the first three series, rather than heat 98834, which had been used in Series 4. Test 575 contained four tube support simulants having the same fritted design used in the Series 1 tests. The test was conducted because the test piece had been assembled for a previous program, but had not been used. Since this design was known to produce accelerated corrosion, it was hoped that the test could also provide additional crevice assemblies having small indications. While through wall cracking was produced after 25 days of boiler operation, the associated eddy current signals were on the order of 20 volts, so that no further evaluation of the crevice assemblies was performed.

Test 576 was configured to produce small cracks, as was the intention of the Series 4 tests, while maintaining local crevice superheats in the same region as in the tests which produced accelerated corrosion. The crevices contained two sludge regions: a larger outer region of chromium oxide and a smaller inner region of simulated plant sludge. The expectation was that the cracking would be confined to the inner region, while the outer region would increase the superheat to values comparable to those in the early tests. Two of the crevice assemblies were specified to have the simulated plant sludge occupying the center of the top half of the chromium oxide region, while the other two crevice assemblies were specified to have the simulated plant sludge centered in the chromium oxide. In both cases, the inner region had a width of 0.125 inch and a height of 0.375 inch.

The test operated for 49 days with the 1X reference cracking chemistry, plus an additional 56 days with the 10X chemistry, when it was shut down because of a primary to secondary leak. Subsequent eddy current evaluation identified indications at two locations, as outlined below:

<u>Location</u>	<u>Voltage</u>	<u>Depth</u>
576-2	8.41	80%
576-4	8.43	86%

Both locations correspond to crevice configurations in which the simulated plant sludge was centered both axially and circumferentially with respect to the chromium oxide. An RPC evaluation identified three cracks; two were in the simulated sludge, while the third was in the

chromium oxide. These results suggest that the two-region sludge configuration is a promising means of producing smaller cracks in model boiler specimens.

7.2 Doped Steam Specimens

Axial stress corrosion cracks and crack networks were produced in 30 mil-annealed Alloy 600 tubes through exposure to a doped steam environment. The steam was produced from water containing 30 ppm each of chloride, fluoride, sulfate and nitrate anions as salts of sodium. The steam pressure was 3000 psi at a temperature of 750° F. Individual specimens were eight inches in length. Stressing was accomplished by clamping the tube at mid length between two flat steel plates, as shown in Figure 7-2. Ovalization of the tube resulted in outer fiber tensile yielding on the OD surface of the tube at the maximum diameter. The tube ends were sealed to permit internal pressurization of the tube during the autoclave exposure. The OD surface of the clamped tube was exposed to the 3000 psi doped steam environment in a one gallon autoclave. Nitrogen gas was used to pressurize the inside of the tube to 4500 psi producing a differential pressure across the tube wall of 1500 psi. The development of through wall cracking was detected by a drop in the internal pressure of the tube.

Table 7.5 summarizes the specimens tested in the doped steam environment. Two heats of mill annealed Alloy 600 tubing were used, Heat 2675 and Heat 96834L. The width of the clamp in contact with the tube was typically 0.25 inch but larger clamp widths were also used in an attempt to vary the crack morphology. All of the tube displacements were sufficient to cause outer fiber yielding. These displacements ranged from 0.030 to 0.005 inch. In general, the smaller displacements resulted in shorter crack lengths and an increase in the test exposure time. The eddy current voltages listed in Table 7.5 are preliminary values used to help decide the disposition of the tube and should not be confused with the eddy current results provided in Section 8 for the same samples using representative field inspection procedures. The crack lengths shown in Table 7.5 are from optical measurements on the tube OD surface obtained at low magnification and may differ significantly from later destructive examinations. Some attempt to control the length of crack initiation sites was made on the last 7 specimens listed in Table 7.5. Selected portions of these specimens were grit blasted. This procedure had no discernible effect on crack initiation. Selected specimens from Table 7.5 were forwarded for full NDE examination, leak and burst testing as described later.

7.3 Fatigue Pre-cracked Specimens

Through wall axial fatigue pre-cracks were developed in 12 mil annealed Alloy 600 tubes by cyclic internal pressurization. A starter flaw was spark machined half way through the wall of the 0.050 inch thick by 0.875 inch diameter tubing with a length of 0.25 inch. Cyclic internal pressurization then was used to grow the crack through wall. A soft plastic bladder seal was then inserted in the tube and cracks up to 0.70 inch in length were grown by a fatigue process. The pressure was adjusted during fatigue pre-cracking to maintain the maximum applied stress intensity factor below 25 ksi- $\sqrt{\text{in.}}$. The maximum plastic blunting of the crack tip was thus kept below 0.0003 inch. Table 7.6 lists the fatigue pre-cracked specimens, crack lengths and number of cycles. The fatigue pre-cracked samples have well characterized leak rates from previous evaluations, although the leak rates are large compared to those for ODSCC cracks. These samples were used in studies of the effect of denting at tube support plate intersections on leak rates through cracked tubes.

7.4 Chemically Dented Tubes

Fatigue precracked tubes and tubes with stress corrosion cracks were used to simulate cracked tubes in tube support plates which are also dented. Carbon steel collars were used to simulate tube support plates. These collars were drilled with the nominal clearance hole for 0.875 inch diameter tubing. The collars were then packed with magnetite using a hydraulic press. The pressed magnetite was drilled out to a tight fit hole for insertion of a precracked tube. The final configuration was a carbon steel collar with an inside diameter of 0.016 inch, a pressed 0.014 inch layer of magnetite and a gap in the tube wall. The ends of these specimens were sealed and the specimens were exposed to a 0.2 M cupric chloride solution in an autoclave at 572°F.

Corrosion of the carbon steel and a smaller reaction layer with the Alloy 600 tubing resulted in corrosion products which tightly packed the tube-tube support plate crevice and led to a small amount of denting of the tube. Table 7.7 lists the dented specimens, the eddy current dent voltage and the average estimated radial dent size.

A section through the tube and collar of specimen Trial-1 is shown in Figure 7-3. The packed magnetite and the corrosion products in the crevice are clearly evident. A scanning electron photograph of the polished section in Figure 7-3a shows several layers of corrosion products along with the starting layer of packed magnetite. The EDS nickel and iron maps of Figure 7-4 show that the corrosion product adjacent to the Alloy 600 tube is enriched in nickel and somewhat depleted in iron. The corrosion product adjacent to the carbon steel collar appears to have the same iron content as the packed magnetite layer. The corrosion product layers in general appear to be relatively dense and any leakage path would be highly tortuous.

7.5 Crack Morphologies

Plugging criteria which are partially based upon eddy current characterization, leak rate and burst strength testing of laboratory specimens depend upon a reasonable simulation of actual service produced cracks. The crack morphologies of service tubes, doped steam test specimens and model boiler test specimens are presented in this section. An intergranular mode of cracking is common to cracks produced in these three environments. Figure 7-5 illustrates this fact with scanning electron fractographs. A further illustration of intergranular cracking is provided by the metallographic details shown in Figure 7-6.

Stress corrosion cracking patterns on the OD of Alloy 600 tubes at tube support plate intersections range from a few to many axial cracks distributed around the circumference of the tube. The model boiler test specimens also show this characteristic. Figure 7-7 shows several arrays of cracks and a larger single crack. The cracks in the doped steam specimens tend to be either a single axial network or axial cracks 180 degrees apart. This is due to the nature of loading of the doped steam specimens. Clamping of the tube leads to a 180 degree symmetry of stresses, bending across the wall thickness and outer fiber bending stresses beyond the yield point. In model boiler specimens, as in actual tube support plate intersections, the stresses are uniform around the circumference of the tube and the occurrence of single or multiple axial cracks is controlled by the crevice conditions. The differential pressure hoop stress is relatively low, about 12 ksi. Hence the model boiler specimens experience essentially prototypic loading while the doped steam specimens experience stresses far beyond actual service conditions. The doped steam environment is substantially less aggressive than that produced in the model boiler tests and thus clamping loads are required in addition to the

pressure stress to produce cracking in reasonable lengths of time. The high stress clamped condition of the doped steam specimens led to a higher degree of through wall cracking for a given total axial crack network length and hence higher leak rates at a given eddy current bobbin coil voltage. Another complicating factor is the fact that relaxation of the through wall bending stresses when the clamping fixture is removed can lead to contact across the faces of the crack. This would provide an eddy current path and reduce the bobbin coil voltage relative to a crack with non-contacting faces, as in the model boiler specimens. The above considerations indicate that the clamped test condition of the doped steam specimens produced non-prototypic stress corrosion cracks, particularly as they relate to any correlation between bobbin coil voltage, leak rate and burst strength. Therefore, only the model boiler specimen test results were added to the data base used to develop tube plugging criteria.

Table 7.1

MODEL BOILER THERMAL AND HYDRAULIC SPECIFICATIONS

Primary loop temperature	327°C (620°F)
Primary loop pressure	13.8 MPa (2000 psi)
Primary boiler inlet temperature	324°C ± 3°C (610°F ± 5°F)
Primary boiler outlet temperature	313°C ± 3°C (595°F ± 5°F)
Secondary T _{sat} at 5.5 MPa (800 psi)	271°C ± 3°C (520°F ± 5°F)
Steam bleed	8 cm ³ /min (continuous)
Blowdown	1 cm ³ /min (8 hr/day)
Nominal heat flux	16.28 x 10 ⁴ kcal/m ² -hr (60,000 Btu/ft ² -hr)

Table 7.2

DIRECTORY OF SINGLE TUBE MCDEL BOILER TEST SERIES 1, 2, AND 3

<u>TUBE PIECE</u>	<u>INITIAL SLUDGE TYPE</u>	<u>THRU WALL</u>	<u>LEAK TESTED</u>
Series 1 - Archive Test Pieces			
528-1-1	Chem. Cons.	Yes	No
528-2-1	Chem. Cons.	Yes	No
530-1-1	Chem. Cons.	Yes	Yes
530-1-2	Chem. Cons.	Yes	No
530-2-1	Chem. Cons.	Yes	No
530-3-1	Chem. Cons.	Yes	No
530-4-1	Chem. Cons.	Unknown	No
533-1-1	Mech. Cons.	Yes	No
533-2-2	Mech. Cons.	Yes	No
533-4-1	Mech. Cons.	Yes	Yes
532-1-1	Frit	Yes	No
532-2-1	Frit	Yes	No
500-1-1	Mech. Cons.	No	Yes
525-1-1	Frit	No	Yes
535-1-1	Mech. Cons.	No	No
536-1-1	Eccentric	No	Yes
Series 2 Test Pieces			
542-1-1	Mech. Cons.	Yes	Yes
542-2-1	Teflon Sup.	Yes	Yes
542-2-2	Mech. Cons.	No	No
542-3-1	Mech. Cons.	No	Yes
542-4-1	Mech. Cons.	Yes	Yes
543-1-1	Chem. Cons.	Yes	Yes
543-2-1	Chem. Cons.	Yes	Yes
543-3-1	Chem. Cons.	No	Yes
543-3-2	Chem. Cons.	No	Yes
543-3-3	Teflon Sup.	No	Yes
543-4-1	Chem. Cons.	No	Yes
Series 3 Test Pieces			
557-1-1	Chem. Cons.	No	Yes
557-2-1	Chem. Cons.	Yes	Yes
558-1-1	Chromium Oxide	Yes	Yes
558-2-1	Chromium Oxide	No	No
558-3-1	Chromium Oxide	No	No
558-4-1	Chromium Oxide	No	No

Table 7.3

SUMMARY OF SERIES 4 TEST SPECIFICATIONS

Test	Sludge	Collar Material	No. of TSP's	Thick. (in.)	Crevice * Configuration	Chemistry
1	5	Teflon	2	0.25	Snug Fit	10X
			2	0.5	Snug Fit	
			2	0.75	Snug Fit	
2	567/9	Teflon	3	0.75	Mach'd Rings	10X
			3	0.75	Mach'd Helix	
3	568/12	Teflon	2	0.75	Snug Fit	10X
			2	0.75	10 Mil Gap	
			2	0.75	20 Mil Gap	
4	570/2	C. Steel	2	0.25	Sim. Plant	10X
			2	0.5	Sim. Plant	
			2	0.75	Sim. Plant	
5	571/4	C. Steel	2	0.25	Sim. Plant	10X
			2	0.5	Sim. Plant	
			2	0.75	Sim. Plant	
6	572/5	C. Steel	3	0.25	Frits, Empty	10X
			3	0.5	Frits, Empty	
7	573/6	Teflon	2	0.25	Snug Fit	1X
			1	0.5	Snug Fit	
			1	0.75	Snug Fit	
			1	0.75	Mach'd Rings	
8	574/3	C. Steel	2	0.75	Sim. Plant	1X
			1	0.25	Frits, Empty	
			1	0.75	Frits, Empty	

* Where a sludge type is listed, the sludge is chemically consolidated over a 40 to 60 degree arc width within the crevice; the machined rings are formed by dividing the tube support plate circumferentially into six land regions, and axially into two rings having a 1/8 inch thickness; the machined helix is formed by dividing the tube support plate circumferentially into six land regions, and axially into a helical pattern having a 1/8 inch thickness and a 1/4 inch pitch.

Table 7.4

SUMMARY OF SERIES 4 TEST PIECES HAVING EDDY CURRENT SIGNALS

<u>Test</u>	<u>Tube</u>	<u>Collar Material</u>	<u>Thick. (in.)</u>	<u>Crevice Configuration</u>	<u>Thru wall Leak</u>	<u>Leak Tested</u>
3	568-1-1	Teflon	0.75	Snug Fit	Yes	Yes
	568-2-1	Teflon	0.75	10 Mil Gap	Yes	Yes
	568-4-1	Teflon	0.75	Snug Fit	Yes *	Yes *
	568-5-1	Teflon	0.75	10 Mil Gap	No	No
	568-6-1	Teflon	0.75	20 Mil Gap	No	No
5	571-1-1	C. Steel	0.75	Sim. Plant	Yes	Yes
8	574-2-1	C. Steel	0.75	Sim. Plant	No	No
	574-3-1	C. Steel	0.25	Frits, Empty	No	No
	574-4-1	C. Steel	0.75	Frits, Empty	Yes	Yes

* Separate cracks were identified at the top and bottom ends of the crevice. Both cracks were included in the leak test.

Table 7.5

Summary of SCC Behavior in Doped Steam at 750°F

Specimen ^a Number	Alloy Heat	Clamp Width (in)	OD min Deflection (in)	TW SCC (hours)	Crack Network Lengths (in)	Bobbin Mix Voltage
SL-FH-1	2675	0.75	0.030	143.1	1.20	5.7
SL-FH-2	2675	2.0	0.030	93.3 ^b	2.67; 2.96	0.48; 0.27
SL-FH-3	2675	0.25	0.020	169.9	0.90	0.95; 8.5; 1.1
SL-FH-4	2675	0.25	0.010	261.3	0.63	7.4; 3.2
SL-FH-5	2675	0.25	0.010	170.5	0.47	4.0
SL-FH-6	2675	0.25	0.005	98.2	1.45	5.2; 7.5; 1.6
SL-FH-7	2675	0.25	0.005	175.0	1.22	6.1
SL-FH-8	2675	0.25	0.005	207.8 ^c	0.27; 0.34	1.5; 0.44
SL-FH-9	2675	0.25	0.005	118.0	0.97; 1.15	1.3; 3.9; 2.0
SL-FH-10	2675	0.25	0.005	118.0	1.20; 1.16	0.69; 1.7; 1.0
SL-FH-11	2675	0.25	0.005	119.9	0.50; 1.10	2.3; 5.7
SL-FH-12	2675	0.25	0.005	119.9	0.57; 1.63	1.9; 0.48
SL-FH-14	2675	0.25	0.005	307.8	0.67	2.2; 5.2
SL-BW-1	96834L	0.75	0.030	95.1	0.72; 0.85	5.4; 1.5
SL-BW-2	96834L	2.0	0.030	35.3	1.65	6.0
SL-BW-3	96834L	2.0	0.020	58.0	0.78	7.4
SL-BW-4	96834L	0.25	0.020	64.5	1.10; 1.44	7.5; 4.1; 4.3
SL-BW-5	96834L	0.25	0.020	164.4 ^c	0.37; 0.50	0.48; 0.53
SL-BW-6	96834L	0.25	0.030	71.0	2.07; 2.35	8.0; 7.2
SL-BW-7	96834L	0.25	0.010	97.8	0.72	4.2
SL-BW-8	96834L	0.25	0.005	132.8	0.28	2.6
SL-BW-9	96834L	0.25	0.005	334.0	0.65	4.3
SL-BW-10	96834L	0.25	0.005	424.0	0.62; 0.17; 0.28	7.0
SL-BW-11	96834L	0.25	0.005	213.0	0.34	2.2
SL-BW-12	96834L	0.25	0.005	693.0 ^d	e	2.5
SL-BW-13	96834L	0.25	0.005	87.0	0.63	3.2; 3.4; 7.6
SL-BW-14	96834L	0.25	0.005	64.0	0.33	4.6
SL-BW-15	96834L	0.25	0.005	667.0 ^d	e	1.2; 2.3
SL-BW-16	96834L	0.25	0.005	500.0	0.20	4.9; 7.7; 4.6
SL-BW-17	96834L	0.25	0.005	146.0	0.14	1.8; 1.7; 2.8
SL-BW-18	96834L	0.25	0.005	366.0 ^d	e	NDD ^f
SL-BW-19	96834L	0.25	0.005	480.0 ^d	e	NDD ^f
SL-BW-20	96834L	0.25	0.005	257.0 ^d	e	NDD ^f
SL-BW-21	96834L	0.25	0.005	243.0 ^d	e	NDD ^f

a SL-FH-13 rejected because of baseline NDE indication.

b Leaked at 750°F; did not leak at room temp.; but had visible OD cracks.

c Did not leak at 750°F. Test periodically shut down till OD cracks visible.

d Did not leak at 750°F.

e No OD cracks visible at termination.

f No detectable degradation.

Table 7.6

Fatigue Precracked Specimens

<u>Specimen</u>	<u>Crack Length</u>	<u>Number of Cycles</u>
FAT-1	0.500	323,700
FAT-2	0.299	85,000
FAT-3	0.300	25,100
FAT-4	0.697	1,278,000
FAT-5	0.300	22,600
FAT-6	0.302	110,000
FAT-7	0.509	410,000
FAT-8	0.707	710,000
FAT-9	0.513	370,000
FAT-10	0.701	726,000
FAT-11	0.499	226,000

Table 7.7

Summary of Dented Specimens

<u>Specimen</u>	<u>Dent Voltage</u>	<u>Average Radial Dent (inches)</u>	<u>Exposure Time (hours)</u>
Trial-1	.	.	24
FAT-1	7.39	0.00037	24
FAT-2	6.09	0.00030	24
FAT-3	12.11	0.00061	48
FAT-4	12.0	0.00061	58
FAT-5	4.55	0.00023	6
FAT-6	0.00	0.0	6
FAT-7	9.43	0.00047	24
FAT-8	17.42	0.00087	48
FAT-9	3.40	0.00017	6
FAT-10	2.50	0.00012	6
FAT-11	2.75	0.00014	6
BW-1	14.67	0.00073	24
BW-3	6.27	0.00031	24
BW-9	6.38	0.00032	48
BW-14	7.03	0.00035	48

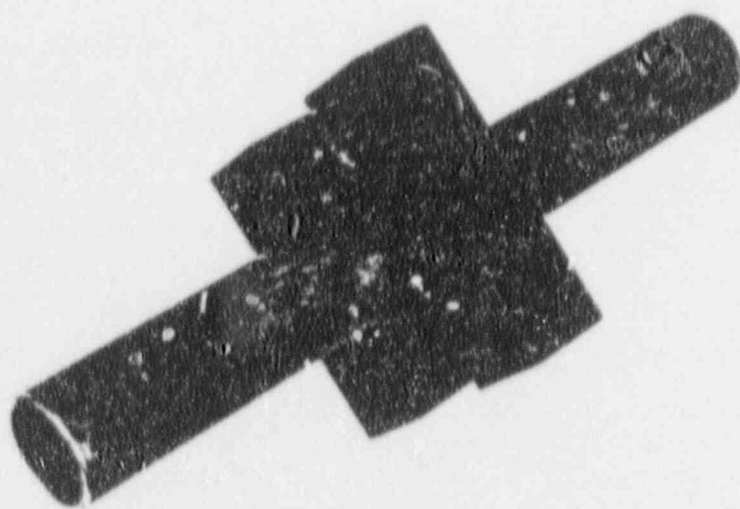


Figure 7-2. Clamped Specimen Used For Doped Steam Test

Specimen Trial-1

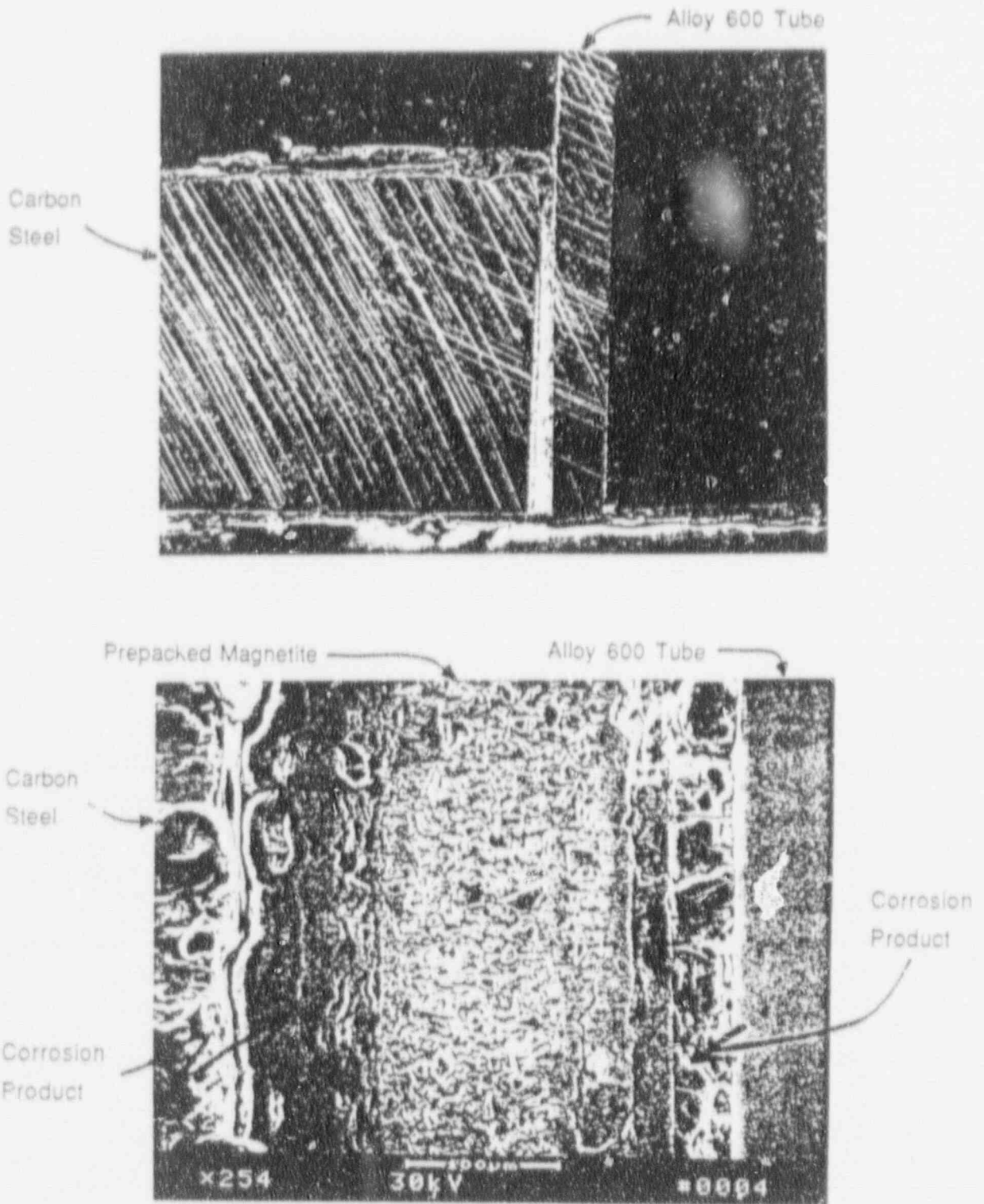


Figure 7-3. Section Through a Dented Tube Support Plate Intersection

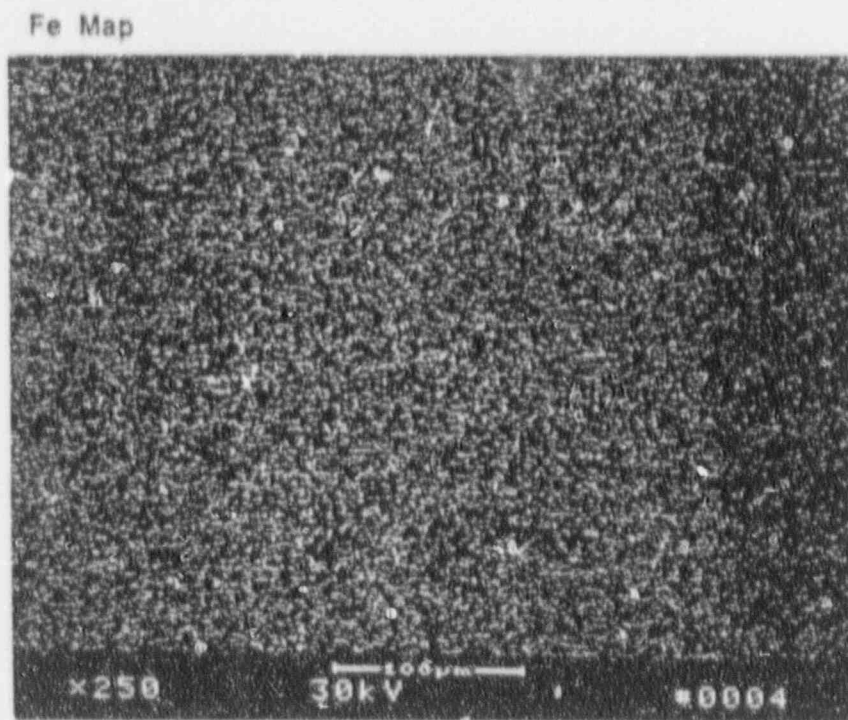
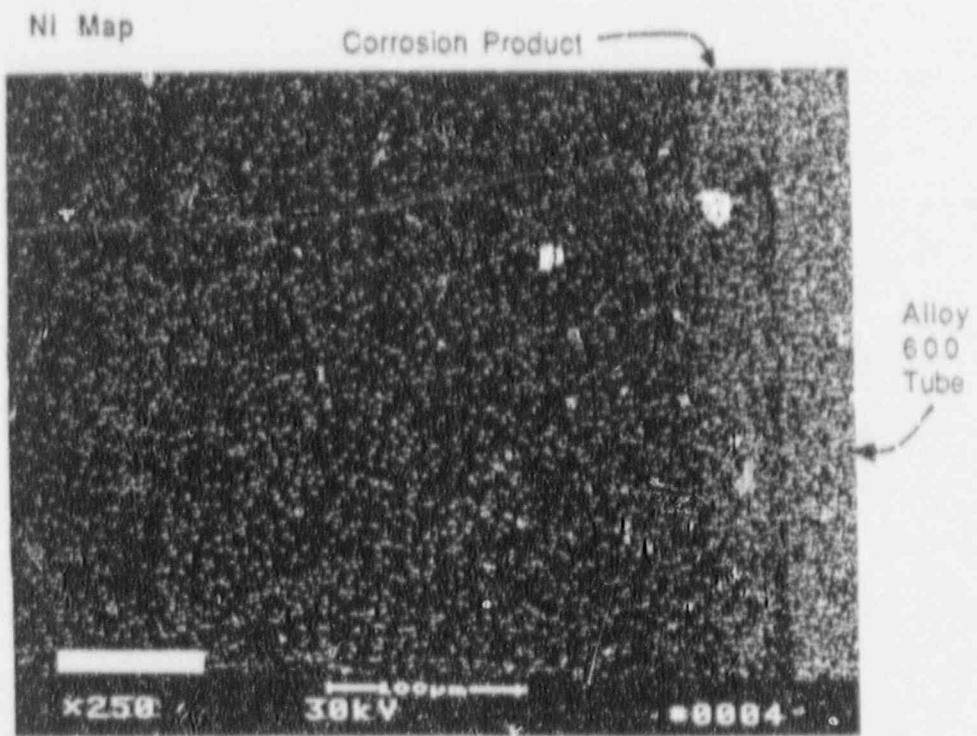
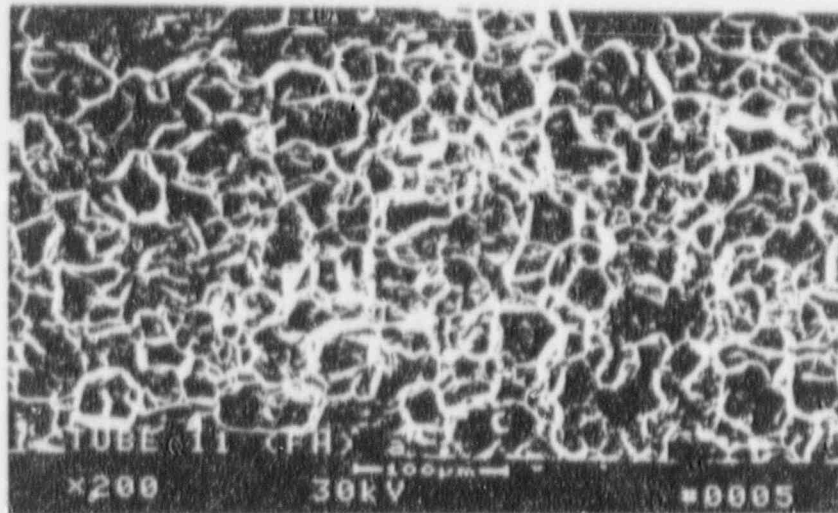
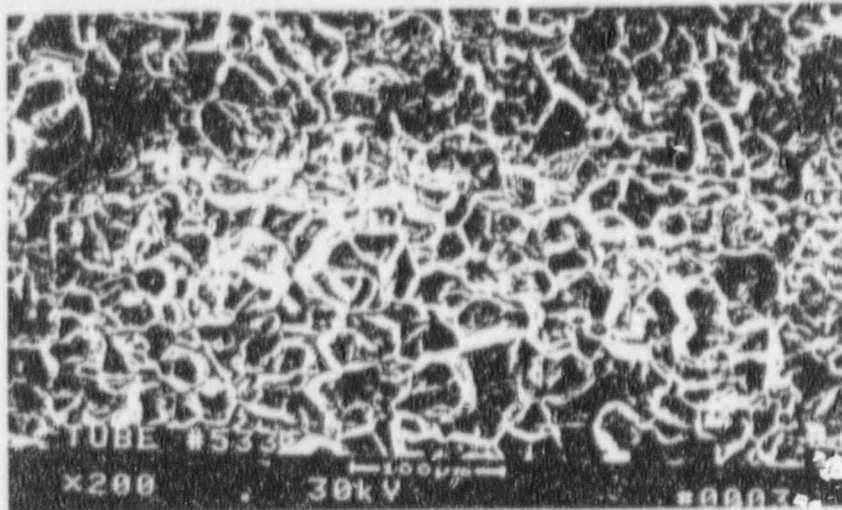


Figure 7-4 . EDS Elemental Maps Across a Dented Crevice; Specimen Trial-1



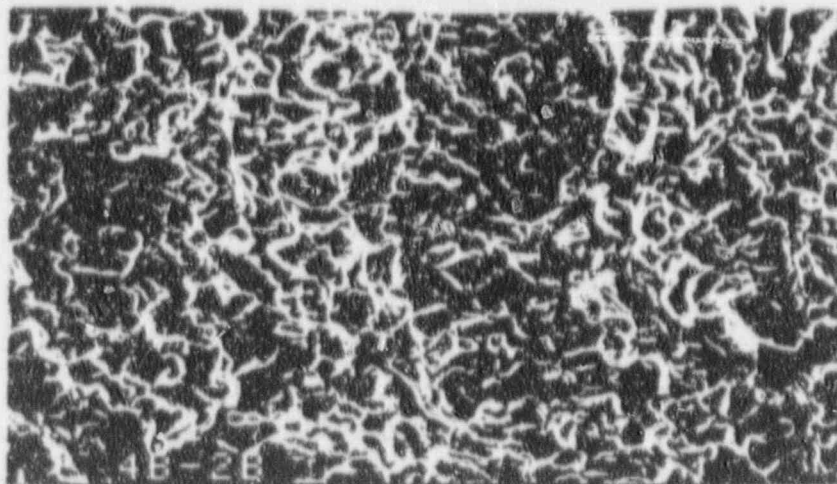
Doped
Steam
Specimen

200X



Model
Boiler
Specimen

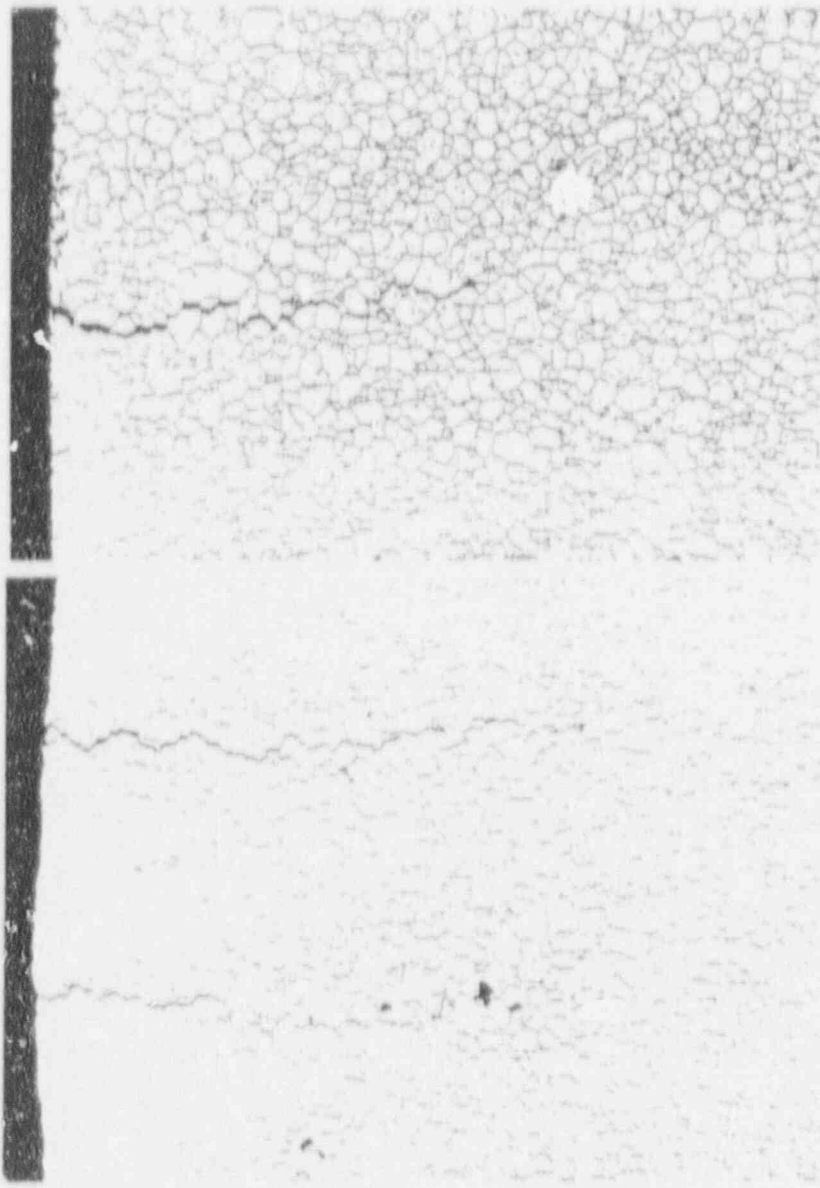
200X



Pulled
Tube
(Field)

200X

Figure 7-5. SEM Fractographs of Cracks in Doped Steam Specimen,
Model Boiler Specimen and Service Tube



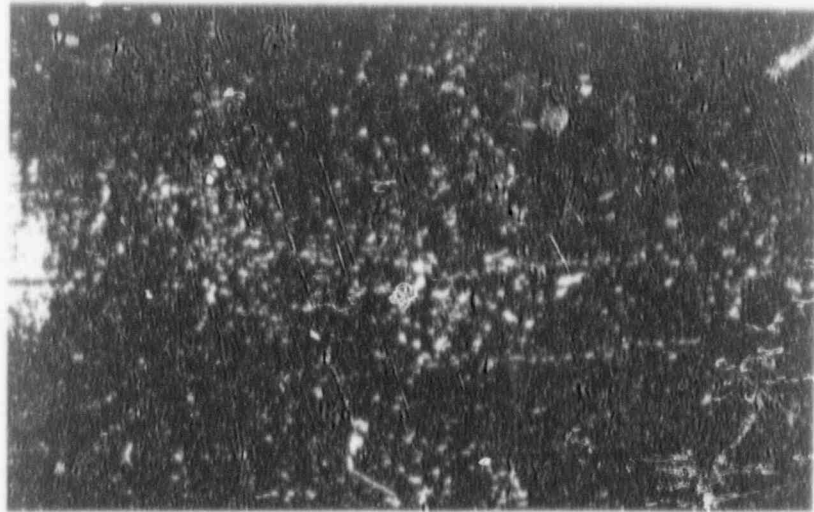
Doped
Steam
Specimen

100X

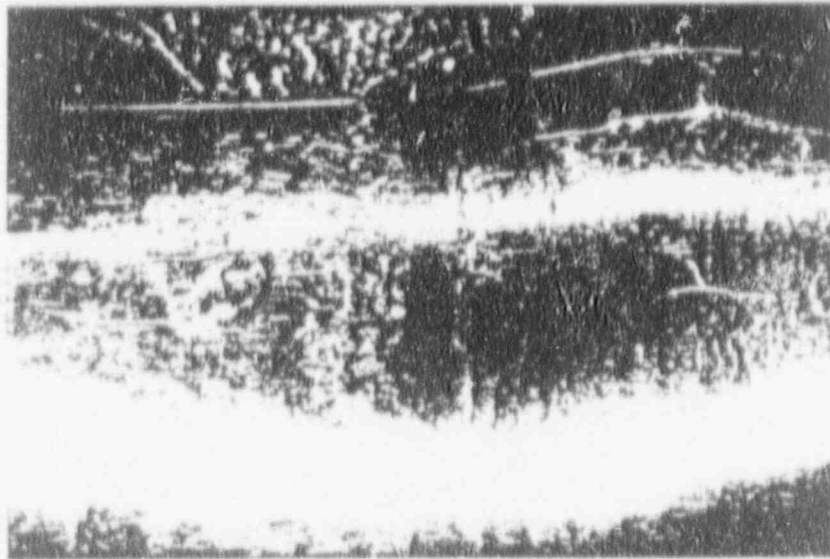
Model
Boiler
Specimen

100X

Figure 7-6. Metallograph of Cracked Specimens



Axial
Crack
Network



Distributed
Cracks Under
Steel Collar

Large Cracks
Under Teflon
Spacer Pins



Single
Large
Crack

Figure 7-7. Cracks in Model Boiler Specimens

8.0 NON-DESTRUCTIVE EXAMINATION (NDE)

An extensive NDE program was implemented to characterize the laboratory cracked specimens and to assess the sensitivity associated with application of the bobbin coil voltage limits for the tube plugging criteria.

The test program included tests to address some of the variables associated with field characterization of degradation as follows:

1. Bobbin probe voltage sensitivity to the length of the cracks, depth of the cracks, presence of ligaments in the cracks and parallel cracks.
2. Multiple probes to address probe-to-probe variations utilizing probes from Echoram and Zetec.
3. Influence of tube to TSP crevice condition on bobbin coil response including open crevices, packed crevices, incipient denting and fully developed denting.
4. Bobbin probe voltage sensitivity to probe wear to establish field inspection requirements for acceptable NDE uncertainties.
5. Variability among calibration standards, and normalization of the frequency mix.
6. Use of RPC to augment bobbin probe inspections, although the RPC data are not considered essential to the development of the plugging criteria of this report.

The NDE results for the laboratory specimens are utilized in later sections to establish the relation of eddy current voltage response to potential leakage and burst pressure as the basis for the tube plugging criteria.

Establishing a relationship between the bobbin coil response and tube integrity (leakage, burst considerations) is important to inspection planning. A relationship helps determine the importance of detecting degradation with a small amplitude. That is, can "small" indications be left in service and have negligible consequences for safe operation. Degradation exceeding present plugging limits, whose bobbin coil eddy current response was not detected as readily as would equivalent size notches, has been confirmed by destructive examinations.

The morphology of the intergranular corrosion can explain the lack of an eddy current response for small cracks. The observed field degradation, multiple short cracks coupled with an intergranular nature, allows paths for the eddy currents to pass uninterrupted through the degradation. An appreciation for why this phenomenon can account for the lack of an eddy current response has come from the use of liquid metal modeling techniques. Using this technique, degradation is simulated as inserts in liquid metal, and degradation morphologies that are difficult or impossible to machine can be easily simulated. The difference in response between "real" cracks and notches have been modeled by varying the contact between the faces of the crack. In that work, interfacial contact of 50% reduced the eddy current response by a factor of 5. An example of this behavior was found for the doped steam specimens cracked with high applied hoop stresses. These specimens were found to have lower voltages than expected

for the crack sizes present in these tubes. This result is judged to be the consequence of crack face contact as a result of removing the applied stress. Similarly, the voltage sensitivity tests reported here show large voltage increases as ligaments between cracks are lost. The presence of crack ligaments and partially degraded grain boundaries provides an explanation for the lack of eddy current response associated with field induced degradation. Further, the presence of these ligaments and the low instance of primary coolant leakage associated with this degradation mode suggests that there is residual strength associated with these ligaments. Thus, significant degradation depths may result in less severe loss of strength than was assumed in determining the plugging limits based only on crack depth. For this reason, plugging criteria are based on voltage responses correlated to tube integrity through the voltage versus burst pressure and leakage correlations.

8.1 Voltage Sensitivity to Crack Morphology

A series of eddy current tests were performed to establish voltage trends with crack morphology to characterize voltage as a measure of tube integrity. In most cases, machined specimens were used to simulate degradation features. This section describes using simulated cracks and volumetric indications. In addition, voltage measurements for laboratory specimens and pulled tubes with IGA degradation are summarized to assess detectability of IGA.

Voltage Sensitivity Using Slits in Copper Foil

To establish the general trends of bobbin coil voltage amplitude to crack morphology, sensitivity tests were performed using slits in a cylindrical copper foil to simulate varying crack lengths, ligaments and parallel cracks around the tube circumference. The copper foil was placed around a plastic tube. The various crack morphologies simulated by the slits in the copper foil, and the associated voltage responses are shown in Figure 8-1. For each combination of simulated cracks in Figure 8-1, the total crack network length is equal to the TSP thickness of 0.75 inch. The vertical cuts between the parallel axial cracks simulate loss of ligaments between cracks.

The voltage trends of Figure 8-1 show that the voltage increases with:

- o Increasing crack length
- o Increasing number of cracks around the tube circumference, and
- o Loss of ligaments between cracks.

Voltage Sensitivity Tests Using Slots in Inconel 600 Tubing

Additional information on the functional dependence of bobbin signal voltage on length and depth of axial cracks was obtained using EDM slots in 7/8 inch OD, 50 mil wall alloy 600 tubing as shown in Figure 8-2. The signal voltages for the slots represent the upper bound for the signal voltages expected for actual cracks of similar length. For the 100% deep slots, the signal voltage increases steeply with slot length up to about 0.5 inch and continues to increase up to one inch, after which it tends to level off. For the 50% deep OD slots, the signal amplitude increases with slot length up to about 0.250 inch after which it levels off. The signal amplitude increases by a factor of about 50 for 100% deep slots as the slot length increases from 0.03 inch to 1.0 inch; it increases by

a factor of about 4 for the 50% deep OD slots over the same range of crack length. It may further be noted that for longer slots, there is a greater increase in the signal voltage as the depth increases from a shallow depth to 100%. For example, for 1/4" slots, the voltage increases by a factor of about 50 as the depth increases from 50% to 100% whereas for 60 mil long slot it increases by only a factor of 10 for the same range of depth change. Voltage increases in an exponential manner with depth for a given slot length. For example, the voltage for a 1/4" slot increases from about 5 volts at 80% depth to 40 volts at 100% depth. Overall, the voltage amplitude is particularly sensitive to deep wall penetration and crack length; this is the desired dependence for voltage as a severity index for tube integrity.

Figure 8-2 also shows the data for three slots with depth varying along slot length. The central 1/3 of each slot had 100% depth which tapered off to 0% at the ends. The signal amplitudes for these slots with tapered ends are, as expected, higher than for the through wall rectangular slots when plotted against the through wall slot lengths. Figure 8-3 shows a plot of percentage increase in signal amplitude above that for a uniform through wall slot resulting from the tapers as a function of the 100% deep portion of the slot length. As the through wall length increases, the influence of the partial depth slot decreases such that for lengths greater than about 0.25 inch, the partial depth slot length has negligible influence on the voltage amplitude.

Figure 8-4 shows the signal amplitudes for the axial slots obtained by using the rotating pancake probe with a 125 mil diameter coil. This data is qualitatively similar to the bobbin data of Figure 8-2. Figure 8-5 gives a plot of bobbin voltage vs. RPC voltage for the slots showing a correlation between the signal amplitudes expected from the two types of probes. It may be noted that for both the bobbin and the RPC probes, the amplitudes are dominated by the deepest part of the slots. A well defined correlation between bobbin and RPC voltages is seen for the single slot data.

The effect of ligament within the crack length on the eddy current signal voltage was studied by varying the axial distance between two 0.125 inch long 100% deep axial slots. The results are shown in Figure 8-6. The bobbin signal voltage drops by approximately a factor of 2 when a ligament as thin as 8 mils is placed in the middle of the two slots. The bobbin signal voltage is relatively insensitive to any significant increase in the spacing beyond 16 mil. RPC voltage is also seen to decrease with increasing ligament size although the rate of decrease for small ligaments is less than for the bobbin coil. Since the small 16 mil ligaments within a crack cannot be distinguished by eddy current, the voltage increase with loss of ligaments supports voltage as a severity index for tube integrity.

A variation in bobbin signal amplitude is expected in case of parallel, multiple axial cracks, spaced around the circumference of the tube. This effect can again be studied using EDM slots in the Alloy 600 tubing. Figure 8-7 shows the effect on the signal amplitude of varying the spacings between four through wall axial slots. The signal amplitude increases by a factor of about three as the spacing between parallel slots increases from a few mils to 700 mils. Closely spaced parallel slots do not show an increase in voltage above that of a single slot. The signal phase angle decreased by about 10 degrees at 400 kHz for this entire range of spacing. Qualitatively, a similar result was found for 50% deep OD axial slots.

The positioning of degradation near the end of the TSP can potentially influence the responses, with the principal contribution due to the mix residual that occurs at these locations. This response is typically small compared to the voltage plugging limits so that the amplitude of significant degradation will not be influenced by this residual. To demonstrate the TSP edge effects, bobbin coil measurements were made on 1/4-inch EDM slots of 50% and 100% depth. Measurements were made for the slot at the center of the TSP and at the inside and outside edges of the TSP. Results of these measurements are given in Table 8.1. It is seen that the voltage values for the crack within the TSP are essentially the same at the center and at the inside edge of the TSP. Variations in voltage with the slot inside the TSP are <2% for both 50% and 100% deep notches, while moving the slot outside the TSP increased voltages by 5%-10%. The bobbin coil indicated depth changed by 29% as the 50% slot was moved from the center to the outside edge of the TSP. These results support the conclusion that amplitude responses to degradation on the order of the 3-4 volt plugging limits will not be significantly impacted by the location of a crack within the TSP.

General conclusions from this eddy current evaluation of axial slots are:

- o Both bobbin and RPC voltage amplitudes increase sharply with axial crack length up to about one inch for 100% deep slots.
- o The voltage increase is much smaller for partial depth OD axial slots and voltage does not increase significantly with length for slots greater than about 1/4" long.
- o Signal amplitude is dominated by the 100% deep portion of the slot.
- o Bobbin coil signal voltage is a function of spatial separation between parallel axial slots. Very closely spaced slots show an insignificant increase in the voltage over that of a single slot.
- o A correlation exists between RPC and bobbin voltages for single slots. However, bobbin voltages increase for parallel slots while RPC voltage can be isolated to a single slot provided slots are adequately separated to permit resolution of each slot.
- o The presence of a small ligament between two axial slots reduces the signal voltage.
- o Amplitude responses to degradation on the order of the voltage plugging limits are not significantly influenced by the location of a crack within the TSP.
- o Both bobbin and RPC voltage amplitudes from slots represent an upper bound for the voltages expected from cracks of similar length and depth.

These results demonstrate the use of the voltage amplitude as a crack severity index for tube integrity assessments. Voltages increase with increasing crack length, with increasing depth particularly near through wall penetration and with loss of ligaments between cracks.

The general concept of relating voltage to burst pressure can be demonstrated by combining data for voltage vs. slot length with burst pressure vs. slot length data. Figure 8-8 demonstrates the resulting voltage/burst correlation. Voltages for slots are

not typical of cracks but the general trend and slope are similar to that later developed in Section 9.6 from burst testing of cracked tubes. As intuitively expected and shown even for machined specimens, a given voltage amplitude does not define a unique crack morphology. Thus a spread in burst pressures for a given voltage is expected. This spread is accommodated in the plugging criteria by using a voltage/burst correlation at the lower 95% confidence band of the test data. Various crack morphologies involving variables such as length, depth, ligaments, multiple cracks, etc. influence the spread of the data and thus the resulting tube plugging limit.

Voltage Sensitivity to Volumetric (Non-Crack) Indications

It is desirable to compare voltage amplitudes for volumetric indications to those associated with plugging levels for ODSCC cracks. Given defect specific plugging criteria for ODSCC at TSPs, these comparisons help to guide the importance of distinguishing ODSCC from other types of degradation. The plugging criteria of this report establish a bobbin voltage threshold above which RPC inspection is required to facilitate identification of ODSCC from other types of degradation. These voltage comparisons provide guidance on setting the bobbin voltage threshold for RPC inspection.

Typical bobbin coil voltage amplitudes were developed from laboratory simulations of volumetric degradation as developed below and summarized in Table 8.2:

- Wastage: Flat rectangular shaped flaws of different depths were machined to simulate tube wastage: the bobbin signal amplitudes as a function of maximum OD depth for 1/4" and 1/8" long machined flaws are shown in Figure 8-9. In addition, tapered flaws were machined to simulate the tube wastage shapes observed at some plates. Figure 8-9 shows the bobbin signal voltage as a function of maximum OD depth for tapered flaws.
- Fretting: Flaw shapes somewhat similar to the tapered flaws shown in Figure 8-9 are observed from fretting. Thus, the data of Figure 8-9 can be used to assess the voltages expected from fretting.
- Pitting: Figure 8-10 shows the bobbin signal amplitude vs diameter of machined through wall holes simulating 100% deep pits. Pitting observed in operating SGs occurs as multiple pits for which voltages are significantly higher as noted in Table 8.2. The data from ASME flat-bottom holes of partial depths may be used for estimating the signal voltages expected from partial depth OD pits.
- Cold Leg Thinning: Pulled tube data from two different plants with cold leg thinning were reviewed and are summarized as follows. In one case, a flaw at the second TSP in the cold leg with a maximum wall penetration of 48% had a bobbin amplitude of 4 volts. Figure 8-11 shows a photograph of the OD surface at the degraded location. In the second example, which was also at the second TSP in the cold leg, a maximum wall penetration of 59% yielded a bobbin amplitude of 4.9 volts. Examination of the pulled tubes showed no cracks in these tubes and the degradation was identified as "cold leg thinning."

The data of Table 8.2 and supporting figures indicate that bobbin voltages would exceed about 2 volts, as limiting by pitting degradation, before being a concern for tube integrity. A single pit, simulated by 30 and 60 mil through wall holes would have voltages of about 2 and 7.5 volts, respectively. Pitting typically occurs as multiple pits in operating SGs with higher voltage levels as noted in Table 8.2. Cold leg thinning at 50% depth will yield a bobbin amplitude of over 4 volts.

Based upon the above noted voltage levels, volumetric indications with bobbin voltage amplitudes exceeding about 2 volts should be inspected using RPC probes to aid characterization of the causative mechanism. Volumetric indications less than this voltage amplitude would be expected to be acceptable for continued service and separation of the causative mechanism from ODSCC would not be critical to assure tube integrity.

IGA Detectability

Limited laboratory specimens and pulled tubes at TSP intersections with significant IGA are currently available for assessments of detectability and tube integrity. Available laboratory IGA specimens were prepared as long (4 to 6 inches), uniform IGA to assess detectability in unexpanded regions of tubesheet crevices. Data from three domestic tube to TSP intersections from Plant L (R12C8), one from Plant M-2 (see Section 4), and three from non-Westinghouse plant N are available. Some French data obtained from EdF for TSP indications can also be assessed. The general morphology of the French indications is more like Plant L tube R12C8 than Farley indications.

Three sets of laboratory IGA specimens are available for NDE assessments. Two are Westinghouse samples and the third represents samples prepared by Westinghouse under EPRI sponsorship (EPRI NP-5503). The two Westinghouse sets of specimens represent laboratory IGA under accelerated conditions and provide uniform wall penetration IGA over 4 to 6 inch lengths. Bobbin coil detection for these specimens is shown in Figure 8-12. Figure 8-12a represents specimens prepared using sensitized tubing and shows very high bobbin coil amplitudes. Figure 8-12b shows bobbin coil responses using non-sensitized material. The non-sensitized material shows much lower amplitudes. Methods of sample preparation were refined for the EPRI program to further improve comparisons with field experience.

The EPRI specimens were prepared using a 50% caustic and 12% chromium oxide environment at 650°F for up to 10,000 hours. Temperatures in some cases were increased to 700 °F to accelerate the corrosion rate such that 21% penetration was obtained in 35 days. Even under the accelerated laboratory conditions, the times to create IGA are very long compared to preparation of ODSCC specimens. Specimens in the range of 2 to 30% nearly uniform wall penetration were obtained in this program. Figure 8-13 shows typical NDE results for 29% deep IGA. The bobbin coil differential tests reveal the uniform IGA whereas the RPC results are not particularly revealing. Voltage amplitudes are not available for the samples that were destructively examined. Bobbin coil measurements of library samples were performed with the results given in Table 8.3. These samples show voltage amplitudes of about 1-2 volts where the IGA depth is expected to be <30% deep and are NDD where depths of a few percent are expected. Deep cracks within the samples were detected with amplitudes of 4-40 volts. These samples are more representative of field IGA than the Figure 8-12 samples although limited pulled tube data for uniform IGA are available for direct comparisons.

As described in Section 4.0, a pulled tube from Plant M-2 shows IGA with cracks up to 26% depth. This tube had a voltage amplitude of 1.8 volts, which is high compared to tubes with principally ODSCC at comparable depths as shown in Figure 6-9. The signal amplitude is comparable to the laboratory specimens of Table 8.3, although lower than the specimens of Figure 8-12.

Three pulled tube results from Plant N-1 with egg crate supports are also shown in Figure 6-9. These data for IGA degradation also support IGA detectability at voltage levels comparable or higher than that for ODSCC with minor IGA.

The recent pulled tube (R12C8) indications from Plant L were detected by pre-pull bobbin coil inspection. Table 8.2 and Figure 6-9 show the voltage and maximum depth for the three Plant L indications that were found to have significant IGA involvement as compared to other pulled tube results including Plant L data with negligible IGA involvement. It is seen that the Plant L voltage levels are typical of the rest of the population of pulled tubes with less IGA involvement than the Plant L tube R12C8 crack morphology, which shows patches of IGA with IGA/SCC cracks.

Figure 6-9 also shows voltage amplitudes for tube to TSP intersections removed from French units. The French data show voltage responses toward the high range of the data. The French crack morphology is IGA + SCC as shown in Figure 6-8. The Plant L (R12C8), Plant M and Plant N morphologies also show IGA + SCC.

Tube removal analyses performed in Belgian Plant E during 1991 showed ODSCC with shallow IGA at all support plate intersections examined. Again the affected areas were located entirely within the TSP intersections, mainly at mid-height. The affected circumference was small when shallow depths prevailed, and reached nearly 360° in strongly affected intersections; at shallow depths, many initiation sites with multi-directional cracking were visible; in strongly affected areas, axial cracking was predominant, with only shallow associated IGA. For tube R19C35-2H, the burst cracks, developed with a loose fitting TSP collar, were axial in direction, showing 100% intergranular depth. Several tube sections broke upon pulling; the maximum corrosion penetration was 60% in a dense array of axial cracks, with shallow IGA in some places.

Figure 4-28 illustrates the cellular morphology of the Belgian tubes and Figure 8-24 shows the EC traces collected in the field for the corresponding location prior to pulling. The 2.27 volt signal recorded at 300 kHz, when converted to US voltage calibration in accordance with Table 6.4, corresponds to approximately 23 volts, well in excess of any voltage-based alternate criteria under consideration. The stated reporting threshold for TSP ODSCC in Belgium is 0.2 volts at 300 kHz, or approximately 2 volts at U.S. voltage calibration. Nearly all tubes plugged in the U.S. for TSP ODSCC would escape reporting under the Belgian criteria.

It is plain that since Belgian practices are based on the absence of safety implications for ODSCC/IGA contained within the TSPs, that only the most severe degradation is plugged and most indications are not recognized. Since the Plant E tubes are pilgered in the manufacturing process, the periodic noise in the tubing complicates detection of minor degradation; thus the relatively high reporting threshold dovetails nicely with the S/N limitations in the EC data.

In summary, the conditions which went "unreported" in the Belgian tubes were not simple IGA, which is easily identified as the U.S. field and lab data show; rather the voltage calibration and reporting threshold combined to create the impression that significant cracking and associated shallow IGA, comprising the cellular degradation observed in the lab, had not been detectable. In the U.S. NDE calibration system, these are prominently visible and pose no challenge to evaluation.

Overall, the available pulled tube results show comparable voltage responses relative to maximum depth with no definable dependence on IGA involvement within the broad scatter of the data. The laboratory uniform IGA samples show significant voltage responses at 30% depth. The available pulled tubes with significant IGA levels show IGA with cracks and have been found to be detectable indications.

8.2 Probe Comparisons

To address concerns that the results of the study might be limited to a specific probe, probes from different eddy current probe vendors (Echom and Zetec) were used. Both probes were nominally 0.720 inch in diameter and incorporated the latest technology for centering. The coils on each probe were nominally 0.06 inch wide and were spaced by 0.06 inch. Initially each of the probes was used with two different sets of frequencies duplicating typical field inspection configurations. The first set of frequencies (configuration I) was 400, 200, 100 and 10 kHz. The second (configuration II) was 600, 400, 100, and 10 kHz. Review of the data indicated no significant differences in the results from the different probes for the different frequency configurations.

Comparisons were made between the data obtained from probe Er and probe Zt for the cracked tube specimens. The probes have different frequency response characteristics: Probe Zt gives a greater response at 100 kHz, while probe Er has a larger response at 600 kHz since it is designed for higher frequency operation than the Zt probe. This difference is not a significant issue and is noticeable only as a consequence of the way in which the voltage calibrations have been derived. Table 8.4 gives the voltage measured by probe Zt divided by the voltage measured by probe Er for the EDM calibration notches. As can be observed for each frequency, the difference between the probe responses is a constant factor for all notches. This apparent variation between the probes can be eliminated by calibrating each of the frequencies individually rather than using the 400 kHz calibration factor, or by calibration at the planned mix frequency. The latter approach is recommended as a part of the plugging criteria of this report.

The results for the 400/100 kHz Mix channel were fortuitous. Plots of the measured voltage for various indications from probe Zt versus the voltage from probe Er indicate a one to one correspondence for both amplitude and depth (Figures 8-14 to 8-16). The correspondence between the mix channels of the two probes is due to the fact that the 400 kHz channel is being used both as the "primary" mix frequency and to set the calibration factor. If another frequency is used as the primary mix frequency (i.e., 100 kHz) the apparent mix amplitudes will differ. Table 8.5 gives the measurement of the ASME holes using a 100/400 kHz mix for the two different probes. As can be seen the results from the two probes differ by a constant multiplier. As with the individual frequencies, this factor can easily be accommodated by using a different calibration procedure.

The bobbin coil inspections were supplemented by two RPC examinations, also with probes from the two eddy current probe vendors noted above. However only one set of frequencies (400, 200, 100, and 10 kHz) was used for both RPC probes. The data gathered during this phase of the program were used primarily as a qualitative tool in assessing the extent of the degradation.

8.3 Influence of TSP Crevice Condition

For some specimens, the crevice between support and the tube was packed with magnetite and the sample was inspected again with the bobbin coil. Table 8.6 compares the data from the two inspections. With the exception of sample BW-11 the amplitude of the responses from the samples changed by approximately 10%. Sample BW-11 showed a 50% increase in amplitude. When the support was removed from the sample the degradation response had indeed increased, indicating that the presence of magnetite in the crevice did not cause the increase in response, rather the process of packing the crevice had mechanically deformed the sample causing further loss of ligaments and a subsequent increase in response. Additional evidence for the minimal impact of the presence of magnetite in the crevice is derived from the comparison of the data from corrosion samples with tight packed crevices. A comparison of the samples prior to support removal and after magnetite removal with the packed support ring (Figure 8-17) in place show a 5% increase in response with the magnetite present with a scatter of approximately 10%.

As part of the test program, 6 fatigue crack and 3 doped steam corrosion crack samples were leak tested to determine the influence of the dented support plate crevice condition. Table 8.7 summarizes the results of the eddy current inspection of these samples before and after denting (note samples FAT 1, 2, and 3 had been leak tested previously). Denting resulted in a significant change in the amplitude of the fatigue crack eddy current responses. Prior to denting, all but one of the fatigue cracks had amplitudes which approached that of the through wall EDM notch (80 Volts). After denting, two of the fatigue cracks could not be distinguished from the response of the dents, despite the fact that the dent response was an order of magnitude smaller than the initial fatigue crack response. Initially the corrosion cracks produced smaller responses than the fatigue cracks. However, their responses on average, could be detected in the presence of the dent. However, these results represent large indications in the presence of small dents. In field applications, small to moderate indications typically cannot be separated from dent signals that exceed the amplitude of the indication.

The difference in the behavior of the two crack types (and further within the fatigue samples) in the presence of denting is a consequence of the heavier oxide coating on the crack faces of the corrosion samples and the leak tested fatigue cracks. Under the compressive loads of denting, the crack faces are forced together. The presence of the oxides on the corrosion crack faces prevents interfacial contact and therefore results in a minimal change in the crack response. On the other hand, the faces of a virgin fatigue crack, being free of oxides, come into intimate contact permitting the eddy currents to flow unimpeded across the crack, significantly reducing the response. The significant loss in response of the fatigue cracks demonstrates that interfacial contact does indeed result in a reduced eddy current response. However, it is not expected that service-induced degradation will respond to denting as have the fatigue cracks. Rather, it is

expected that the compressive stresses from denting would not play a direct role in providing interfacial contact such that field induced cracking would respond as did the doped steam samples, with little change in amplitude. However, it is recognized that bobbin coil detection of cracks at dented intersections is unreliable when the degradation amplitude is smaller than the dent amplitude.

8.4 Sensitivity to Probe Wear

Eddy current test parameters can exist over which there is little systematic control and which may vary between tubes or along the length of a tube. The centering of the eddy current probe as it passes the degradation poses the greatest concern of this type. This study has shown that probe centering can vary the amplitude of a signal, in the worst case, by a factor of two. The laboratory study used field probes which had excellent centering characteristics. At the beginning of a field inspection sequence, the probe centering characteristics would mimic those found in the laboratory. As the inspection continues the probe is expected to wear and its centering capability to degrade. The time frame for this to occur is unknown because it is a complex function of inspection extent, tube geometry, the presence of oxides, etc.

A means of assessing the probe's centering capability is through the use of an appropriate verification standard as illustrated in Figure 8-18. Such a standard can be as simple as four holes drilled in a segment of tubing. The holes are displaced axially in different planes with each spaced 90 degrees around the circumference from its neighbor. The amplitude ratio between the holes then determines the degree of centering of the probe. A standard of this type could be put in-line to provide continual verification of probe centering during the inspection.

The bobbin probe centering mechanism wears with usage. This could affect the eddy current signal. The effect of wear on signal voltage was evaluated using a "4-hole standard." The four holes were 67 mils in diameter, 100% deep, 90 degrees apart circumferentially, and 1.5 inches apart, axially. A 0.720 inch diameter probe fabricated by Echoram was used for this evaluation. The centering mechanism in this probe consists of three sets of spring loaded plastic buttons (hemispherical) 120 degrees apart and protruding approximately []^a mils from the probe body. The test runs were made with the tubes in a vertical position. The probe pull speed was approximately 12 inches per second. The tubes were rotated 90 degrees after each run to provide for the randomness in the probe to tube orientation expected in the field. For the new probe, the scatter in the voltages obtained from the four identical holes for repeated test runs was []^a.

The centering buttons of the probe were worn by repeatedly running the probe through a tube with an abrasive tape on the tube ID. The data from the 4-hole standard was collected for different levels of probe wear. At the level of probe wear represented by a 50% reduction in height of the centering plastic buttons (ie, about []^a mils), the scatter in the voltage for the four holes was found to be within []^a. Increasing the probe wear beyond this level resulted in rapid deterioration of the quality of the data. The results of the tests are shown in Figure 8-19. The 50% reduction in height of the centering buttons appears to envelope typical field wear between probe changeouts

during an inspection. Thus an allowance of []^a uncertainty for probe wear is consistent with current field experience.

The remaining concerns over probe centering are the impact of tube to tube variations in diameter and ovality. Neither of these are anticipated to be major sources of variability.

8.5 Eddy Current Inspection and Analysis Practices

To lay the foundation for field application of the results of the tests, the data collection and analysis procedures were made as close to those used in the field as possible, while conducted under laboratory conditions. A formal procedure for instrument calibration, data acquisition and analysis was developed. The key point to note from the procedure is that the analysis of the bobbin coil data was conducted from a 400/100 kHz mix data channel. The mix was established by eliminating the response from the support ring on the calibration tube. The voltage for all channels was calculated from the conversion factor found by setting the response of the 20% ASME calibration holes at 400 kHz to 4 volts (save/store mode). This corresponds to setting the 400/100 kHz mix channel voltage of the 20% holes to 2.75 volts or to 6.4 volts for the four-100% hole standard. A similar procedure, in which the 400 kHz response of a through wall EDM notch was set to 20 volts and the voltage of all other channels established using the Save/Store mode, was used to calibrate the RPC probes.

Because each calibration standard is potentially unique, a system to assure consistency of the data must be established. Currently, the field data is tied to the calibration tube used during an inspection and the correlation between that standard and all others used in the industry is through the certification provided by the manufacturer of the standard. Typically, the controlled parameters of the standards are the depth of the calibration holes and whether the phase response of the machined holes is within acceptable tolerance. The amplitudes of the hole responses are not controlled parameters. Small changes in tube dimensions, hole placement and other subtleties can cause variations in the hole response and therefore systematic offsets in the measured degradation response with respect to the data obtained in this study. An examination of a limited number of field standards has found a variation in the 20% hole response as large as 18% above that of the standard used in the laboratory work.

Evaluations of alternate practices for voltage normalization have found minimum uncertainties between standards when the 4 in-plane hole standards are utilized. This eliminates the depth variation between standards and limits voltage variations to principally the hole diameter tolerance. Consequently, the recommended voltage normalization is 6.4 volts for the 4 hole (0.033" dia.) ASME standard. For further limitation on the uncertainty, the ASME allowable hole tolerance of 0.003" is reduced to 0.001".

8.6 Alternate Inspection Methods: Rotating Pancake Coil (RPC)

The primary objective of this program was to arrive at inspection and plugging criteria based upon the bobbin coil inspection. However supplemental RPC data was also acquired from the test samples. This data served two objectives: (1) to establish a data base that

if needed could be used to augment the bobbin coil data in establishing tube plugging criteria, and (2) to gather information that might be used as additional support for the current practice of using RPC data as the final arbiter for determining tube plugging.

It is believed that combinations of deposits and other spurious conditions can result in bobbin coil responses that have degradation-like characteristics. To minimize plugging tubes with such responses, the philosophy of using the bobbin coil as a screening tool and the RPC as a confirmatory tool has developed. For the samples that were leak checked, indications were present in both the bobbin and RPC data. Table 8.8 contains a comparison of the bobbin coil and RPC results respectively for the leak tested model boiler samples. It is noted that the samples with the largest bobbin responses generally had large multiple RPC responses. It can be noted that specimen 568-4 had a crack length greater than the 0.75 inch TSP thickness. In this case, the crack extended below the TSP into a teflon spacer used to support the TSP in the test. Figure 8-20 shows examples of RPC traces for typical model boiler specimens at 6.5, 12.7, 26.5, and 67.7 volts (bobbin coil). It can be seen that for bobbin coil voltages above about 25 volts, multiple axial cracks of considerable amplitudes are found. Specimen 543-2 (67.7 volts) shows closely spaced, long axial cracks with large RPC amplitudes.

An example of an apparently spurious bobbin response, similar to those observed in pulled tubes, was identified in the model boiler tests (model boiler 568). During interim inspections these samples displayed the characteristic of having bobbin coil indications without showing discrete RPC indications. The bobbin coil mix response had amplitudes on the order of one volt, with depths based upon phase measurements on the order of 80% through wall. However, the phase of the indications did not change as a function of frequency consistent with that of the calibration tubes. Rather, as the frequency was decreased the phase remained essentially constant such that the apparent depth of the indication decreased. RPC inspection in the helicoil scan mode did not confirm 80% flaws and gave a response indicating a band of material with electromagnetic properties different from that of the remaining portion of the tube but with no discrete indications. When the pancake probe was pulled axially through the band, a response was obtained which behaved similar to the bobbin coil response. That is, as the inspection frequency is decreased the apparent depths of the indications appeared shallower. The underlying mechanism which causes this shift in the tubing's electromagnetic properties has not been identified. Ultimately the samples showed both significant bobbin coil responses and discrete RPC indications. Post-burst examination of the samples showed no apparent degradation apart from that identified in the RPC results. This example tends to lend support for the RPC probe as an arbiter of bobbin coil indications for identifying discrete crack responses and to avoid plugging of undegraded tubes with electromagnetic property changes.

8.7 Field Considerations

Proper implementation of the tube plugging criteria depends on consistent data acquisition between the field and the laboratory. Although field NDE procedures were utilized in the laboratory, the test program yielded the following modifications to the field eddy current procedures which are necessary to assure proper control of the uncertainties in the data acquisition:

1. The 4-hole ASME (Section XI, Article IV-3220) standard with 0.033 inch diameter holes spaced 90° apart in a single plane should be used for field voltage normalizations. However, hole diameter tolerance must be ± 0.001 inch rather than the ASME value of 0.003 inch.
2. An additional standard should be used in line with the ASME standard to limit the effect of probe wear (probe centering) on the field data. Use of this standard indicates when data uncertainties due to probe wear become greater than acceptable for the tube plugging criteria, requiring use of a new probe.
3. The calibration should be normalized to 6.4 V for the 400/100 kHz mix channel for the 100% 4 hole ASME standard to eliminate the uncertainties introduced by depth uncertainties in the standards and by calibration to 4 V for the 400 kHz channel and carrying over the conversion factor for the mix channels.
4. The NDE data acquisition and analysis guidelines of Appendix A should be implemented to enhance consistency and repeatability of the inspection data.

8.8 Eddy Current Uncertainties for Tube Plugging Criteria

In most prior evaluations, SG NDE uncertainty is determined as the difference between bobbin coil indicated depth versus actual depth from destructive tube examinations. This is not the case for voltage measurements such that the NDE uncertainties for voltage do not have such a unique interpretation. For voltage plugging criteria based upon voltage versus burst pressure correlations, the NDE voltage "uncertainties" affect both the voltage measurement and the spread or uncertainty in the burst pressure correlation. The goal for the voltage measurements is to minimize the uncertainty on repeating a measurement so that the uncertainty on the burst correlation is reduced to the extent practical. The remaining voltage measurement uncertainties end up as part of the burst correlation uncertainty. For example, assume that a number of perfectly identical samples were prepared such that burst pressures would be identical. If voltage measurements were then made with different probe diameters, calibration standards, open crevices, packed crevices, copper deposits in crevices, etc., the voltage measurement variability would then result in a spread in the voltage versus burst correlation. Clearly the goal is to minimize the burst correlation uncertainty (lower 95% confidence limit used for plugging criteria) by controlling the voltage variability. The voltage measurement procedures must be consistent between laboratory and field implementation to apply the laboratory specimen NDE/burst data for developing plugging limits. Inclusion of field voltage measurements for tubes pulled prior to implementing the procedures to improve measurement repeatability tend to increase the spread in the burst correlation. The NDE voltage uncertainty is defined as the uncertainty in voltage repeatability emphasizing differences between the laboratory and the field measurements.

As applied for the plugging limit development, the variables affecting the burst correlation are split into NDE uncertainties for determining voltage and burst correlation uncertainties as given in Table 8.9. The potential contributors to the NDE repeatability uncertainty are probe centering (principally probe wear), calibration standards, probe design differences, eddy current analysis variability and eddy current

system variability. Eddy current system variability results from noise due to instrumentation and cabling. This contributor is on the order of 0.1 V and can be ignored when combined with the probe wear uncertainty for applications to plugging limits above a few volts.

Probe design differences are eliminated by requiring that only bobbin coil probes with 0.06 inch coils and 0.06 inch spacing between coils be used for voltage measurements. These values are commonly used by nearly all probe vendors. The voltage amplitude is a function of coil to coil spacing. For differential responses and a center to center coil spacing of 0.12 inch, the influence of small changes in coil spacing such as associated with manufacturing tolerances is small. This sensitivity is shown in Figure 8-21.

The calibration uncertainty results from dimensional tolerances in fabricating the standards. The effects of dimensional tolerances in SG tubing result in a spread of the voltage/burst correlation as the tolerances may affect both voltages and burst pressure and thus are not categorized as an NDE uncertainty. The calibration standard variability for Farley SG applications is eliminated by requiring that the field standards be calibrated against the laboratory standard. The differences between calibration correction factors have been minimized by normalizing voltages to a four through wall hole ASME standard rather than the 20% depth holes. Voltage sensitivity to manufacturing tolerances for the through wall holes is expected to be smaller than found for 20% depth holes.

The probe centering uncertainty is limited by requiring probe replacement if individual hole voltages for an axially staggered, four through wall hole wear standard vary by more than a probe wear allowance between the initial or new probe values and subsequent measurements. Thus this uncertainty is limited by field data collection requirements. The probe wear allowance includes voltage repeatability uncertainties (found to be ~ 5% as shown in Figure 8-19) for a new probe and a wear allowance for additional repeatability variation. For the Farley data acquisition requirements, the probe wear allowance represents the principal NDE voltage uncertainty. Pending additional field experience with the probe wear standard, the expected 10% NDE uncertainty for probe wear has been increased to a 15% limit for the EC guidelines of Appendix A. From Figure 8-19, the standard deviation on probe wear is about 10%. At 90% cumulative probability, which is applied for the NDE uncertainty in establishing the Farley SG repair limits, the probe wear uncertainty would be about 13%. Probe replacement is required by Appendix A at a 15% voltage change using the probe wear standard.

Table 8.9 identifies variables which led to spread or uncertainties in the voltage/burst correlation. Crack morphology variations are the principal contributor to spread in the burst correlation. Voltage amplitude does not define a unique crack morphology but rather a range of morphologies and associated burst pressures with the empirical relationship established by the voltage/burst correlation. The correlation relates the EC sensitivity and burst pressures to the degradation morphology. To date, emphasis in developing the burst correlation has been placed on axial ODSCC with crack branching and limited IGA which represents the Farley SGs and many other plants based upon pulled tube examinations. Assessments of increased IGA involvement will be performed as data become available. The history of crack morphology for Farley pulled tubes shows limited IGA involvement such that the model boiler generated ODSCC specimens are

representative of Farley SG tube degradation. Currently available data for increased IGA involvement indicates that the current burst correlation tends to envelope IGA and IGA/SCC modes of degradation. Section 8.1 shows comparable voltage responses, Section 9.8 includes available Plant L data with IGA/SCC in the burst correlation and Section 9.9 shows that burst results for IGA specimens are consistent with the burst correlation.

Data analysis guidelines for voltage amplitudes are given in Appendix A to minimize the human factor variability in interpreting signal amplitudes. Some uncertainty will remain and becomes reflected in the spread of the voltage/burst correlation. For conservatism, the eddy current analyst variability is also included in the NDE voltage uncertainty. However, this issue is less significant for application of the voltage plugging limits than for 40% depth limits, for which indications are evaluated at or near the detection threshold such that many of the indications have a poor signal to noise ratio. Under these circumstances, it is expected that the human factor plays a greater role in determining the accuracy of the inspection. The voltage limits move the amplitude of concern for tube plugging to generally higher signal to noise ratios so that human factors and details of interpretation guidelines become less significant.

To assess analyst variability on voltage measurements, an assessment was performed by comparing voltage measurements from two analysts evaluating Farley-1 and Farley-2 indications at TSP intersections. The Appendix A guidelines were provided to both analysts who independently evaluated the same indications for voltage amplitudes. The differences in amplitudes between the two analysts were then evaluated as a function of a lower cutoff voltage. Table 8.10 summarizes the analyst variability evaluated at the 90% cumulative probability of the difference in voltage measurements. Examples of distributions for the voltage differences between analysts are shown in Figures 8-22 and 8-23 for Farley-1 and Farley-2. The mean difference between analysts is approximately zero. The percentage difference from the mean is essentially independent of the lower cutoff voltage. The cumulative probability of the measurement differences is used as NDE uncertainty since the distributions are not representative of a normal distribution. From Table 8.10, the eddy current analyst variability at 90% cumulative probability is about 10%.

As noted in Table 8-9, the use of field voltage measurements for pulled tubes obtained prior to implementing the present voltage calibration requirements contributes to the spread or uncertainty in the burst correlation. Uncertainties associated with field crevice conditions, like the human factors, are more significant at the low amplitudes near detection thresholds than at the voltage plugging limits. This has been the experience in Farley SGs where distorted indications have been primarily low amplitude indications. Again, the larger amplitudes near voltage plugging limits provide more reliable quantification of the indications than associated with current experience with depth limits for tube plugging.

An uncertainty in the burst correlation that adds some measure of conservatism to the correlation is the effect of tube pull forces on crack morphology and potential reduction in burst pressures. Although not a major concern for axial indications, effects of the tube pull such as loss of ligaments can occur. Since pre-pull field voltage amplitudes rather than post-pull values are used in the burst correlation, the pull force effects add conservatism. Post-pull voltages are commonly higher than pre-pull values.

Based on the above, the contributions to the NDE uncertainty at 90% cumulative probability are 13% for probe wear and 10% for analyst variability. These contributors can be considered independent variables and combined as root-mean-square (RMS) to obtain the net NDE uncertainty of 16%. This value is applied in Section 12 to develop the tube plugging voltage limit. For probabilistic SLB leak rate and tube burst evaluations, as required for the tube plugging criteria of Section 12, a distribution for the NDE uncertainty is required. The 16% uncertainty at 90% probability can be approximately equivalenced to 1.3 standard deviations to obtain a standard deviation of about 12%. The probe wear uncertainty is bounded at 15% by the probe replacement requirement. Thus an upper bound on the total NDE uncertainty distribution is estimated at about 25% representing the RMS of 15% for probe wear and 20% for analyst variability. The distribution for the NDE uncertainty is then represented by a standard deviation of 12% with a distribution cutoff at 25%.

Overall, the NDE uncertainty reflects measurement repeatability and is dominated by probe wear allowances which can be limited to 10-15% by field implementation of a probe wear standard. Burst correlation uncertainties are dominated by crack morphology variations which are accounted for by application at the lower 95% uncertainty on the burst correlation for tube plugging limits.

8.9 Conclusions

1. The use of probes from Echoram or Zetec has negligible influence on the data acquisition for the tube plugging criteria.
2. For indications with amplitudes greater than 2 volts the presence of the tube support causes only small changes in indication response for the ODSCC specimens.
3. Small indications, where their amplitude approaches the size of the mix residual, can be influenced by the presence of the support.
4. The eddy current response is essentially unaffected for a packed tube to tube support plate crevice as compared to an open crevice.
5. Large amplitude cracks which are likely to have oxide coating on the crack surfaces remain detectable by eddy current in the presence of minor denting. Small amplitude cracks and cracks with clean crack surfaces (i.e., fatigue generated cracks) may be masked by the dent signal for dented intersections.
6. Probe centering characteristics, related to probe wear, can contribute to the uncertainty of the eddy current signal. This uncertainty was found by probe wear test simulations to be about []^a to envelope typical field probe replacements. Probe wear influence on the signal uncertainty can be controlled by the use of an appropriate wear standard. The staggered 4-hole standard will be applied at Farley to limit the voltage amplitude uncertainties from probe wear to []^a for the 720 mil diameter probe.
7. Use of a reference ASME standard for voltage calibration and calibration of the 400/100 kHz mix channel are recommended for application of the tube plugging

criteria. Calibration at the mix frequency is recommended to minimize effects of variations in frequency response between probes.

8. NDE uncertainties contribute to the spread or uncertainty in the voltage vs. burst pressure correlation and tend to lower the structural limit for tube burst which is based on the lower 95% confidence interval. The use of reference calibration standards, frequency mixes, etc. are directed toward minimizing the NDE uncertainties associated with voltage measurement repeatability. Other NDE considerations remain as part of the burst correlation uncertainty although the principal variable in the burst uncertainty is crack morphology differences.
9. The NDE voltage uncertainty can be represented by a standard deviation of 12% with a distribution cutoff at 25% or 16% at a 90% cumulative probability.

Table 8.1

Effect of Flaw Location on Bobbin Coil Measurements*

Flaw Location	50% Deep Slot		100% Deep Slot	
	Voltage	Depth	Voltage	Depth
1. Slot centered in TSP	0.75	43%	47.4	100%
2. Slot extending from TSP edge inside TSP	0.95	72%	48.1	100%
3. Slot extending from TSP edge outside of TSP	1.07	36%	49.3	99%
4. Slot without a TSP	1.07	49%	48.7	99%

* Measurements for 0.25 inch long EDM slot in 0.75 inch diameter tubing.

Table 8.2

Typical Voltage Amplitude for Volumetric Type 1 of Degradation

<u>Type of Degradation</u>	<u>Voltage Examples</u>	<u>Comments</u>
Wastage		
o Characterized by machined rectangular flaws	~4.5-7.5 volts @60% depth ⁽¹⁾	Data of Figure 8-8
Fretting		
o Characterized by machined, tapered flaws	~10 volts @60% depth ⁽¹⁾	Data of Figure 8-8
Pitting		
o Single drilled hole simulation	~7.5 volts for 60 mil dia., 100% deep ~5.3 volts for 109 mil dia., 60% deep ~2 volts for 30 mil dia., 100% deep	Data of Figure 8-9 Data of Table 6.4 Data of Figure 8-9
o Multiple pits	~2 volts multiple indications for multiple pits up to 60 mils in diameter and 64% deep	Pulled tube example

Note 1. Typical limiting depths for continued service allowing 10% for EC uncertainty and about 5% for growth between inspections.

Table 8.4

Comparison of the EDM Notch Amplitude Response of Probe ZT and Probe ER
(Ratio Probe ZT / Probe ER)

<u>Notch Depth</u>	<u>400 kHz</u>	<u>200 kHz</u>	<u>100 kHz</u>	<u>Mix 400/100 kHz</u>
100 %	1.07	1.70	1.94	1.01
80 %	0.98	1.67	1.93	0.94
60 %	1.05	1.67	1.90	0.94
40 %	1.04	1.70	1.87	0.91

Table 8.5

Comparison of ASME Hole Amplitude Response of Probe ZT and Probe ER
(Ratio Probe ZT / Probe ER)

<u>Hole Depth</u>	<u>Response Ratio 100/400 kHz Mix</u>
100 %	1.58
80 %	1.54
60 %	1.55
40 %	1.53

Table 8.6

**Comparison of Tight (Magnetite Packed) and Open Crevices
for Probe ZT and Probe ER
(Ratio Tight/Open)**

<u>Sample</u>	<u>400/100 kHz Mix</u>	
	<u>Probe Zt</u>	<u>Probe Er</u>
BW-8	1.05	1.08
BW-10	1.01	1.03
BW-11	1.51*	1.54*
BW-14	0.95	0.97
BW-17	0.96	0.94
BW-12	0.91	0.93

- * Caused by process of packing the crevice as verified by comparing the pre-packed response with the tube response after removal of the magnetite packing.

Table 8.7

Influence of Denting on Indication Response

<u>Sample</u>	<u>400/100 Mix Amplitude</u>		<u>Dent Size</u> volts
	<u>Before Denting</u> volts	<u>After Denting</u> volts	
Fatigue			
FAT-1	61.5	18.6	7.39
FAT-2	14.6	5.42	6.09
FAT-3	NT	2.39	12.1
FAT-4	59.0	NDD	12.08
FAT-7	NT	1.24	9.43
FAT-8	69.0	NDD	17.4
Doped Steam			
BW-1	8.9	18.0	14.7
BW-3	12.5	3.97	6.27
BW-9	4.21	4.9	6.36

NT No Test
NDD No Detectable Degradation

Table 8.8

Laboratory Specimen NDE Summary⁽¹⁾

Sample Number	Probe Type	Flaw Amplitude (V)	Phase (°)	Flaw ⁽⁴⁾ Depth (%)	Flaw ⁽⁴⁾ Length (in)	No. of Flaws
500-1	Bobbin RPC					9
500-1 ⁽³⁾	Bobbin RPC					
509-2	Bobbin RPC					
509-3	Bobbin RPC					
510-1	Bobbin RPC					
525-1	Bobbin RPC					
525-1 ⁽³⁾	Bobbin RPC					
528-1	Bobbin RPC					
528-2	Bobbin RPC					
530-1	Bobbin RPC					
532-1	Bobbin RPC					
532-2	Bobbin RPC					

- (1) As received Specimens, Prior to Leak Test. Data are average results for Echoram EE-720-FsbM-UF and Zetec A-720-ULC(775) Bobbin Probes and Echoram EB-720-2XSRPC and Zetec 720-MRPC Pancake Probes.
- (2) Specimen had multiple axial indications.
- (3) Second NDE after initial leak testing.
- (4) RPC depth is for the deepest crack and length for the total crack network.

Table 8.8 (continued)

Laboratory Specimen NDE Summary⁽¹⁾

Sample Number	Probe Type	Flaw Amplitude (V)	Phase (°)	Flaw ⁽⁴⁾ Depth (%)	Flaw ⁽⁴⁾ Length (in.)	No. of Flaws
533-4	Bobbin RPC					
533-4(3)	Bobbin RPC					
535-1	Bobbin RPC					
536-1	Bobbin RPC					
542-1	Bobbin RPC					
542-2	Bobbin RPC					
542-3	Bobbin RPC					
542-4	Bobbin RPC					
542-4(3)	Bobbin RPC					
543-1	Bobbin RPC					
543-1(3)	Bobbin RPC					
543-2	Bobbin RPC					
543-2(3)	Bobbin RPC					

- (1) As received Specimens, Prior to Leak Test. Data are average results for Echoram EE-720-FsbM-UF and Zetec A-720-ULC(775) Bobbin Probes and Echoram EB-720-2XSRPC and Zetec 720-MRPC Pancake Probes.
- (2) Specimen had multiple axial indications.
- (3) Second NDE after initial leak testing.
- (4) RPC depth is for the deepest crack and length for the total crack network.

Table 8.8 (continued)

Laboratory Specimen NDE Summary⁽¹⁾

Sample Number	Probe Type	Flaw Amplitude (V)	Phase (°)	Flaw ⁽⁴⁾ Depth (%)	Flaw ⁽⁴⁾ Length (in.)	No. of Flaws
543-3	Bobbin RPC	[]]]	9
543-4	Bobbin RPC					
543-4(3)	Bobbin RPC					
555-1	Bobbin RPC					
555-3	Bobbin RPC					
557-1	Bobbin RPC					
557-2	Bobbin RPC					
557-4	Bobbin RPC					
558-1	Bobbin RPC					
568-1	Bobbin RPC					
568-2	Bobbin RPC					
568-4	Bobbin RPC					
568-6	Bobbin RPC					

(1) As received Specimens, Prior to Leak Test. Data are average results for Echoram EE-720-FsbM-UF and Zetec A-720-UJC(775) Bobbin Probes and Echoram EB-720-2XSRPC and Zetec 720-MRPC Pancake Probes.

(4) RPC depth is for the deepest crack and length for the total crack network.

Table 8.8 (continued)

Laboratory Specimen NDE Summary⁽¹⁾

Sample Number	Probe Type	Flaw Amplitude (V)	Phase (°)	Flaw ⁽⁴⁾ Depth (%)	Flaw ⁽⁴⁾ Length (in.)	No. of Flaws
571-1	Bobbin RPC	[9
574-4	Bobbin RPC					
576-2	Bobbin RPC					
576-4	Bobbin RPC					

- (1) As received Specimens, Prior to Leak Test. Data are average results for Echoram EE-720-FsbM-UF and Zetec A-720-ULC(775) Bobbin Probes and Echoram EB-720-2XSRPC and Zetec 720-MRPC Pancake Probes.
- (4) RPC depth is for the deepest crack and length for the total crack network.

Table 8.9

Variables Influencing NDE Voltage and Burst Correlation Uncertainties

NDE Voltage Uncertainties (Voltage Repeatability)

- o Probe centering: probe diameter and wear considerations⁽¹⁾
- o Calibration standards: dimensional tolerances⁽²⁾
- o Probe design differences⁽³⁾
- o Human factors affecting voltage repeatability that are not adequately controlled by data analysis guidelines

Burst Correlation Uncertainties

- o Crack morphology (length, depth, ligaments, multiple cracks, WGA involvement) variability for same voltage amplitude
- o Tubing dimensional tolerance⁽⁴⁾
- o Variations in field crevice conditions (open, packed, deposits, TSP corrosion, small dents, etc.)⁽⁵⁾
- o Effects of tube pull forces on crack morphology and associated burst pressures⁽⁶⁾
- o Utilization of voltage measurements for pulled tubes obtained prior to implementing voltage measurement standards of this report⁽⁷⁾

Notes:

1. Minimized in the field during APC implementation by use of a 4-hole probe wear standard.
2. The influence of dimensional tolerances of the calibration standards on voltage normalization is eliminated by calibrating the field standards to the laboratory reference standard.
3. Uncertainty minimized by specifying coil to coil spacing (coil centers are separated by 120 mils).
4. The influence of tubing dimensional tolerances as they affect burst pressure are inherently included in the spread of burst pressures from pulled tubes and laboratory specimens.
5. The influence of field crevice conditions as they affect burst pressure are inherently included in the spread of burst pressures from pulled tubes.
6. Results as pre-pull field measured voltages rather than post-pull voltages are used in burst correlation.
7. The use of field voltage measurements for pulled tubes obtained prior to implementing the voltage calibration requirements contributes to the spread or uncertainty contained in the burst correlation.

Table 8.10

EC Analyst Variability for Farley SG Bobbin Voltage

<u>Voltage Range</u>	<u>Mean Volts</u>	<u>90% Cumulative Probability (Volts)</u>	<u>90% Cumulative Probability (%)</u>
<u>Unit 1</u>			
>0.00	1.15	0.15	13%
>0.78	1.31	0.13	9.9%
>1.00	1.42	0.16	11%
>1.25	1.60	0.20	12%
>1.50	1.82	0.19	10%
<u>Unit 2</u>			
>0.0	0.91	0.08	8.8%
>0.75	1.14	0.10	8.8%
>1.00	1.37	0.13	9.5%
>1.25	1.58	0.21	13.3%
>1.50	1.96	0.13	6.6%

Figure 8-1

Voltage Sensitivity to Crack Network Morphology

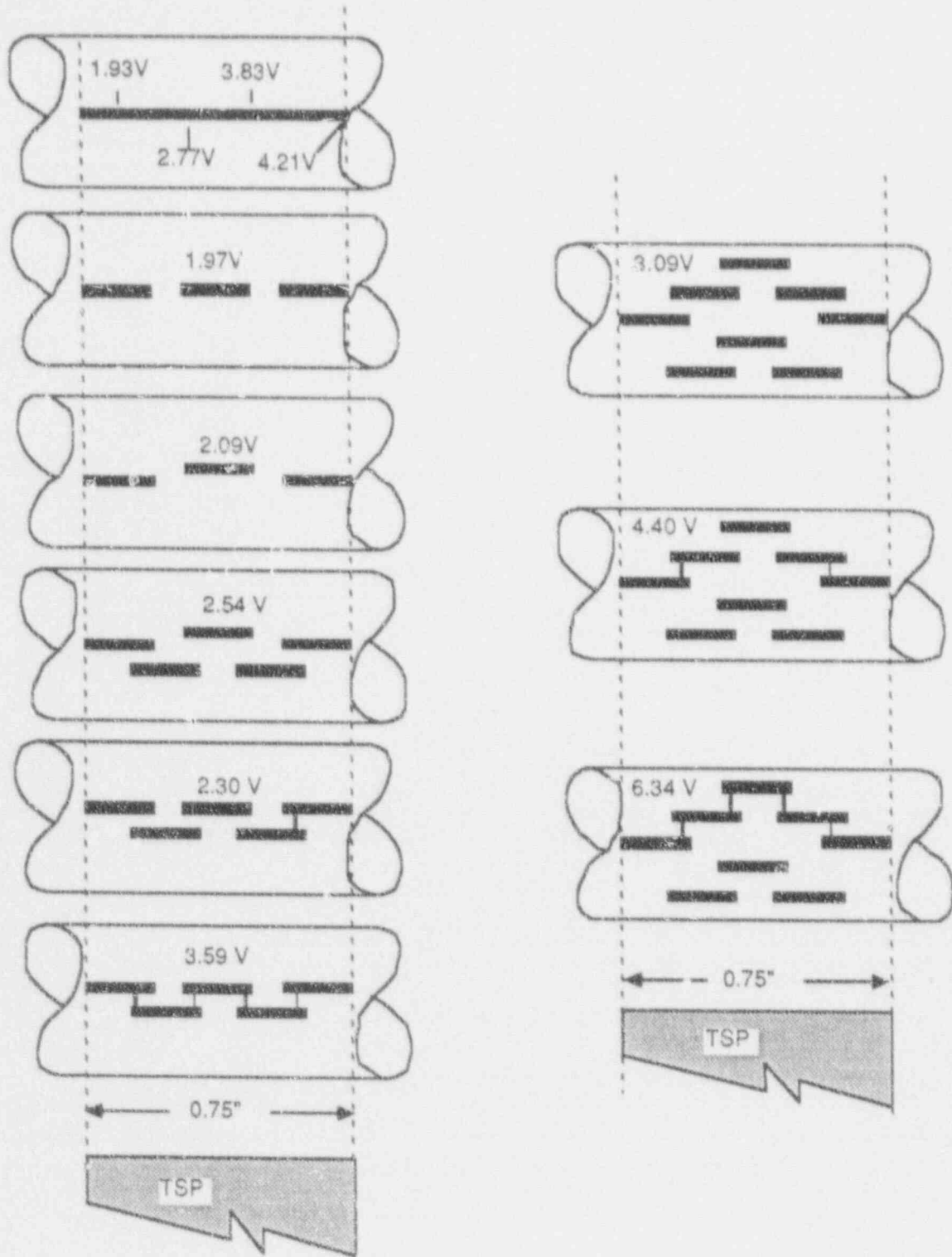


Figure 8-2

Bobbin Coil Voltage Dependence on Slot Length and Depth

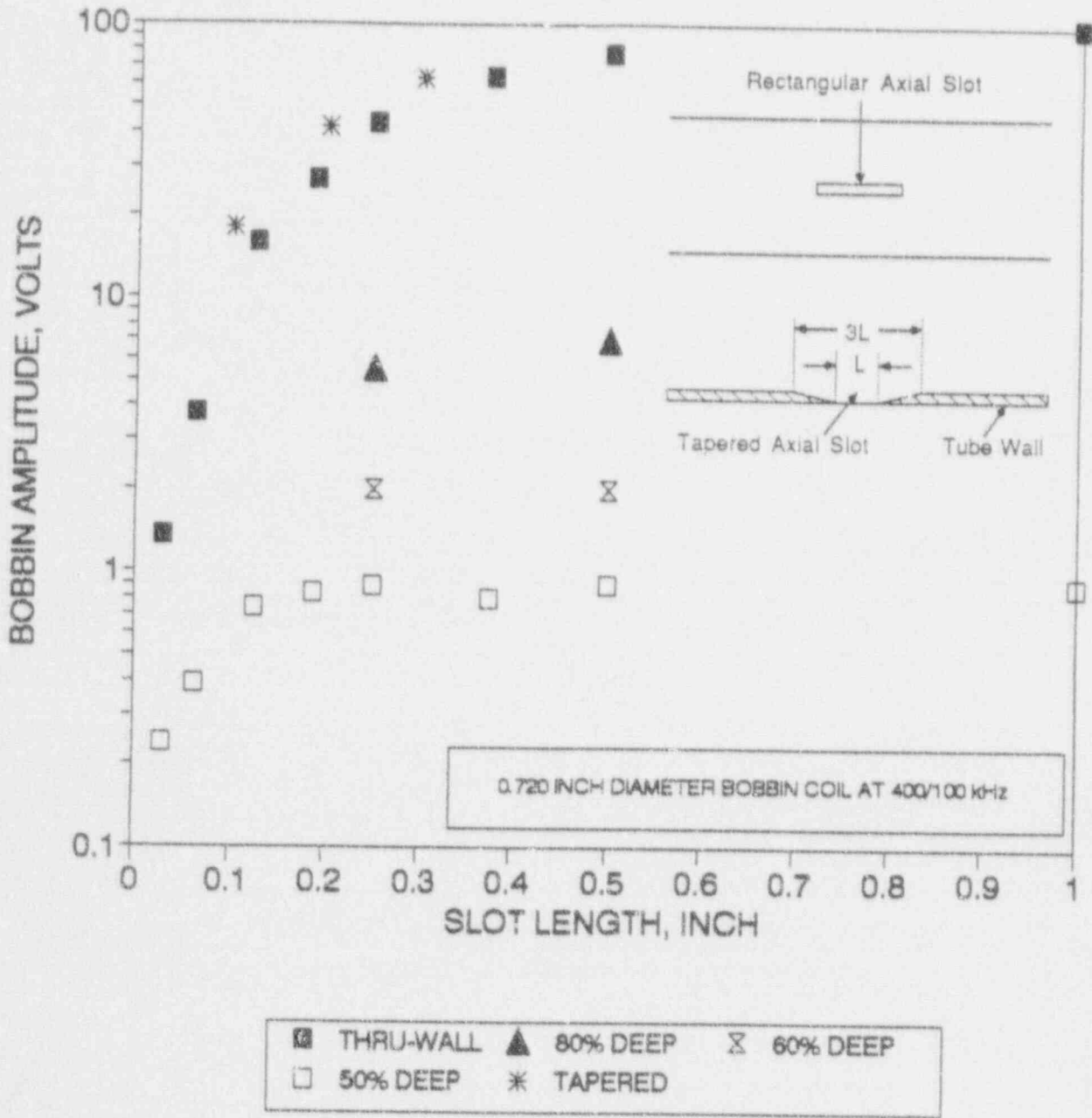


Figure 8-3

Bobbin Coil Voltage Increase due to Tapers at Ends of Through Wall Axial Slots

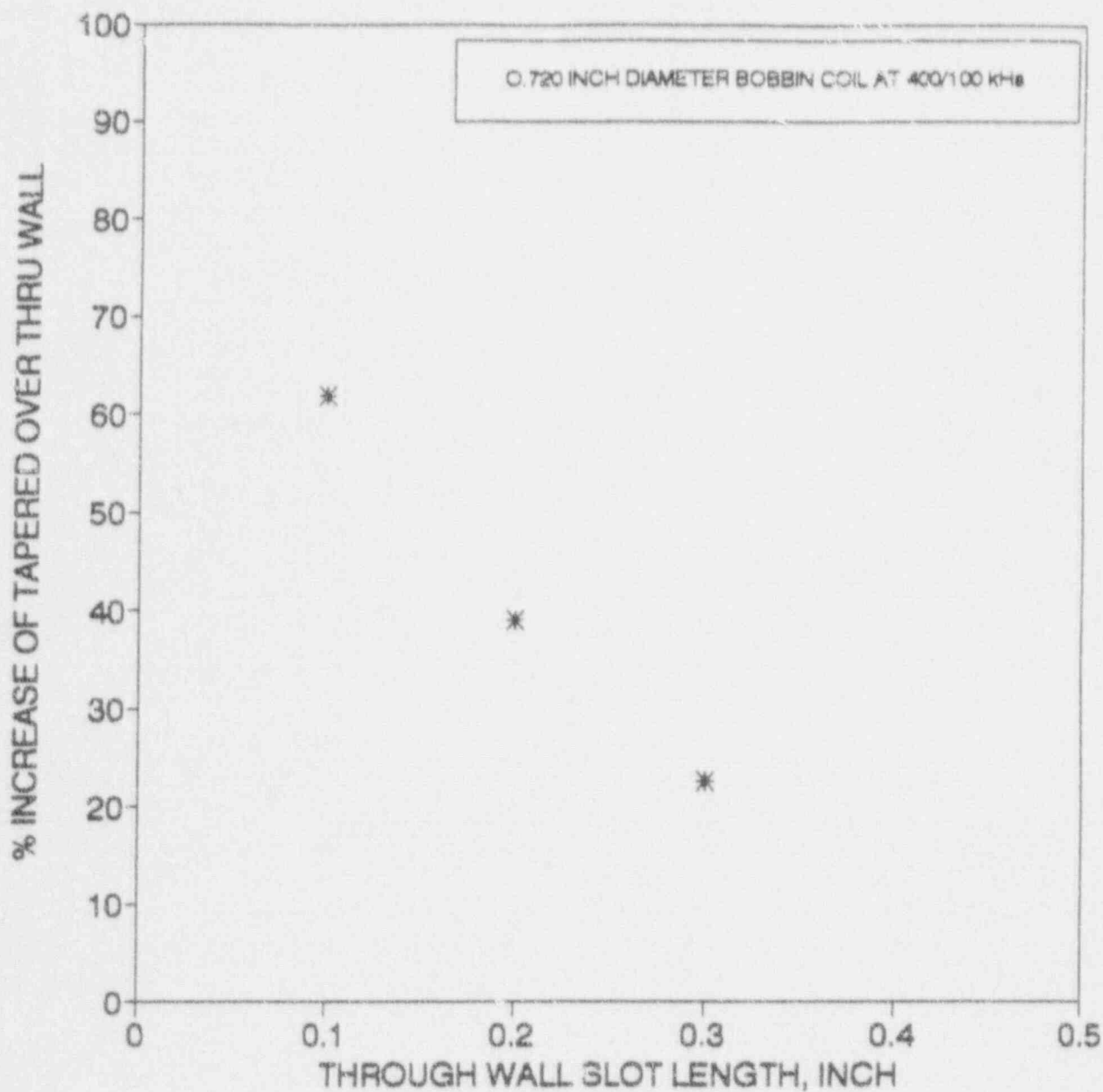


Figure 8-4

RPC Voltage Dependence on Slot Length and Depth

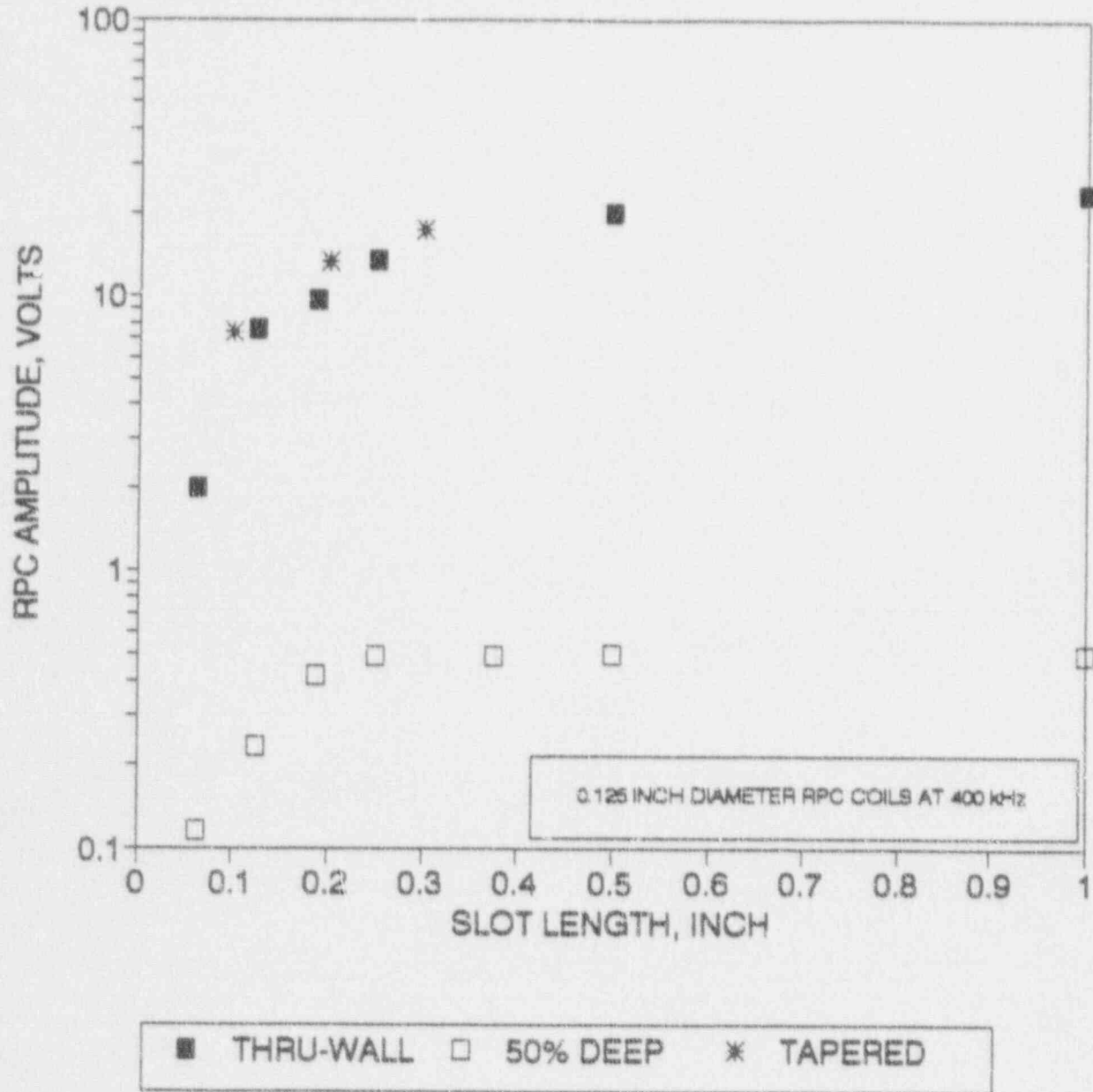


Figure 8-5

Correlation of Bobbin Coil to RPC Voltage

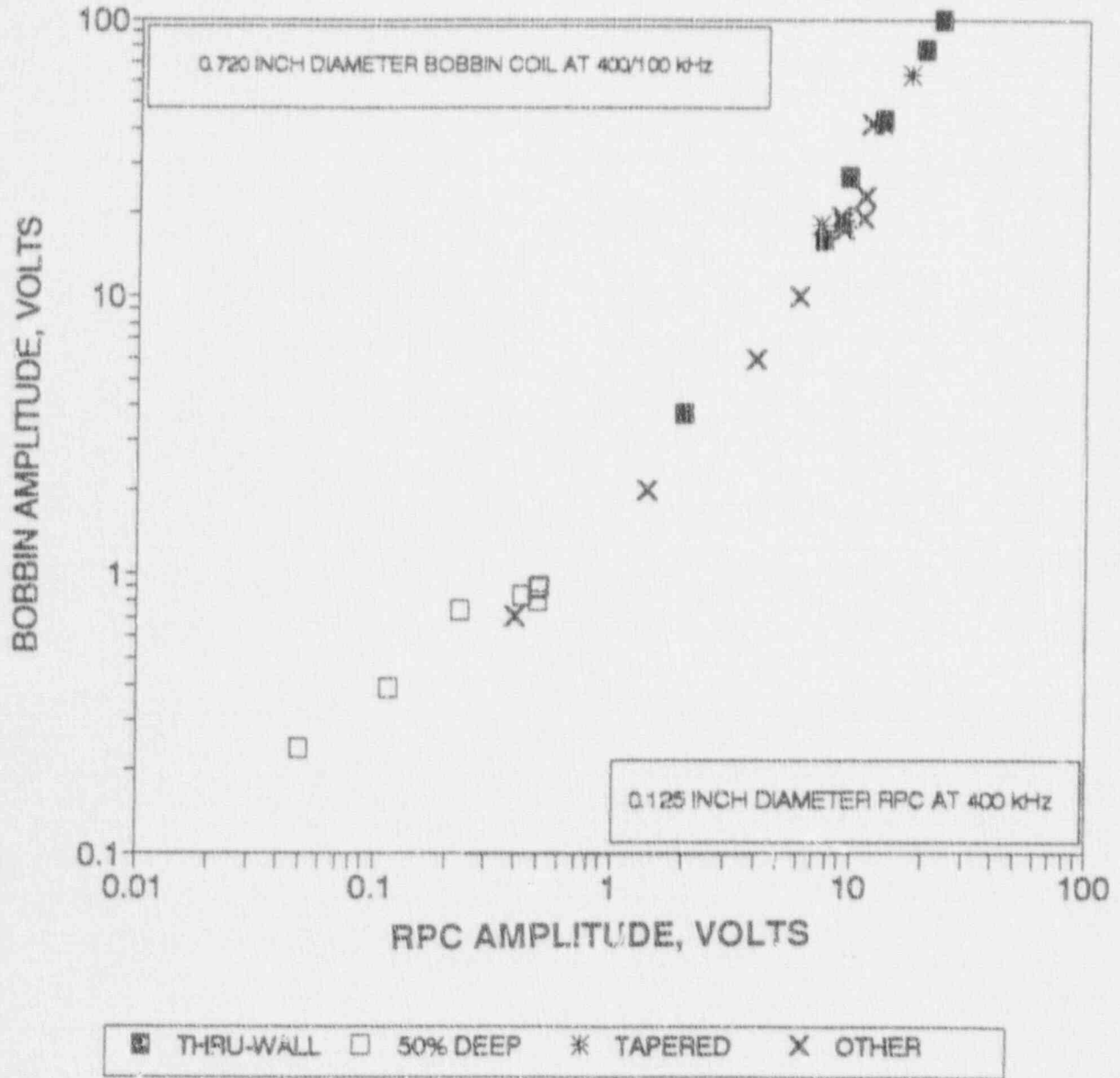


Figure 8-6

Voltage Dependence on Ligament Size Between Axial Slots

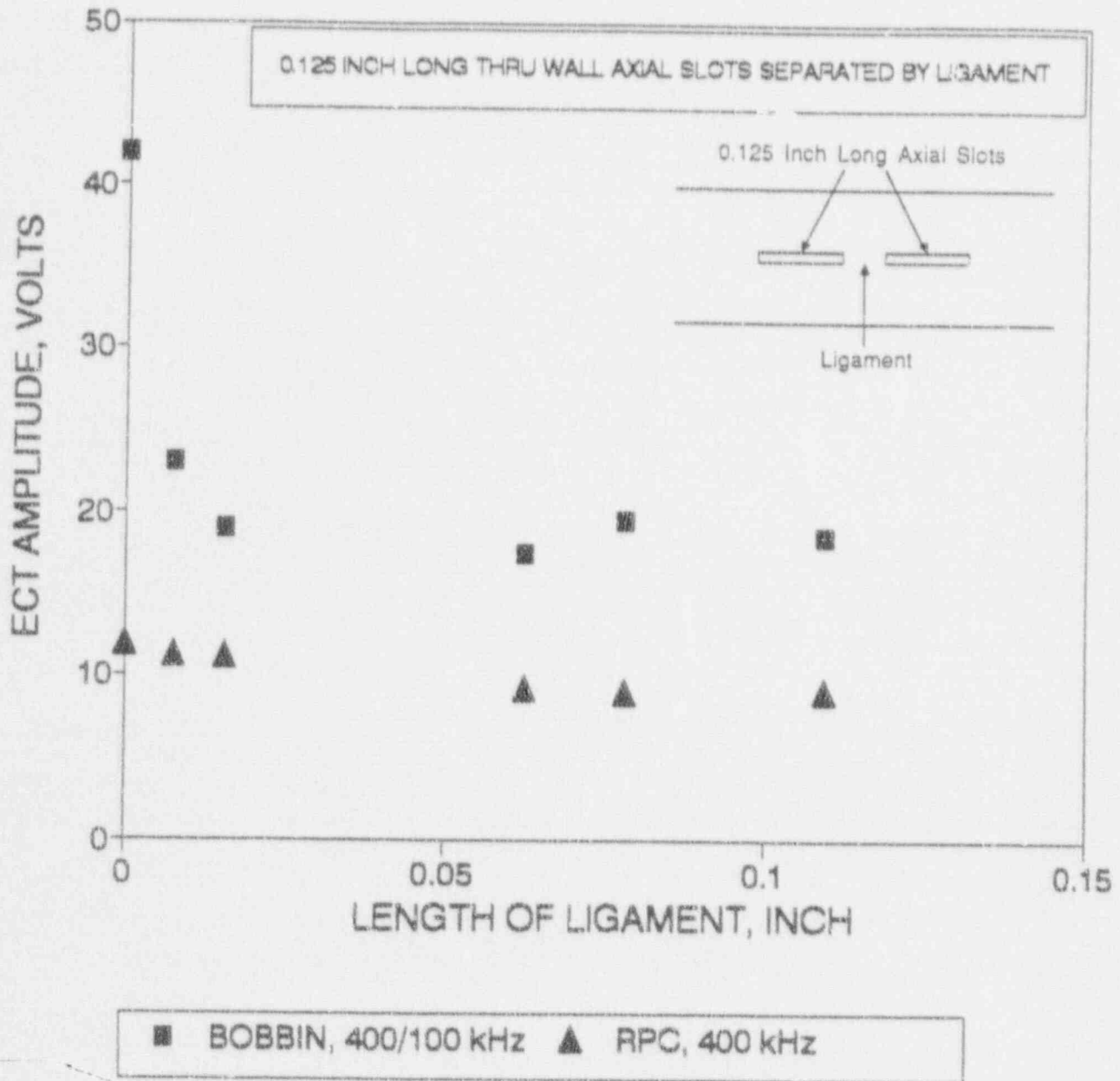


Figure 8-7

Bobbin Coil Voltage Dependence on Circumferential Spacing Between Axial Slots

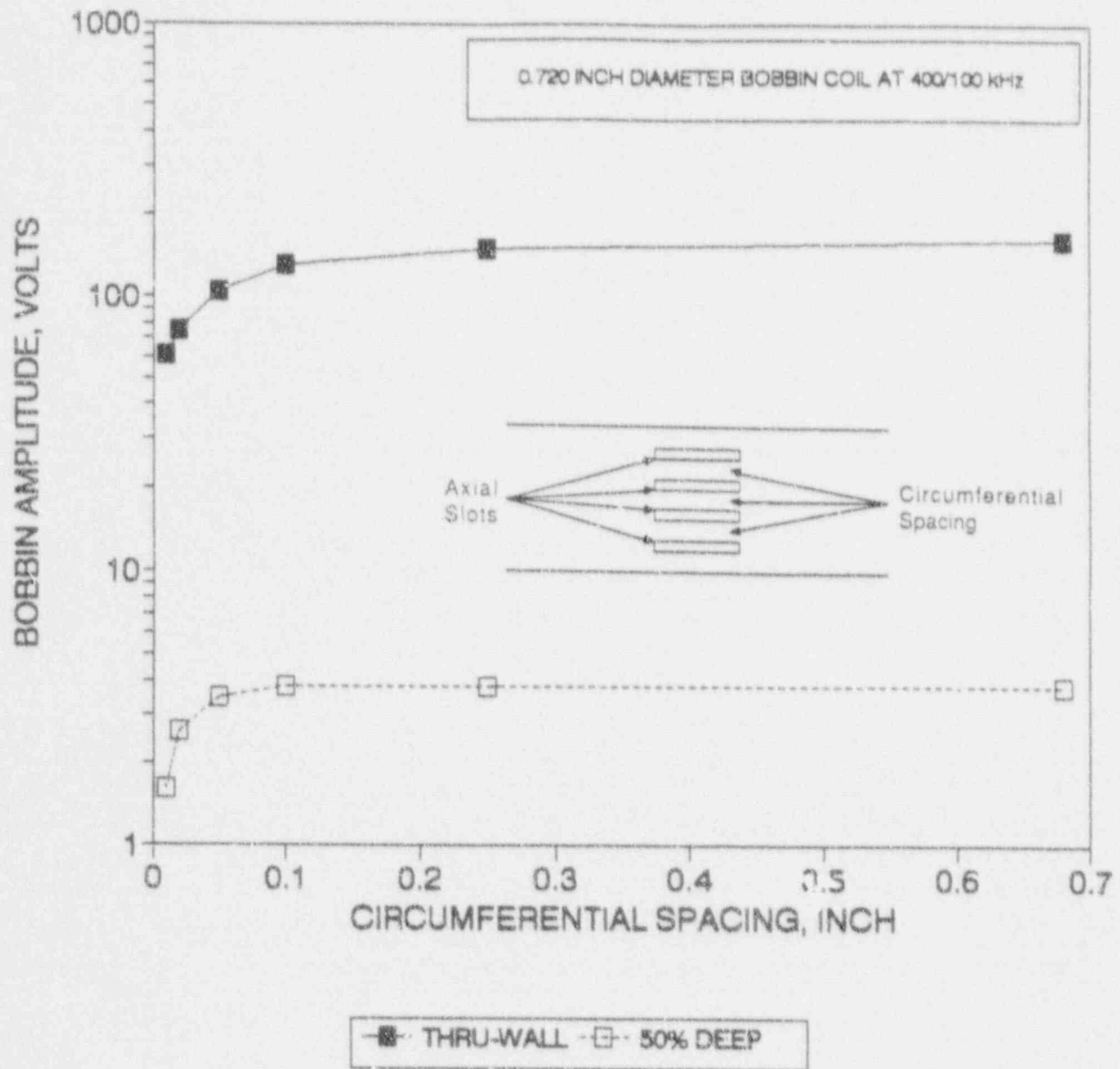
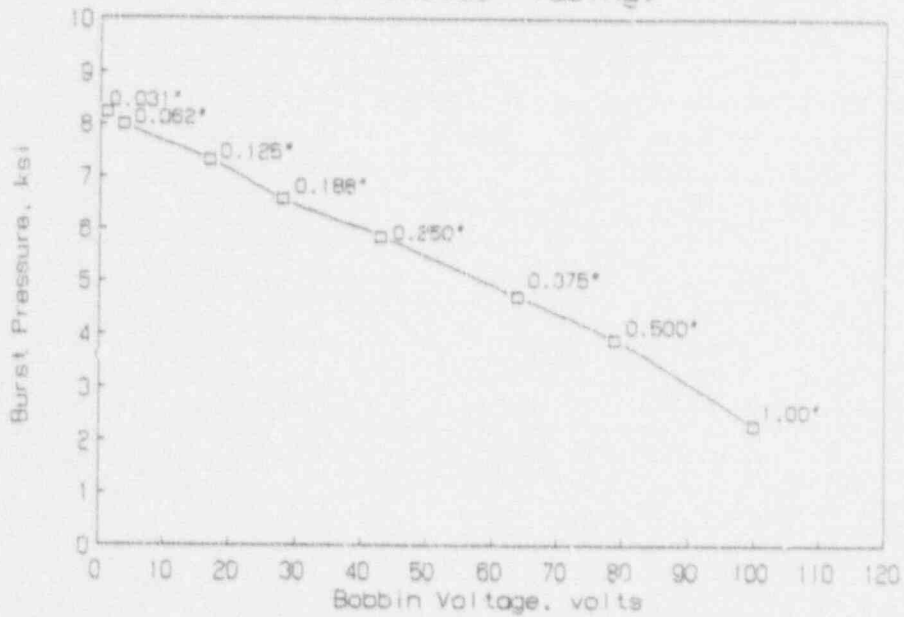


Figure 8-8

Burst Pressure vs. Voltage for EDM Slots

Burst Pressure Vs Bobbin Voltage

Thru-wall Slots
(7/8X0.05" Tubing)



Burst Pressure Vs Bobbin Voltage

Partial Wall Slots
(7/8X0.05" Tubing)

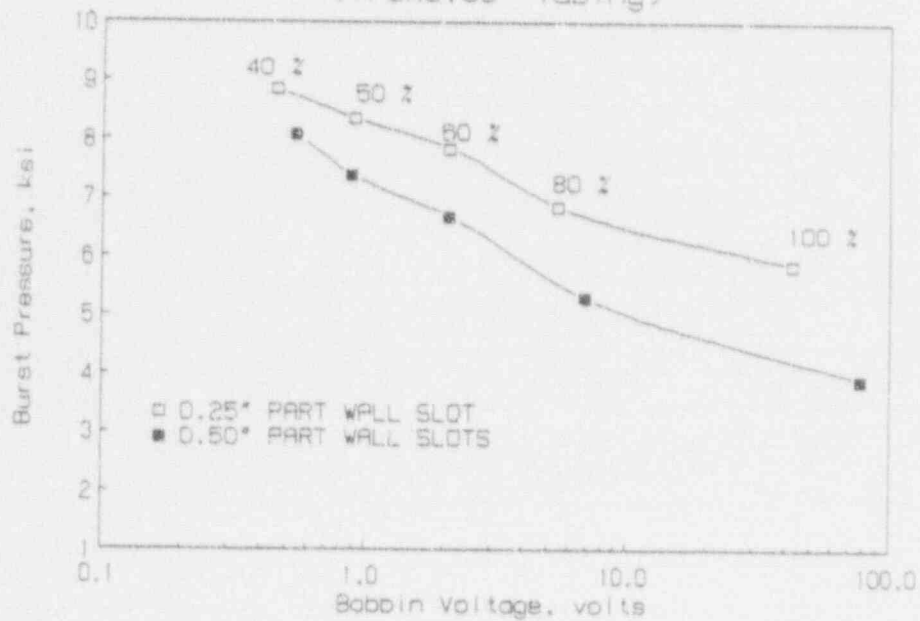


Figure 8-9

Typical Bobbin Coil Voltage vs Depth for Simulated Volumetric Tube Degradation

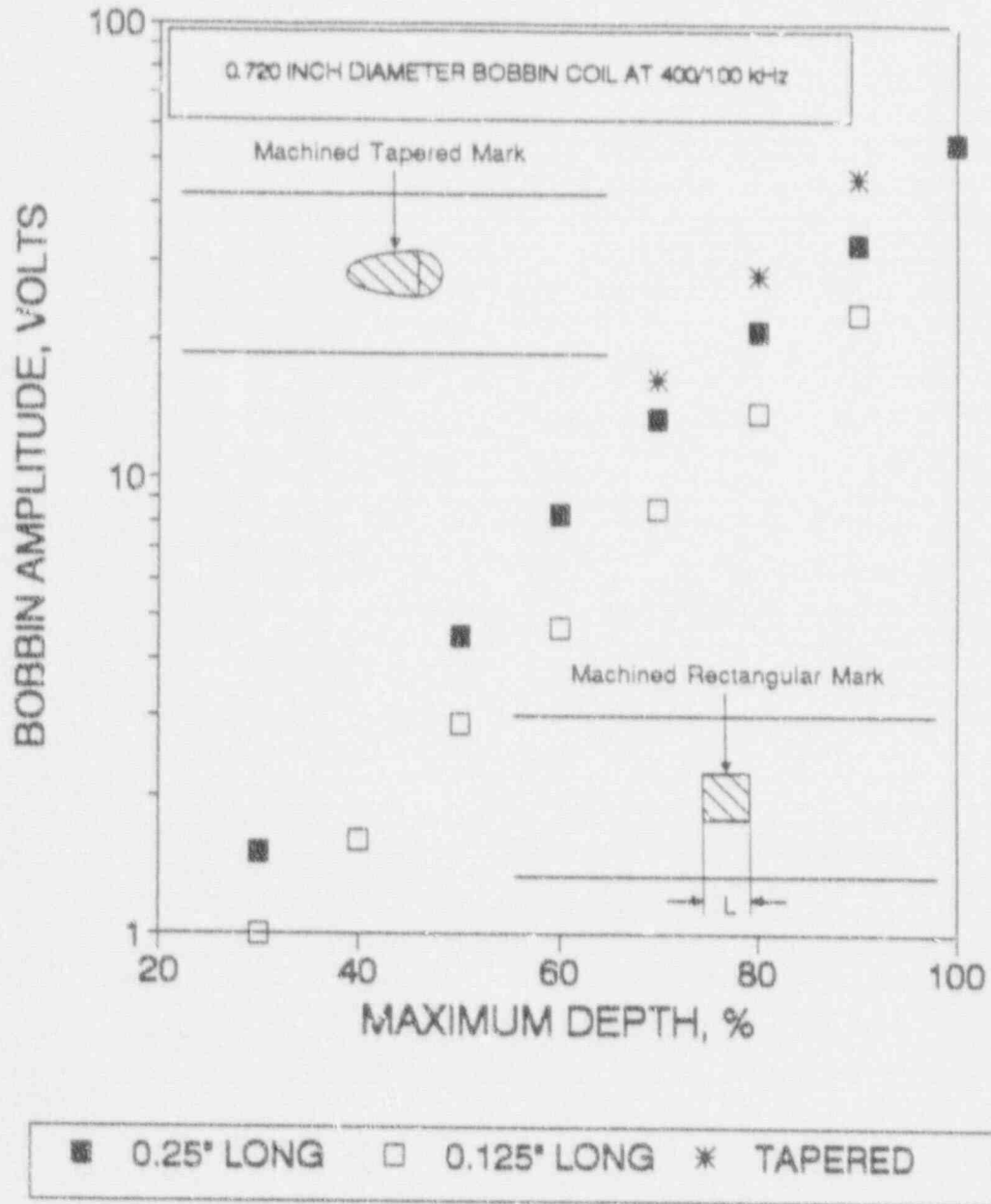


Figure 8-10

Bobbin Coil Voltage Dependence on Diameter of Through Wall Holes

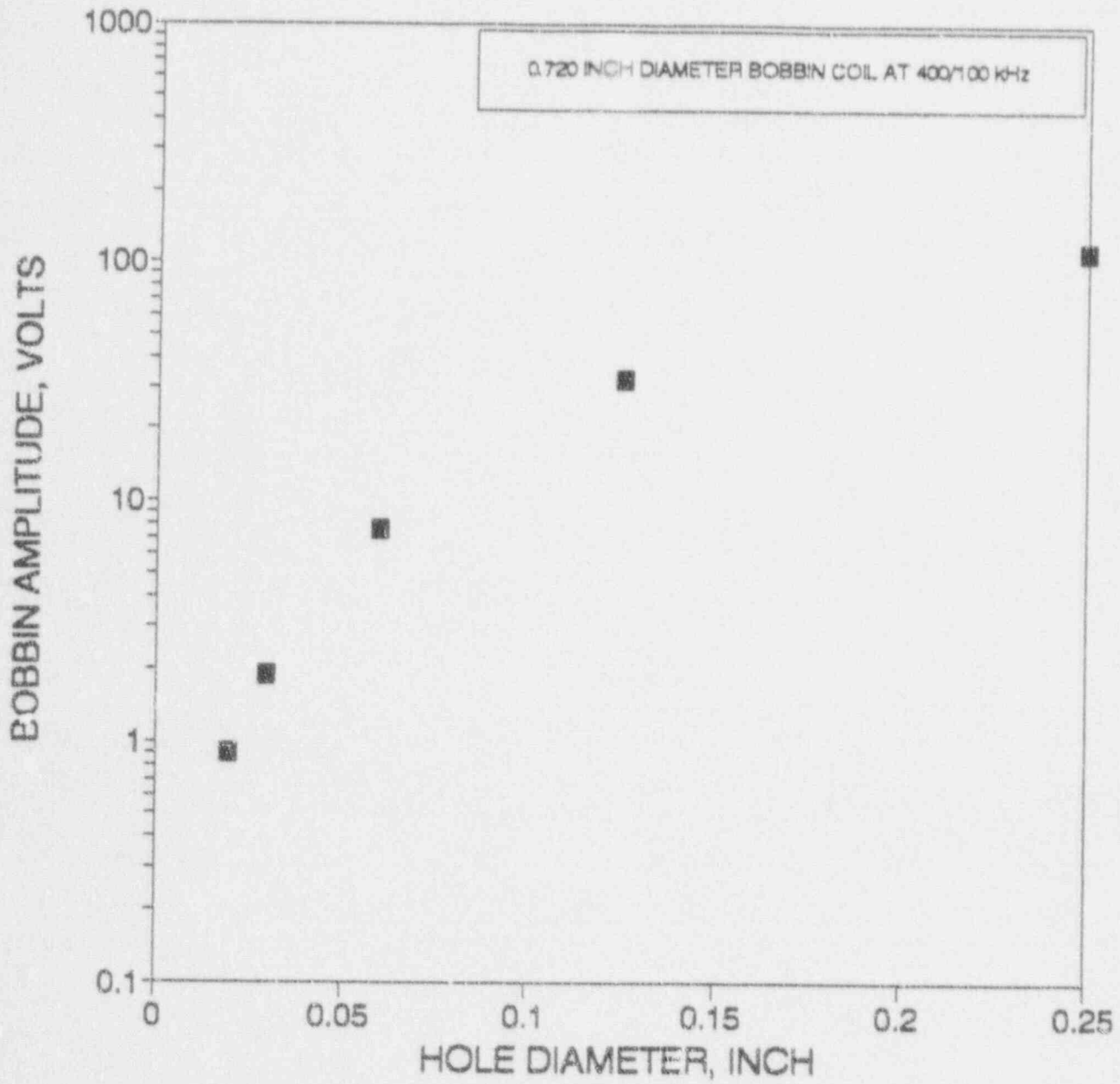


Figure 8- 11

Photograph of the OD Surface of a Pulled Tube With Cold Leg Thinning

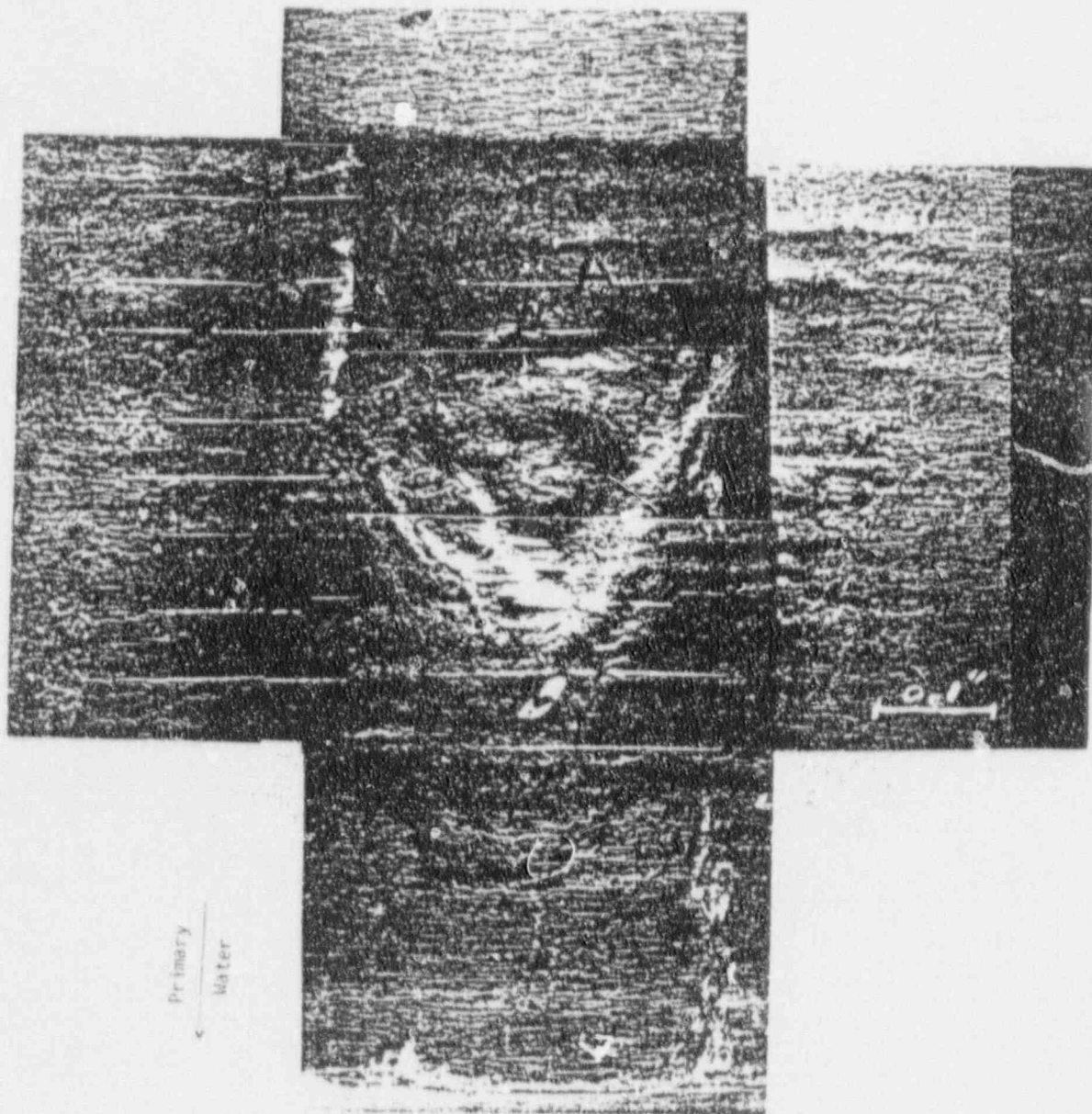


Figure 8-12a

Bobbin Data and Typical Metallographic Sections of Simulated IGA Specimens Using Sensitized Alloy 600MA Tubing

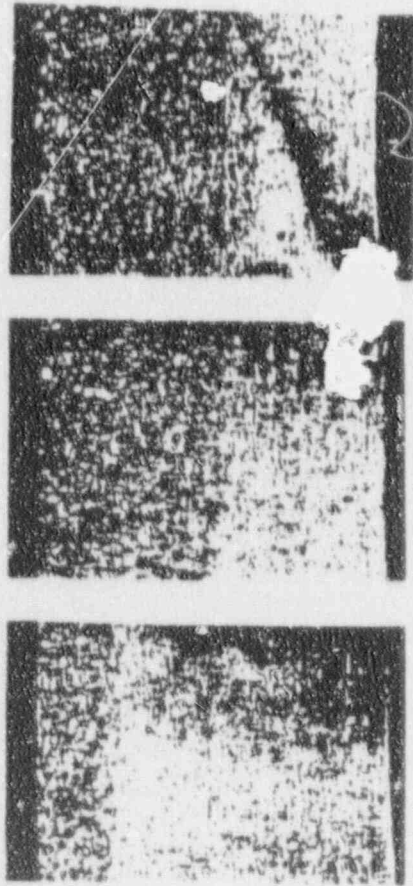


Figure 8-12b

Bobbin Data from Simulated IGA Specimens Using Non-Sensitized Alloy 600MA Tubing

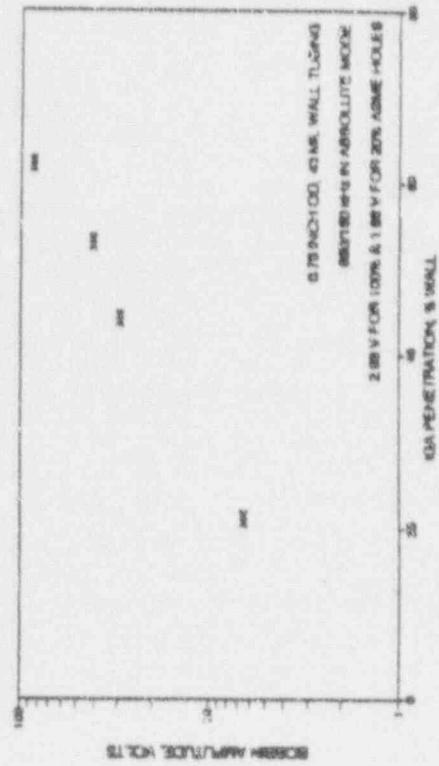
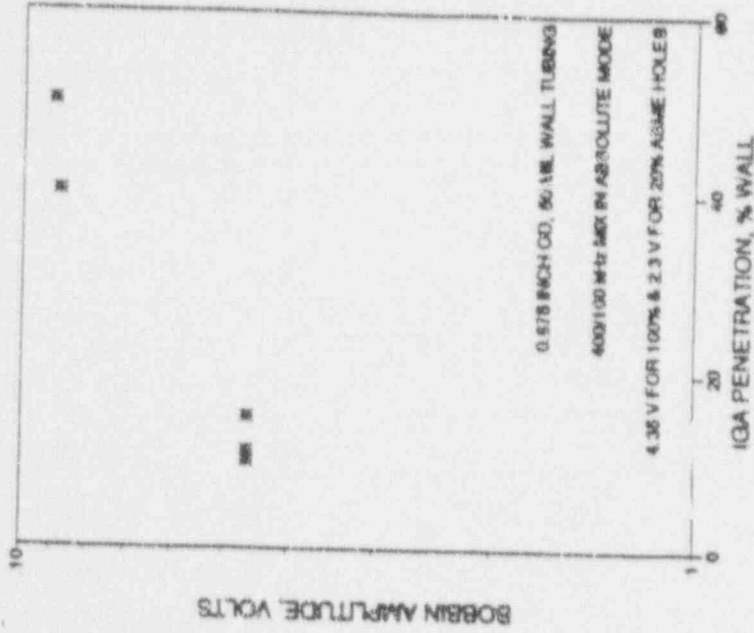


Figure 8-12. Bobbin Coil Results for Laboratory IGA Specimens

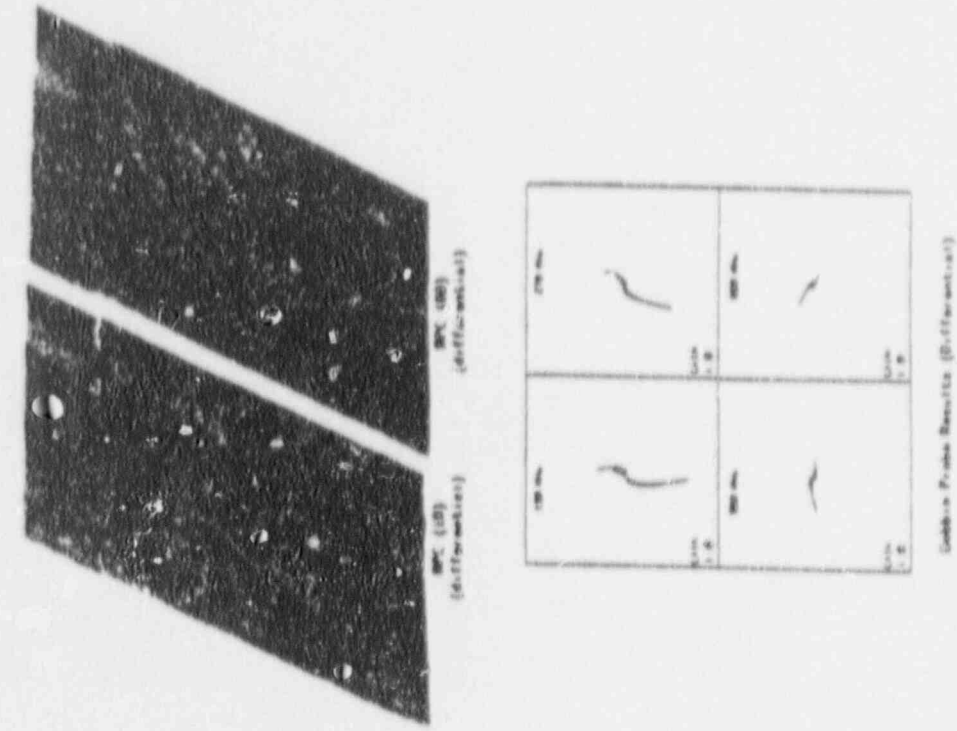


Figure 8-13a. NDE Results for a Type 1 (IGA) Sample (Dye Penetrant and X-Ray)

Figure 8-13b. NDE Results for Type 1 (IGA) Damage (Eddy Current)

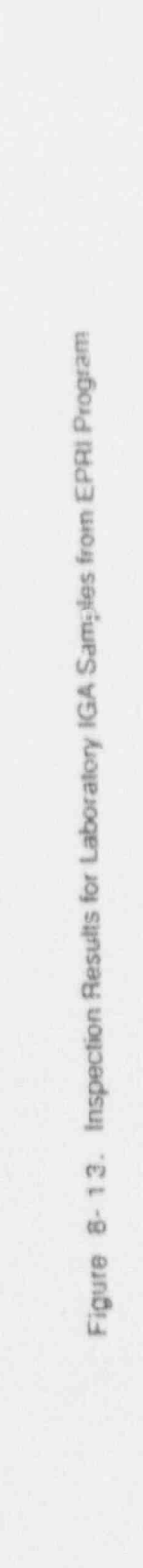


Figure 8-13. Inspection Results for Laboratory IGA Samples from EPRI Program

Figure 8-14

Voltage Comparison of Indications Found With Two Eddy Current Probes
(400/100 kHz Mix)

PROBE 1 VOLTAGE VERSUS PROBE 2 VOLTAGE

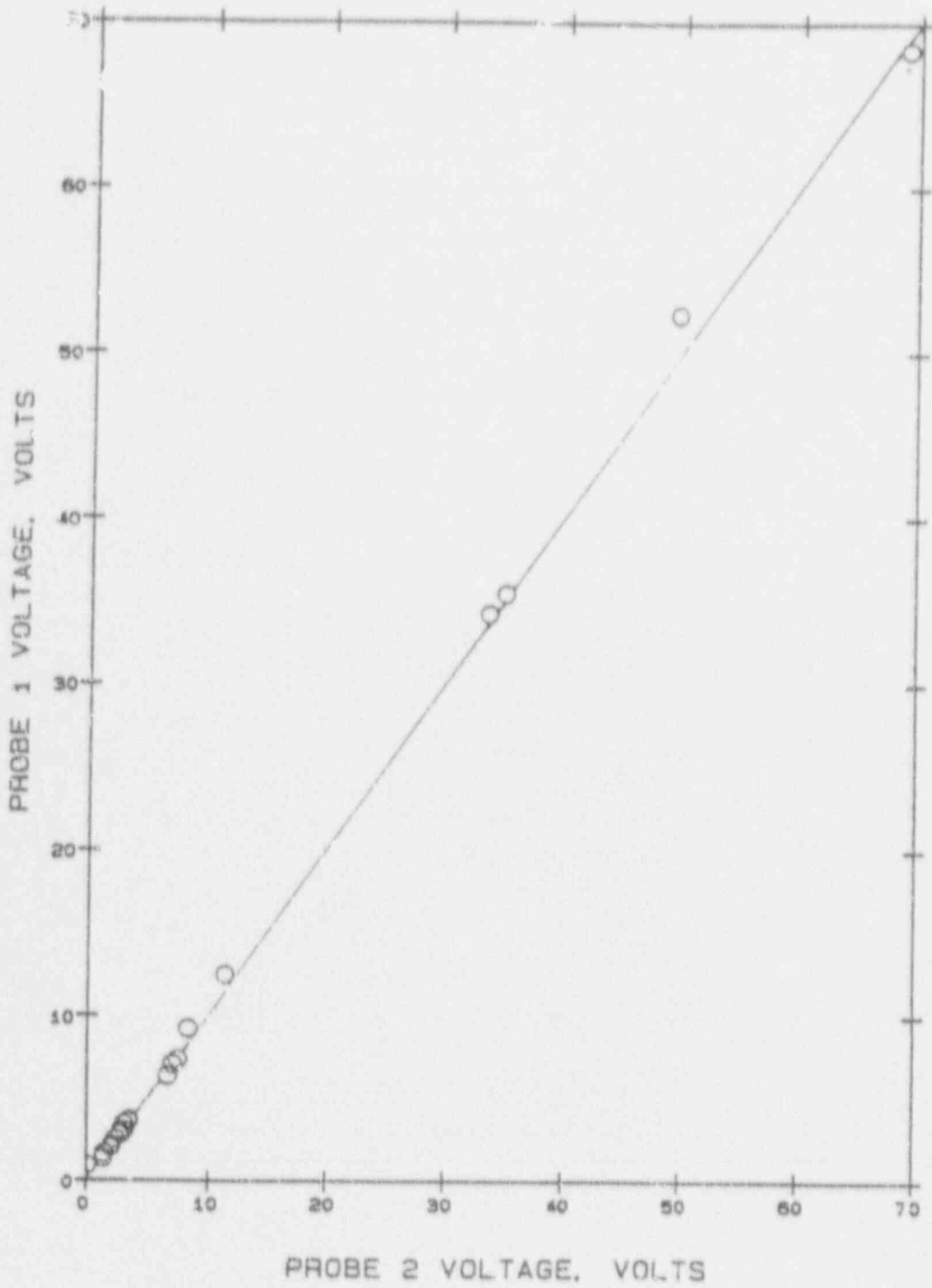


Figure 8-15

Comparison of 400/100 kHz Mix Amplitude Response from Two Probes (Model Boiler Sample)

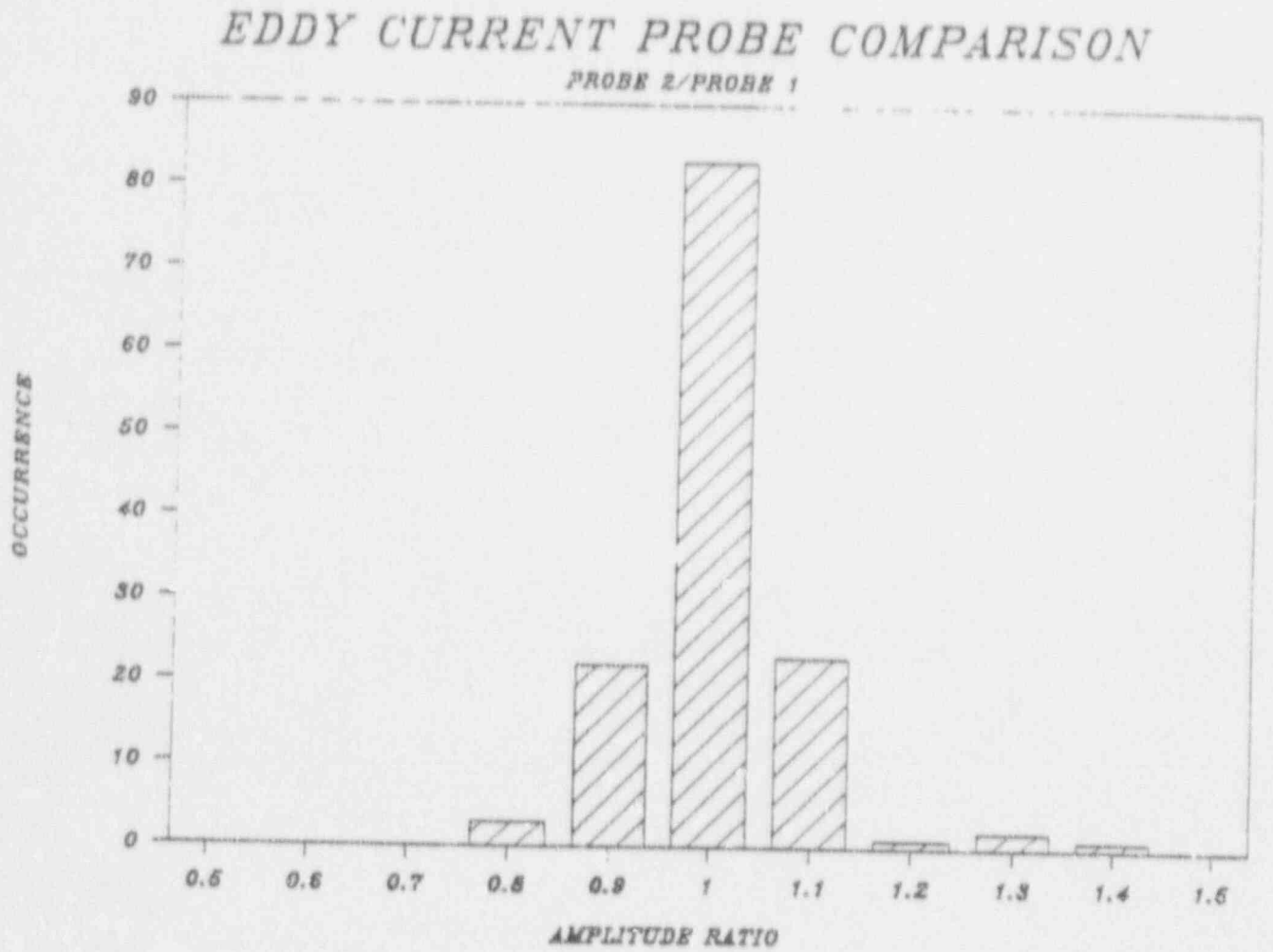


Figure 8-16

Comparison of 400/100 kHz Mix Phase Response form Two Probes (Model Boiler Sample)

EDDY CURRENT PROBE COMPARISON

PROBE 2-PROBE 1

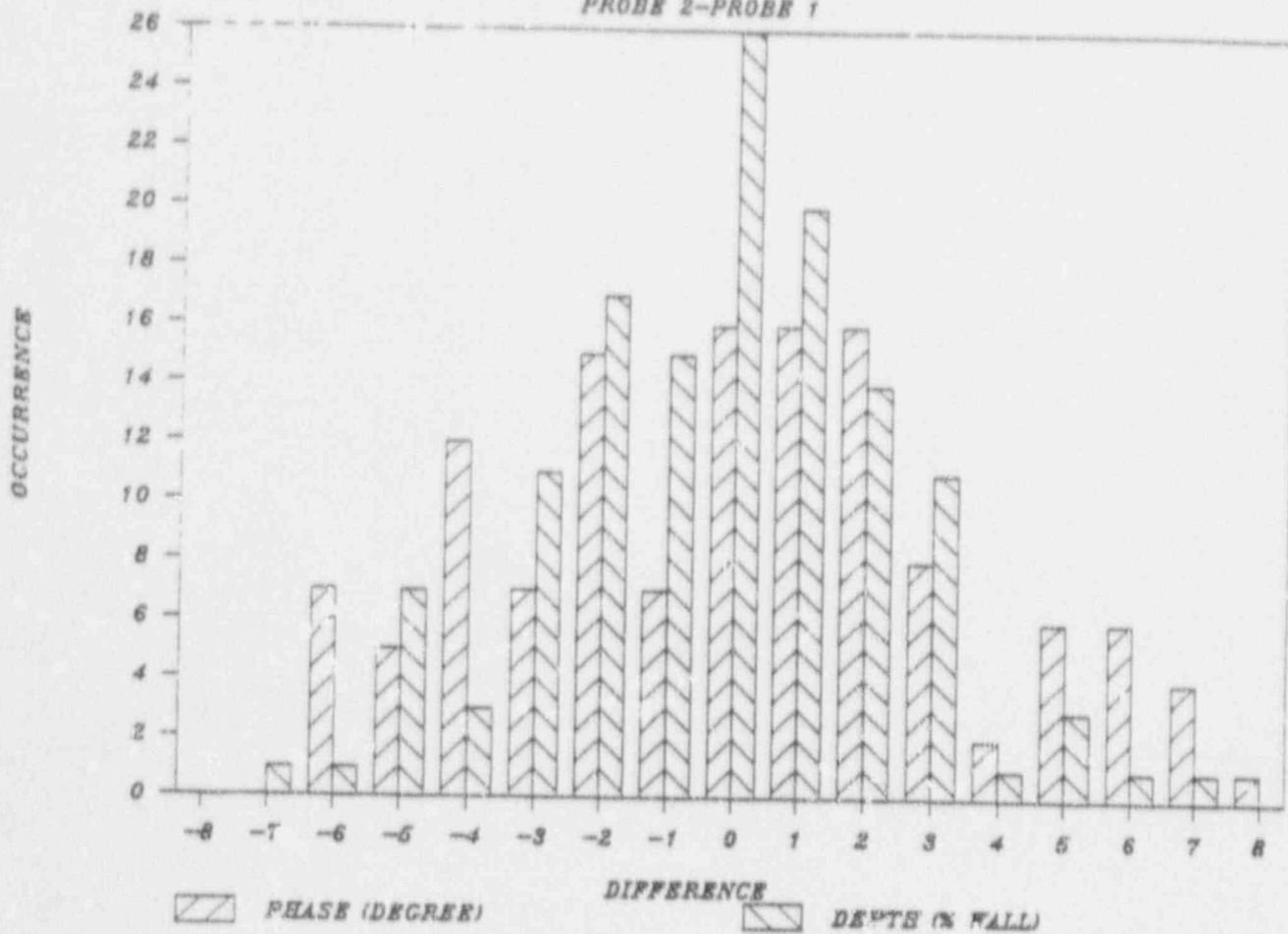


Figure 8-17

Comparison of Tight and Open Crevice Indication Response

VOLTAGES RELATIVE TO DRILLED CARBON STEEL SUPPORTS

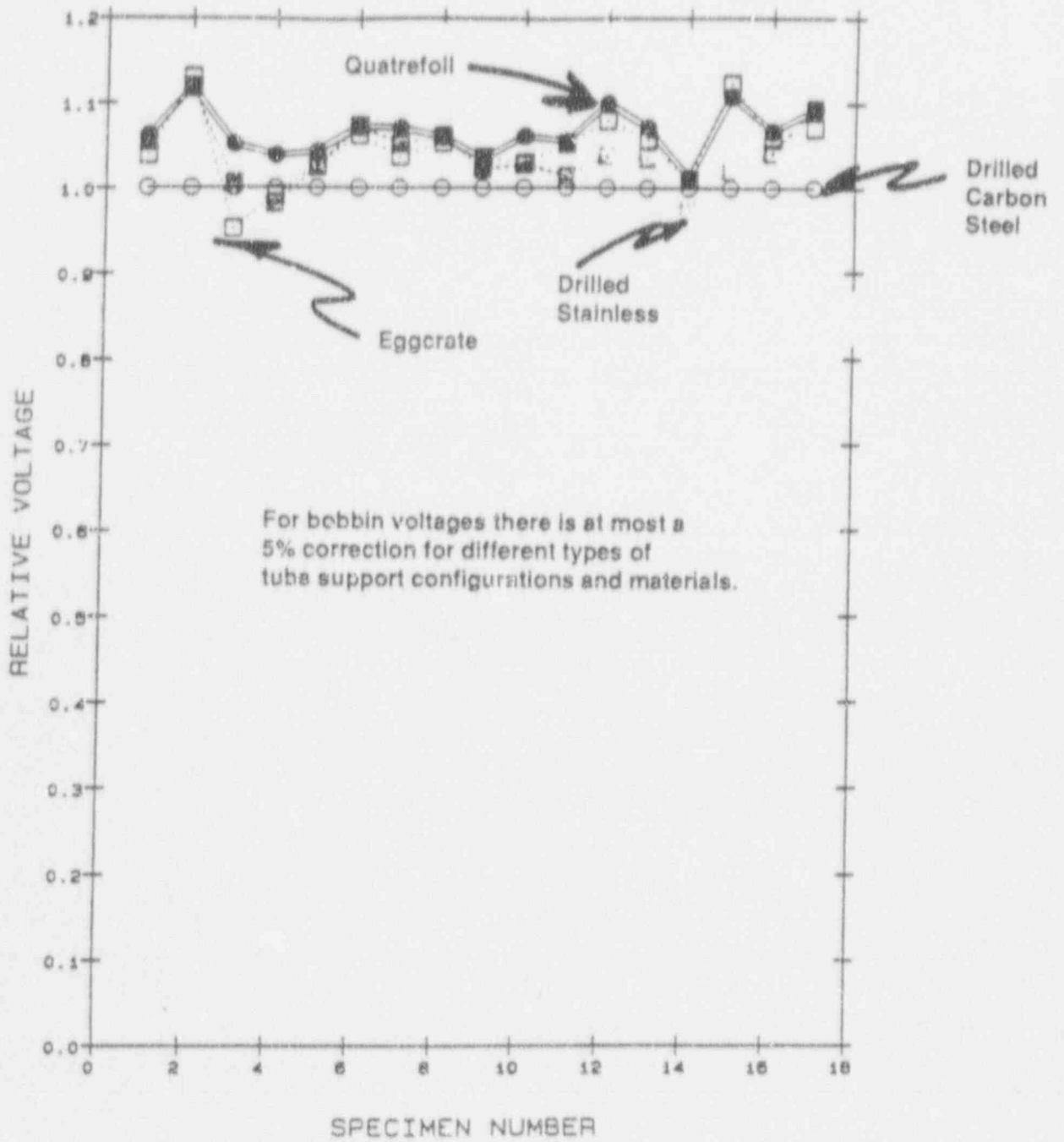


Figure 8-18

Probe Wear Calibration Standard

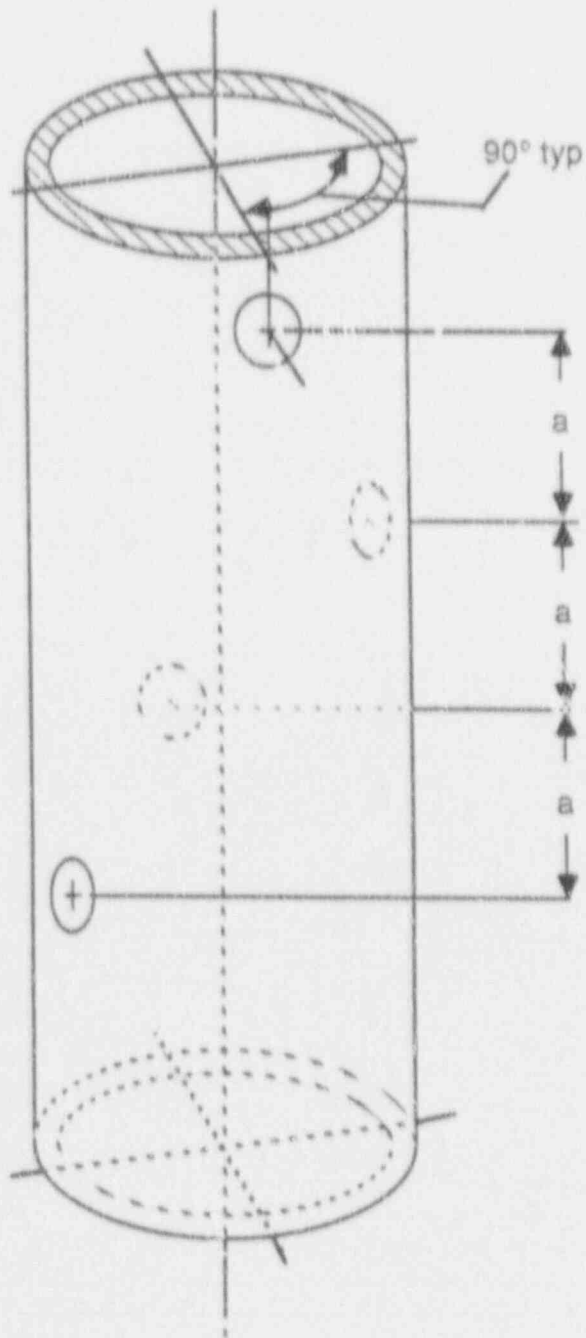


Figure 8-19

Bobbin Coil Amplitude Dependence on Probe Wear

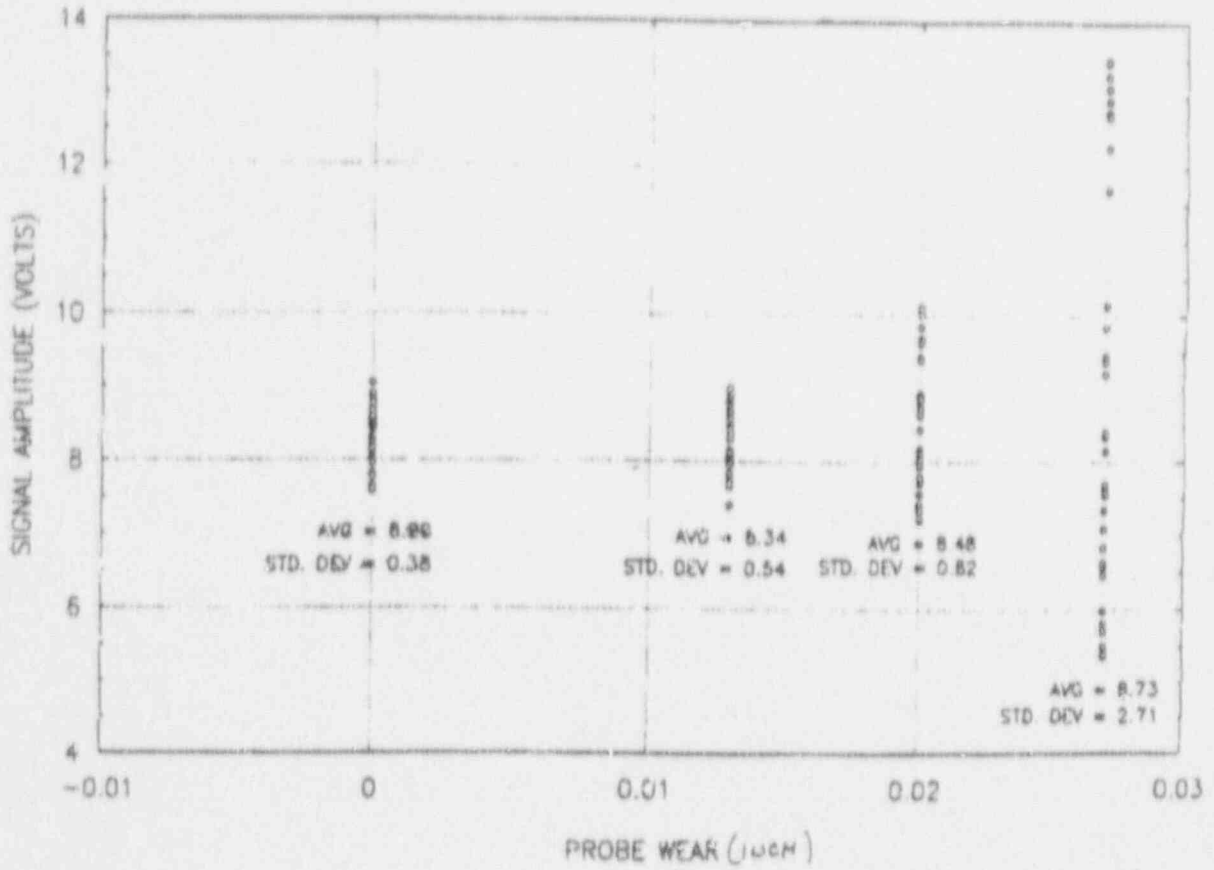
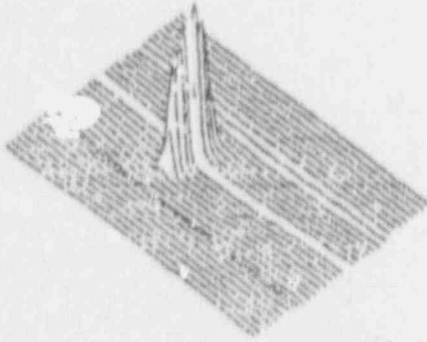


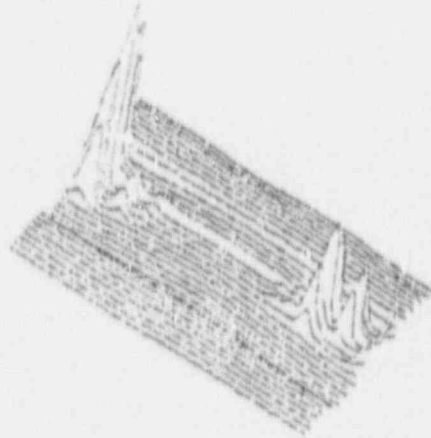
Figure 8-20

RPC Traces of Typical Model Boiler Specimens

574-4



568-2



542-4

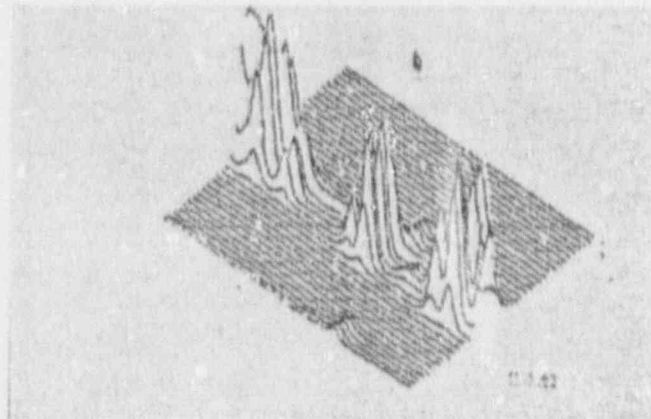


Figure 8-21

Signal Amplitude (Arbitrary Units) vs. Center-to-Center Coil Spacing

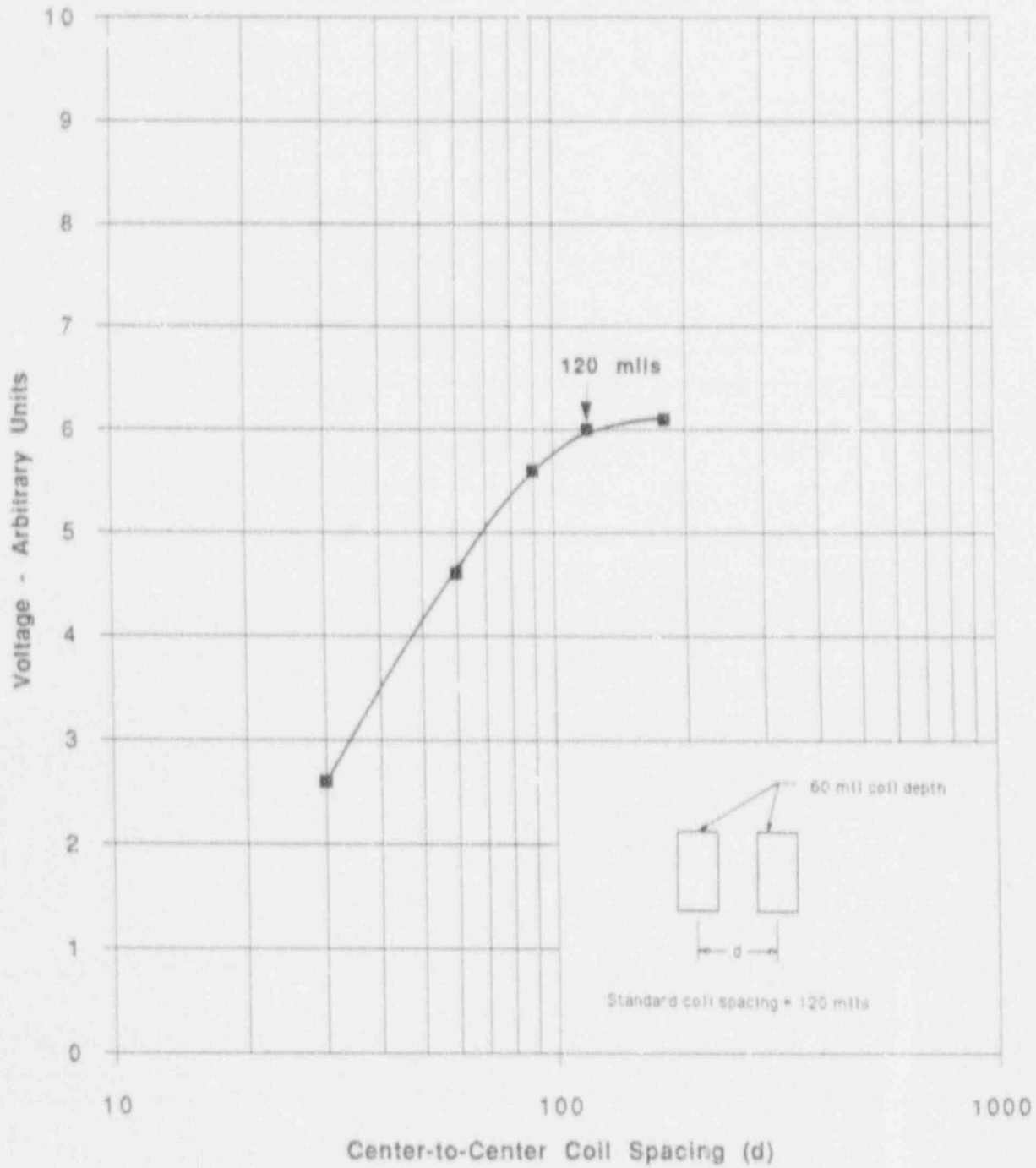


Figure 8-22. Examples of Analyst Variability for Farley-1 Voltage Measurements

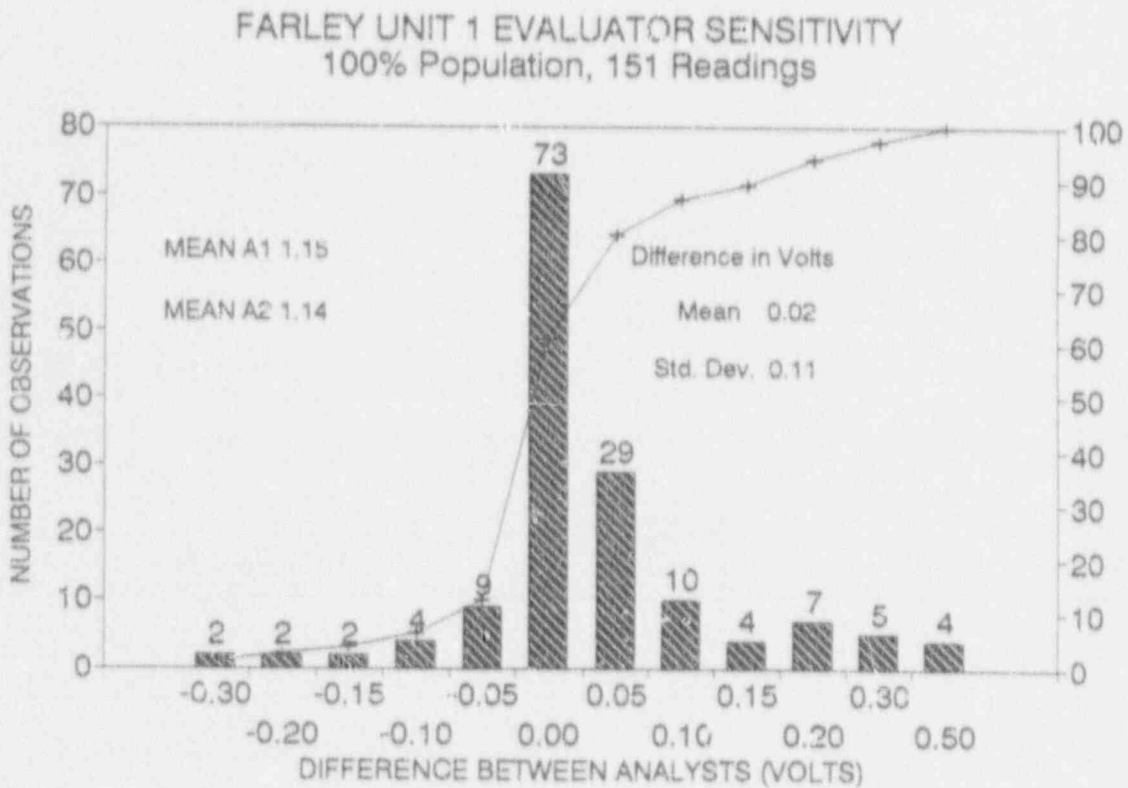
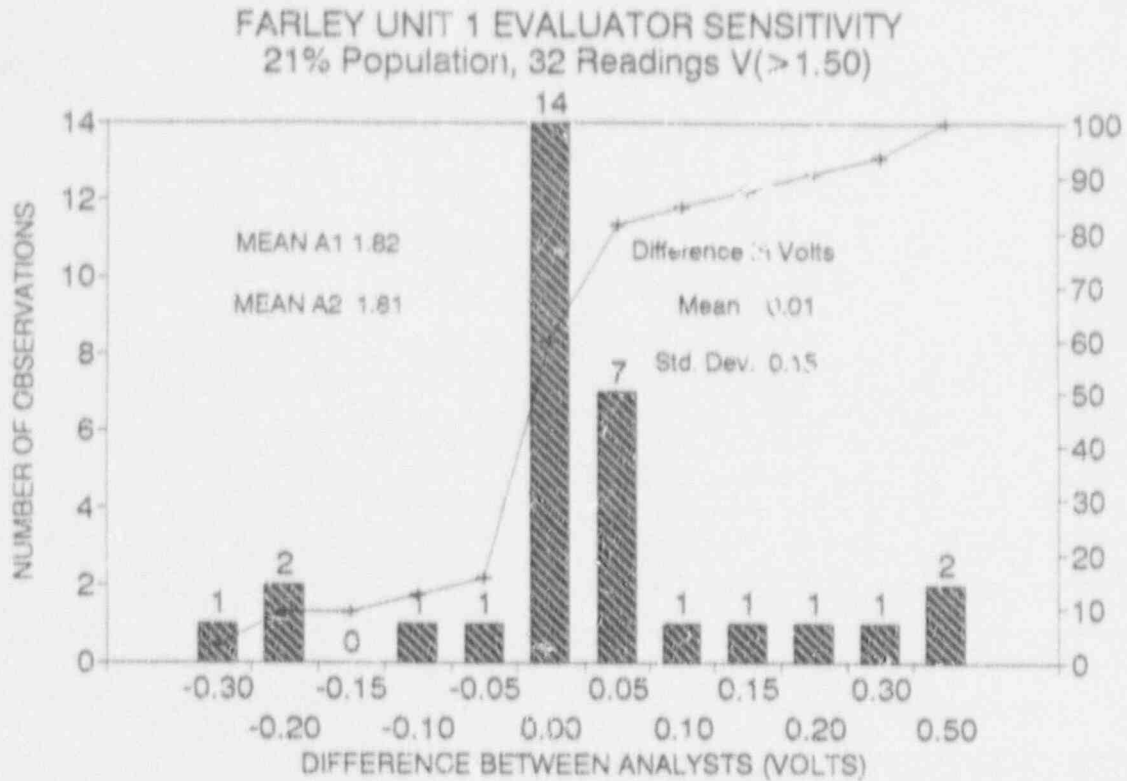
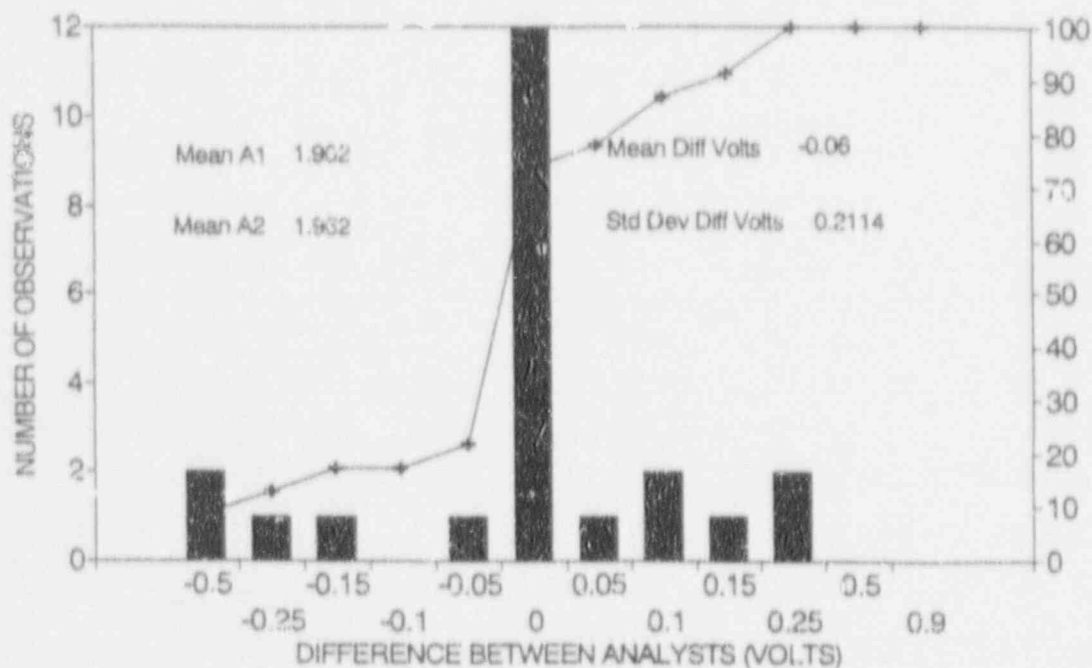
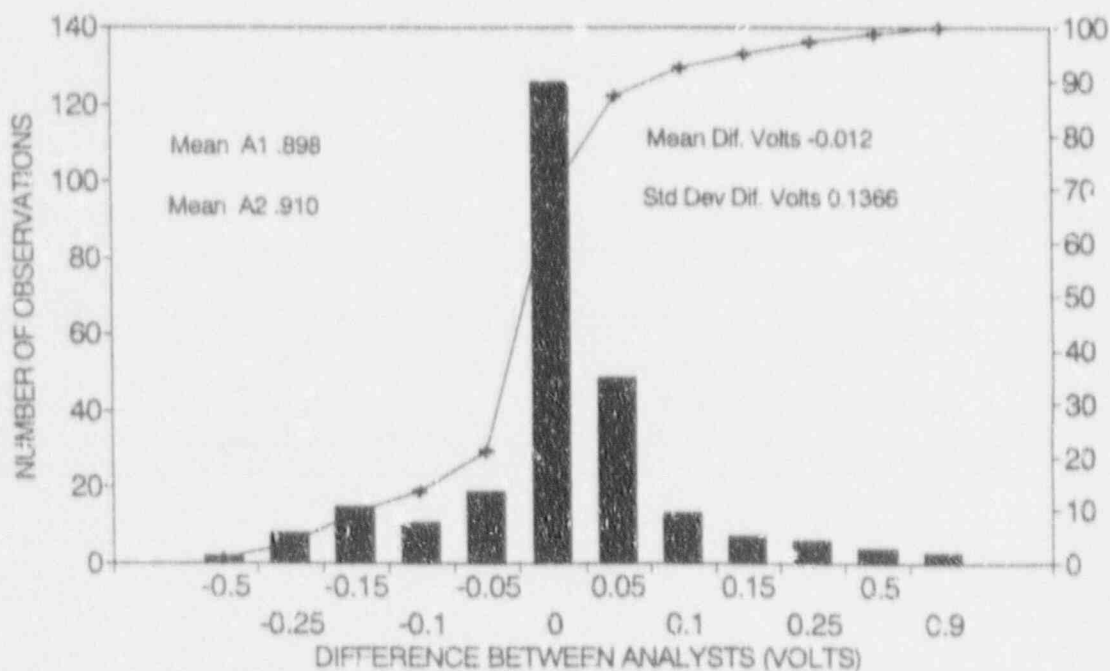


Figure 8-23. Examples of Analyst Variability for Farley-2 Voltage Measurements

FARLEY UNIT 2 EVALUATOR SENSITIVITY
9% Population, 23 Readings, V (>1.50)



FARLEY UNIT 2 EVALUATOR SENSITIVITY
100% Population, 263 Readings



9.0 LEAK AND BURST TESTS

9.1 Objectives

The objective of the leak rate tests is to determine the relationship between eddy current characteristics and the leak rates of tubes with stress corrosion cracks. Leak rates at normal operating pressure differentials and under steam line break conditions are both of interest, since leakage limits are imposed under both circumstances. The SLB leak rate data are used to develop a formulation between leak rate and bobbin coil voltage.

Crevice condition is another important factor. Tightly packed or dented crevices are expected to significantly impede leakage through cracked tubes. Since denting is readily detectable by non destructive means while crevice gaps cannot be readily assessed, the emphasis is placed upon open crevices and dented crevices as the limiting cases.

Given the assumption that significant support plate displacements cannot be excluded under accident conditions, burst tests of tubes with stress corrosion cracks are conducted in the free span condition and burst pressure is correlated with bobbin coil voltage. This burst pressure correlation is then applied to determine the voltage amplitude that satisfies the guidelines of Reg. Guide 1.121 for tube burst margins.

9.2 Leak Test Procedure

Leak testing of cracked tubes is accomplished as follows. The ends of the tube are plug welded. One end has a fitting for a supply of lithiated (2 ppm Li), borated (1200 ppm B) and hydrogenated (1 psia) water to the tube inner diameter. The specimen is placed in an autoclave and brought to a temperature of 616°F and a pressure of 2250 psi. The pressure on the outer diameter is brought to 1000 psi. A back pressure regulator on the secondary side maintains the 1000 psi pressure. Any leakage from the primary side of the tube tends to increase the secondary pressure because of the superheated conditions. The back pressure regulator then opens, the fluid is released, condensed, collected and measured as a function of time. This provides the measured leak rate. The cooling coil is located prior to the back pressure regulator to prevent overheating and to provide good pressure control. Typical leakage duration is one hour unless leak rate is excessive and overheating of the back pressure regulator occurs. Pressure is controlled on the primary side of the tube by continuous pumping against another back pressure regulator set at 2250 psi. The bypass fluid from this regulator is returned to the makeup tank.

To simulate steam line break conditions the primary pressure is increased 100 psi by a simple adjustment of the back pressure regulator and secondary side is vented within one to three minutes to a pressure of 350 psi. The pressure differential across the tube is thus 2650 psi. Temperature fluctuations settle out in several minutes and the leakage test period lasts for approximately 30 minutes.

9.3 Leak Test Results

A summary of leak test results is provided in Table 9.1. The first series of leak rate tests were conducted at the normal operating pressure differential. Some of the model boiler specimens had tightly packed crevices as a result of corrosion product buildup. These specimens were tested as is. Following this first series of leak rate tests, the welded end plugs were cut from the leak specimens. Because of the crack location and short length of some specimens additional leak rate testing could not be performed. Specimens with tight collars were subjected to extensive eddy current testing. This required removal of the tight collars. Hence all repeat testing of the first series of test specimens was performed under open crevice conditions. Repeat testing led to higher leak rates either as a consequence of the test itself, handling or forceful removal of tight collars. Only in one case did a non-leaker become converted into a leaker as a result of retesting. This case, 533-4, is one of forceful removal of a tight collar.

The steam line break conditions increased the leak rates by about a factor of three compared to normal operating conditions. More variation in this factor can be expected. Prolonged leak rate testing under operating conditions is expected to lead to lower rates. The increase in the leak rate upon transition to accident conditions then becomes more variable.

From Table 9.1, tight crevices are seen to be sometimes of benefit in reducing leak rates. Specimen 542-4 had a very high eddy current voltage and a low leak rate of [] g, while specimen 543-2 had a high voltage and a leak rate of [] g. The four other tight crevice specimens were non-leakers. Three of these remained non-leakers after removal of the tight collars. Damage during removal of the tight collar is suspected as the reason the fourth non-leaker became a leaker. Tight crevices can be of benefit in reducing leak rate but cannot be relied upon. Further, pending future developments, eddy current techniques have not been shown to be able to confidently distinguish between open and tight crevices although the presence of magnetite can often be detected in the crevice.

Eddy current inspection techniques are very sensitive to denting at tube support plate intersections. Dents of a fraction of a mil are easily detectable. Specimens with large through wall cracks which were then dented to less than one mil have not leaked significantly either at operating pressure or under steam line break conditions. A tight through wall fatigue crack 0.50 inch in length will leak at more than the typical tech spec limit of 0.35 gpm. From Table 9.1 it is evident that a small dent has turned such a cracked tube into a non-leaker. []

]g.

9.4 Burst Test Procedure

Burst tests were conducted using an air driven differential piston water pump at room temperature. Pressure was recorded as a function of time on an X-Y plotter. Sealing was accomplished by use of a soft plastic bladder. Burst tests of tubes with stress corrosion cracks were done in the free span condition. No foil reinforcement of the sealing bladders was used since the crack location which was to dominate the burst

behavior was not always readily apparent. Some of the maximum openings developed during burst testing were not sufficient to cause extensive crack tearing and thus represent lower bounds to the burst pressures. The openings were large enough in all cases to lead to large leakage.

9.5 Burst Test Results

Burst test results are summarized in Table 9.1. Figure 9-1 illustrates a plot of burst pressure versus bobbin coil voltage for specimens from model boiler and pulled tube tests which are considered reasonably representative of the range of field observations of ODSCC of tubes at tube support plate intersections. Note that some of the burst data points are lower bound estimates since extensive crack tearing did not develop. In these cases the crack openings were large enough to cause large leakage events in service. From Figure 9-1 it is seen that burst pressures remain above about [5000 psi even for voltage up to 30 volts]9. As discussed later, reasonable limits on bobbin coil voltage can assure maintenance of required burst pressure margins with respect to both operating and accident pressure differentials.

9.6 Correlation of Burst Pressure with Bobbin Coil Voltage

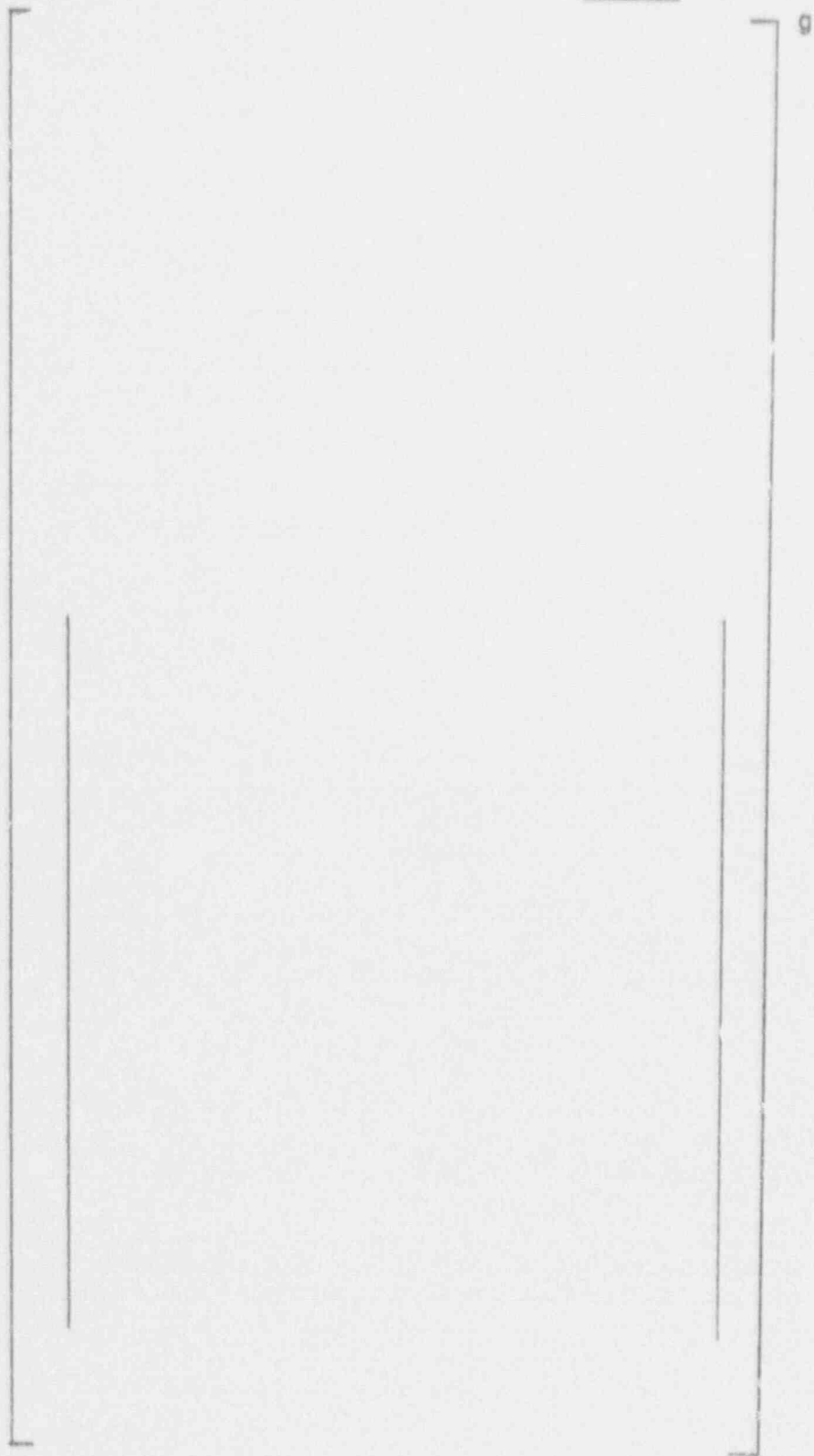
As noted in an earlier section, a broad interpretation of the physical significance of the voltage of an eddy current bobbin coil indication is that the voltage reflects the volume of material over which eddy currents are perturbed by the geometrical configuration producing the indication. Hence, there is a very broad range of different geometries which can produce the same voltage indication. In the case of ODSCC in the tube support plate crevices of steam generator tubes, the bobbin coil indication voltage reflects the volume of material where eddy currents are perturbed by cracking. Obviously at a given indication voltage any one of a broad range of crack morphologies can be present. From past experience with pulled tubes it is known that cracking patterns are typically confined to the crevice region and thus restricted in length. Additionally, the typical cracking pattern consists of an array of essentially axial cracks. Thus, at a given indication voltage one of many crack patterns can be present but there are observed restrictions in these patterns. Other restrictions to cracking patterns, within perhaps rather wide limits, are expected from the operation of a given SCC cracking mechanism. Just as crack morphologies are many valued but bounded at a given indication voltage, burst pressures will be consequently many valued but bounded.

The burst data of Table 9.1 (model boiler samples) and of Table 6.2 (pulled 7/8-inch tubes) provide a total of 60 data points that have been used to develop a correlation between bobbin coil voltage and burst pressure. The data used in the correlation for 7/8-inch tubing are as follows:

Bobbin Volts (V)	Burst Pressure (ksi)
[]

Bobbin
Volts
(V)

Burst
Pressure
(ksi)





An higher order regression analysis of this data has been performed providing an equation for the mean curve using a second order polynomial equation. The equation for burst pressure (BP) as a function of volts (v) obtained is:

$$[\quad \quad \quad]^{\circ}$$

The coefficient of correlation for this regression fit is 0.90 and the error of the BP estimate is 1.014. A -95% prediction interval is established using the expression:

$$[\quad \quad \quad]^{\circ}$$

where,

$$[\quad \quad \quad]^{\circ}$$

Since the burst tests were performed at room temperature conditions, the -95% prediction interval curve was factored by the LTL-to-room temperature strength property ratio of 0.857, the Lower Tolerance Limit (LTL) strength properties of Alloy 600, 7/8 x 0.050 inch, mill annealed tubing at 650 °F divided by the strength at room temperature of the tubing material tested. The strength properties utilized are the sum

of the yield and ultimate strengths. The LTL strength (yield + ultimate) of the tubing material at 650°F is 126.0 ksi and the room temperature strength of the material tested is 147.0 ksi giving a ratio of 0.857.

The curves and data points are plotted in Figure 9-2. The -95% prediction interval, LTL, is used to establish the voltage corresponding to the burst pressure capability required of three times normal operation pressure differential (4380 psi), 3ΔP. The 3ΔP voltage at the -95% prediction interval is []^a. The voltage corresponding to steam line break (SLB) pressure (2650 psi) is []^a. It should be noted that the burst capabilities developed conservatively assume no potential benefit of interaction with the tube support plate.

A probabilistic analysis (similar to Section 11.4) has been performed using Monte Carlo techniques for the Farley 2 voltage distributions (Figure 5-5) including indication voltages < 3.6 volts at BOC to determine the probability of burst during a postulated SLB. Accounting for EC uncertainty and growth rate, 100,000 cases were run to obtain the lowest burst pressure for each BOC distribution. A probability of 3×10^{-5} of having burst capability < 2650 psi (SLB) was determined for the worst SG. This compares favorably with NUREG-0844, which indicates a required probability of less than 2.5×10^{-2} for single tube ruptures.

9.7 Correlation of Leak Rates with Bobbin Coil Voltage

The distribution of crack morphologies at a given indication voltage gives rise to an associated distribution of burst pressures. Since some of these crack morphologies may involve through wall cracking, a distribution of leak rates will be also associated with each indication voltage level. The expected total leak rate from a given population of eddy current indication voltages can be determined in a statistical fashion from a knowledge of the distribution of possible leak rates at each voltage level. This is accomplished by performing a probabilistic analysis for SLB conditions as described in section 11.4. The basis for the probabilistic analysis is the correlation of leak rates at SLB conditions to bobbin coil voltage. The correlation is established utilizing linear regression analysis of the logarithms of the corresponding leak rates and voltages thereby establishing a leakage rate model of the form:

$$[]^a$$

where,

$$[]^a$$

Prediction intervals for leakage rate at a given voltage are then established to statistically define the range of potential leakage rates.

where,

[

] ⁹

Figure 9-3 is a plot of the SLB leakage rate correlation versus bobbin voltage showing the data, the linear regression fit and $\pm 95\%$ confidence prediction intervals. Since a fit using logarithms of voltage and leakage is required to obtain the correlation, the zero leakers were input with a leak rate of 0.0001 liters per hour, three orders of magnitude below the lowest measured leak rate. This is judged to be sufficiently low to maximize the slope of the curve providing conservative predictions of leakage rate in the higher voltage range. In addition, only the zero leakage tubes with 90% or greater through wall penetration are included, again, maximizing the slope of the curve. Plant L data taken at room temperature were added to the data base since CRACKFLO analysis of leakage through axial cracks indicates a very small difference between hot conditions and room temperature for the same crack lengths.

At sufficiently low voltages the degree of through wall cracking will not be sufficient to lead to measurable leakage. There will be a threshold voltage below which leakage is not expected. [From the model boiler specimen leak rate data of this section and the pulled tube test data of Section 6, the threshold voltage for leakage at a normal operating conditions is somewhere between 6.5 and 2.3 volts. Consideration of field experience argues for a threshold closer to the 6.5 volt value as discussed in Section 12. In Section 12.4 it is shown that SLB leakage below about 3.2 volts is negligible and 3.2 volts can be applied as an acceptable EOC threshold for SLB leakage analyses. This corresponds to about 2.0 volts at BOC as the leakage threshold applied for SLB analyses. The indicated thresholds to date point to the conservative nature of the regression analysis.]⁹

Section 12 discusses the application of burst pressure and leakage correlations in the formulation of plugging criteria. The subsections above show that burst pressure margins can be maintained by limiting maximum allowable bobbin voltage indications and that any associated leak rates with a given population of voltage indications can be appropriately determined.

9.8 Burst Testing of IGA Specimens

Three sets of laboratory IGA specimens were utilized for NDE measurements and burst testing. Two are Westinghouse samples and the third represents samples prepared by Westinghouse under EPRI sponsorship (EPRI N-5503). The two Westinghouse sets of specimens represent laboratory IGA under accelerated conditions and provide uniform wall penetration IGA over 4 to 6 inch lengths. Figure 8-12 shows bobbin coil voltage amplitudes for these specimens. The IGA specimens from Figure 8-12b with depths of

9%, 36% and 52% in non-sensitized, 7/8 inch OD tubing were burst tested to compare the voltage vs burst characteristics of IGA to that of the available data base of Figure 9-2. In addition, a 3/4 inch diameter, sensitized specimen of Figure 8-12a with 58% deep uniform IGA was burst tested. These specimens had IGA from one end of the tube to the other and hence differential bobbin coil measurements could not be made. As noted in Figure 8-12, the bobbin amplitudes were measured in the absolute mode. It is expected that for uniform step change in depth, the peak to peak differential voltages would be a factor of 1 to 2 higher than the absolute voltages which are typical of single peak amplitudes. Thus using the absolute amplitudes of the specimens appears to be a conservative representation of the differential amplitudes. However, due to the associated uncertainty on the voltage amplitudes, the burst results for these specimens are provided for information only and are not included in the voltage/burst correlation used to establish tube repair limits.

The EPRI specimens were prepared using a 50% caustic and 12% chromium oxide environment at 650°F for up to 10,000 hours. Temperatures in some cases were increased to 700°F to accelerate the corrosion rate such that 21% penetration was obtained in 35 days. Even under the accelerated laboratory conditions, the times to create IGA are very long compared to preparation of ODS/SCC specimens. Specimens in the range of 2 to 30% nearly uniform wall penetration were obtained in this program. Bobbin coil measurements of library samples were performed with the results given in Table 8.3. These samples show voltage amplitudes of about 1-2 volts where the IGA depth is expected to be < 30% deep and are NDD where depths of a few percent are expected. Deep cracks within the samples were detected with amplitudes of 4-40 volts.

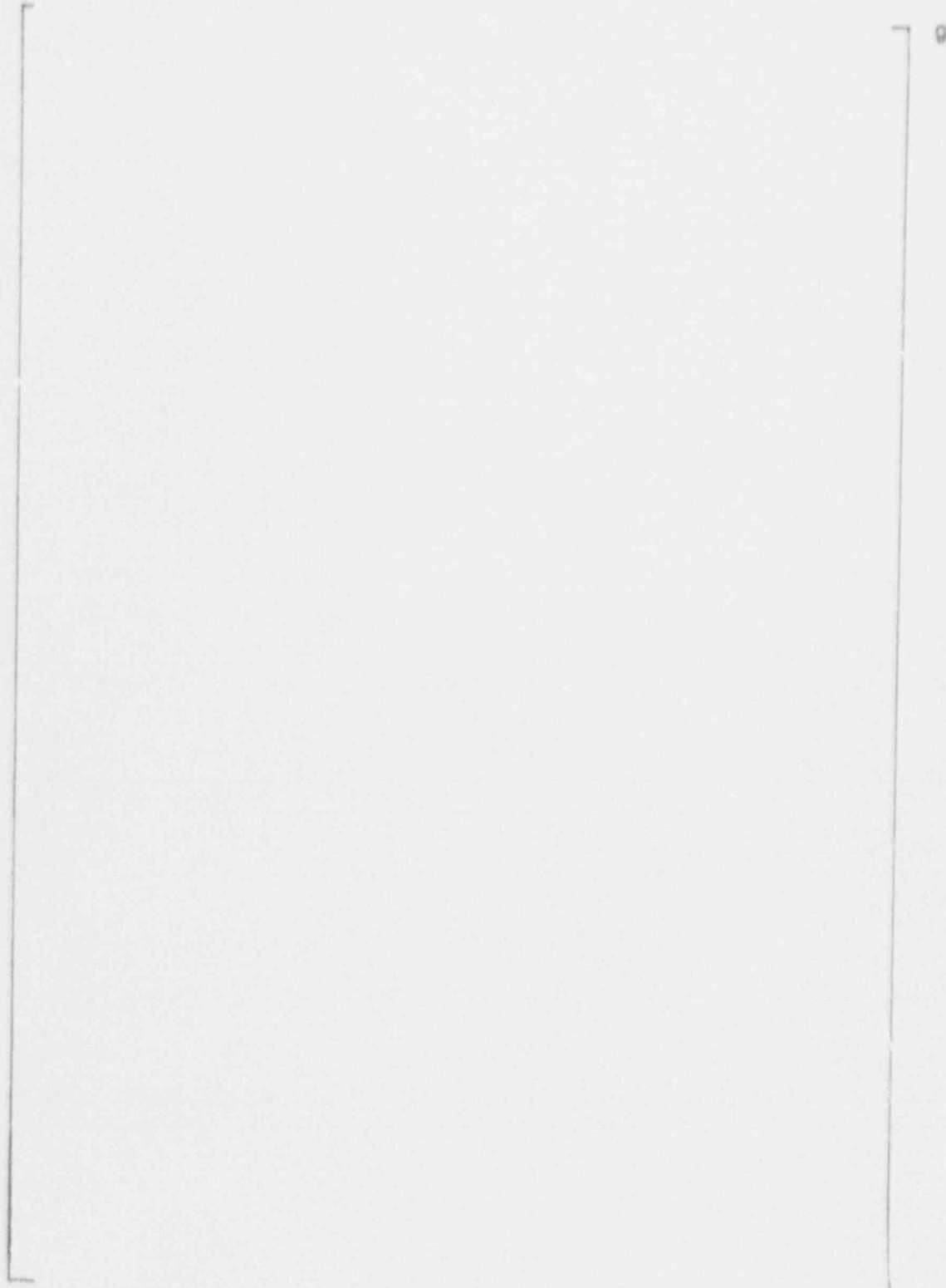
Destructive examination of Sample 1-3 found that the 40 volt crack (See Table 8.3) was 0.86 inch long and 0.42 inches through wall with no significant IGA at the crack location. Sample 1-4 was cut and the 9 volt crack was found to be 1.32 inch long and approximately 90% deep with no significant IGA. The 4 volt crack was isolated to obtain a burst specimen. NDE performed on the isolated specimen identified a 1.8 volt bobbin indication. This sample burst at 8,315 psi. Destructive examination identified a 0.38 inch crack partial through wall with 80% maximum depth. This sample had cellular IGA/SCC in a patch about 0.4" by 0.4" at the crack location. The IGA/SCC ranged in depth from 10 to 20%.

Figure 9-4 shows the IGA burst results added to the best fit and 95% confidence bands of data of Figure 9-2. From Figure 9-4, it is seen that the IGA specimen burst pressures are near or above the mean fit to the Figure 9-2 data even though the IGA specimens are degraded over 4 inches compared to the 0.75 inch TSP thickness which controls the indication lengths for the data points of Figure 9-2. These results indicate that the burst pressure correlation of Figure 9-2 represents a lower bound for uniform or general IGA tube degradation. In general, this result is expected at least for IGA depths up to about 65%. The voltage amplitude for uniform IGA is expected to be as high or higher than for a single axial IGSCC indication of the same depth due to the greater volumetric involvement of the IGA. Burst test results for uniform thinning of 0.75 inch length show higher burst pressures than for a 0.75 inch long single crack of the same depth for depths up to about 65% while cracks have higher burst pressures above ~65% depth. Thus the higher burst pressures for uniform IGA together with equal or higher voltage amplitudes would yield the Figure 9-4 results showing IGA burst pressures at or above the ODS/SCC results.

The IGA burst test results show that uniform IGA behaves effectively as uniform thinning of a tube. That is, the burst pressures of uniform IGA correlate well with results obtained for uniform wall thinning. For local IGA patches of limited circumferential involvement, the burst pressures would be expected to approach that of a crack. Thus an IGA patch combined with an SCC emanating deeper than the IGA patch, as commonly found in pulled tubes, would be expected to reflect burst properties associated with the crack alone. This is shown for the EPRI specimen 1-4 which had cellular IGA/SCC up to 20% depth. Similarly, the bobbin coil voltage responses are expected to be comparable. The results for the Plant L tube R12-C8 of Table 6.2 reflect this effect. The maximum IGA patch depth was about 30% for this tube in the region of the maximum 55% crack depth. The voltage amplitude was 1.30 volts with a burst pressure of 10,500 psi. This data point lies just above the mean fit to the data of Figure 9-2. Overall, the results for IGA and IGA/SCC types of degradation are enveloped by the burst pressure correlation of Figure 9-2.

Table 9.1

Summary of Leak and Burst Test Results



9

Figure 9-1

Bursi Test Results Versus Bobbin Coil Voltage

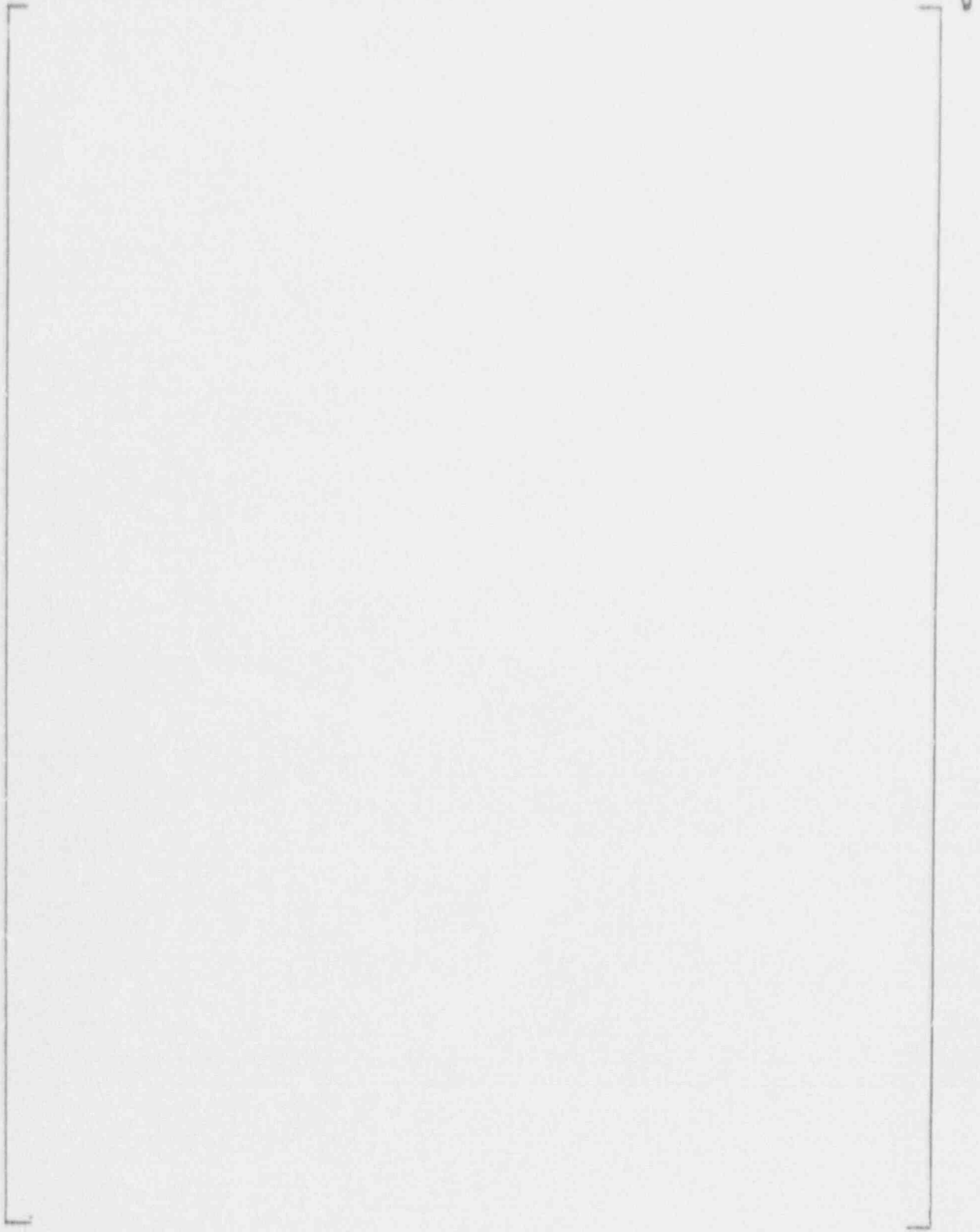


Figure 9-2

Burst Pressure Correlation With Bobbin Voltage



Figure 9-3

SLB Leak Rate Correlation With Bobbin Voltage



Figure 9-4

Burst Pressure Correlation With Bobbin Voltage -
IGA Specimen Burst Test Results Included



10.0 SPECIMEN DESTRUCTIVE EXAMINATIONS

10.1 Objective

The objective of this task is to characterize the size, shape, and morphology of the laboratory created corrosion in Alloy 600 tube specimens which have been tested for leak rate and burst pressure. The crack morphology is also to be compared generally to the corrosion morphology observed in tubes pulled from operating steam generators. A summary of the results is presented in this section. From information in Section 9, one can compare the leak rate and burst pressure data to the actual size of the crack opening and relate eddy current voltage to crack aggregate size and corrosion morphology.

10.2 Examination Methods

Examination methods included visual examinations, macrophotography, light microscopy and/or SEM (scanning electron microscopy) examinations, SEM fractography, and metallography. A total of seven model boiler test specimens and one doped steam specimen were selected for destructive examinations. Six of these were examined following leak and burst testing (all model boiler specimens), one following leak testing but without burst testing being performed, and one (the doped steam specimen) with neither leak nor burst testing being performed. The six model boiler specimens were 543-4, 525-1, 533-4, 536-1, 558-1, and 571-1. The leak and burst data for these specimens are given in Table 9.1 which also lists the eddy current (bobbin coil) test results. The two specimens without burst test data were 533-3 (model boiler specimen) and SL-FH-11 (doped steam specimen).

The specimens were initially examined visually and with a low power microscope. The burst opening and visible cracks around the circumference of the tube within the tube support plate intersection were photographed and their location in relation to the burst crack noted. The major burst crack was then opened for fractographic observations including crack surface morphology, crack length, and crack depth using SEM. One metallographic cross section containing the majority of secondary cracks within the tube support plate region was selected for each tube specimen. The location of the cracks within this metallographic cross section was noted, the cracks measured as to their depth and a crack was photographed to show the typical crack morphology. Note that the one metallographic section through each specimen will provide the secondary crack distribution at that location. Secondary cracks at other elevations would not be recorded unless the burst test happened to open the secondary cracks sufficiently for visual examination to record their location.

10.3 Results

Tube 543-4

Visual examination was performed on the secondary cracks adjacent to the main burst opening. The short secondary cracks were orientated axially, inclined, and in some instances circumferentially.

Fractography of the burst crack showed that the environmental cracks were intergranular and typical of stress corrosion cracking (SCC). The macrocrack was 0.52 inch long and did not penetrate the ID wall although it was at many locations up to 98% through wall. The macrocrack was composed of at least six microcracks, all of which had joined together by intergranular SCC. A metallographic cross section through many of the secondary cracks observed on the circumference within the tube support plate intersection revealed five cracks with depths ranging from 45 to 98% through wall. A sketch of the location of these cracks relative to the burst fracture is shown in Figure 10-1. A typical crack morphology of a secondary crack is shown in this figure. A summary of the observed OD origin secondary cracks and morphology of the main crack are shown in Figure 10-2.

Tube 525-1

The burst crack was relatively short (0.16 inch long) but deep, 95% through wall. The secondary cracks were all short and axially orientated. A summary of the shape, morphology and distribution of cracks found in Tube 525-1 is presented in Figures 10-3 and 10-4.

Tube 533-4

A network of axial and circumferential secondary cracks was observed near the burst crack. The axially orientated burst crack was composed of at least five microcracks which joined to form the macrocrack. Ledges separating these microcracks showed partially ductile features (shear dimples) while the crack face was entirely intergranular. The OD origin major crack was 0.34 inch long and penetrated the ID wall for a length of 0.14 inch. The secondary crack distribution found in the metallographic cross section is shown in Figure 10-5, and a summary of the cracks observed in the support plate region of the tube is presented in Figure 10-6.

Tube 536-1

Many secondary cracks were observed clustered around the burst opening. Other larger secondary cracks were found at a location 95 and 145 degrees from the burst opening. The main burst crack and the largest of the secondary cracks located at 95 degrees were opened and examined by SEM. In both cases the OD origin cracking morphology was intergranular and typical of SCC. The burst crack was 0.4 inch long and 90% through wall. The macrocrack consisted of at least five microcracks which joined to form the macrocrack. The largest secondary crack was 0.3 inch long and 95% through wall. It was also formed by several microcracks. A metallographic cross section through the remaining piece of the tube at the support plate intersection showed two intergranular OD origin stress corrosion cracks (Figure 10-7). The largest of these two cracks was 40% through wall. A summary of crack observations on Tube 536-1 is shown in Figure 10-8.

Tube 558-1

The major burst crack originated from a cluster of smaller secondary cracks which joined in an irregular pattern to form the macrocrack. Visual examinations of the remaining tube support plate region indicated that no other secondary cracks existed.

The opened fracture face of the burst crack revealed more clearly the irregular cracking pattern of this major crack caused by the joining of the widely separated microcracks. The macrocrack showed at least five major ledges which separated six microcracks. The features of the OD origin macrocrack were intergranular. The crack was 0.4 inch long and it penetrated the ID wall for approximately 0.32 inch. A summary of the crack observations is shown in Figure 10-9. No metallography was done on this tube since so secondary cracks away from the burst opening were observed.

Tube 571-1

The burst crack may have consisted of one major axial crack whose length was 0.44 inch. No obvious separation of the macrocrack into different microcracks was found within the burst crack, alternatively there were hints of four microcracks which nucleated in almost identical planes. The OD origin crack penetrated the wall for a length of 0.36 inch. It was entirely intergranular and typical of stress corrosion cracking, based on examination of the fractographic details. No other secondary cracks were observed along the circumference of the tube. A metallographic cross section through the center of the support plate region found no secondary cracking. A summary of crack observations is shown in Figure 10-10.

Tube 523-3

The collars of this model boiler specimen were removed. Several small axial cracks were observed at the support intersections. One was located under the Teflon collar and a few were located under the top steel collar. The crack within the Teflon collar intersection was opened and examined by SEM. The macrocrack face exhibited intergranular features with some ductile tearing on a ligament separating two microcracks. The macrocrack was 0.27 inch long and was composed of four microcracks. The OD origin crack penetrated the ID wall for a length of 0.17 inch. A metallographic cross section through the center of the Teflon collar intersection revealed numerous small intergranular stress corrosion cracks. The location (on the tube) and depth of these cracks is shown in Figure 10-11, together with one micrograph. A metallographic cross section through some of the cracks within the steel collar intersection revealed the crack distribution shown in Figure 10-12. The characterization of the through wall crack found under the Teflon collar and the crack distribution around the tube within the Teflon support is shown in Figure 10-13.

Tube SL-FH-11

Doped steam specimen SL-FH-11 developed a large number of OD origin cracks. The largest agglomeration of these cracks was opened for fractographic examinations and cross sectional cuts made above (A) and below (B) the opened section of the macrocrack for metallographic examinations. The opened section of the tube showed an intergranular macrocrack, 0.37 inch long. The ligaments separating the 4 microcracks of the macrocrack had only intergranular features. The crack penetrated the ID wall for a length of 0.23 inch. The cross sectional cuts made through planes A and B produced the crack distributions and typical crack morphologies depicted in Figures 10-14 and 10-15. A summary of the crack observations made at this location is shown in Figure 10-16.

A longer but more shallow crack network was found on the opposite side of the tube from the crack described above. The crack network was opened and examined fractographically. The macrocrack was 0.46 inch long and was composed of at least three microcracks. It penetrated the ID wall locally for a length of only 0.03 inch. The crack morphology was intergranular with some ductile features at ligament locations. A summary sketch of the crack profile and character is shown in Figure 10-17.

10.3.1 Additional Results

Results for specimens completed since WCAP-12871 was originally issued are presented below.

Tube 528-2

A grouping of many small, OD origin, axial cracks, interconnected by ligaments and by circumferential extensions, formed the curved major burst opening shown in Figure 10-18. Other small axial cracks with circumferential involvement were observed in other areas of the simulated tube support plate crevice region. Fractographic observations of the major burst opening revealed that the corrosion crack consisted of at least six microcracks with intergranular ligaments. These ligaments often ran $1/4$ the circumferential direction. The combined length of the microcracks that formed the macrocrack was 0.67 inch. Through wall cracking extended for 0.50 inch. The morphology of the macrocrack was IGSCC with no to negligible IGA components, as shown in Figure 10-19. Circumferential, intergranular extension of the axial crack can be observed in this figure. Transverse metallography through the center of the crevice region revealed the crack distribution shown by a sketch in Figure 10-20. An example of the morphology of one of the secondary OD cracks is also shown in this figure; the morphology is again that of IGSCC with negligible IGA aspects. A summary of the major crack shape and corrosion morphology and the distribution of OD cracks observed within the crevice region of tube 528-2 is shown in Figure 10-21.

Tube 532-1

A large number of long and short, OD origin, axial cracks were observed in the simulated tube support plate crevice region of tube 532-1, many of which were through wall. Figures 10-22 and 10-23 show photographs of the burst tube and convey the extensive cracking around the tube within the crevice region. A grouping of small axial cracks combined to form two of the burst crack openings which were examined in some detail. The longer of the two was 0.70 inch long and was formed from five microcracks interconnected with ledges having intergranular features. The length through wall was 0.52 inch. The crack morphology was IGSCC. Cracks seen on a metallographic cross section through the center of the crevice are depicted in a sketch in Figure 10-24 together with two micrographs showing typical crack morphologies of secondary cracks. A summary of the major burst crack and its morphology together with a distribution of cracks seen the crevice region is shown in Figure 10-25.

Tube 532-2

OD origin cracking within the simulated tube support plate crevice region of tube 532-2 can be seen in the post-burst test photographs of the tube in Figures 10-26 and

10-27. Many of the axial cracks were through wall. A group of many through wall microcracks formed the weakest area in the tube where the main burst fracture occurred. Fractography of this macrocrack showed at least six microcracks combined to form the main burst fracture. The total length of the macrocrack was 0.75 inch and it was through wall for 0.58 inch. While most ligament features were intergranular, occasional areas had ductile features, indicating that the ligaments had not completely interconnected by intergranular corrosion. The corrosion crack morphology of the main burst crack was that of IGSCC. A metallographic cross section through the center of the crevice region revealed many cracks shown by a sketch in Figure 10-28. Typical crack morphologies are also shown in photomicrographs for two secondary cracks in this figure. A summary of the main burst crack description, morphology, and crack distribution are given in Figure 10-29.

Tube 535-1

The burst opening in tube 535-1 formed from a cluster of small axial OD origin cracks. Numerous secondary, but small, microcracks were observed around the circumference within the crevice region. Fractography of the burst crack showed that it was composed of three microcracks which joined together by intergranular corrosion to form the macrocrack. The morphology of the macrocrack was IGSCC with negligible IGA aspects. The macrocrack was relatively short (0.28 inch in length) and penetrated through wall for 0.11 inch. A metallographic cross section through the center of the crevice showed numerous secondary cracks, some of which had negligible to minor IGA aspects. The distribution of cracks from the cross section is shown in Figure 10-30 along with a photomicrograph of a secondary crack. A summary of the of burst crack description and of the overall crack distribution is given in Figure 10-31.

Tube 555-3

The most degraded area in tube 555-3 was confined to one location within the crevice region, the location where the burst opening occurred. The crack distribution was complex in this region, with numerous parallel axial cracks with short circumferential branches. All cracks were of OD origin. Fractography showed the major macrocrack to be composed of two or three microcracks which joined in an irregular pattern to form the burst crack. The connecting ledges showed ductile fracture features, while the individual microcracks had intergranular features. The macrocrack was 0.75 inch long with a through wall length of 0.42 inch. Metallography of a tube cross section through the region with the highest crack density showed a morphology of IGSCC with some IGA or SCC branch characteristics near the main fracture. A photograph of the burst opening and a photomicrograph of the fracture face at the burst opening is shown in Figure 10-32. Other crack details are summarized in Figures 10-33 and 10-34.

Tube 576-2

Only one, OD origin, single axial crack was observed after burst testing of tube 576-2. Fractography showed the crack to be 0.30 inch long and it was through wall for 0.22 inch. The macrocrack appeared to be composed of a single microcrack and its morphology was that of IGSCC. A metallographic cross section through the center of the crevice region found no secondary cracks around the circumference. Crack details and crack morphology data are shown in Figures 10-35 and 10-36.

Tube 576-4

Tube 576-4 also burst with a single crack. Fractography showed the crack to be 0.60 inch long and it was through wall for 0.43 inch. The OD origin intergranular macrocrack was composed of three axial microcracks joined together by ligaments with intergranular features. Visual examination showed a few small axial cracks nearby the burst crack. The morphology of the burst crack was IGSCC with negligible IGA aspects. A metallographic section cut through the center of the crevice found only the burst crack. Crack details and crack morphology data are shown in Figures 10-37 and 10-38.

10.4 Comparison with Pulled Tube Crack Morphology and Conclusions

Section 4.0 of this report describes the crack morphology observed on tubes pulled from operating steam generators. Most of the support plate cracking was OD origin, intergranular stress corrosion cracking that was axially orientated. Most cracks had minimal IGA features in addition to the overall stress corrosion features. Even when the IGA was present in significant amounts, it usually did not dominate over the overall SCC morphology. Large macrocracks were composed of numerous short microcracks (typically < 0.1 inch long) separated by ledges or ligaments. The ledges could have either intergranular or dimple rupture features depending on whether or not the microcracks had grown together during plant operation.

It is concluded that the laboratory generated corrosion cracks described in Section 10.3 have these same basic features. The laboratory created specimens possibly had even less of a tendency to develop IGA components to the overall stress corrosion crack features.

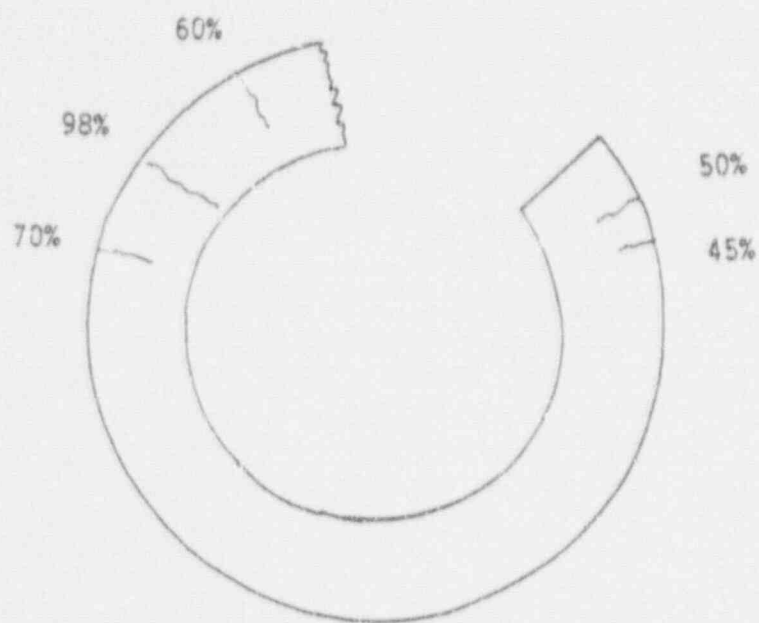
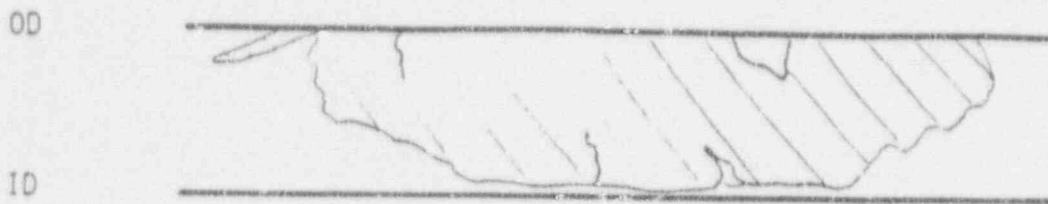


Figure 10-1 Sketch of a metallographic cross section through secondary support plate crevice cracks in Tube 543-4. A typical crack micrograph is also shown.



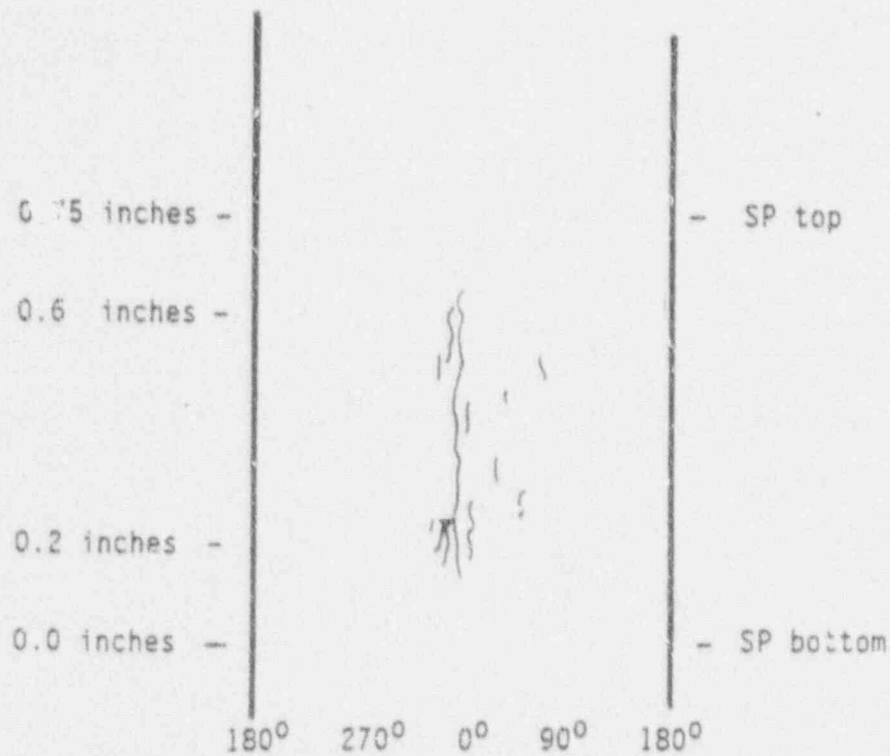
Sketch of Burst Crack

Macrocrack Length = 0.52 inches

Throughwall Length = 0 (98% throughwall)

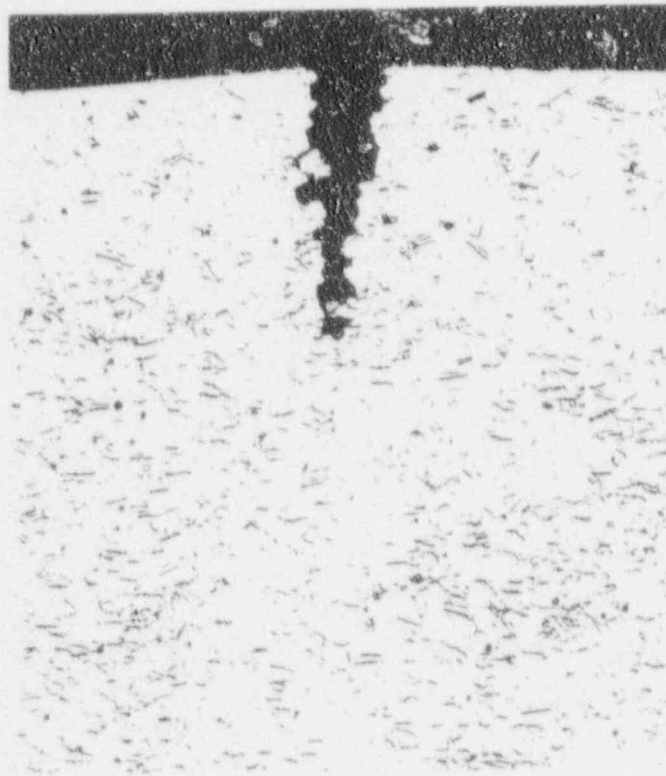
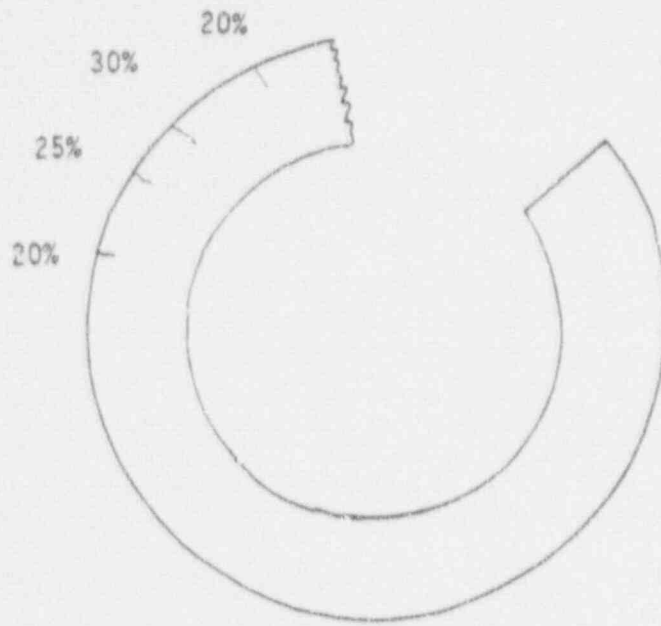
Number of Microcracks = 6 (all ligaments with intergranular features)

Morphology = IGSCC



Sketch of Crack Distribution

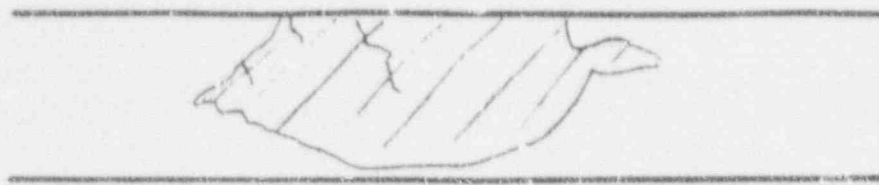
Figure 10-2 Summary of crack distribution and morphology observed on Tube 543-4.



100X

Figure 10-3 Secondary crack distribution in a metallographic cross section of Tube 525 in the support plate crevice region. A micrograph of one of the cracks is also shown.

OD



ID

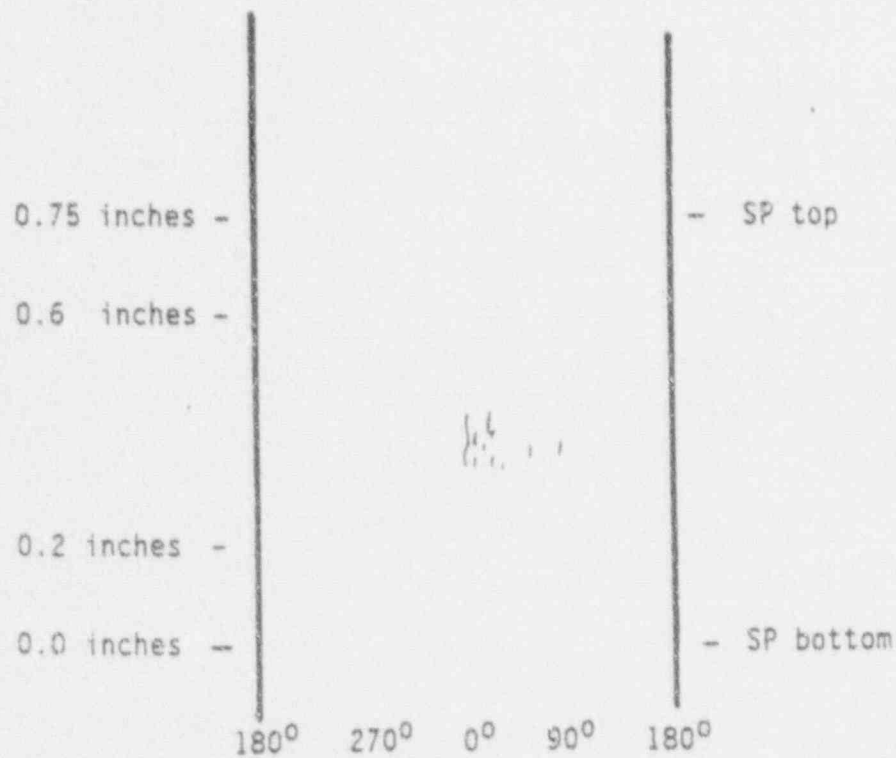
Sketch of Burst Crack

Macrocrack Length = 0.16 inches

Throughwall Length = 0 (95% throughwall)

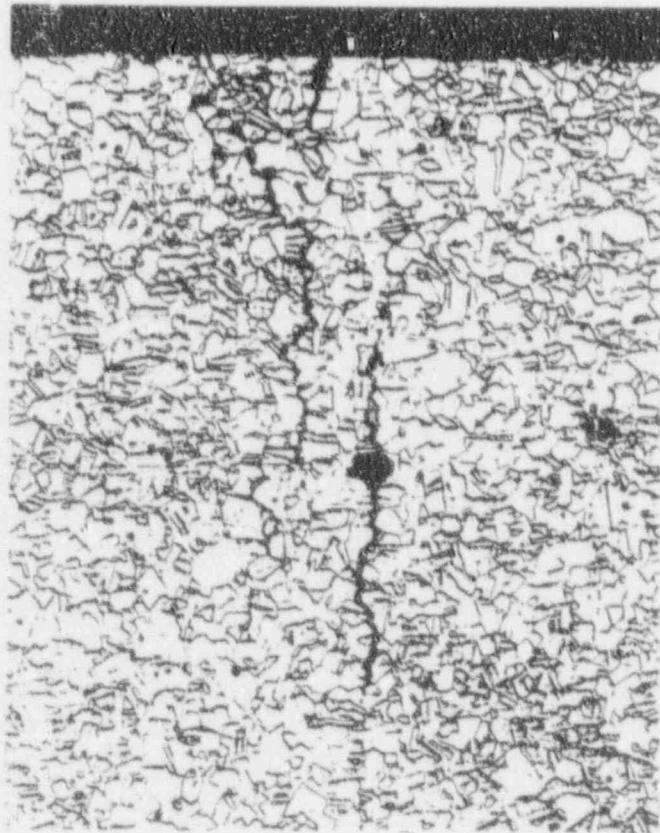
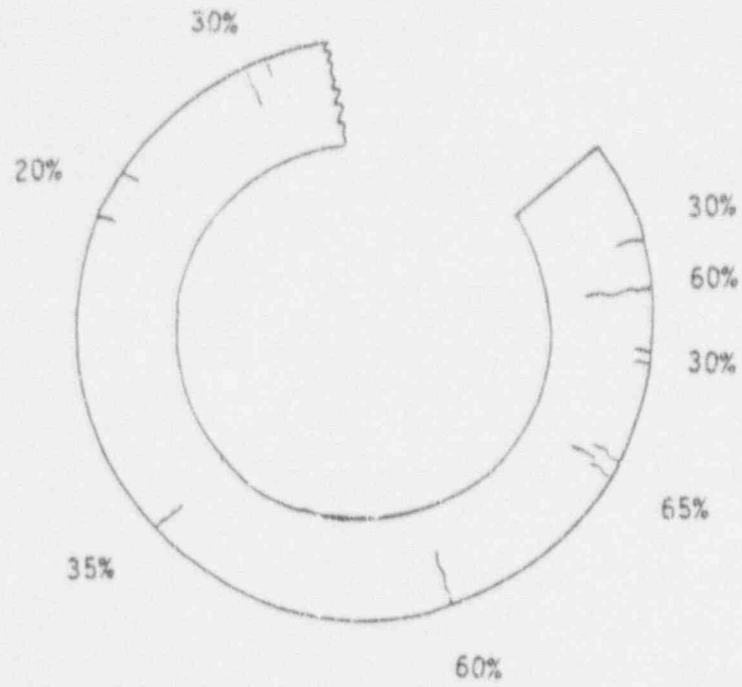
Number of Microcracks = 3 (all ligaments with intergranular features)

Morphology = IGSCC



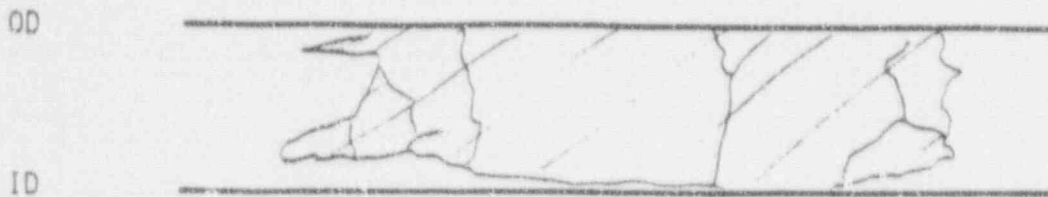
Sketch of Crack Distribution

Figure 10-4 Summary of crack distribution and morphology observed on Tube 525.



100X

Figure 10-5 Secondary crack distribution and a micrograph of one of these cracks in a metallographic cross section of Tube 533-4.



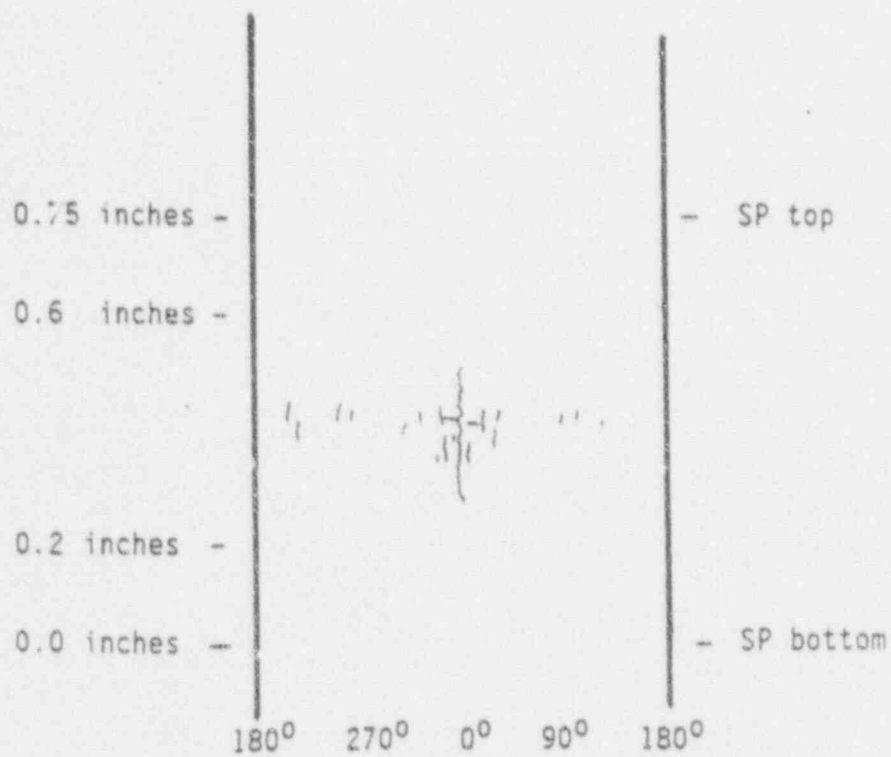
Sketch of Burst Crack

Macrocrack Length = 0.34 inches

Throughwall Length = 0.14 inches

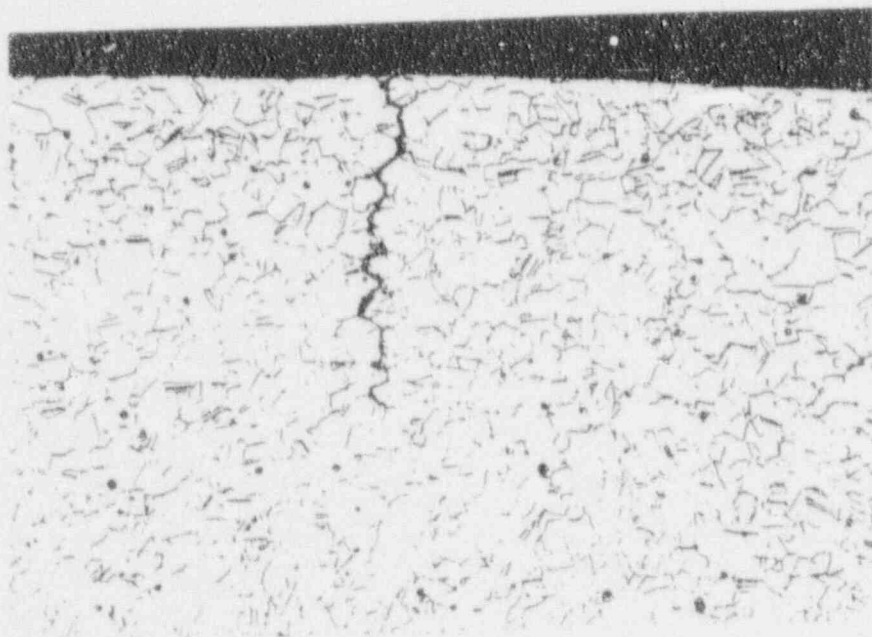
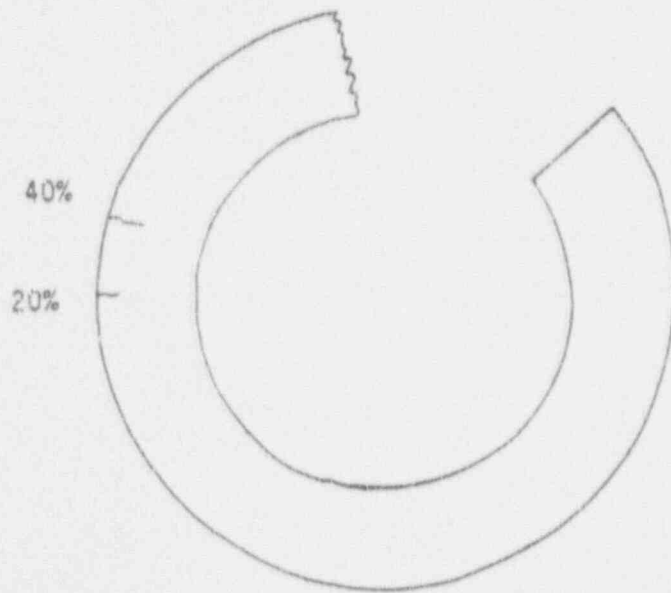
Number of Microcracks = 5 (3 ligaments have ductile features)

Morphology = IGSCC



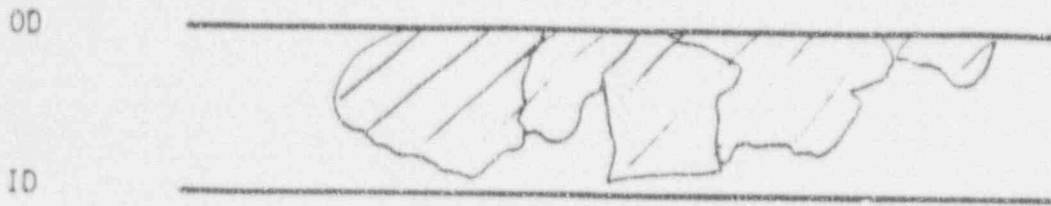
Sketch of Crack Distribution

Figure 10-6 Summary of crack distribution and morphology observed on Tube 533-4.



100X

Figure 10-7 Secondary crack distribution and a micrograph of one of these cracks in a metallographic cross section of Tube 536-1.



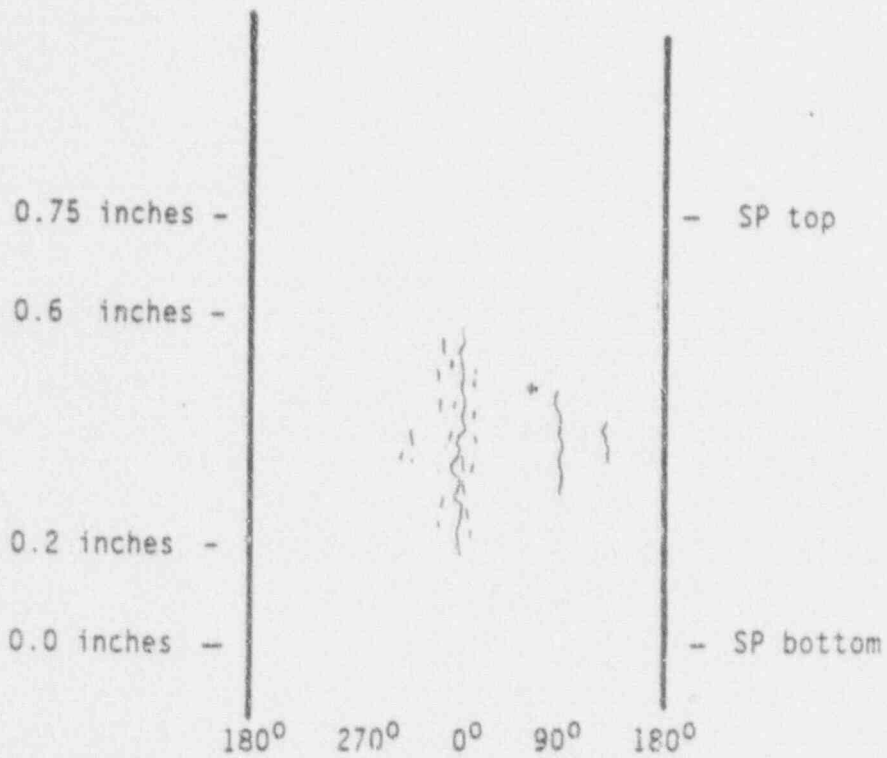
Sketch of Burst Crack

Macrocrack Length = 0.4 inches

Throughwall Length = 0 (90% throughwall)

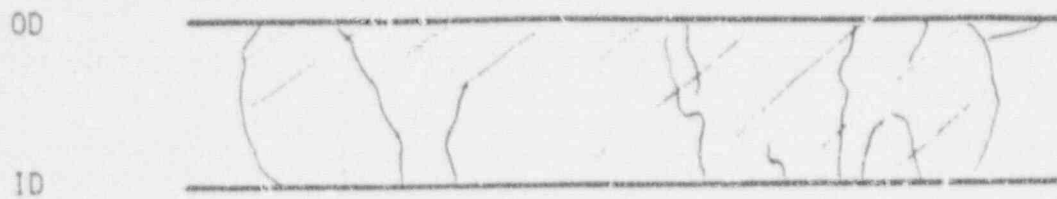
Number of Microcracks = 5 (ligaments have mostly ductile features)

Morphology = IGSCC



Sketch of Crack Distribution

Figure 10-8 Summary of crack distribution and morphology observed on Tube 536-1.



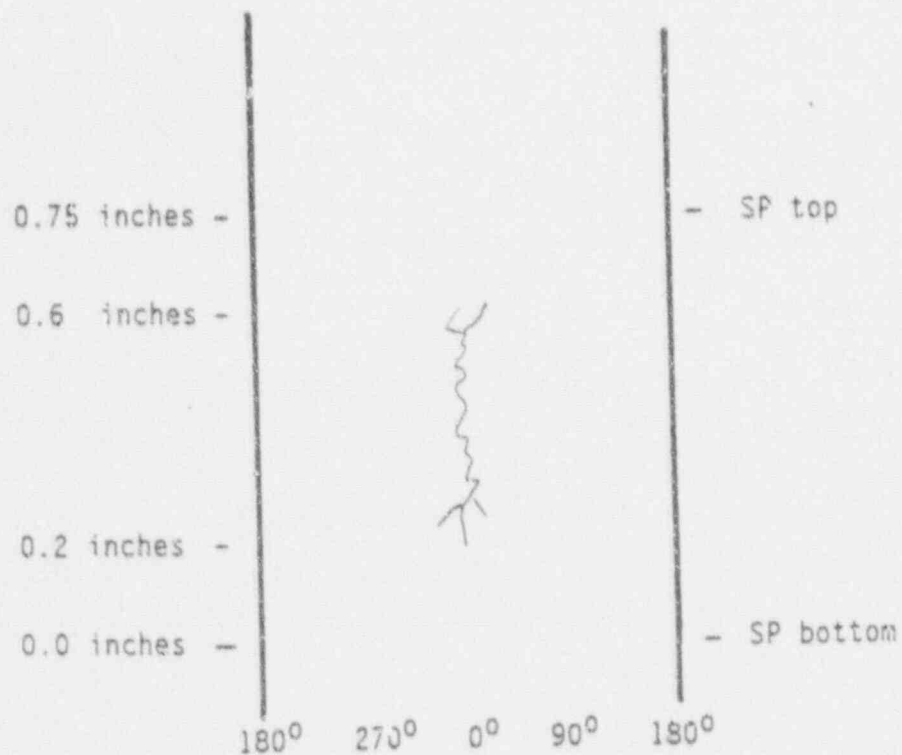
Sketch of Burst Crack

Macrocrack Length = 0.4 inches

Throughwall Length = 0.32 inches

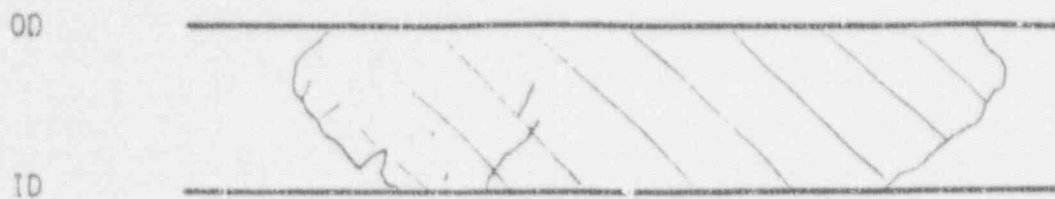
Number of Microcracks = 6 (ligaments have mostly intergranular features with some ductile features)

Morphology = IGSCC



Sketch of Crack Distribution

Figure 10-9 Summary of crack distribution and morphology observed on Tube 558-1.



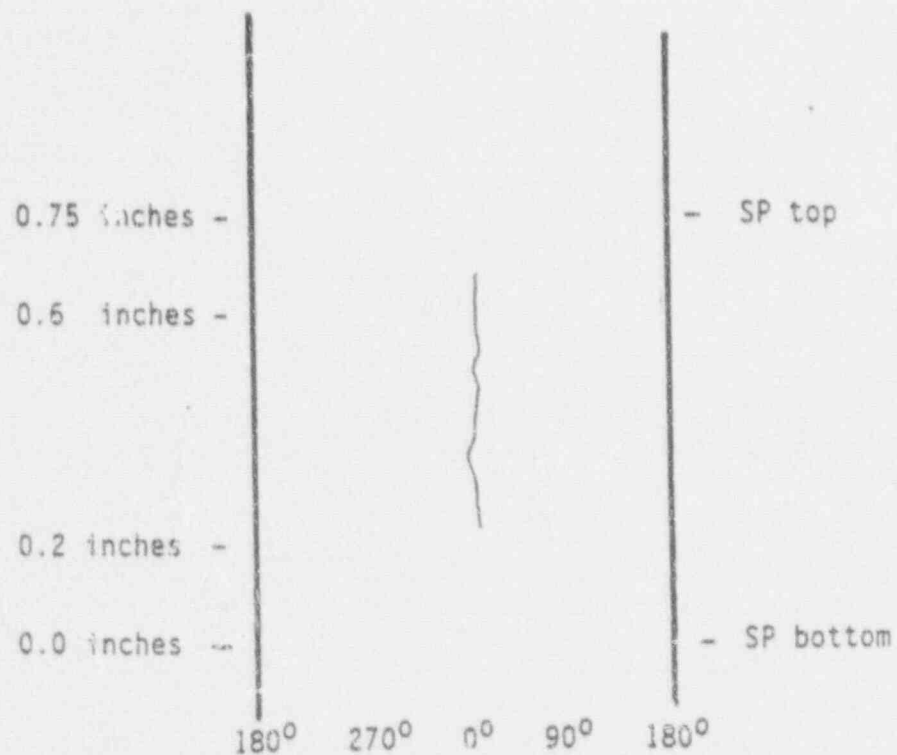
Sketch of Burst Crack

Macrocrack Length = 0.44 inches

Throughwall Length = 0.35 inches

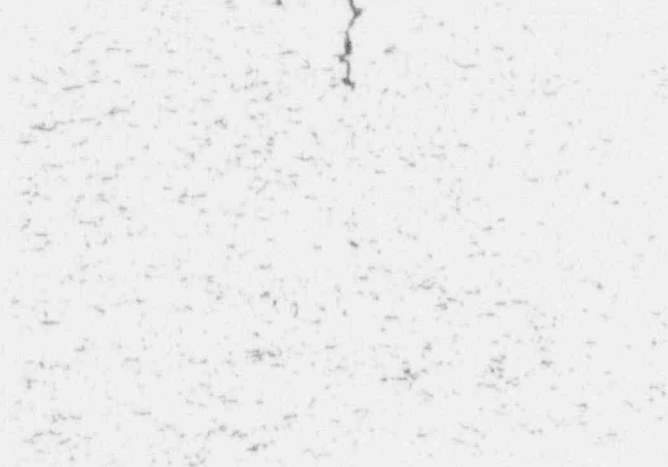
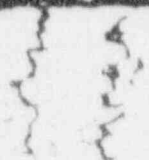
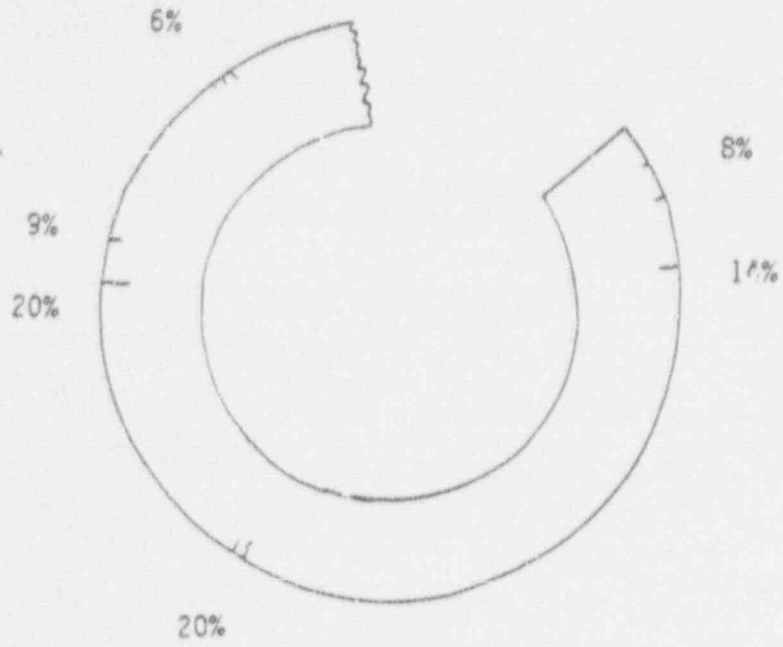
Number of Microcracks = 1 to 4

Morphology = IGSCC



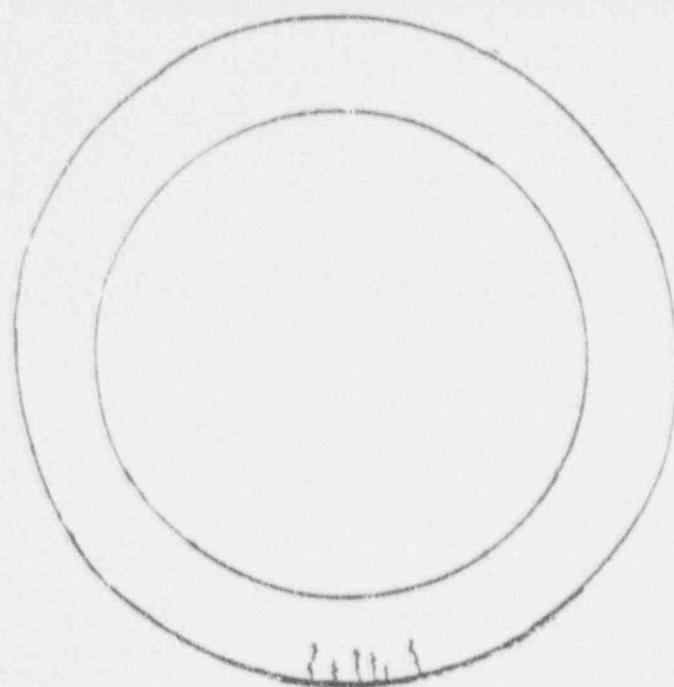
Sketch of Crack Distribution

Figure 10-10 Summary of crack distribution and morphology observed on Tube 571-1.

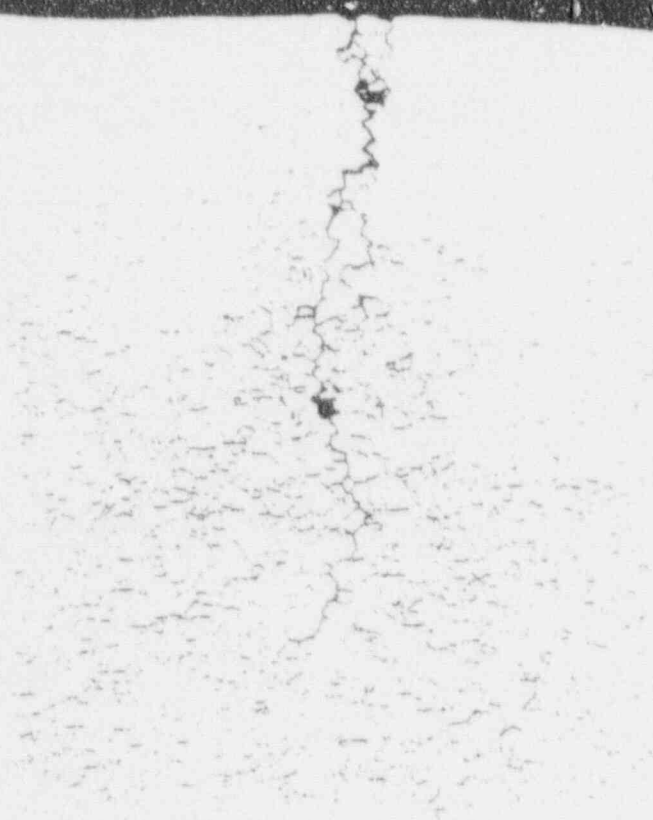


100X

Figure 10-11 Secondary crack distribution and a micrograph of one of these cracks in a metallographic cross section of Tube 533-3 within the Teflon support plate region.



51% 51% 66%



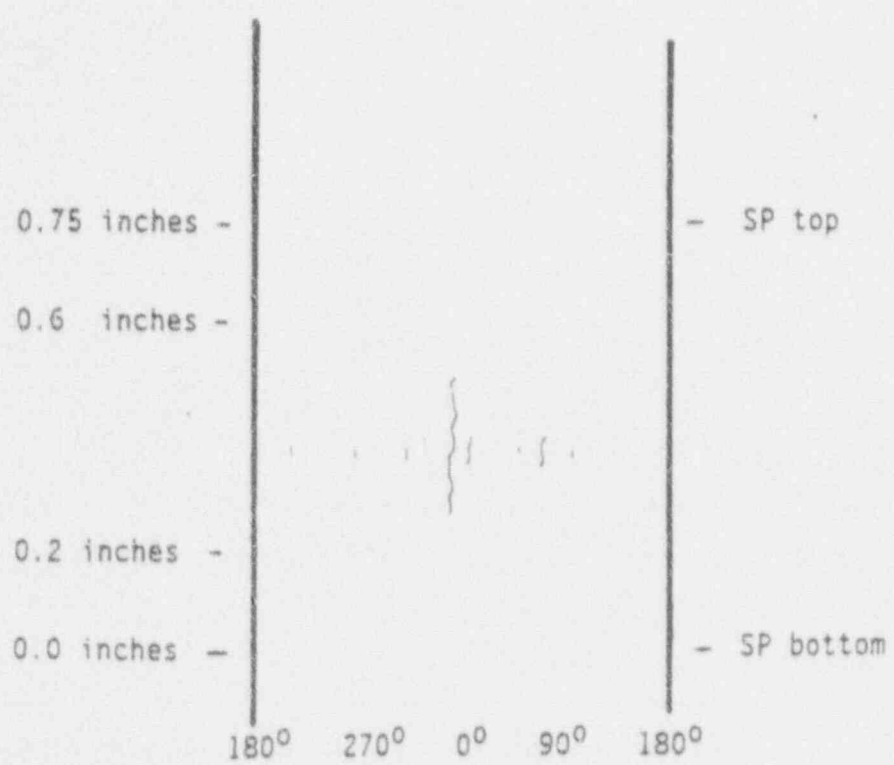
100X

Figure 10-12 Crack distribution and a micrograph of one of these cracks in a metallographic cross section of Tube 533-3 within the steel collar region.



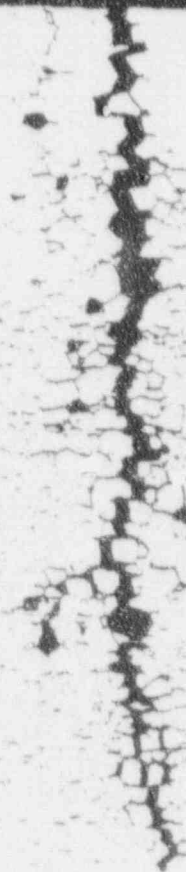
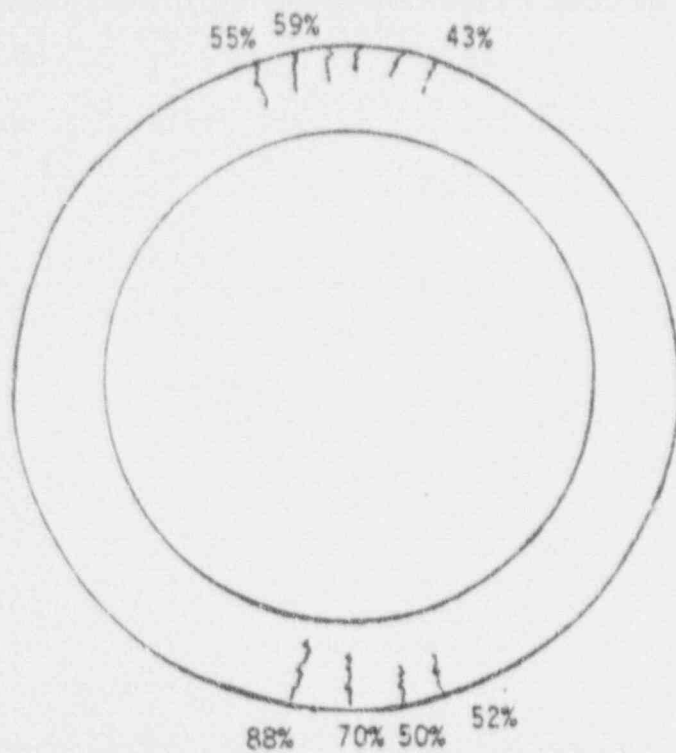
Sketch of Crack

Macrocrack Length = 0.27 inches
 Throughwall Length = 0.17 inches
 Number of Microcracks = 4 (2 ligaments with intergranular features; 1 with ductile features)
 Morphology = IGSCC



Sketch of Crack Distribution

Figure 10-13 Summary of crack distribution and morphology observed on Tube 533-3 at the Teflon intersection.



10GX

Figure 10-14 Crack distribution and a micrograph of one of these cracks in a metallographic cross section of Tube SL-FH-11 in Plane A.

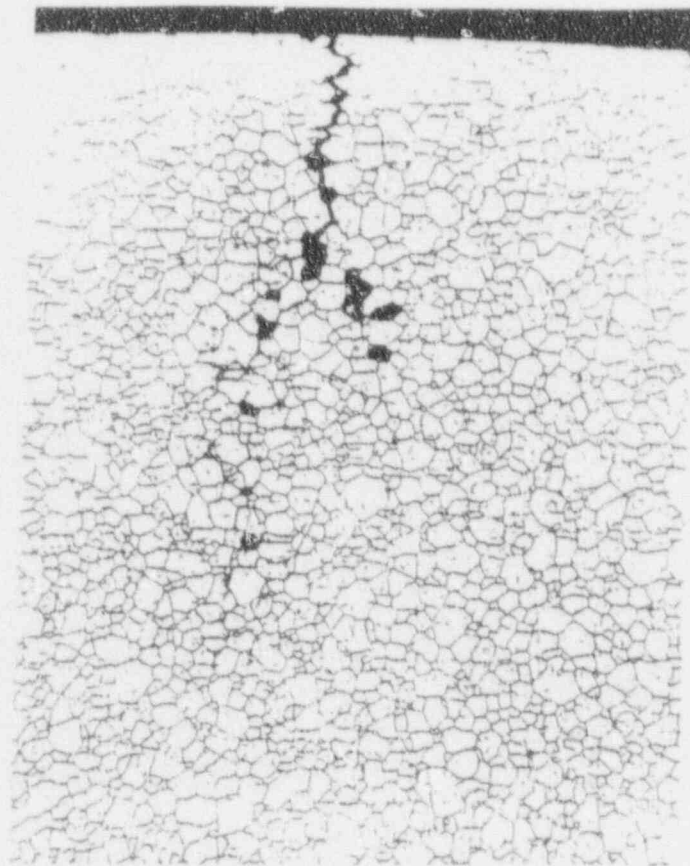
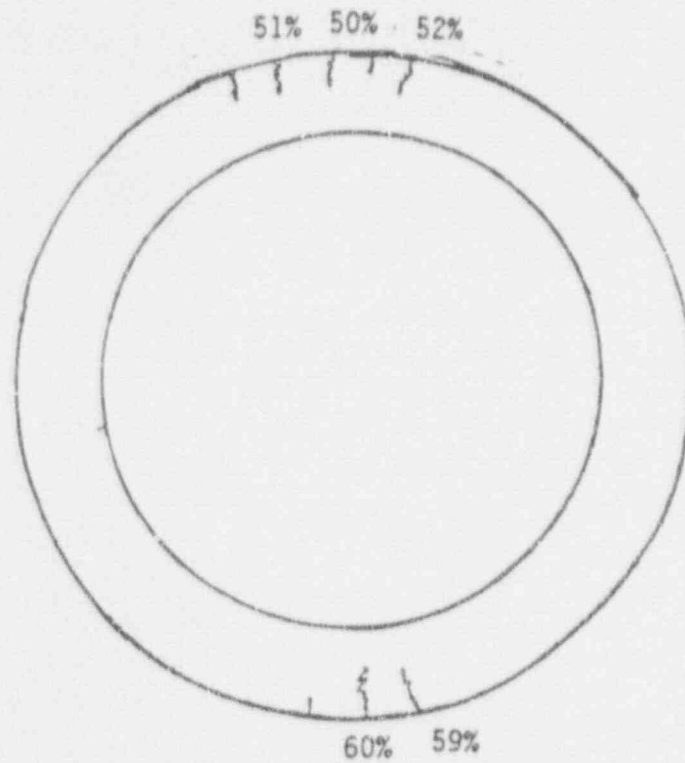


Figure 10-15 Crack distribution and a micrograph of one of these cracks in a metallographic cross section of Tube SL-FH-11 in Plane B.



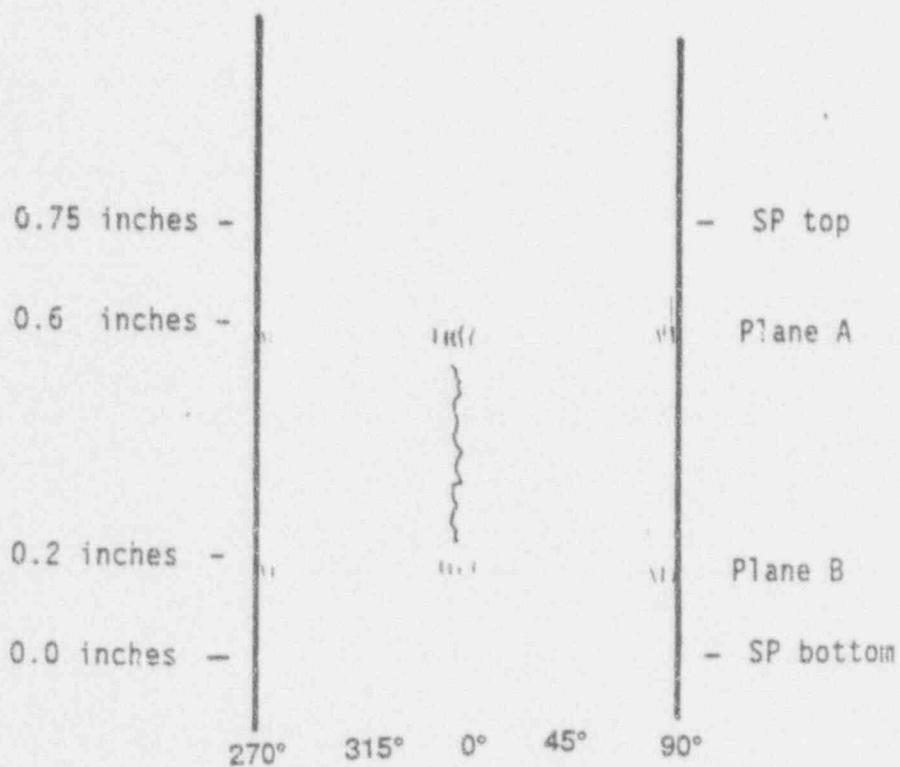
Sketch of Crack

Macrocrack Length = 0.37 inches

Throughwall Length = 0.23 inches

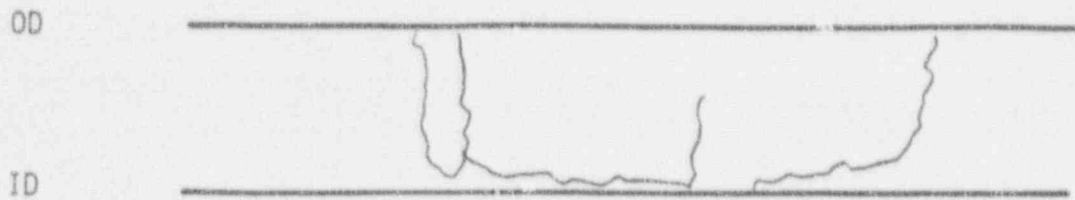
Number of Microcracks = 4 (all ligaments with intergranular features)

Morphology = IGSCC



Sketch of Crack Distribution

Figure 10-16 Summary of crack distribution and morphology observed on Tube SL-FH-11.



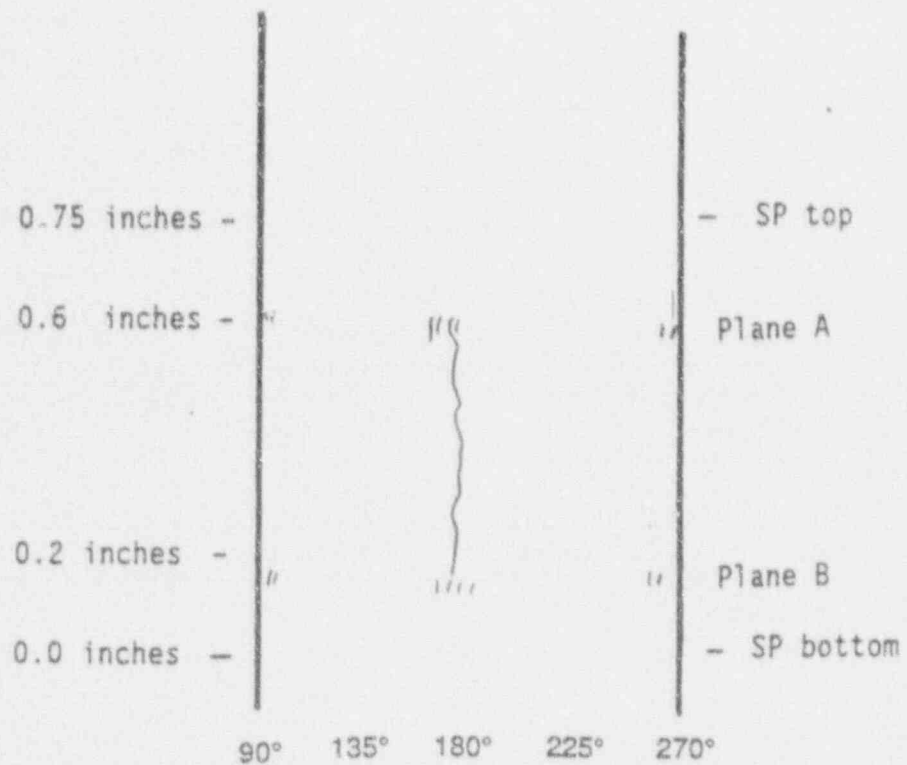
Sketch of Crack

Macrocrack Length = 0.46 inches

Throughwall Length = 0.03 inches

Number of Microcracks = 3 (ligaments have intergranular and ductile features)

Morphology = IGSCC



Sketch of Crack Distribution

Figure 10-17 Summary of crack distribution and morphology observed on second crack opened in the laboratory on Tube SL-FH-11.

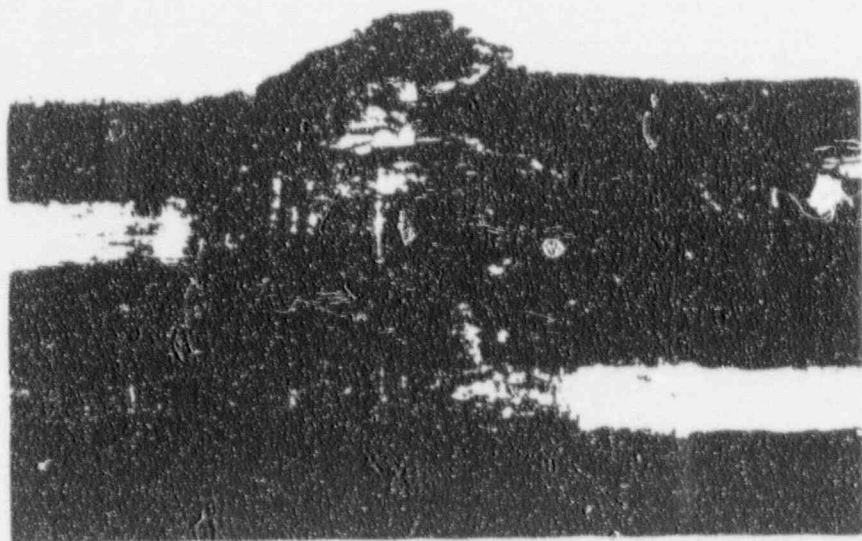
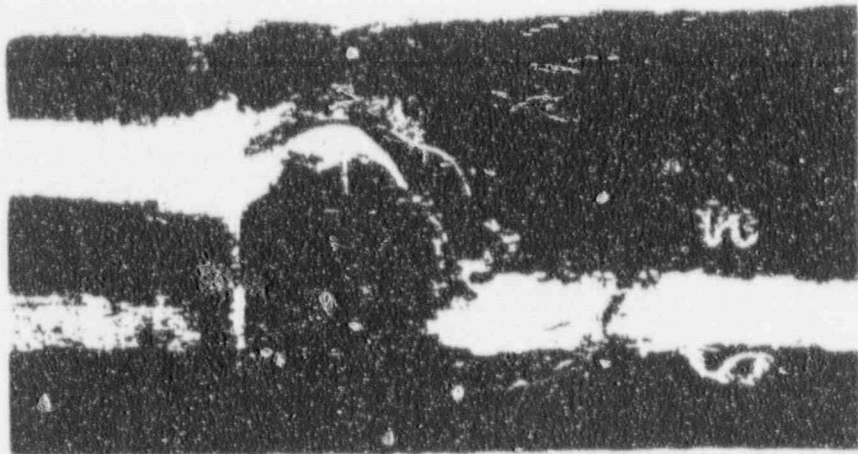
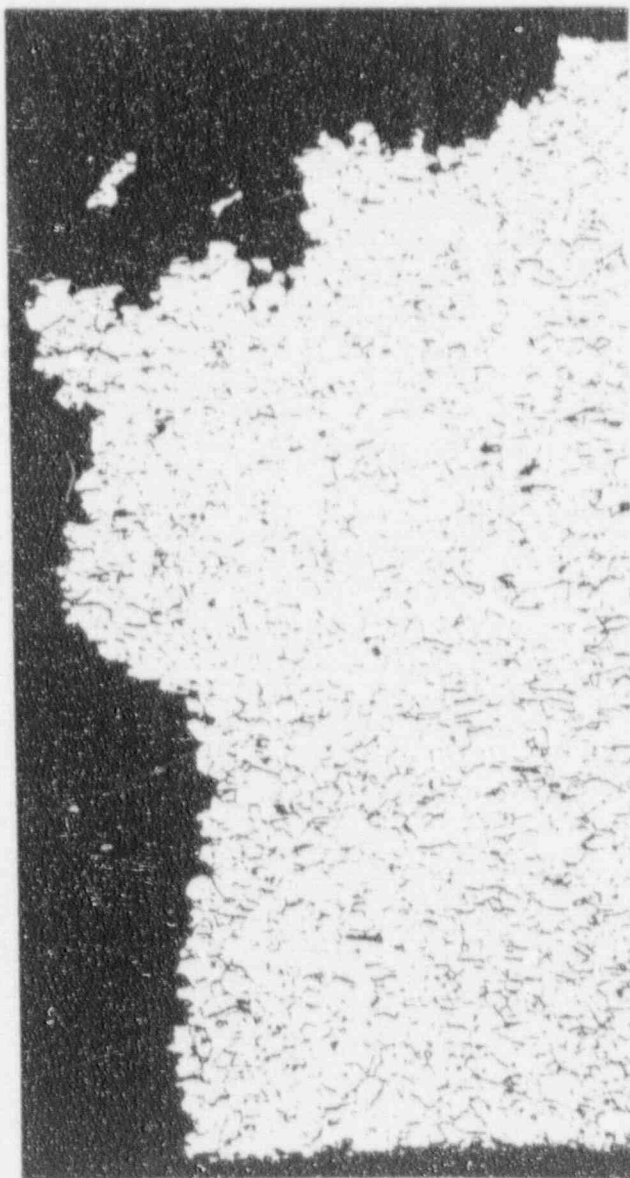


Figure 10-18 Appearance of the major burst crack opening on tube 528-2.



00

Figure 10-19 Crack morphology of the major burst opening crack (transverse section) showing it to be IGSCC with circumferential extension. Mag. 100X

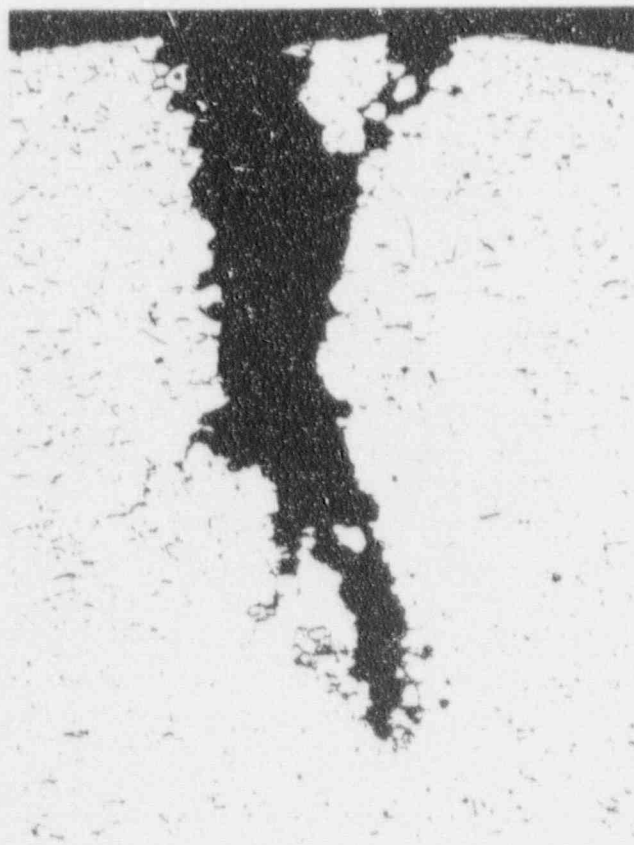
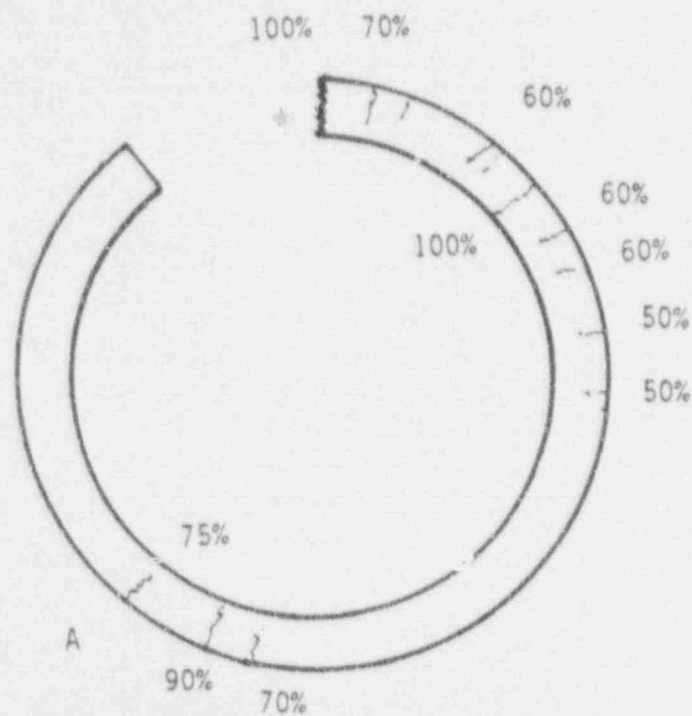
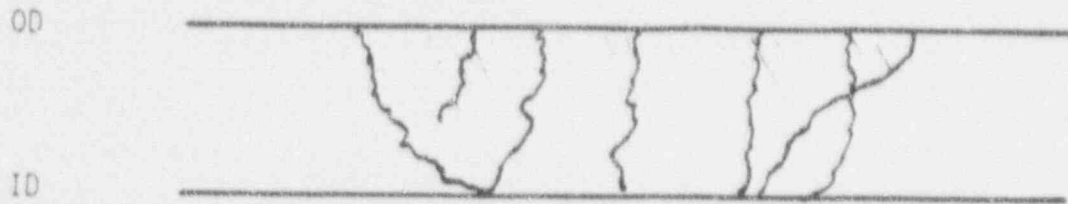


Figure 10-20 Crack distribution found by a metallographic cross section through the center of the crevice (top) and a photomicrograph of one of the secondary cracks (bottom) showing IGSCC with negligible IGA aspects. Mag. 100X



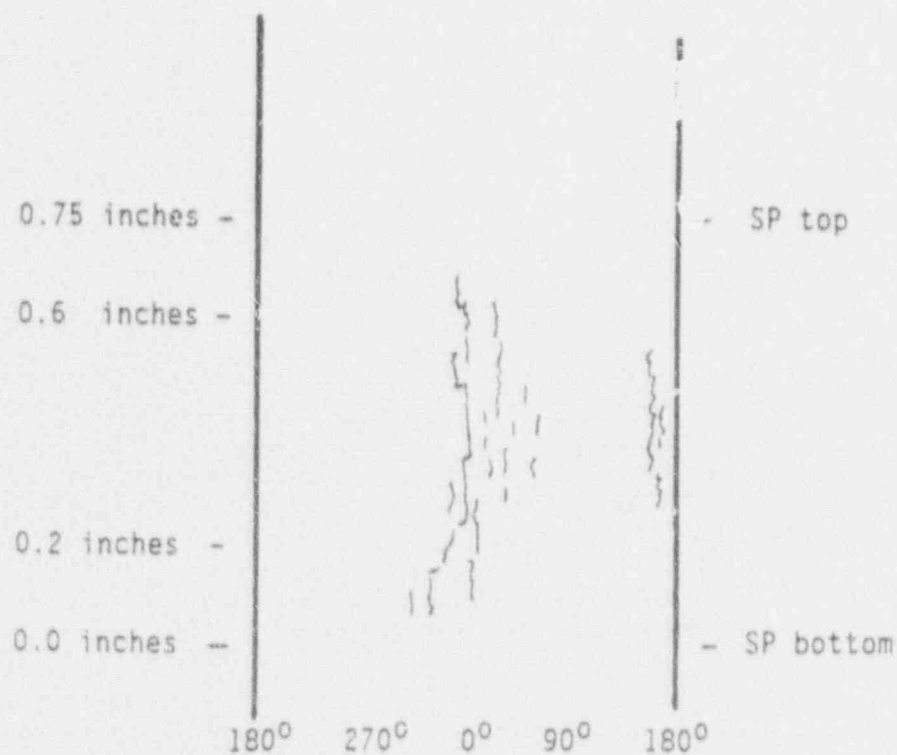
Sketch of Burst Crack

Macrocrack Length = 0.67 inch

Throughwall Length = 0.50 inch

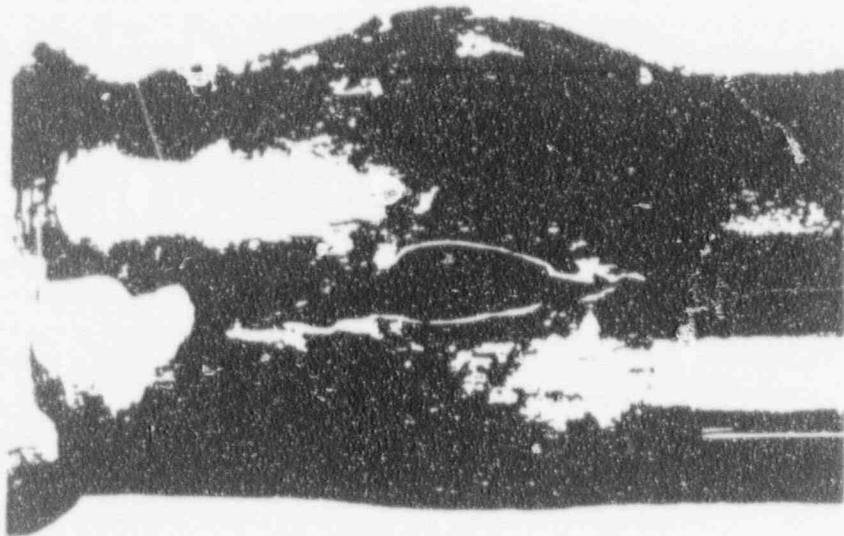
Number of Microcracks = at least 6 (ligaments have intergranular features)

Morphology = IGSCC

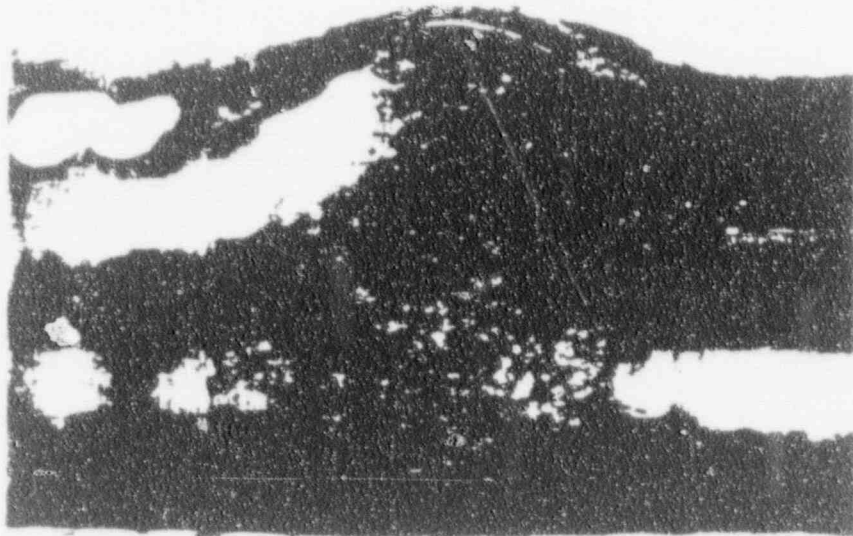


Sketch of Crack Distribution

Figure 10-21 Summary of overall crack distribution and morphology observed on tube 528-2.

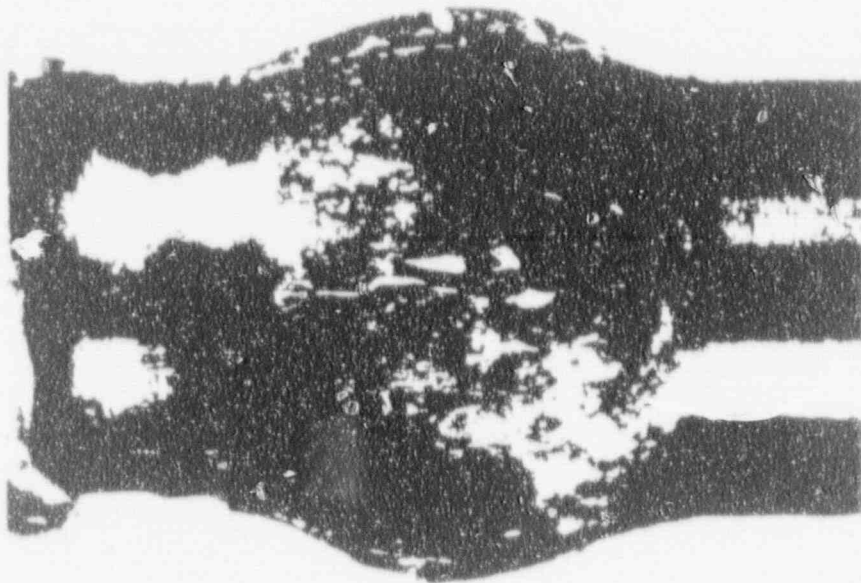


0°

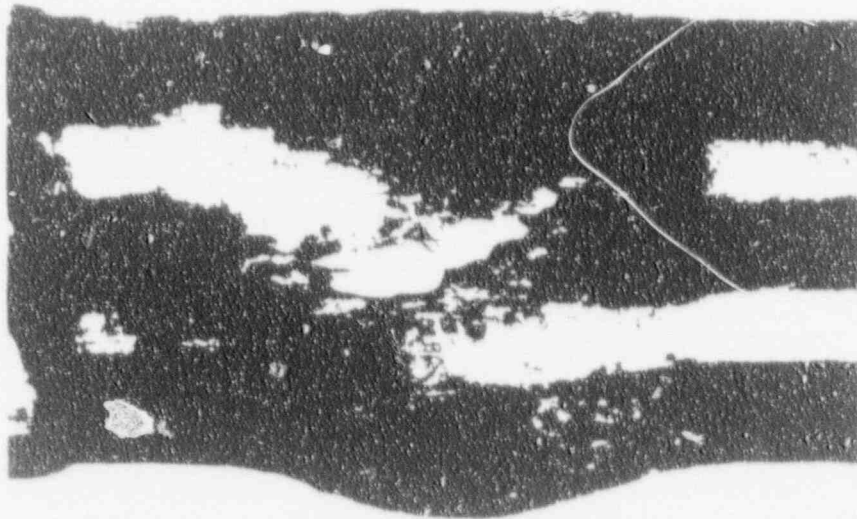


60°

Figure 10-22 Photographs of the longest burst opening and of nearby secondary cracks in tube 532-1 and 0 and 60° locations.



220°



330°

Figure 10-23 Photographs of additional secondary cracks and of a second major burst crack in the crevice region of tube 532-1.

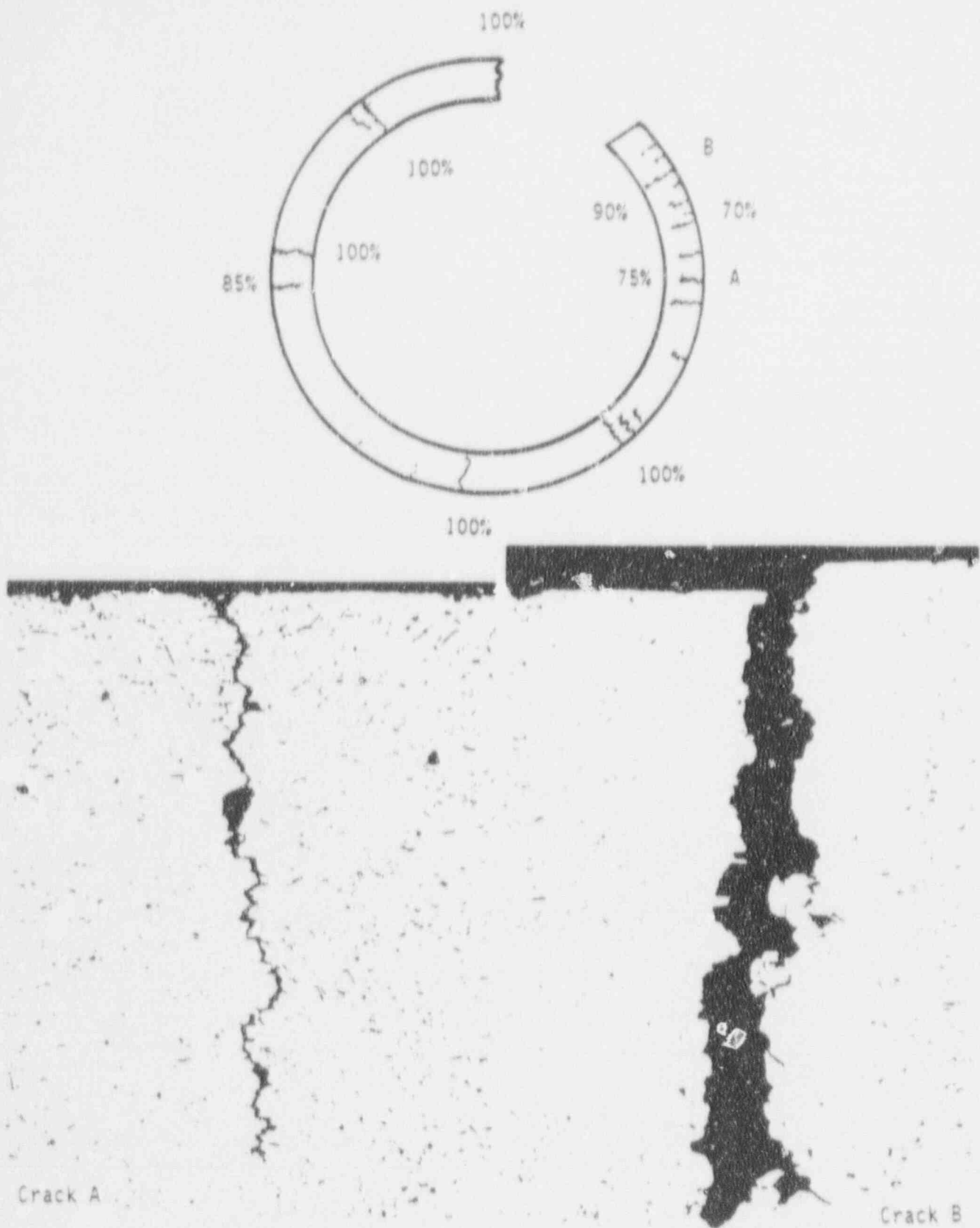
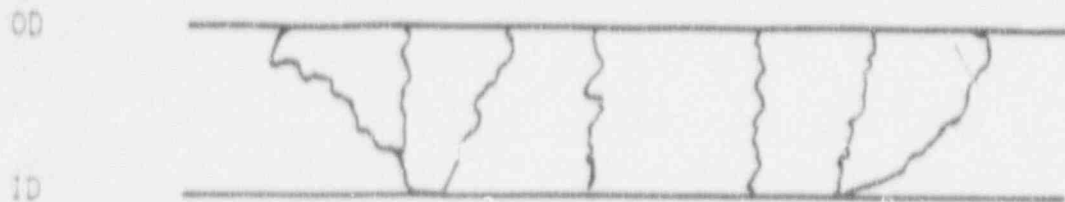


Figure 10-24 Sketch of crack distribution found by a metallographic cross section through the center of the crevice region of tube 532-1 and two photomicrographs of secondary cracks. Mag. 100X



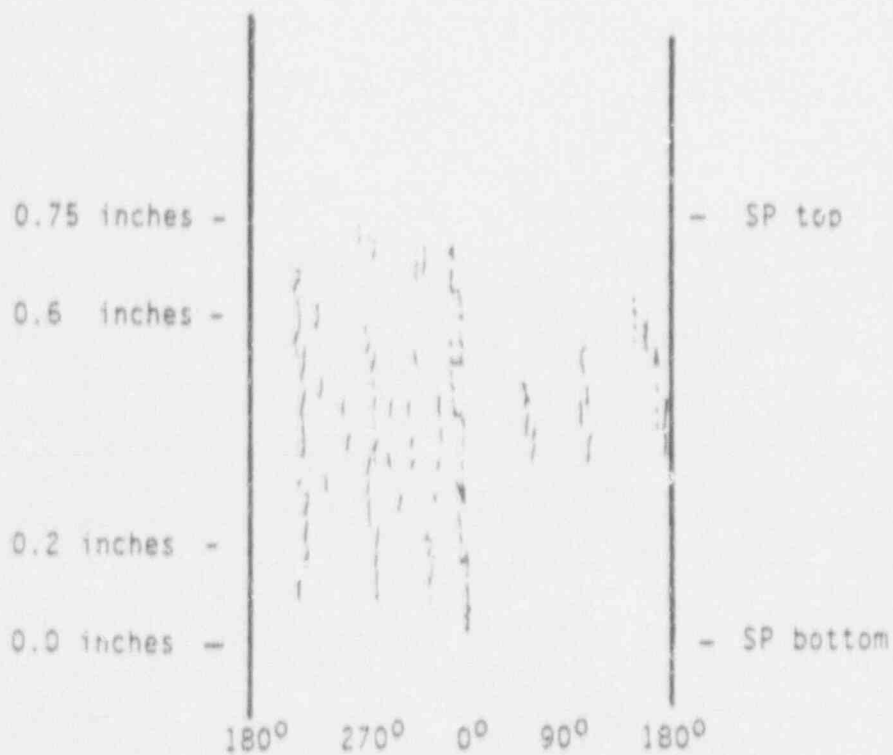
Sketch of Burst Crack

Macrocrack Length = 0.70 inch

Throughwall Length = 0.52 inch

Number of Microcracks = 5 (ligaments have intergranular features)

Morphology = IGSCC

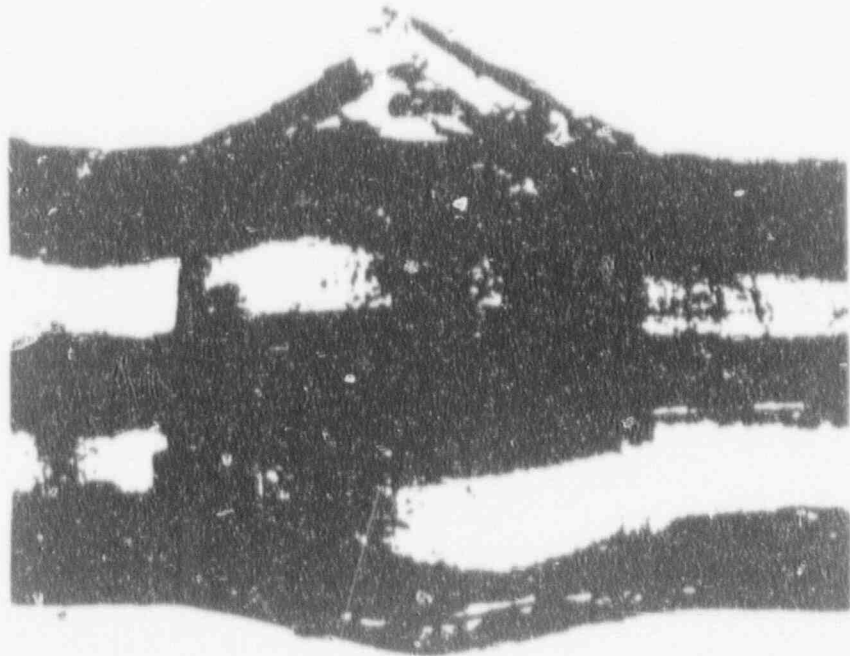


Sketch of Crack Distribution

Figure 10-25 Summary of overall crack distribution and morphology observed on tube 532-1.



0°

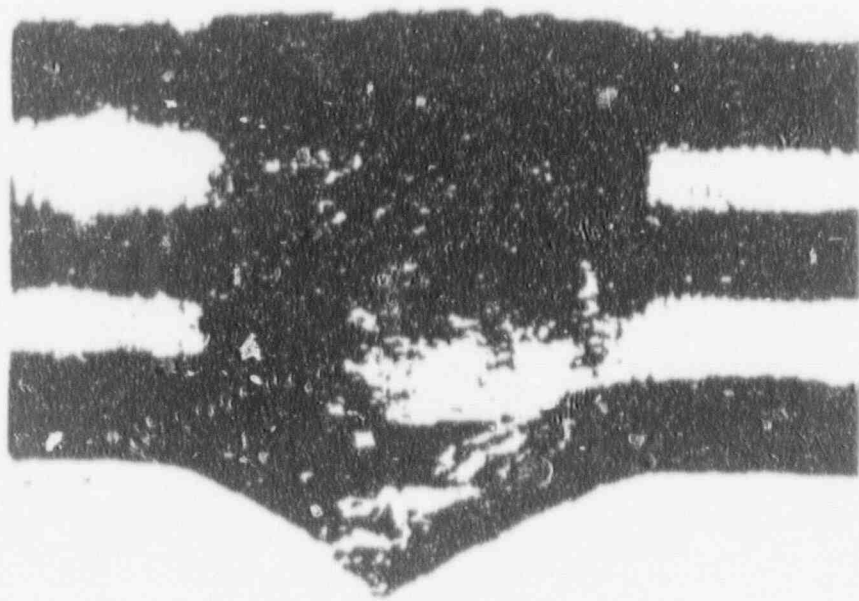


90°

Figure 10-26 Photographs of the major burst opening and secondary cracks within the crevice region of tube 532-2 at 0 and 90°.



180°



270°

Figure 10-27 Additional photographs of secondary cracks in tube 532-2 at 180 and 270°.

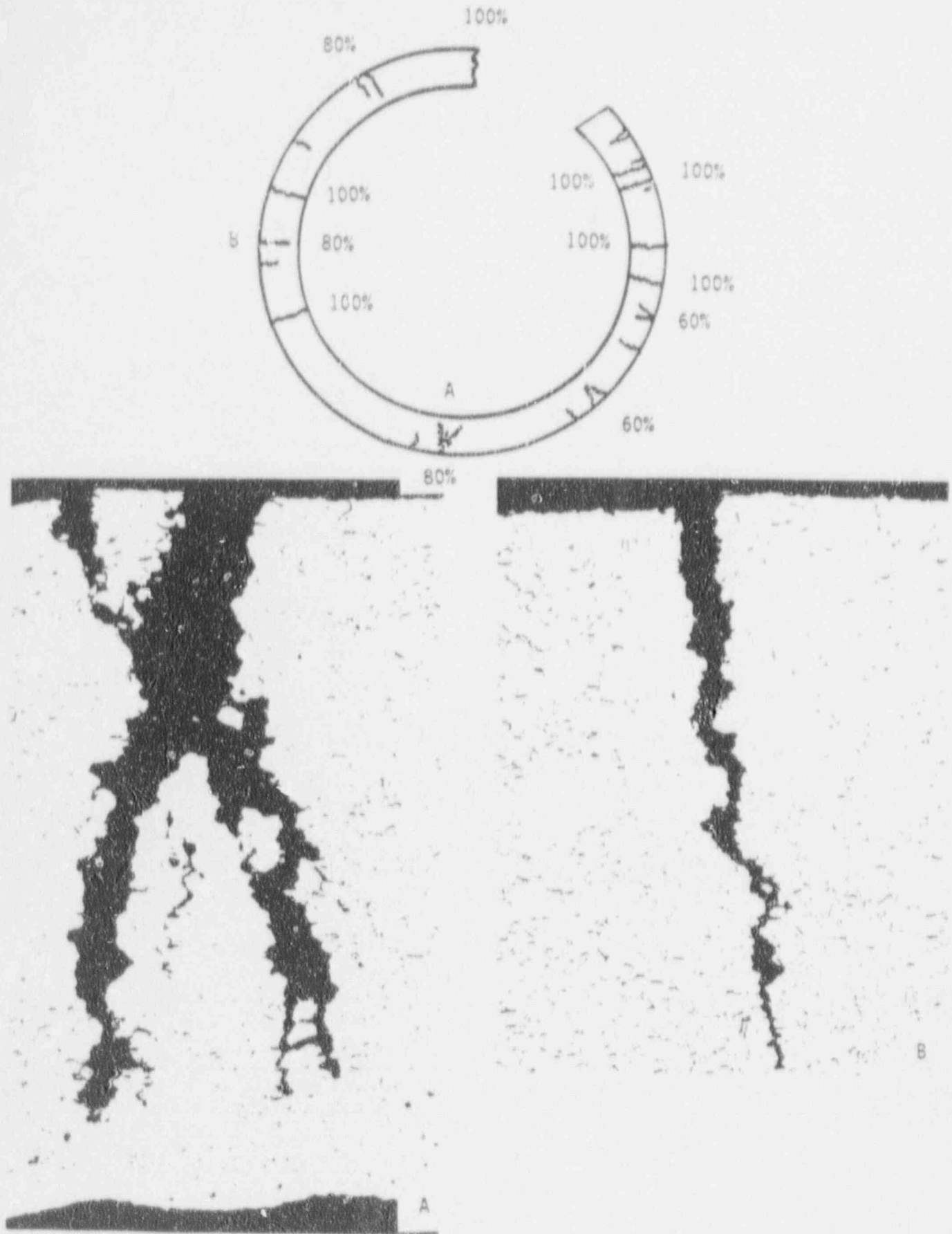


Figure 10-28 Crack distribution as revealed by a metallographic cross section through the center of the crevice of tube 532-2 and photomicrographs of secondary cracks A and B. Mag. 100X



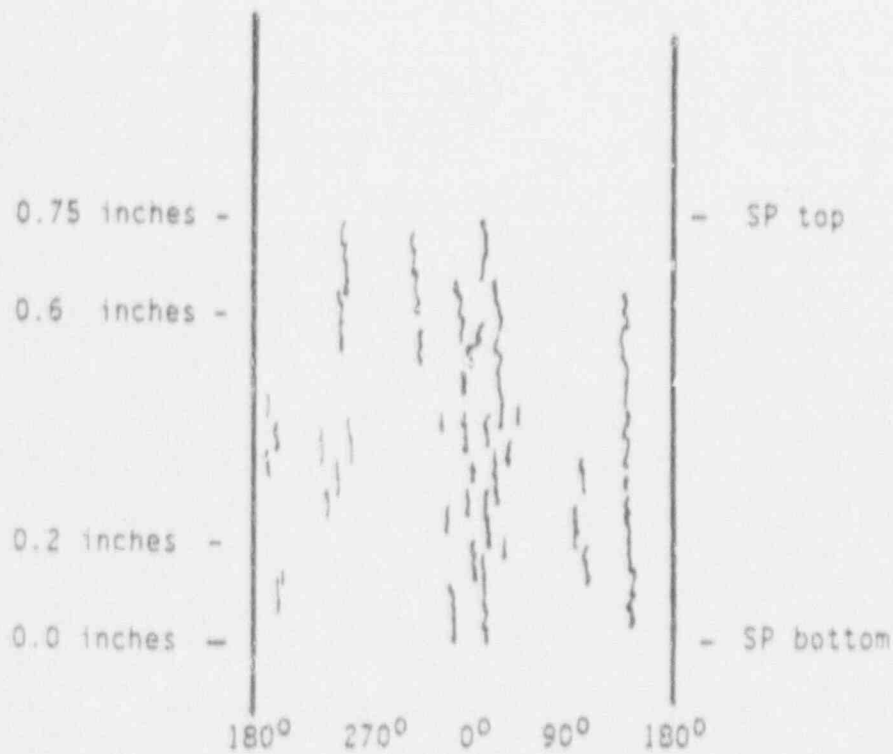
Sketch of Burst Crack

Macrocrack Length = 0.75 inch

Throughwall Length = 0.58 inch (combined through wall length)

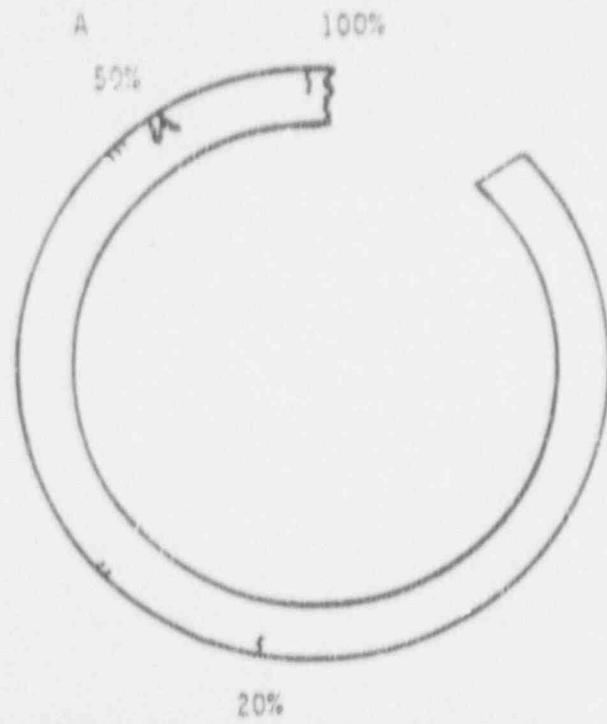
Number of Microcracks = at least 6 (ligaments have mostly intergranular features)

Morphology = IGSCC



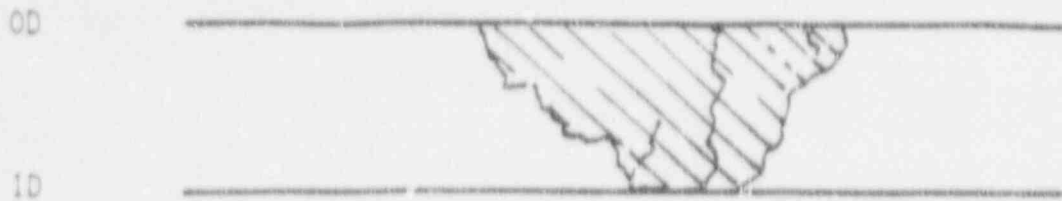
Sketch of Crack Distribution

Figure 10-29 Summary of the burst crack and overall crack distribution in the crevice region of tube 532-2.



Crack A

Figure 10-30 Sketch of crack distribution in a metallographic cross section of tube 535 through the center of the crevice and a photomicrograph of crack A. Mag. 100X



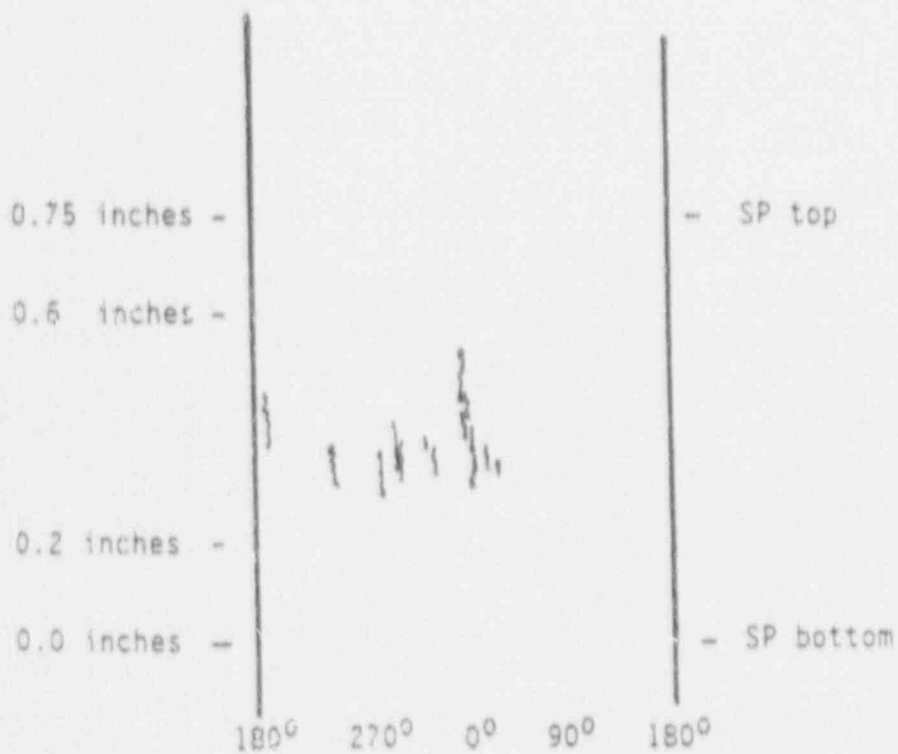
Sketch of Burst Crack

Macrocrack Length = 0.28 inch

Throughwall Length = 0.11 inch

Number of Microcracks = 3 (ligaments have intergranular features)

Morphology = IGSCC



Sketch of Crack Distribution

Figure 10-31 Summary of the burst crack and overall crack distribution in the crevice region of tube 535-1.

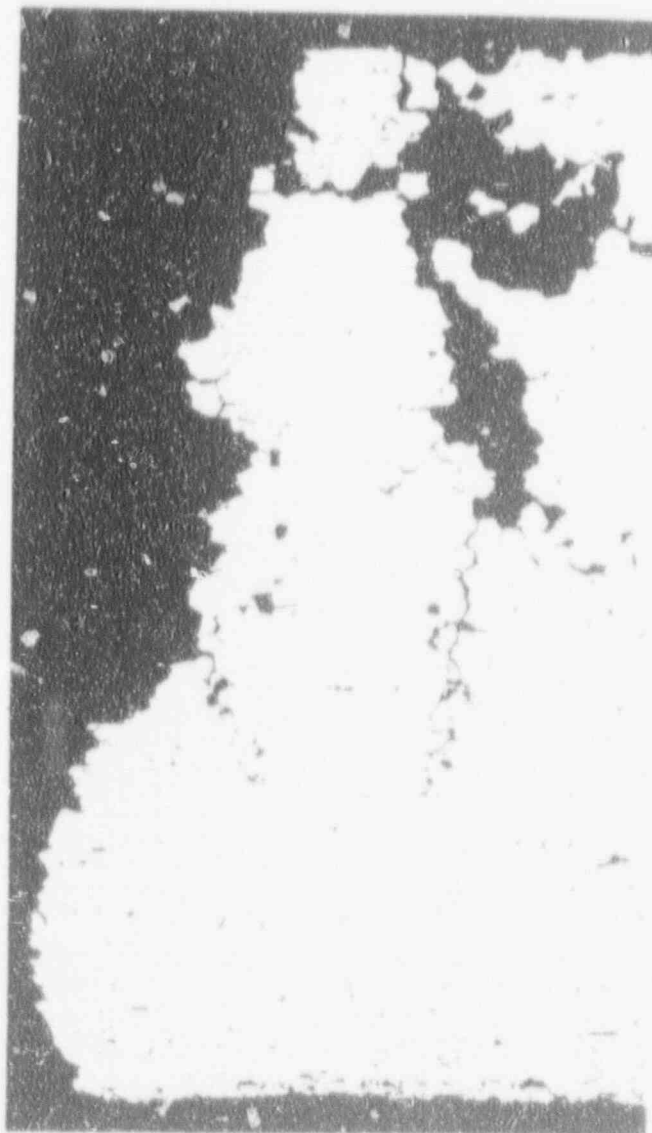
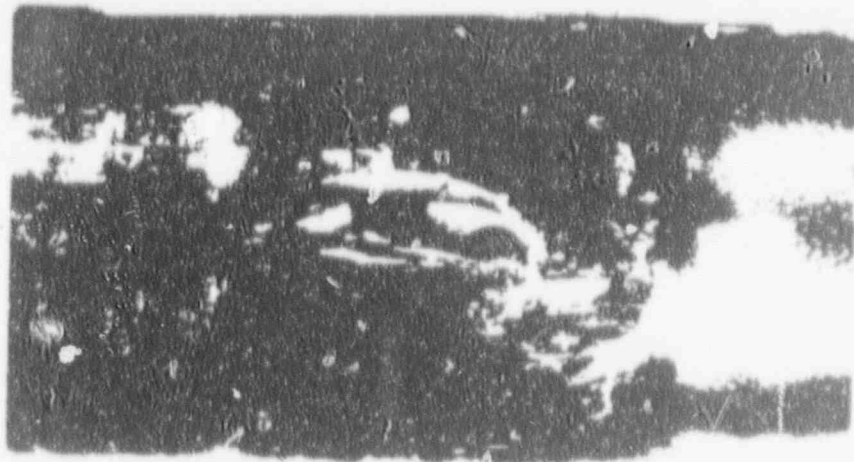


Figure 10-32 Photograph of the burst opening and a transverse photomicrograph of the burst fracture in tube 555-3. Mag. 100X

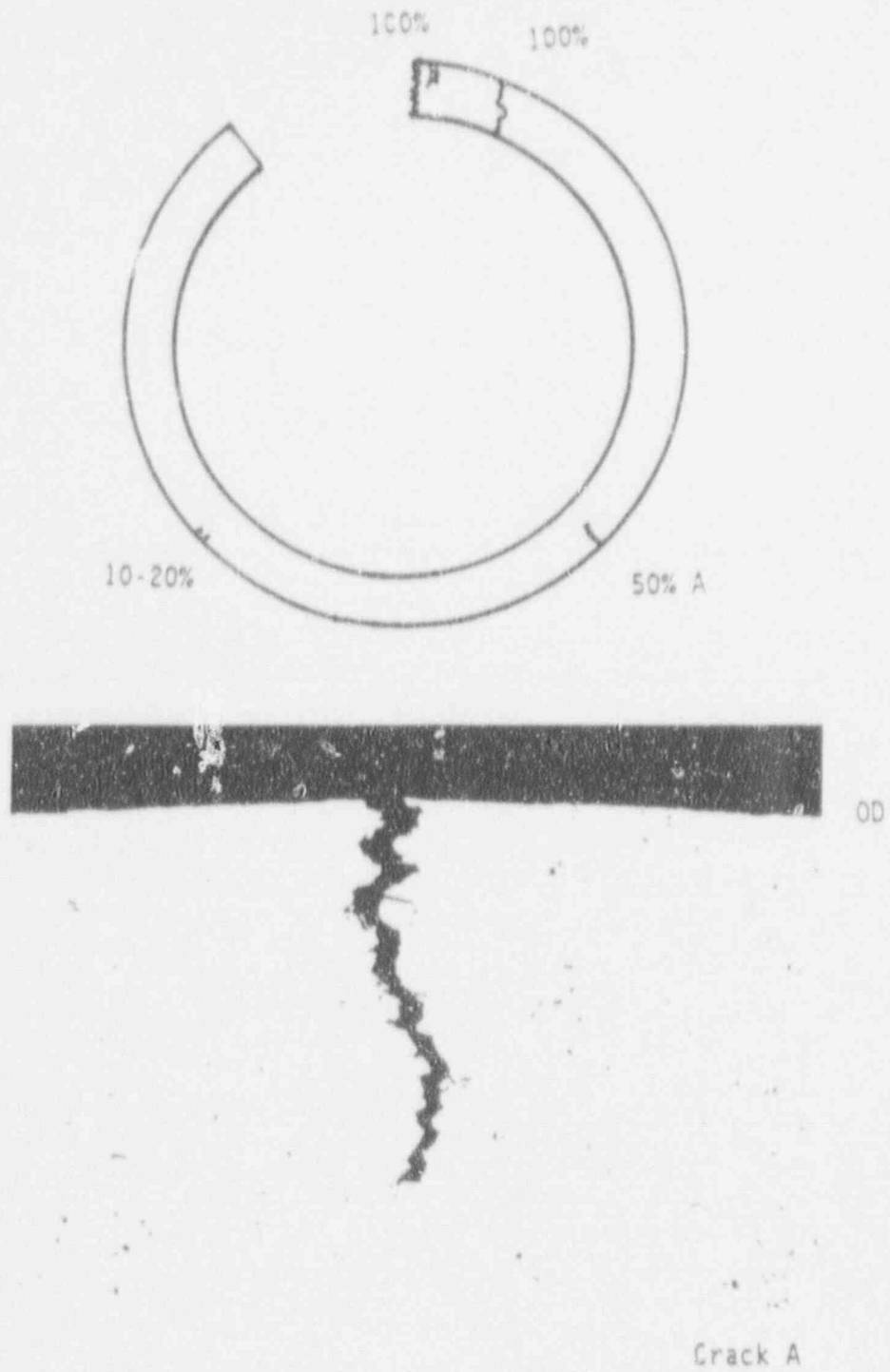
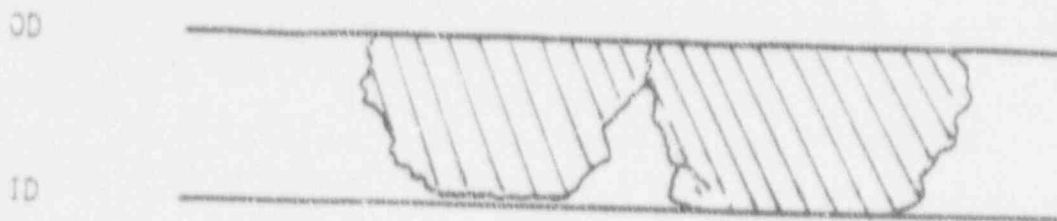


Figure 10-33 Sketch of the metallographic cross section through secondary crevice region cracks in tube 555-3. A photomicrograph of Crack A is also shown. Mag. 100X



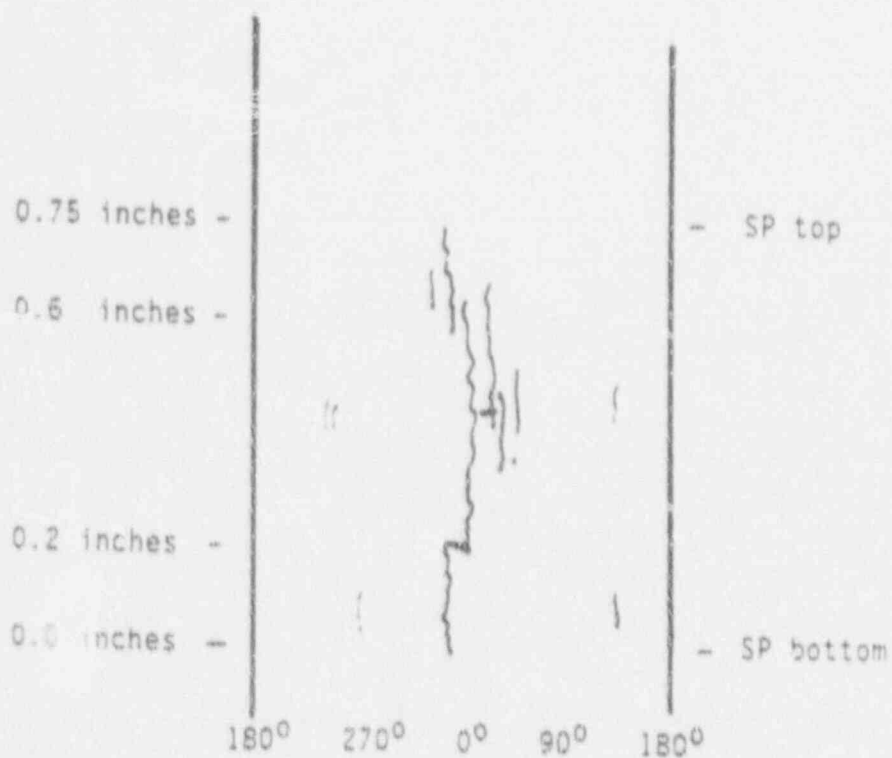
Sketch of Burst Crack

Macrocrack Length = 0.75 inch

Throughwall Length = 0.42 inch

Number of Microcracks = 2 (separated by ductile ligaments)

Morphology = IGSCC



Sketch of Crack Distribution

Figure 10-34 Summary of burst crack observations and the overall crack distribution observed at the crevice region of tube 555-3.

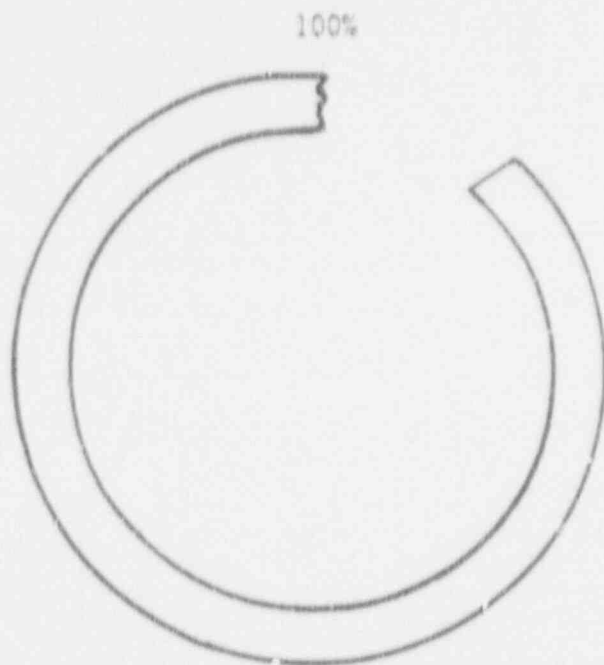
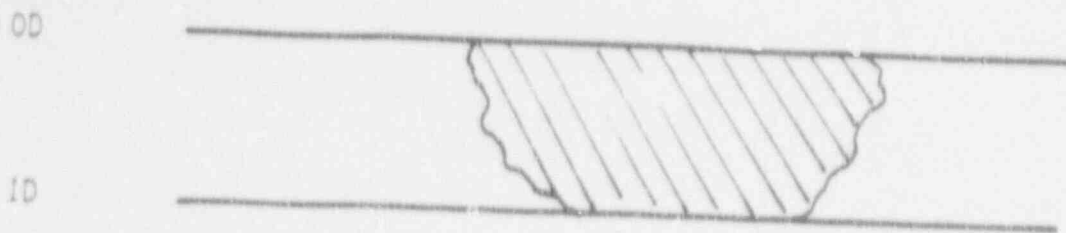


Figure 10-35 Sketch of a metallographic cross section through the center of the crevice region of tube 576-2, showing only the single burst crack, and a photomicrograph of the burst crack.
Mag. 100X



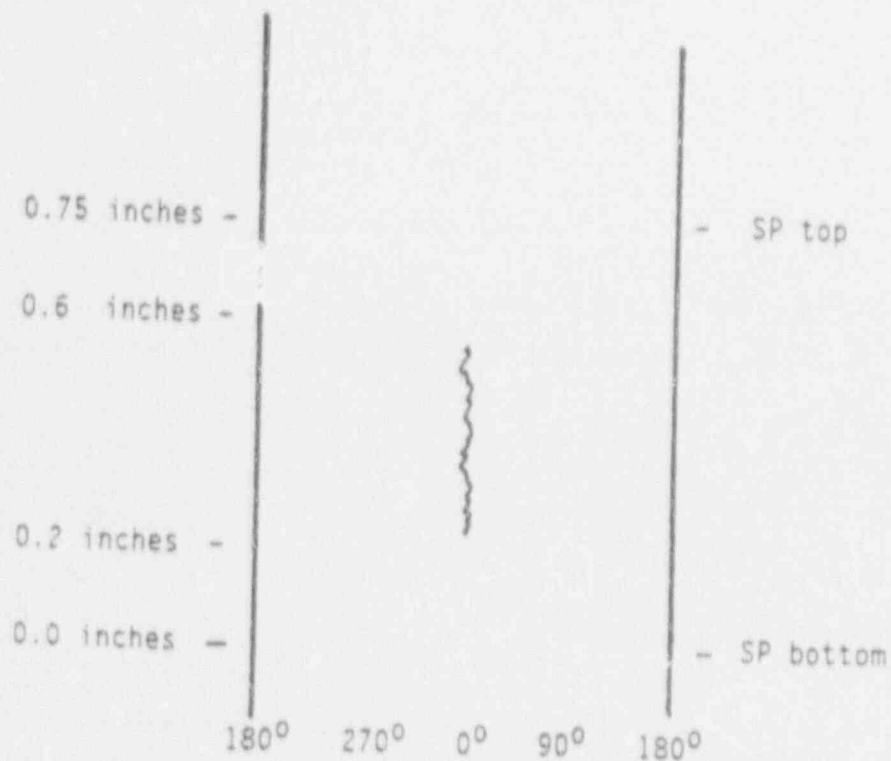
Sketch of Burst Crack

Macrocrack Length = 0.22 inch

Throughwall Length = 0.22 inch

Number of Microcracks = 1 (no ligaments)

Morphology = IGSCC



Sketch of Crack Distribution

Figure 10-36 Summary of burst crack observations and the overall crack distribution observed within the crevice region of tube 576-2.

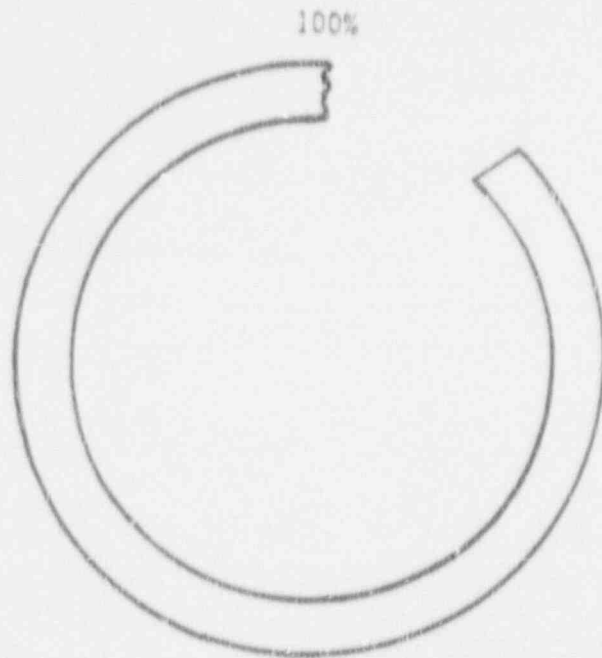
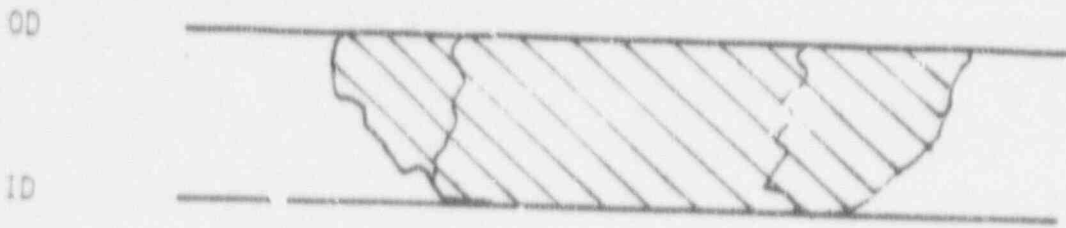
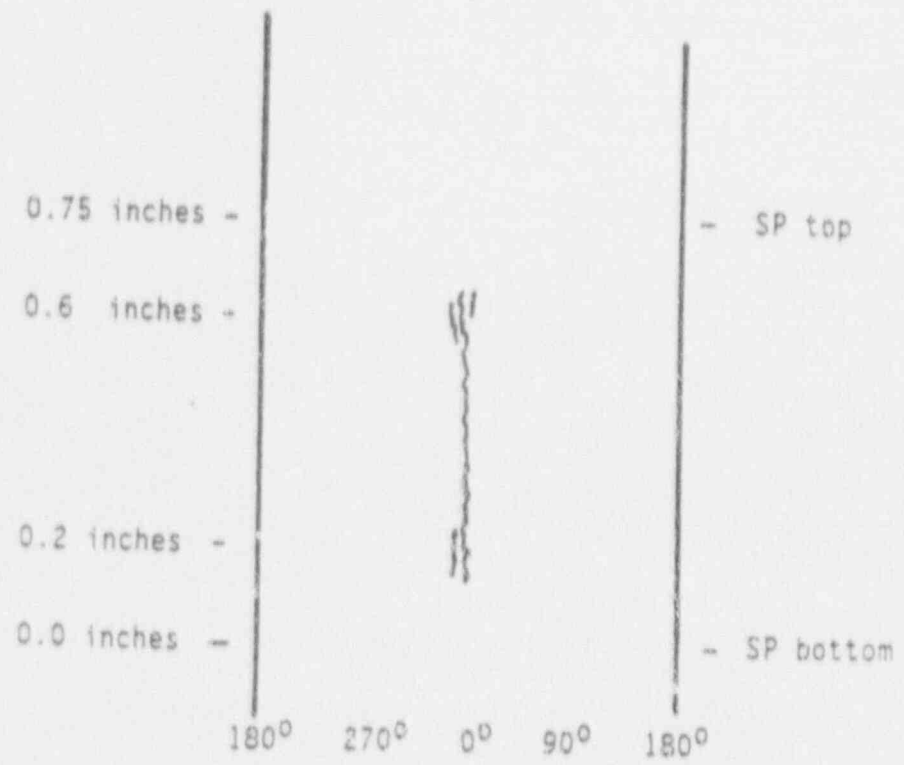


Figure 10-37 Sketch of a metallographic cross section through the center of the crevice region of tube 576-4, showing only the single burst crack, and a photomicrograph of the burst crack.
Mag. 100X



Sketch of Burst Crack

Macrocrack Length = 0.60 inch
 Throughwall Length = 0.43 inch
 Number of Microcracks = 3 (ligaments have intergranular features)
 Morphology = IGSCC with negligible IGA features



Sketch of Crack Distribution

Figure 10-38 Summary of burst crack observations and the overall crack distribution observed at the crevice region of tube 576-4.

11.0 STEAM LINE BREAK (SLB) AND COMBINED ACCIDENT CONSIDERATIONS

This section gives consideration to the limiting accident conditions and their implications relative to tube plugging criteria applications. First, the SLB analysis methods to obtain the tube support plate (TSP) displacement responses (motions) are described and results are developed for open crevices, small gap, and dented tube conditions.

If the TSP displacements from normal operation positions are large compared to the crack lengths which can lead to burst during an SLB event, then the benefit of the TSP to prevent tube rupture could be lost. Assuming uniform through wall cracks (not typical of ODSCC) approaching the TSP thickness, displacement of the TSP during the SLB event exceeding approximately 0.75 inch would expose sufficient length of the crack with the possibility that burst margins would not be met. TSP displacements were evaluated using non-linear, dynamic time history analyses to assess the potential for crack exposure, assuming an open crevice.

The mitigating effects of TSP corrosion and tube denting on these tube support plate motions and associated stresses are quantified based on test results. Then, the evaluation focuses on the constraint against TSP displacement provided by corroded, packed crevices (gap closure).

A combined accident condition evaluation is completed and an allowable SLB leakage is determined. Allowable leakages during an SLB event are developed for Farley plant based upon radiological analyses associated with the determination of acceptable primary to secondary side leakage during accident conditions. An example calculation of a probabilistic analysis of potential end of cycle SLB leakage is presented for Farley-2 based on the 1990 inspection results for comparison to the allowable value.

11.1 Tube Support Plate Loads and Responses During Steam Line Break

Tube support plate deflection will occur during a postulated steam line break as a result of the loads generated by the blowdown transient. These loads were determined for a typical 51 Series steam generator using the TRANFLO code. This is a network flow based code which models the hydraulic characteristics of the steam generator internals. Code output gives pressures and support plate pressure drops as a function of time for the simulated steam line break transient.

The initial condition for the simulated steam line break transient is no load with the water level at the top tube support plate. These conditions lead, conservatively, to the highest tube support plate loads for the transient. The water level assumption is more severe than required by the licensing bases as described in the following section. Additionally, the steam line flow area is assumed to be limited by a flow restrictor. The basis for this assumption is also detailed in the following section. The resulting primary and secondary pressures are shown on Figure 11-1 and the tube support plate pressure drops are shown in Figures 11-2 and 11-3.

The licensing bases for two key assumptions used in the determination of the expected deflection of the steam generator tube support plates are provided below. These analyses assumptions are: the pipe break area used in determining blowdown loads and

corresponding stresses on the steam generator internals during a postulated steam line break event is limited to the size of the effective flow area of the restrictor; and the steam generator water level at the beginning of the event is conservatively assumed to be at the top TSP.

Relative to expected blowdown loads, the design bases for the main steam line flow restrictors (whether integral to the steam generators or part of the main steam line) include, among other criteria, the following:

1. Plant protection in the event of a steam line rupture. In such an event, the flow restrictor reduces steam flow from the break, which then reduces the cooling rate of the primary system.
2. The flow restrictor must maintain its integrity in the event of a double ended break of the main steam line immediately downstream of the flow restrictor.

Although not explicitly mentioned in the design bases, the reduced steam generator blowdown rates/loads discussed in (1) above result in reduced stresses developed in the internal steam generator components, particularly the tubes and tube support plates, compared to other scenarios.

The limiting radiological consequence as a result of a secondary system piping failure is during the steam release from a postulated main steam line break outside of containment. The activity from the affected steam generator containing tubes with potential through wall cracks (and hence potential primary to secondary leakage pathways) as a result of the implementation of steam generator tube plugging criteria is released directly to the environment. The unaffected steam generators are assumed to continually discharge steam and contained activity via the safety and relief valves up to when the initiation of the RHR system can be accomplished.

As noted in (1) and (2) above, it is expected that the flow restrictor would remain functional during a postulated main steam line break event. Therefore, any rupture of the steam line downstream of the flow restrictor and outside of containment with a break area greater than the effective area of the flow restrictor would be limited to radiological consequences from a break size equivalent to the effective area of the restrictor. Plants such as Farley with integral steam generator flow restrictors already take credit within FSAR accident analyses for the presence of the flow restrictor and its ability to limit the effects of such a break on core heat flux and reactor coolant system temperature and pressure, steam generator blowdown loads, and radiological consequences as a result of the presence of a direct release path to the environment.

Relative to expected steam generator water level at the beginning of the transient, the accident analyses assume a condition worse than a no load condition at time zero for a postulated steam line break event. Therefore, it is very conservative to utilize steam generator water level at the uppermost tube support plate in establishing the effects of the blowdown loads on the steam generator internals, including the tubes and support plates.

11.1.1 TSP/Tube Relative Displacements for Open Crevices Under SLB Loadings

The purpose of this subsection is to describe how the analyses to obtain the relative motions between the tubes and tube support plates under SLB loadings are performed. These relative motions are required to determine the potential crack length uncovered during the SLB event if relative motion can occur (as in the non-dented case).

Structural Modeling and Methods

The analytical models to which the (above) loadings are applied, the methods of obtaining the analytical solutions, and the results obtained from performing the analyses (both initial static and many dynamic time-history solutions) are presented below.

Two completely different models were used to obtain a final set of TSP displacement solutions during an SLB. First, a conservative solution was obtained using a global (multiple plate) model. Based on this solution it was determined that: 1) the degree of refinement in the plate superelements was not fine enough in the global model to eliminate potential solution inaccuracies due to numerically incompatible stiffnesses (low stiffness from the plate and very large stiffness associated with the local rotation of a large number (~80) of tubes), and, 2) the stay rods remain elastic during response to the SLB loadings.

Based on the observations from the global model solution (above) a 1/8 symmetric single plate model was developed. Individual plate solutions for each of the seven Series 51 TSPs were obtained. This model considers only 16 tubes at any node. Solutions from this model, obtained with the same loadings as input to the global model, and individual plate superelements with stay rod boundary conditions based on the global model solution, provide the bases for the plate displacements reported below.

For the global model, the stiffness and mass representations of all the linear components of the structural system comprised of the tubesheet, tubes, tube support plates, and interconnecting structures are required for the SLB time-history analysis. These required representations are established through the use of a finite element model utilizing superelements (or substructures) to define appropriate mass and stiffness matrices for each linear component. Due to the support system for the tube support plates (TSPs), it is necessary to include the mass and stiffness of the tubesheet, channel head, lower shell, all seven TSPs, and the tie rods and spacer pipes. In order to account for possible interaction (nonlinear gap closures) between the TSPs and the tubes due to denting, or due to TSP local rotations, tube superelements are also included in the analysis. The Westinghouse proprietary general purpose finite element program, WECAN, is used in developing the linear superelements to represent these components in the analytical models.

The TSPs are supported vertically using a central tie rod/spacer group and four outer tie rod/spacer groups located around and near the edge of the plate. The tie rods have a nut on the upper side of the top TSP. Around the outside of the tie rods are spacers which are located between each of the support plates and between the bottom TSP and the tubesheet. For the central tie-rod and spacer groups the spacers are welded to each of the TSPs with the exception of the spacer between the tubesheet and first TSP which is not welded to the tubesheet. For the spacers located at the periphery of the TSPs, there

is no rigid link between the spacer and the support plates. The lack of a rigid link between the spacers and TSPs results in a nonlinear system. In order to treat this nonlinearity, the central spacer between the tubesheet and first TSP, and all the outer spacers, are incorporated explicitly in the dynamic analysis.

The overall finite element global model is shown in Figure 11-4, with an enlarged view of the tubesheet region shown in Figure 11-5. For the central tie rod and spacer, the area and moment of inertia are specified as one-fourth of the properties for the full geometry.

As shown in Figure 11-5, four tube representations are included in the model. These four tubes are used to generate superelements which can be incorporated at different locations in the dynamic model to be representative of tubes anywhere in the bundle. Four separate tubes are needed due to the different displacement boundary conditions at the plate center, along the symmetry boundaries (one for either side), and for internal plate regions. A summary of the different superelements generated for this analysis is provided in Table 11.1.

Relative to the generic nature of this analysis, one of the design variations between the various Series 51 steam generators is the method of tube expansion inside the tubesheet. In some plants, a full depth WEXTEx expansion with a partial hard-roll is used, while other plants have partial hard-roll or full depth hard-roll. The expansion process affects the stiffness of the tubesheet, as well as the stiffness of the tubes between the tubesheet and first TSP. Farley Unit-1 has WEXTEx expansions and Unit-2 has hard roll expansions. For this analysis, it is conservative to consider the case where the tubes are unexpanded inside the tubesheet, as this results in somewhat higher plate motions due to the decreased tube stiffness. This affects several input parameters, one of which is the modeling of the tubes. In preparing the model, the plate representing the tubesheet is located at the base of the plate (rather than at the tubesheet centerline), and the tube span between this plate and the TSP. In the case where the tubes are considered to be unexpanded, the free axial length of the tubes runs from the base of the tubesheet to the first TSP. Because the tubesheet is 21" thick, this can significantly affect the stiffness of the tubes. In order to properly model the tube stiffness, the properties specified for the tubes are modified to account for the length variation. Finally, in order to account for the entire bundle, the tubes are separated into thirty-one separate groups, with the number of tubes in each group based on a ratio of the plate area covered by each group to the total plate area.

The material properties for the analysis are taken from the 1968 edition of the ASME Code, which was determined to be the applicable code edition for the majority of plants with Series 51 steam generators. Since temperature dependent properties cannot be used in superelements, the properties are specified at the average temperature during the transient, approximately 530°F. The material properties for the tubesheet and tube support plates are modified to account for the tube penetrations and flow holes. In the case of the TSPs, the density is further modified to account for the added mass of the secondary side fluid. In setting up the superelements for subsequent dynamic analyses, it is necessary to define dynamic degrees of freedom. The degrees of freedom are of primary importance for interconnecting structures and adequately defining mode shapes for the tubesheet and each of the TSPs. The locations of the nodes specified as dynamic degrees of freedom are shown in Figure 11-6.

The displacement boundary conditions for the various superelements consist primarily of prescribing symmetry conditions along the "X" and "Y" axes for each of the components. Vertical constraint is provided at channel head nodes corresponding to the approximate location of the support pads.

Application of Pressure Loading

The SLB pressure loads will act on the tubesheet and each of the TSPs. To accommodate this, load vectors are prescribed using reference loads of 1 psi. Later, these vectors are scaled to the actual time-history (transient) loading conditions as defined in Section 11.1 for the dynamic analysis.

The loads are applied to each component in separate load vectors to allow for individual component scaling. The transient pressures summarized in Section 11.1 are relative to the control volume for the thermal hydraulic analysis. The area over which the hydraulic pressure acts corresponds to the area inside the wrapper minus the tube area. These pressures must be scaled based on a ratio of the plate area in the structural model to the control volume area in the hydraulic model.

WERWOLF Computer Program and Solution Capabilities

The global model superelements discussed previously, which represent each of the linear structural components of the Series 51 SG tubes, tubesheet and tube support plates physical system, provide the appropriate mass and stiffness matrices necessary to obtain both the required static and dynamic global model solutions.

These superelement properties are appropriately coupled together in the nonlinear solver program WERWOLF. Additional required model capabilities to simulate important physical effects are also implemented in WERWOLF. For these APC SLB analyses, the additional capabilities include: gap elements to simulate geometric nonlinearities; tube bending stiffness; potential for plasticity in the tie rods; a procedure to obtain the nonlinear static solution; and a procedure to obtain the dynamic (time-history) displacement solution. More detail with respect to each of these capabilities follows. It is noted that each of these capabilities are available for use in both the global and local models.

The gap elements (geometric nonlinearities) are used to represent the closure of plates on tubes by way of local plate rotational deformations due to bending. The curvature induced by the bending will eventually cause the plate to contact the tubes simultaneously at both the upper and lower surfaces of the plate. Once one of the above described gaps close in this manner, the bending stiffness associated with the local tube wall deformations will act to oppose further (local) plate rotational motion.

Potential for plasticity response (material nonlinearities) due to large deformations is modeled in the tie rods.

An iterative procedure to obtain the nonlinear static solution is used. This static solution is then used to define the initial conditions for the global model dynamic analysis.

A special fast procedure which integrates the differential equation of motion to obtain the (dynamic) time-history displacement solution (dynamic analysis) is utilized in these analyses.

The local model requires only tube (local) rotational stiffness and plate superelements. Here, the plate is assumed to have 45 degree symmetry. Effects from all other structures are reflected in the displacements of the stay rods which are obtained from the global model and applied to the local (single plate) model as boundary conditions. There is no friction modeled either before or after gap closure. The required level of solution accuracy is obtained by checking for gap closure on a single tube level and integrating these stiffness effects on a 16 tube level. For comparisons, it is noted that the global model uses 80 tube clusters so that the relative stiffnesses are in the ratio of $80/16 = 5$.

Results for Open Crevice Conditions

The global model solution shows that the tie rods remain elastic.

The local model time-history nonlinear solutions provide the basis for the following observations:



The above analyses utilize conservative assumptions. One such particularly conservative assumption concerns the wedge supports for the TSPs. In manufacturing, the wedges are forced into the TSP-to-wrapper gap with welding of the TSP to the wedges. However, neither the wedge installation forces nor the weld size were measured during assembly and have been ignored in the above analyses. This permits the TSPs (in the model) to displace vertically at the edges with no constraints. Tube-to-TSP friction has also been ignored. In addition, local stiffening of the plates along the tubelane and at the periphery where flow holes are not present were also ignored. All these assumptions lead to conservative TSP displacements. Rather than pursue refinements in the analytical models, analyses were performed that reflect the corroded TSP conditions identified by eddy current evaluations for the Farley Unit 1 steam generators. These conditions are typical of operating units with carbon steel TSPs. Analyses reflecting small crevice gaps and tube-to-magnetite contact forces are described in Sections 11.1.2 and 11.1.3 below.

11.1.2 Effects of Gap Size on Tube Support Plate Response

The purpose of this subsection is to establish the effects that corrosion buildup and/or incipient denting would have on TSP response to the SLB loadings.

For this parametric study, a series of time-history solutions were obtained using the local model for the seventh TSP only. Each of these individual solutions was obtained in the same manner as was the initial solution which provided motions and stresses (see previous subsections) using the nominal tube/support plate gap value of 16 mils. However, in each succeeding analysis, the gap was reduced (made smaller) by one mil (1/1000th of an inch).

Results are shown on Figure 11-7. For the nominal gap case (16 mils), the maximum dynamic motion []^a. From the parametric study results, it was determined that when the gaps become smaller, the bonding of the plate starts to be restricted due to multiple gap closures. At each gap closure point (in space), the tube involved becomes active in the stiffness matrix and works to impede further (local) plate rotation (in time). This newly introduced tube wall local deformation stiffness is much greater than the tube bending stiffness and, thus, significantly reduces the plate bending deformation relative to the case of plate bending without tube interactions. []^a.

From the analytical results above, it is seen that corrosion buildup and/or incipient denting would act to significantly restrict plate deformations during an SLB. Further, these restricted deformations would cause them to remain elastic and, hence, to generally return to their original positions following removal of the SLB loadings.

11.1.3 Bases for TSP Motion Constraints During SLB

From the Farley ECT database, it is clear that tube cracks are situated within the length (thickness) of the tube support plates at the ECT conditions. ECT data are obtained at cold temperature conditions.

Steam generator tube denting (due to TSP corrosion) and cracking initiate and progress at high temperature (normal operation) conditions. At these conditions the tube is lengthened by pressure and temperature effects when compared to the TSP system and stayrods. Since the tubes and plates are in an equilibrium situation and the causes of cracking occur at the tube/plate intersections, it is clear that the cracks are also situated within the plates at normal operation conditions (high temperatures).

It follows from the facts presented in the above two paragraphs that cracks both are created and must remain inside the TSPs during the complete plant operating cycle. It also follows that any loads due to differential thermal expansion of the tubes, plates, and stayrods structural system, which are generated during the operating cycle, are in fact supported by the combination of tube/support plate interaction loadings comprised of denting, incipient denting and friction effects, each giving rise to cumulative plate/tube relative axial motion constraints. Further, and perhaps more importantly, plate rotational constraints, which form a backup resistance to plate displacements, are

provided by the corrosion deposits and tubes themselves (see the SLB analyses of Section 11.1.2, above).

These rotational constraints due to corrosion and the tubes work to ensure that existing tube cracks would remain situated within the plates in the unlikely event of an SLB. Finally, it is noted that these cumulative integrated plate displacement constraint forces, even without those due to rotational constraints, typically would quickly sum to magnitudes greater than 2 (dynamic impulse loading factor) times the SLB loads on the plates. This is demonstrated in the following subsection.

11.1.4 Demonstration of Corrosion Induced TSP Motion Constraints

The purpose of this subsection is to show, on the basis of interpretations of field ECT data, that the vertical reaction forces generated by corroded and/or dented tube/TSP intersections will, by themselves (i.e., without friction or plate bending effects), quite readily integrate to values sufficient to counteract the SLB loading and, thus, constrain the TSPs to zero displacements relative to the tubes during and after a postulated SLB.

The above concept is demonstrated numerically in Table 11.2 where the minimum number of tubes required to support SLB quarter-plate (quadrant) dynamic loadings are calculated for use as a criterion for each of the plates. TSP quadrants are considered (rather than the entire plate) because that is the spacing of the stay rods and, therefore, the mode of the relevant displacements under SLB loads. Note that the tubes considered in the region of concern are chosen specifically because their position in the tube bundle will ensure that they have the highest loaded tube/plate intersections under SLB loadings.

The third column of Table 11.2 is obtained as the peak SLB load (on a plate and at any time) divided by 4 to obtain the quarter-plate load, and then multiplied by 2 as a dynamic load factor. Note that the reaction-load-per-dented/corroded intersection is taken as the minimum measured value from a test program performed to measure both breakaway and sliding force values associated with a significant number of laboratory generated tube/TSP intersection corrosion samples. These samples included specimens both with denting and with incipient denting.

Column 4 gives the equivalent static loading divided by the minimum single intersection (tube) reaction load capability. This is rounded up to the next nearest integer in column 5 to provide the criteria sought.

Results from the ECT interpretations from Farley Unit 1, SGs A, B and C, for each quadrant and each of the seven TSPs, were compared to the criteria of Table 11.2. From these comparisons, and based on a rather large number of individual intersection ECT interpretations, it was confirmed that the number of intersections with corrosion and/or denting in the appropriate regions of the various quadrants of each of the plates in each of the SGs exceeds the required number (the criteria of Table 11.2) in the vast majority of cases. For those few cases where this is not confirmed a simple extrapolation of the percentage of affected tubes actually obtained over the region where support is most effective in constraining TSP motions shows that there is an extremely small probability that further interpretations would not produce the deficit number of affected tubes.

Based on the above described results it is judged that the capability of resisting SLB loadings in Unit 1 by means of axial tube restraints arising from corrosion related effects only is fully expected.

11.2 Combined Accident Considerations

This section deals with combined accident condition loadings in terms of tube deformation and the effects on tube burst pressure. The most limiting accident conditions relative to these concerns are seismic (SSE) plus loss of coolant accident (LOCA) for tube deformation, and SSE plus steamline/feedline break (SLB/FLB) for tube burst. Details of the analysis methods used in calculating tube stresses and tube support plate loads for these loading conditions are also provided.

11.2.1 SSE Analysis

Seismic (SSE) loads are developed as a result of the motion of the ground during an earthquake. A seismic analysis specific to Series 51 steam generators has been completed. Response spectra that umbrella a number of plants with Series 51 steam generators, including the Farley Plant, have been used to obtain tube support plate (TSP) loads and the displacement time history response of the tube bundle. A nonlinear time-history analysis is used to account for the effects of radial gaps between the secondary shell and the TSPs, and between the wrapper and shell.

The seismic excitation defined for the steam generators is in the form of acceleration response spectra at the steam generator supports. In order to perform the non-linear time history analysis, it is necessary to convert the response spectrum input into acceleration time history input. Acceleration time histories for the nonlinear analysis are synthesized from El Centro Earthquake motions, using a frequency suppression/raising technique, such that each resulting spectrum closely envelopes the corresponding specified spectrum. The three orthogonal components of the earthquake are then applied simultaneously at each support to perform the analysis.

The seismic analysis is performed using the WECAN computer program. The mathematical model consists of three-dimensional lumped mass, beam, and pipe elements as well as general matrix input to represent the piping and support stiffnesses. In the nonlinear analysis, the TSP/shell, and wrapper/shell interactions are represented by a concentric spring-gap dynamic element, using impact damping to account for energy dissipation at these locations.

The mathematical model which is used is shown in Figure 11-8. The tube bundle straight leg region on both the hot and cold leg sides of the bundle is modeled by two equivalent beams. The U-bend region, however, is modeled as five equivalent tubes of different bend radii, each equivalent tube representing a group of steam generator tubes. In addition, a single tube representing the outermost tube row is also modeled. Continuity between the straight leg and U-bend tubes, as well as between the U-bend tubes themselves, is accomplished through appropriate nodal couplings. Note that the five equivalent tube groups are extended down two support plates before the single tube representation begins. This allows dissipation of tube response differences due to the variation in U-bend stiffnesses.

For reasons that will be discussed later, tube deformation calculations are performed for three TSP groupings, TSP 1, TSP 2-6, and TSP 7. The highest seismically induced TSP forces are 82 kips for TSP 1, 102 kips for TSP 2-6, and 76 kips for TSP 7.

11.2.2 LOCA Analysis

LOCA loads are developed as a result of transient flow, and temperature and pressure fluctuations following a postulated main coolant pipe break. For the Farley analysis, LOCA loads are developed for five different pipe break locations. These include three primary pipe breaks and two minor pipe breaks. The primary pipe break locations include the steam generator inlet and outlet lines, and the reactor coolant pump outlet line, while the minor pipe breaks include the pressurizer surge line and the accumulator line. As a result of a LOCA event, the steam generator tubing is subjected to the following loads:

- 1) Primary fluid rarefaction wave loads.
- 2) Steam generator shaking loads due to the coolant loop motion.
- 3) External hydrostatic pressure loads as the primary side blows down to atmospheric pressure.
- 4) Bending stresses resulting from bow of the tubesheet due to the secondary-to-primary pressure differential.
- 5) Bending of the tube due to differential thermal expansion between the tubesheet and first tube support plate following the drop in primary fluid temperature.
- 6) Axially induced loads resulting from differential thermal expansion between the tubes and tie rods/spacers due to the tube being tight in the first TSP, and the reduction in primary fluid temperature. (Based on available data, the majority of intersections are considered to be tight. Because the majority of the intersections are tight, the TSP will respond with the tubes, and the resulting loads on the tubes are judged to be small for this loading.)

Loading mechanisms 3) through 5) above are not an issue since they are a non-cyclic loading condition and will not result in crack growth, and/or result in a compressive membrane loading on the tube that is beneficial in terms of negating cyclic bending stresses that could result in crack growth.

11.2.2.1 LOCA Rarefaction Wave Analysis

The principal tube loading during a LOCA is caused by the rarefaction wave in the primary fluid. This wave initiates at the postulated break location and travels around the tube U-bends. A differential pressure is created across the two legs of the tube which causes an in-plane horizontal motion of the U-bend. This differential pressure, in turn, induces significant lateral loads on the tubes.

The pressure-time histories to be input in the structural analysis are obtained from transient thermal-hydraulic (T/H) analyses using the MULTIFLEX computer code. A

break opening time of 1.0 msec to full flow area (that is, instantaneous double-ended rupture) is assumed to obtain conservative hydraulic loads. A plot of the tube model for a typical T/H model for determining LOCA pressure time histories for the tubes is shown in Figure 11-9. Pressure time histories are determined for three tube radii, identified as the minimum, medium, and maximum radius tubes. For the structural evaluation, the pressures of concern occur at the hot and cold leg U-bend tangent points. Plots of the hot-to-cold leg pressure drops for the limiting major and minor pipe breaks, the steam generator inlet break and the accumulator line break, are provided in Figures 11-10 and 11-11, respectively, for each of the three tubes considered. These results show that significantly higher pressure drops occur for the primary pipe break than for the minor pipe break.

For the rarefaction wave induced by the break, the predominant motion of the U-bends is in the plane of the U-bend. Thus, the individual tube motions are not coupled by the anti-vibration bars. Also, only the U-bend region is subjected to high bending loads. Therefore, the structural analysis is performed using single tube models limited to the U-bend and the straight leg region over the top two TSP's. The LOCA rarefaction pressure wave imposes a time varying loading condition on the tubes. The tubes are evaluated using the time history analysis capability of the WECAN computer program. The structural tube model consists of three-dimensional beam elements. The mass inertia is input as effective material density and includes the weight of the tube as well as the weight of the primary fluid inside the tube, and the hydrodynamic mass effects of the secondary fluid. The geometry of the three tube models used for the LOCA analysis are shown in Figure 11-12, with the node numbers identified.

To account for the varying nature of the tube/TSP interface with increasing tube deflection, three sets of boundary conditions are considered. For the first case, the tube is assumed to be laterally supported at the top TSP, but is free to rotate. This is designated as the "continuous" condition, in reference to the fact that the finite element model for this case models the tube down to the second TSP location. As the tube is loaded, it moves laterally and rotates within the TSP. After a finite amount of rotation, the tube will become wedged within the TSP and will no longer be able to rotate. The second set of boundary conditions, therefore, considers the tube to be fixed at the top TSP location, and is referred to as the "fixed" case. Continued tube loading causes the tube to yield in bending at the top TSP and eventually a plastic hinge develops. This represents the third set of boundary conditions, and is referred to as the "pinned" case.

Using the pressure time histories from the T/H analyses, lateral loads are calculated for each tube length at each time point and the dynamic response of the tube is calculated. The analysis shows the continuous set of boundary conditions to give the largest TSP loads for the minimum and medium tubes. For the maximum radius tube, the fixed condition is found to be most representative due to its increased flexibility and higher tube rotations at the top TSP. Each of the dynamic solutions results in a force time history acting on the TSP. These time histories show that the peak responses do not occur at the same time during the transient. For the Farley analysis, however, it is assumed that the maximum reaction forces occur simultaneously. Using the results for these three tubes, a TSP load corresponding to the overall bundle is then calculated.

Summaries of the resulting TSP forces for the inlet break and the accumulator line break are shown in Table 11-3. Based on the plots shown, a bi-linear representation is

assumed for the peak amplitudes as a function of tube radius. Summaries of the overall TSP forces are provided in Tables 11-4 and 11-5 for the top TSP for the Inlet and Accumulator line breaks, respectively. Note that for tube rows 1-6, the peak response is assumed to be constant and equal to the Row 6 response. Shown in Figure 11-13 is the distribution of TSP load for the inlet break for the top TSP. A summary of the resulting TSP loads for each of the breaks for the top TSP and for the TSP below the top TSP is provided in Table 11-6.

11.2.2.1. LOCA Shaking Loads

Concurrent with the rarefaction wave loading during a LOCA, the tube bundle is subjected to additional bending loads due to the shaking of the steam generator caused by the break hydraulics and reactor coolant loop motion. However, the resulting tube stresses from this motion are small compared to those due to the rarefaction wave induced motion.

To obtain the LOCA induced hydraulic forcing functions, a dynamic blowdown analysis is performed to obtain the system hydraulic forcing functions assuming an instantaneous (1.0 msec break opening time) double-ended guillotine break. The hydraulic forcing functions are then applied, along with the displacement time-history of the reactor pressure vessel (obtained from a separate reactor vessel blowdown analysis), to a system structural model, which includes the steam generator, the reactor coolant pump and the primary piping. This analysis yields the time history displacements of the steam generator at its upper lateral and lower support nodes. These time-history displacements formulate the forcing functions for obtaining the tube stresses due to LOCA shaking of the steam generator.

Past experience has shown that LOCA shaking loads are small when compared to LOCA rarefaction loads. For this analysis, these loads are obtained from the results of a prior analysis for a Model D steam generator. To evaluate the steam generator response to LOCA shaking loads, the WECAN computer code is used along with the seismic analysis model, discussed previously. The steam generator support elements are removed, however, because the LOCA system model accounts for their influence on the steam generator response.

Input to the WECAN model is in the form of acceleration time histories at the tube/tubesheet interface. These accelerations are obtained by differentiation of the system model displacement time histories at this location. Acceleration time histories for all six degrees of freedom are used. The resulting LOCA shaking loads used for the Farley analysis are 17.1 kips for TSP 1-6 and 15.5 kips for TSP 7 for the large break LOCA, and 7.75 kips for all TSP for the minor breaks. The small break loads are scaled from the large break loads based on a comparison of support displacements from system analyses for the two types of breaks.

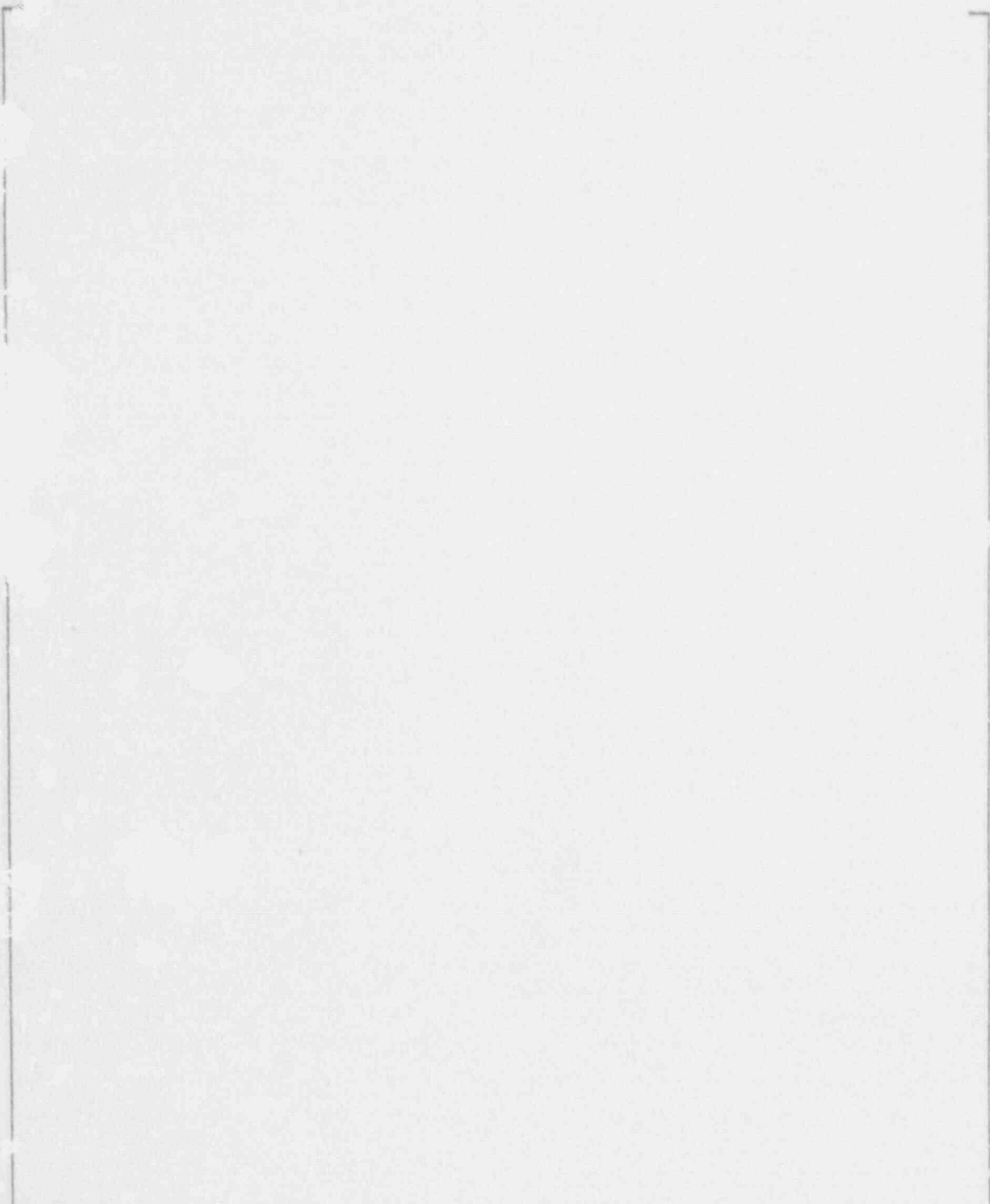
11.2.3 Combined Plate Loads

In calculating a combined TSP load, the LOCA rarefaction and LOCA shaking loads are combined directly, while the LOCA and SSE loads are combined using the square root of the sum of the squares. The overall TSP load is transferred to the steam generator shell through wedge groups located at discrete locations around the plate circumference.

For the Series 51 steam generators, there are six wedge groups located every 60° around the plate circumference (see Figure 11-14). The distribution of load among wedge groups is approximated as a cosine function among those groups reacting the load, which corresponds to half the wedge groups. Except for the bottom TSP, the wedge groups for each of the TSPs are located at the same angular location as for the top TSP. Thus, if TSP deformation occurs at the lower plates, the same tubes are affected as for the top TSP. For the top TSP, however, the wedge groups have a 10 inch width, compared to a 6 inch width for the other plates. This larger wedge group width distributes the load over a larger portion of the plate, resulting in less plate and tube deformation for a given load level. For the bottom TSP, the wedge group width is 6 inches, and the wedge groups are rotated 36° relative to the other TSPs. The distribution of load among the various wedge groups for the LOCA load, which can only act in the plane of the U-bend, is shown in Figure 11-15 for TSP 2-7. Although, the wedges are rotated for TSP 1, the rotations are such that the same load factors result. For seismic loads, which can have a random orientation, the maximum wedge load is 2/3 of the maximum TSP load.

Summaries of the resulting TSP and wedge loads for the Inlet and Accumulator line breaks are provided in Tables 11-7 and 11-8, respectively.

11.2.4 Tube Deformation



11.2.5 Effect on Burst Pressure (SSE + FLB/SLB)

Since the tube support plates provide lateral support to tube deformation that may occur during postulated accident conditions, tube bending stress is induced at the TSP intersections. This bending stress is distributed around the circumference of the tube cross section, tension on one side and compression on the other side, and is oriented in

the axial (along the tube axis) direction. Axial cracks distributed around the circumference will therefore either experience tension stress that tends to close the crack or compressive stress that tends to open the crack. The compressive stress has the potential then to reduce the burst capability of the cracked tube due to the crack opening.

11.3 Allowable Leak Rate for Accident Conditions

A calculation has been completed to determine the maximum permissible steam generator primary-to-secondary leak rate during a steam line break for the Farley Unit 1 and 2 steam generators. Based on a 30 rem thyroid dose at the site boundary, a leak rate of 55 gpm is determined to be the upper limit for allowable primary to secondary leakage in the ruptured loop. Each SG in the intact loops is assumed to have primary to secondary leakage of 150 gpd, which is the maximum value defined by the proposed technical specification. The leakage in the faulted loop may be distributed among tubesheet and support plate locations as shown in Figure 11-19.

Thirty rem was selected as the thyroid dose acceptance criteria based on the guidance of Standard Review Plan (NUREG-0800) Section 15.1.5, Appendix A. Only the release of iodine and the resulting thyroid dose was considered in the leak rate determination. Whole-body doses due to noble gas immersion have been determined, in other evaluations, to be considerably less limiting than the corresponding thyroid doses. The salient assumptions used in the evaluation are those presented in FSAR Section 15.4.2,

with the exception of the iodine partition coefficient assumed for the steam generator in the ruptured loop (1.0 versus 0.1). The salient assumptions follow:

- o Initial primary coolant iodine activity - 1% fuel defects
- o Initial secondary coolant activity - 0.1 $\mu\text{Ci/gm}$ of dose equivalent I-131 (Tech. Spec. LCO)
- o Steam released to the environment (0 to 2 hours)
 - from 2 SGs in intact loops - 479,000 lb (plus primary-secondary leakage)
 - from ruptured loop - 91,000 lb + primary-secondary leakage (the entire initial SG water mass)
- o Iodine partition coefficient for primary-secondary leakage during accident
 - SGs in intact loops - 1.0 since leakage is assumed to be at or above the mixture level
 - SG in ruptured loop - 1.0 since dry SG (FSAR assumes 0.1)
- o Atmospheric dispersion factor - $7.6\text{E-}4$ sec/cu m
- o Thyroid dose conversion factor for I-131 - $1.48\text{E}6$ rem/Ci (TID-14844)

The radioactivity released to the environment due to a main steam line break can be separated into two distinct releases: the release of the initial iodine activity contained in the secondary coolant and the release of primary coolant iodine activity that is transferred by tube leakage. Based on the assumptions stated previously, the release of the activity initially contained in the secondary coolant (3 SGs) results in a site boundary thyroid dose of approximately 2.4 rem. This is independent of the leak location.

Relative to activity release due to primary-to-secondary leakage, the dose contribution from the ruptured loop SG or from the intact loop SGs when the leaks are at a TSP is approximately 0.5 rem/cu m. Because of the SG tube uncover issue that is currently under investigation, treatment of the leakage is different depending on its location. Following a reactor trip, the mixture level in the SG can drop below the apex of the tube bundle. For the SGs in the non-faulted loops, leakage that occurs in the tubesheet region is assumed to mix with the secondary coolant (partition coefficient associated with steaming is 0.1), while the leakage that occurs at a support plate is conservatively assumed to transfer directly to the environment without mixing or partitioning since the leakage site is assumed to be above the mixture level. Although less than 4 feet of the bundle is expected to be above the mixture level, any leakage at a support plate is conservatively assumed to be uncovered for the duration of the accident recovery. The SG in the ruptured loop is assumed to steam dry (no mixture level). Hence leakage to this SG is assumed to transfer directly to the environment regardless of the location of the leak.

The radiological assumptions utilized in the allowable leak rate determination are more conservative than those of the FSAR steam line break offsite dose analysis. The salient differences are with regard to the treatment of primary to secondary leakage and the offsite dose acceptance criteria. These differences are described in detail below:

- o Ruptured loop primary to secondary leakage

The allowable leak rate determination assumes that all primary coolant leakage activity is released directly to the environment. The SG is assumed to steam dry.

Thus, there is no iodine retention or partitioning in the SG. The current FSAR steam line break analysis assumes a retention factor of 0.1; i.e., only 10% of the primary coolant leakage activity is released to the environment.

o Intact loops primary to secondary leakage

The allowable leak rate determination assumes that all primary coolant leakage activity is released directly to the environment. The tube leakage sites are assumed to be located above the mixture level. Thus, leakage activity is assumed to neither mix with the secondary coolant nor partition. The current FSAR steam line break analysis assumes complete mixing of the leakage activity with the secondary coolant. The iodine partition coefficient associated with steaming is 0.1.

c Offsite dose acceptance criteria

The allowable leak rate determination is based on a site boundary dose limit of 30 rem to the thyroid (10% of the 10 CFR 100 guideline). The current FSAR analysis specifies an acceptance criteria of less than 10 CFR 100.

The accidents that are affected by primary to secondary leakage are those that include modeling of leakage and secondary steam release to the environment. These accidents include:

- o Loss of External Electrical Load and/or Turbine Trip, FSAR 15.2.7
- o Loss of All AC Power to Station Auxiliaries, FSAR 15.2.9
- o Major Secondary System Pipe Rupture (SLB), FSAR 15.4.2
- o Steam Generator Tube Rupture (SGTR), FSAR 15.4.3
- o Single Reactor Coolant Pump Locked Rotor, FSAR 15.4.4
- o Rupture of a Control Rod Drive Mechanism Housing (Rod Ejection), FSAR 15.4.6

In addition to these list of accidents, there are also the following events which have secondary side releases associated with them:

- o Uncontrolled RCCA Bank Withdrawal from a Subcritical Condition, FSAR 15.2.1
- o Uncontrolled RCCA Bank Withdrawal at Power, FSAR 15.2.2
- o RCCA Misalignment, FSAR 15.2.3
- o Partial Loss of Forced Reactor Coolant Flow, FSAR 15.2.5
- o Loss of Normal Feedwater, FSAR 15.2.8
- o Accidental Depressurization of the Main Steam System, FSAR 15.2.13
- o Complete Loss of Forced Reactor Coolant Flow, FSAR 15.3.4
- o Single RCCA Bank Withdrawal at Power, FSAR 15.3.6

The reason that the steam line break is generally limiting is because of the assumption that leakage to the faulted steam generator is assumed to be released directly to the environment, i.e., no mixing with the secondary coolant or partitioning of activity is assumed, since the steam generator in the faulted loop is subject to dryout. Depending on the elevation of the degradation (at the tubesheet region versus at a TSP), for other accidents in which there is a secondary side steam release there is justification for mixing and iodine partitioning in the steam generators following the potential initial

uncovery of the top of the tube bundle after the reactor trip. These factors, along with a smaller primary to secondary pressure differential, significantly reduce the release of iodine to the environment for accidents other than steam line break.

As noted above, mixing and iodine partitioning is dependent upon the elevation of the degradation. For the non-SLB accidents in which there is a secondary side steam release, there is justification for mixing and partitioning if the primary to secondary leakage is at the tubesheet region (below SG water level). If the degradation is at a TSP, such that the reactor coolant leakage could directly get into the steam space, the release path (for radioactivity) is assumed to be direct to the environment, just as it is for the faulted SG following an SLB.

Following the implementation of the voltage plugging criteria, tubes are not expected to burst during SLB conditions; however, it cannot be assumed that the tubes will not leak. SLB is the bounding accident condition for which the primary to secondary leakage is to be limited so as to maintain radiological consequences within acceptable levels. Thus the potential leakage during an SLB should be kept below the maximum allowable value determined above.

11.4 SLB Leakage Determination

A number of tube intersections with voltage signals that satisfy the proposed criteria will remain in service until inspected and reevaluated at the next refueling outage. Some of these would have the potential for leakage under SLB conditions as leak rate testing has demonstrated some potential for leakage under SLB pressures in tubes with voltage signals less than 3 volts. Therefore, leak rate during SLB conditions is to be evaluated for the tubes remaining in service at each outage. The methodology used in the evaluation is a probabilistic approach based on the leak rate versus voltage correlation and the population of voltage signals at TSP intersections left in service. Uncertainties in voltage signal, growth allowance and leak rate versus voltage are accounted for using Monte Carlo techniques. Thus, an end of cycle voltage population is assessed (accounting for uncertainties) for its potential for leakage during a postulated SLB.

A current distribution of number of indications versus voltage signal will be obtained at each outage similar to those presented in Section 5 for each steam generator for Farley 1 and 2. Also, the most recent change in voltage (voltage rate) between the last two inspections will be obtained on an effective full power year (EFPY) basis. The current voltage distribution (cut off at the voltage plugging limit) is then combined with the voltage growth rate using Monte Carlo techniques to establish an end of cycle (EOC) voltage distribution. Uncertainty in the voltage signals for the current inspection is accounted for in a statistical manner via the Monte Carlo simulations. Finally, the EOC voltage distribution is evaluated for leak rate using Monte Carlo techniques and the leak rate versus voltage relationship given in Section 9.7 and in Figure 9-3. This is repeated 1000 times to establish a cumulative probability distribution of potential leak rate for each SG.

The method described above has been applied to the Farley 2 voltage distribution in 1990 for each steam generator. The methodology utilizes the following data for Farley-2.

Current Outage Distribution of Indications

The outage inspection of Figure 5-5 with a voltage cutoff of 3.6 volts defines the distribution of indications returned to service assuming application of the current plugging criteria.

Eddy Current Uncertainty

Section 8.8 defines the eddy current uncertainty distribution for the voltage amplitudes as 12% (one standard deviation) with an upper limit of 25%. This uncertainty is applied statistically (by Monte Carlo simulation) to the above distribution of indications.

Growth Rate

Figure 5-15 shows the growth rate distribution from the last Farley-2 operating cycle. The distribution is provided as a cumulative distribution function for voltage growth. The Figure 5-15 growth rates are given as growth over the last cycle and are multiplied by the ratio of the planned EFPYs for the next cycle to the EFPYs for the prior cycle. This growth rate distribution is randomly sampled for the Monte Carlo analysis and added to the current distribution with sampling for eddy current uncertainties to obtain the projected end of cycle distribution. Equal operating cycle lengths for prior and projected cycles are assumed for the Farley-2 analysis results given below.

SLB Leak Rate

The linear regression fit of Figure 9-3 is used to obtain a leak rate for a projected voltage. A Monte Carlo selection is made at each voltage within the prediction interval described by the statistical parameters developed in Section 9.7, and summed over the total distribution of indications to obtain the projected total leakage at SLB conditions for each steam generator. Cumulative probability is obtained based on 1000 cases.

A maximum expected leak rate of a single SG during a postulated SLB was determined to be 0.4 gpm for Farley Unit-2 at the 90% cumulative probability level, compared to the 55 gpm allowable. Therefore, the cutoff value of 3.6 volts conservatively limits the potential for leakage during SLB and no further restriction would be required.

Table 11.1

Summary of Component Superelements

Component	Superelement
Tubesheet	
a. Perforated Region	1
b. Solid Outer Rim	1
Tube Support Plates	
a. TSP 1	2
b. TSP 2	3
c. TSP 3	4
d. TSP 4	5
e. TSP 5	6
f. TSP 6	7
g. TSP 7	8
Channel Head	
a. Tubesheet Junctionure	1
b. Spherical Bowl	1
Shell	
a. Stub Barrel	1
b. Lower Shell	1
Wraps	-
Stayrod	
a. Central	9
b. Outer	9
Spacer	
a. Central - Pass 1	-
b. Central - Pass 2-7	14
c. Outer - Pass 1	-
d. Outer - Pass 2-7	-
Tubes	
a. Central - Pass 1	10
b. Central - Pass 2-7	10
c. "X" Axis - Pass 1	11
d. "X" Axis - Pass 2-7	11
e. "Y" Axis - Pass 1	12
f. "Y" Axis - Pass 2-7	12
g. Interior - Pass 1	13
h. Interior - Pass 2-7	13

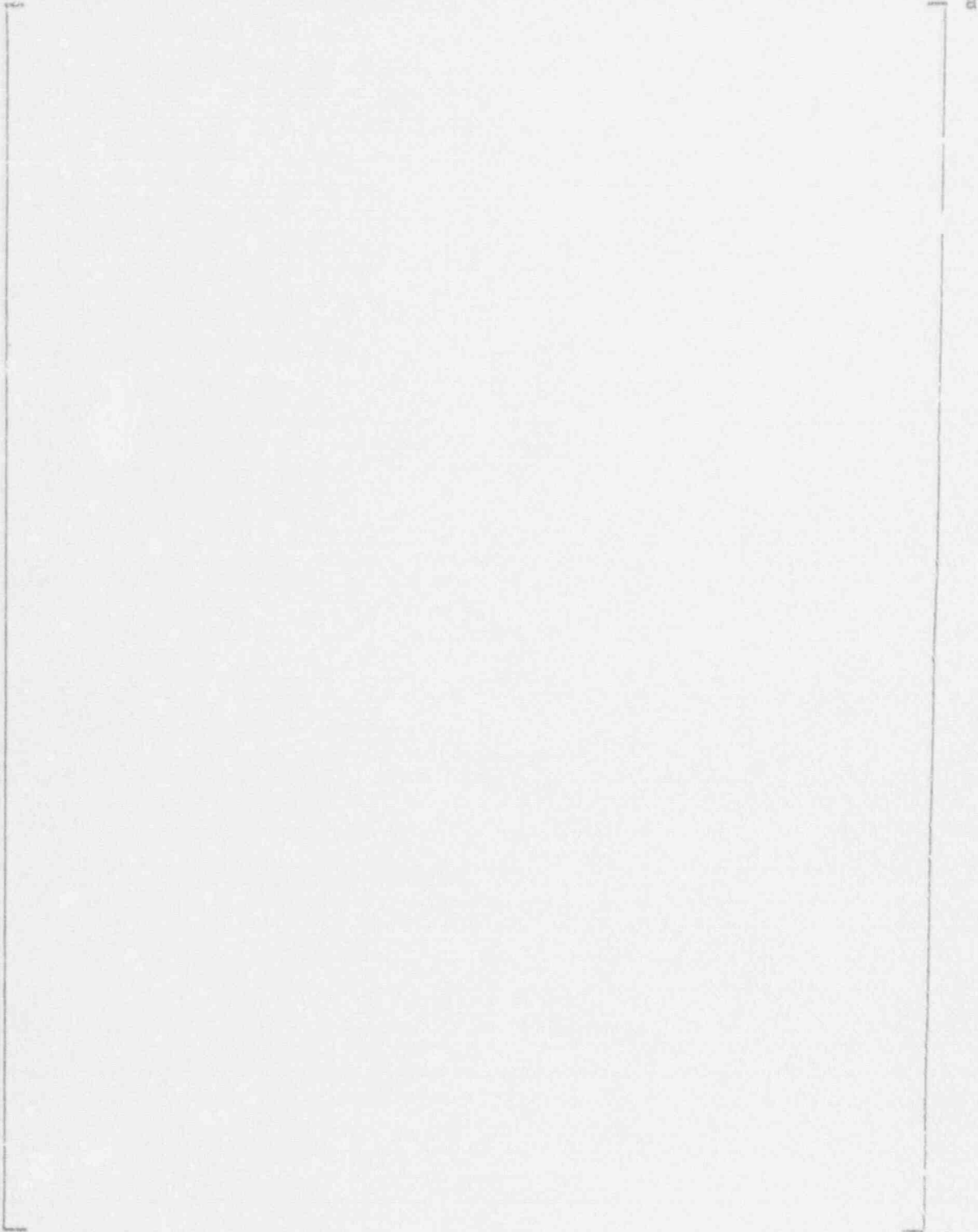
TABLE 11.2 : CRITERIA AND BASES FOR FARLEY (S51) ECT INTERPRETATIONS TO SUPPORT SLB EVALUATIONS.

- RATIONAL : DEMONSTRATE REQUIRED MINIMUM NO. OF ACTIVE INTERSECTIONS TO INDUCE REACTION LOAD TO PREVENT MOTION UNDER SLB LOADING.

- TO BE APPLIED TO EACH PLATE AND EACH QUADRANT.
- ACTIVE (DENTED AND/OR CORRODED) INTERSECTIONS MUST OCCUR WITHIN SPECIFIED "REGION" IN EACH PLATE QUADRANT (AT PERIPHERY OF PLATE AND MIDWAY BETWEEN STAY RODS).
- MIN TEST MEASURED RX LOAD/INTERSECTION = 80 lb.
- DYNAMIC LOAD FACTOR OF 2 USED TO ADDRESS IMPULSIVE LOADING NATURE OF SLB.(1)

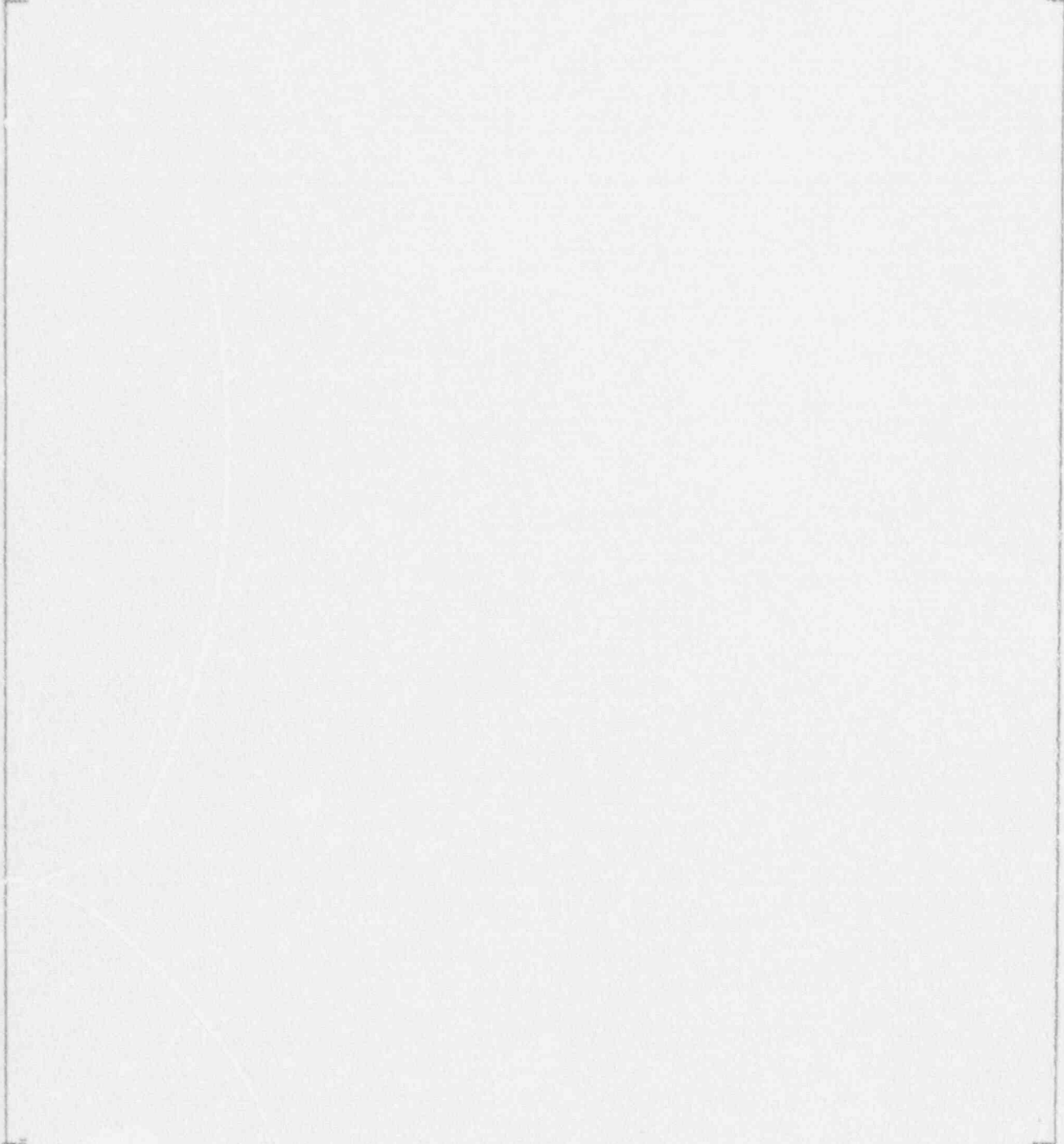
PLATE	PEAK SLB LOAD [lb]	QUARTER-PLATE (QP) DYN LOAD (1) [lb]	QP DYN LOAD MIN RX LOAD	MIN NO TUBES REQ'D TO SUPPORT QUARTER-PLATE DYN LOAD
1	[]
2				
3				
4				
5				
6				
7				

TABLE 11-3
SUMMARY OF TSP FORCES - TOP TSP
DYNAMIC TIME HISTORY ANALYSES
LCCA RAREFACTION PRESSURE WAVE LOADING



a

TABLE 11-4
SUMMARY OF TOTAL TSP FORCE - TOP TSP
LOCA RAREFACTION PRESSURE WAVE LOADING
STEAM GENERATOR INLET BREAK



a

TABLE 11-6
SUMMARY OF TSP FORCES
LOCA RAREFACTION PRESSURE WAVE LOADING



a

TABLE 11-7
SUMMARY OF WEDGE LOADS
COMBINED LOCA + SSE LOADINGS
STEAM GENERATOR INLET BREAK



a

TABLE 11-8
SUMMARY OF WEDGE LOADS
COMBINED LOCA + SSE LOADINGS
ACCUMULATOR LINE BREAK



a

TABLE 11-9
SUMMARY OF CALCULATIONS TO DETERMINE AREA
UNDER FORCE / DEFLECTION CURVE

CRUSH TEST NO. 2

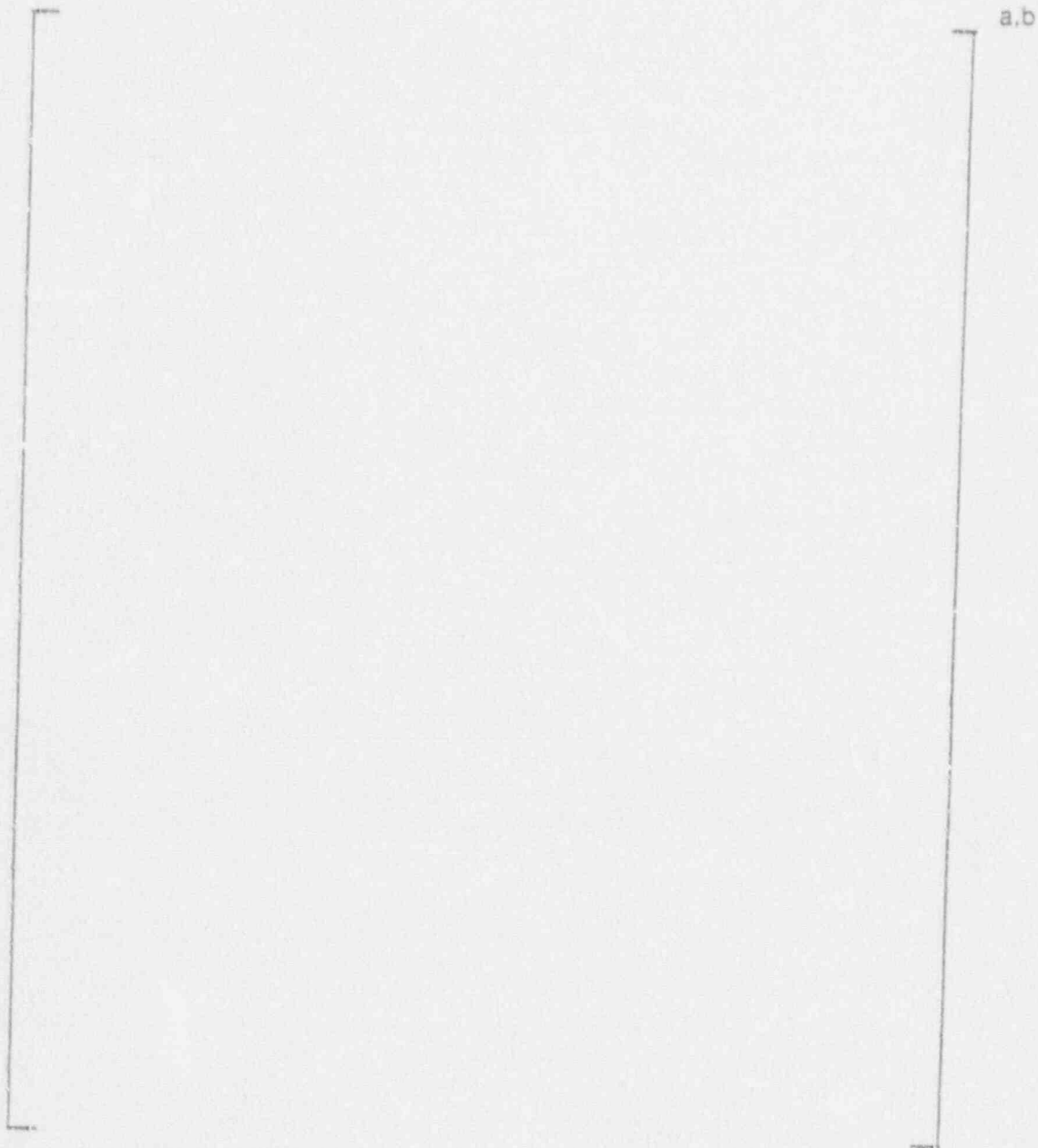


TABLE 11-10
SUMMARY OF NUMBER OF DEFORMED TUBES
AS A FUNCTION OF LOAD

a.b

TABLE 11-11
SUMMARY OF NUMBER OF DEFORMED TUBES AT WEDGE LOCATIONS
TSP 1

a,b

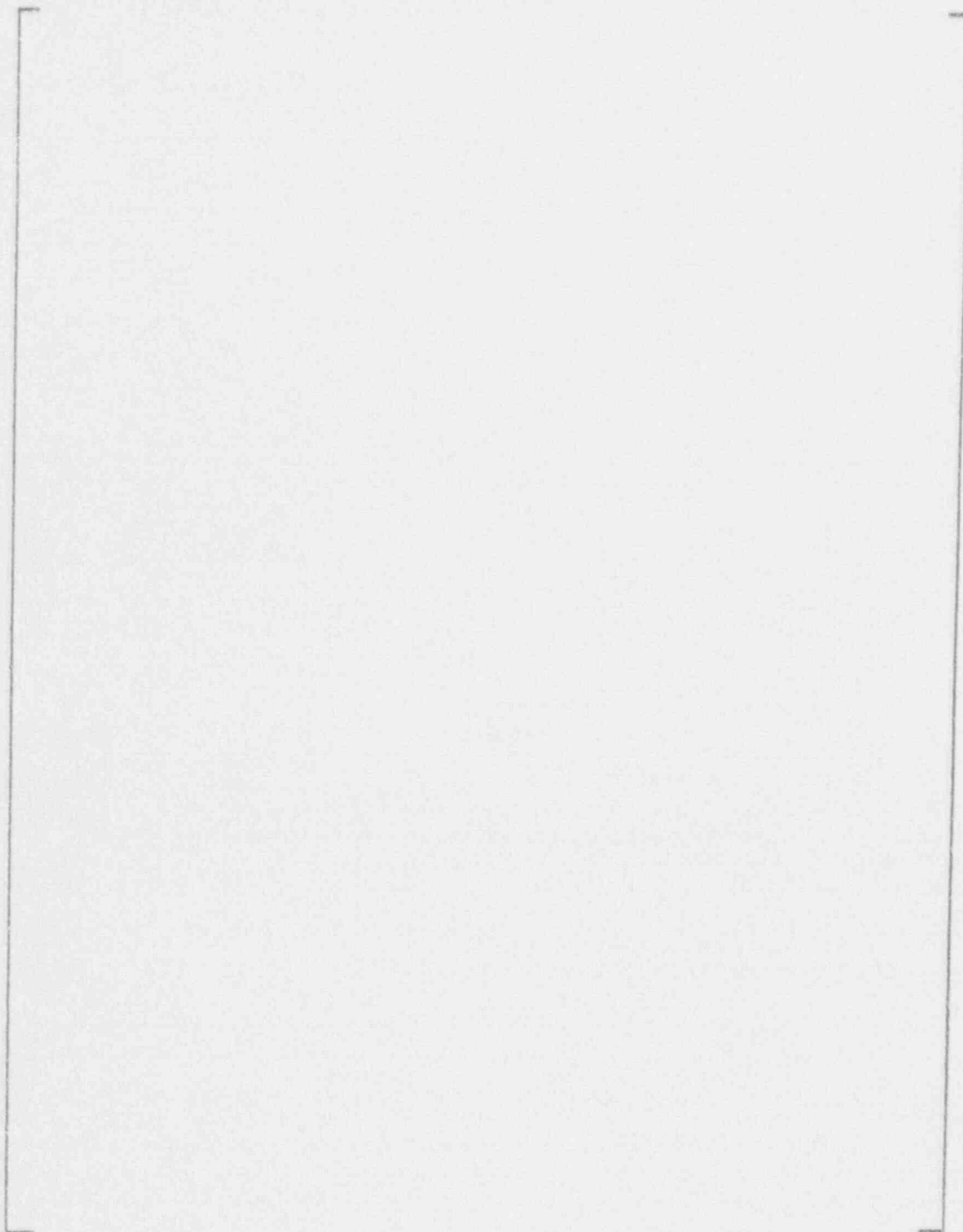
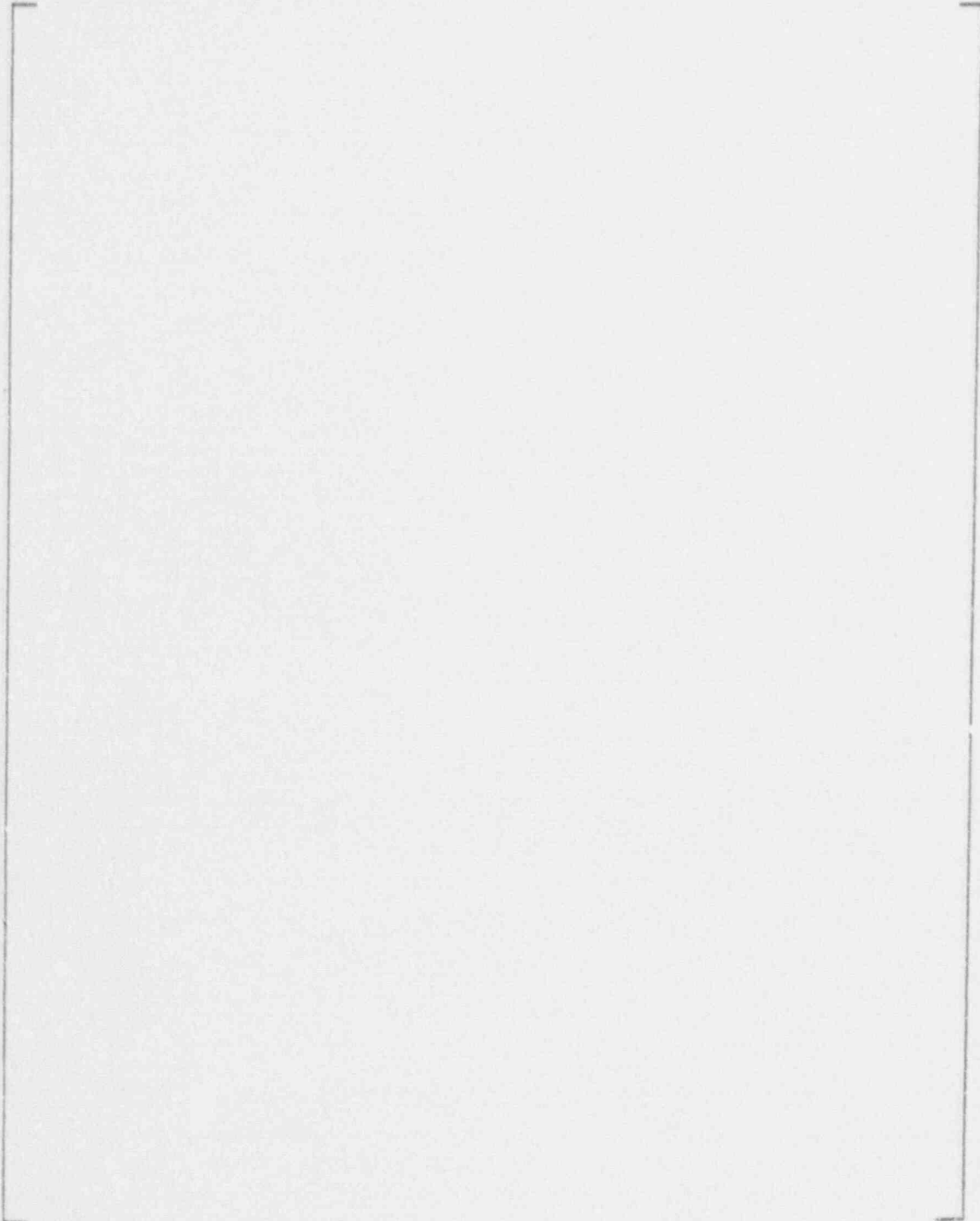


TABLE 11-12
SUMMARY OF NUMBER OF DEFORMED TUBES AT WEDGE LOCATIONS
TSP 2-6

a.b

TABLE 11-13
SUMMARY OF NUMBER OF DEFORMED TUBES AT WEDGE LOCATIONS
TSP 7



a,b

TABLE 11-14
APPLICABILITY OF TEST RESULTS
TO WEDGE LOCATIONS

[

]

a.b

TABLE 11-15
COMBINED BENDING AND INTERNAL PRESSURE BURST TESTS
ON TUBES WITH THROUGH WALL SLOTS



a.b

Figure 11-1

Steam Line Break - Primary and Secondary Pressures



Figure 11-2

TSP Pressure Drops During SLB; TSPs 1 - 4



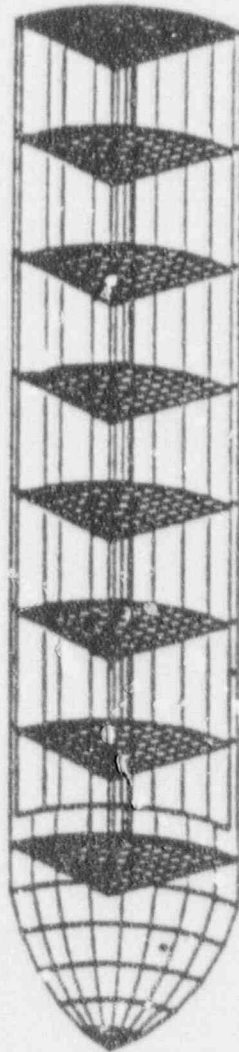
Figure 11-3

TSP Pressure Drops During SLB; TSPs 5 - 7



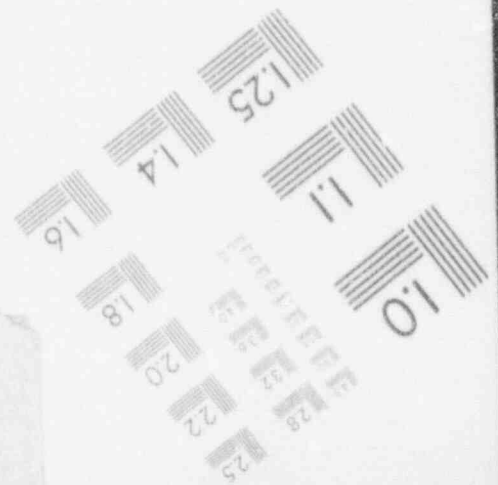
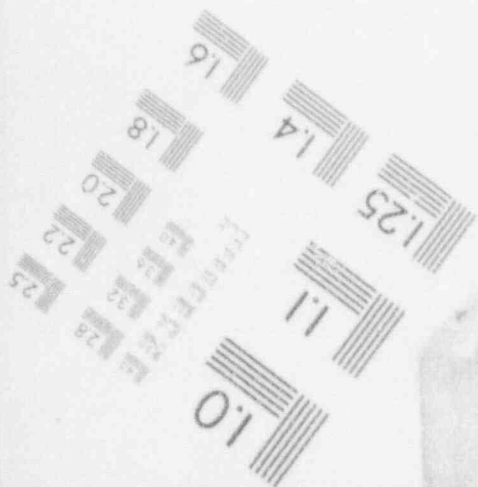
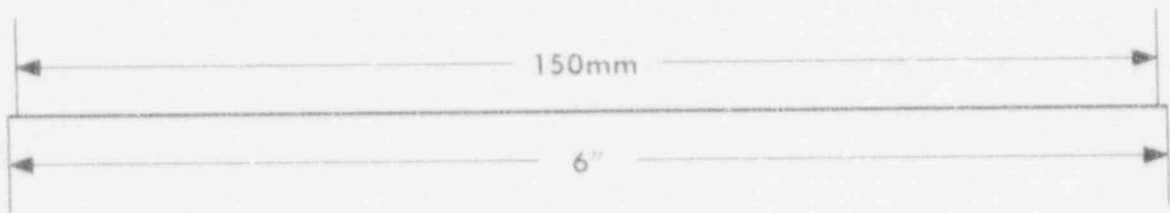
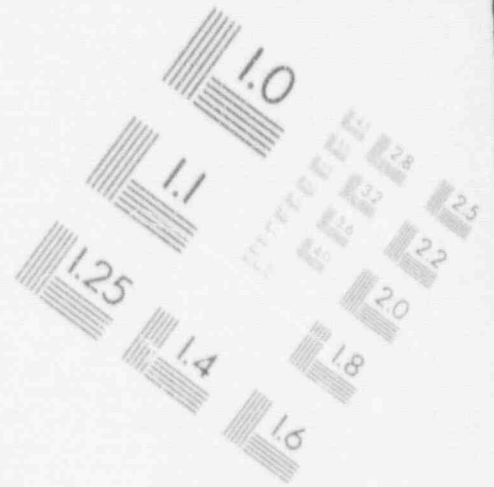
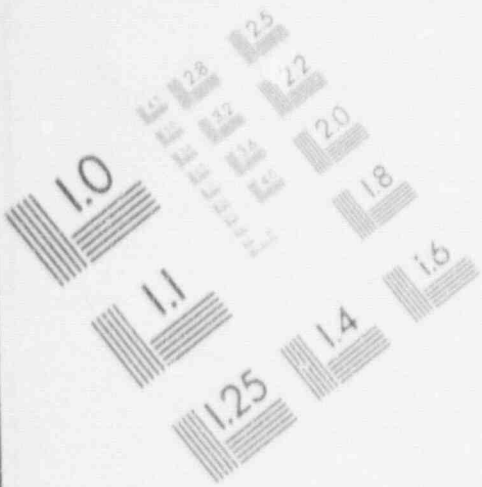
Figure 11- 4

Overall Finite Element Model



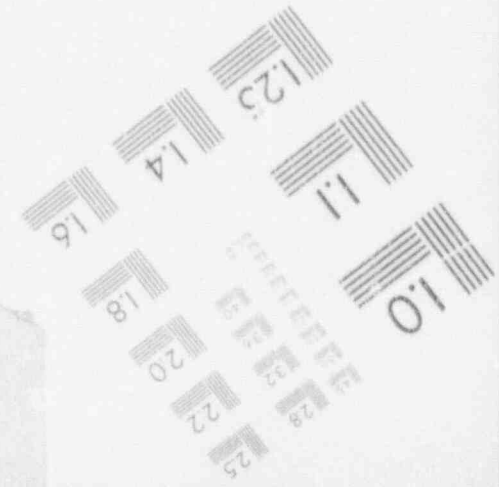
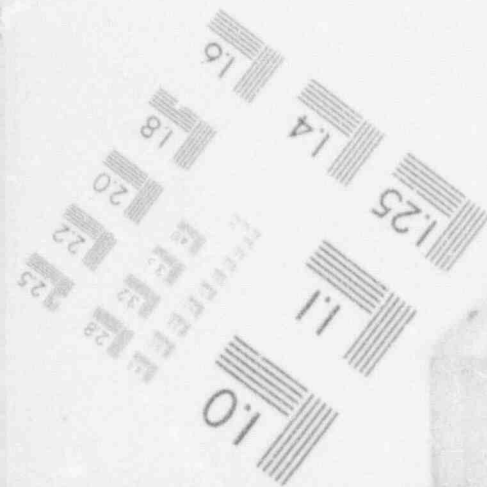
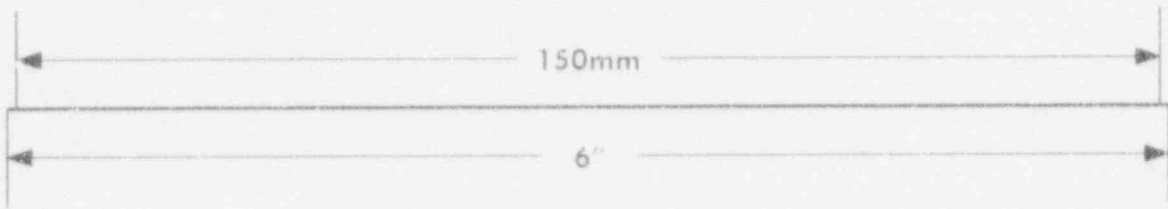
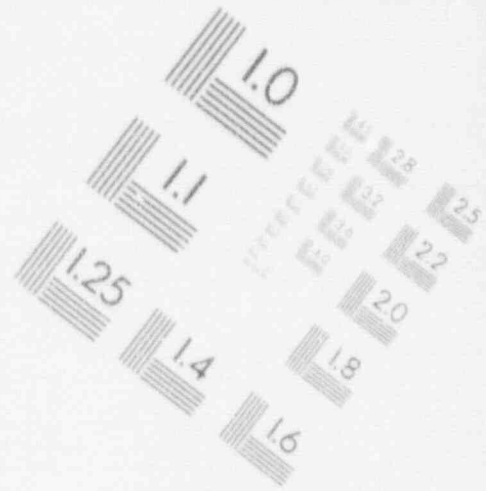
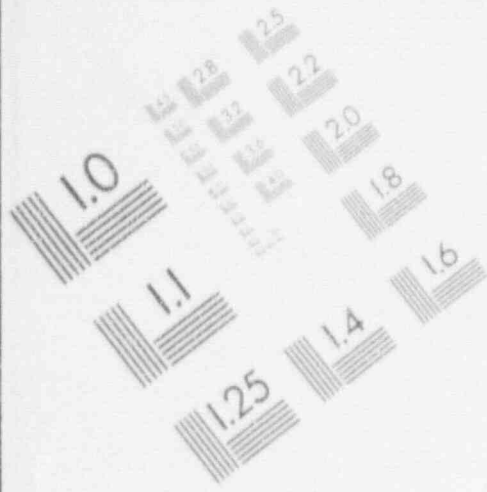
1

IMAGE EVALUATION TEST TARGET (MT-3)



1

IMAGE EVALUATION TEST TARGET (MT-3)



1

IMAGE EVALUATION TEST TARGET (MT-3)

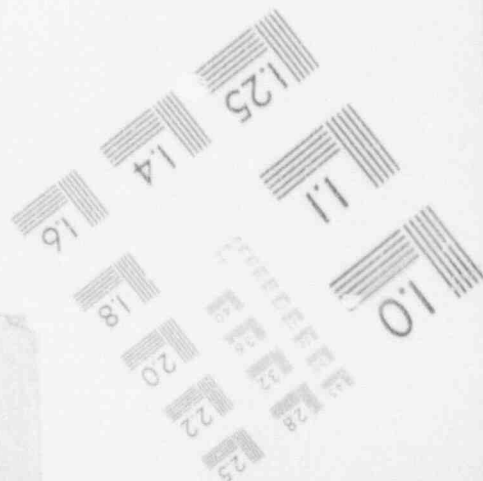
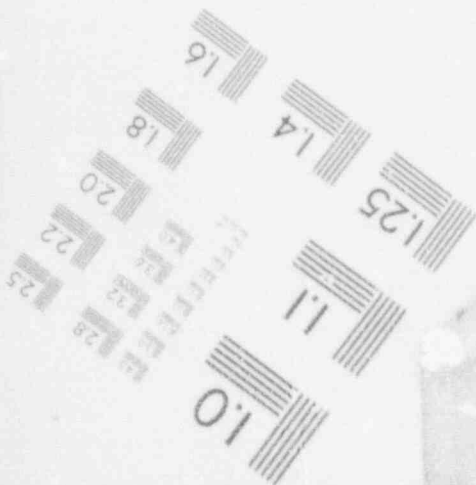
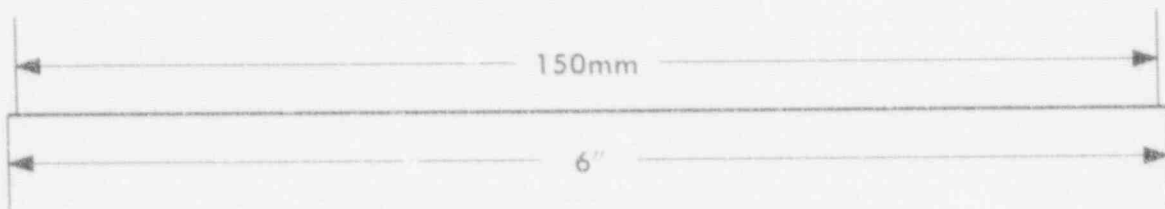
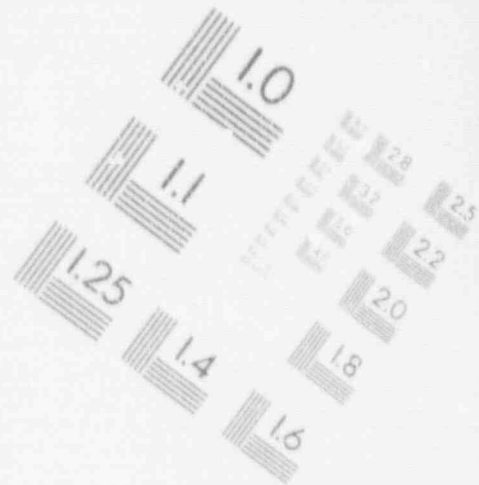
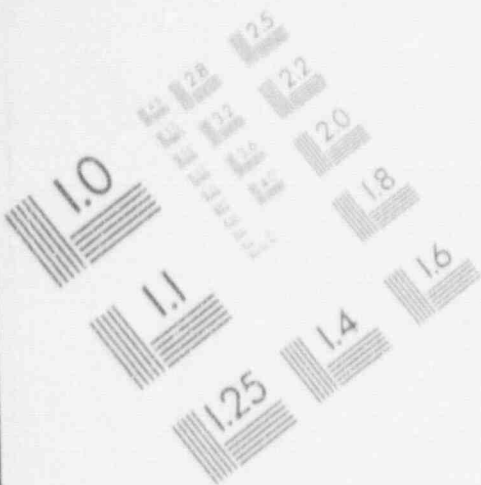


Figure 11 - 5

SLB Finite Element Model
Enlarged View of Tubesheet Region

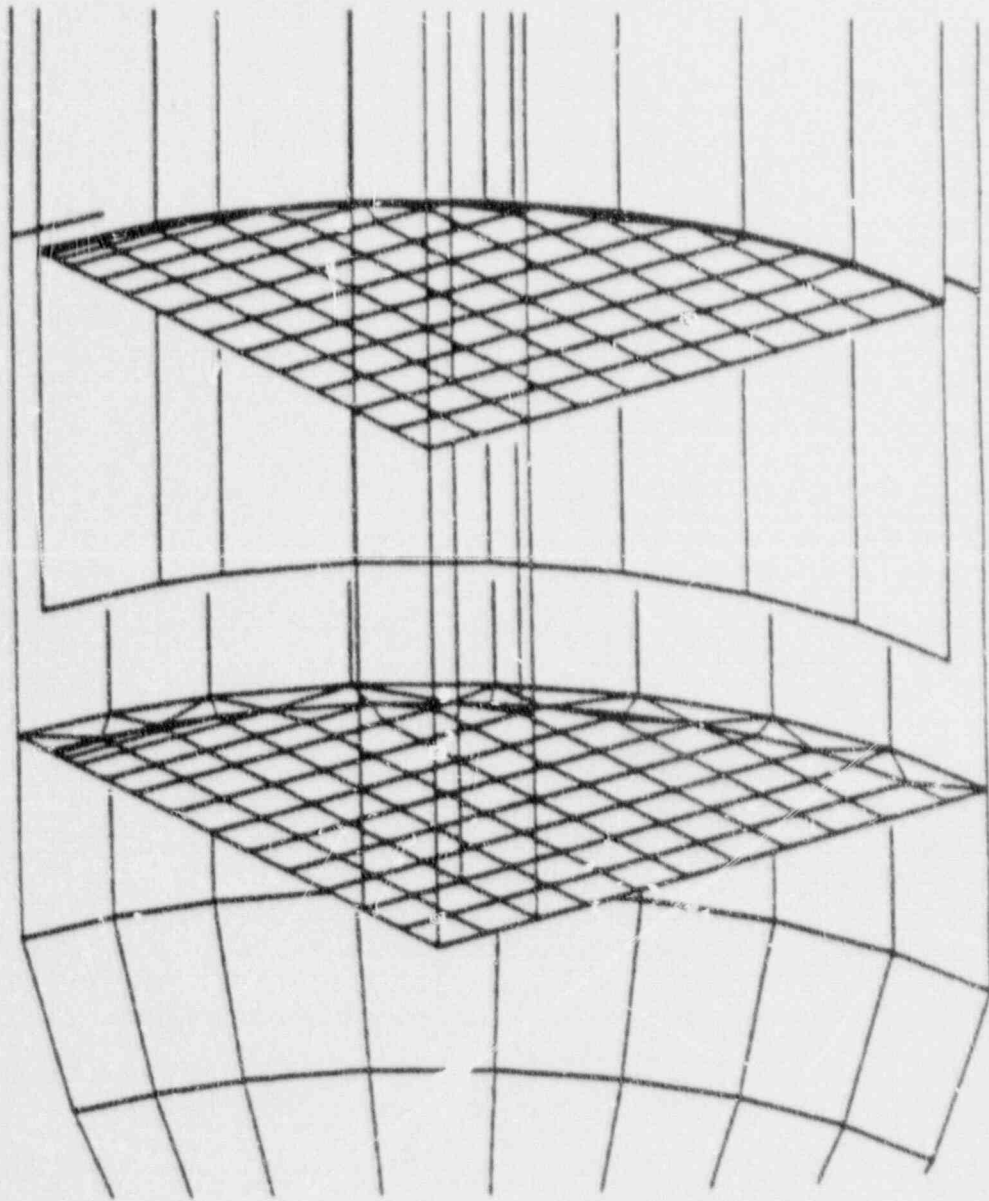
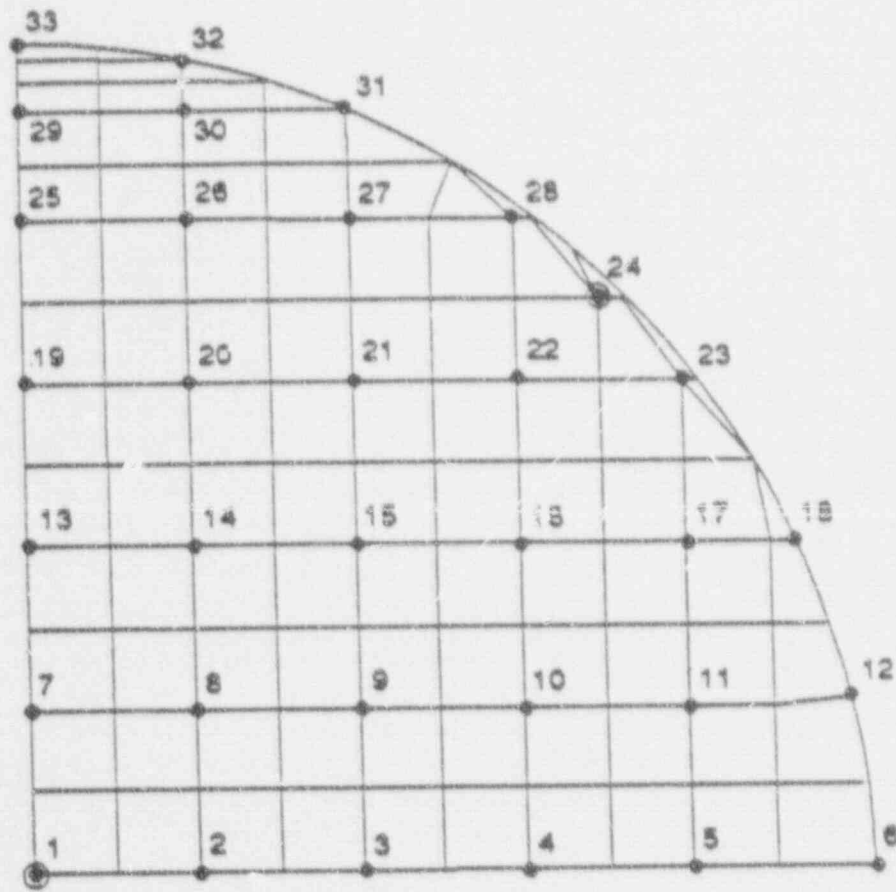


Figure 11 - 6

Tubesheet/TSP Dynamic Degrees of Freedom



⊙ Tie rod / Spacer Location

Figure 11-7

TSP 7 Displacement vs Gap for SLB Loadings



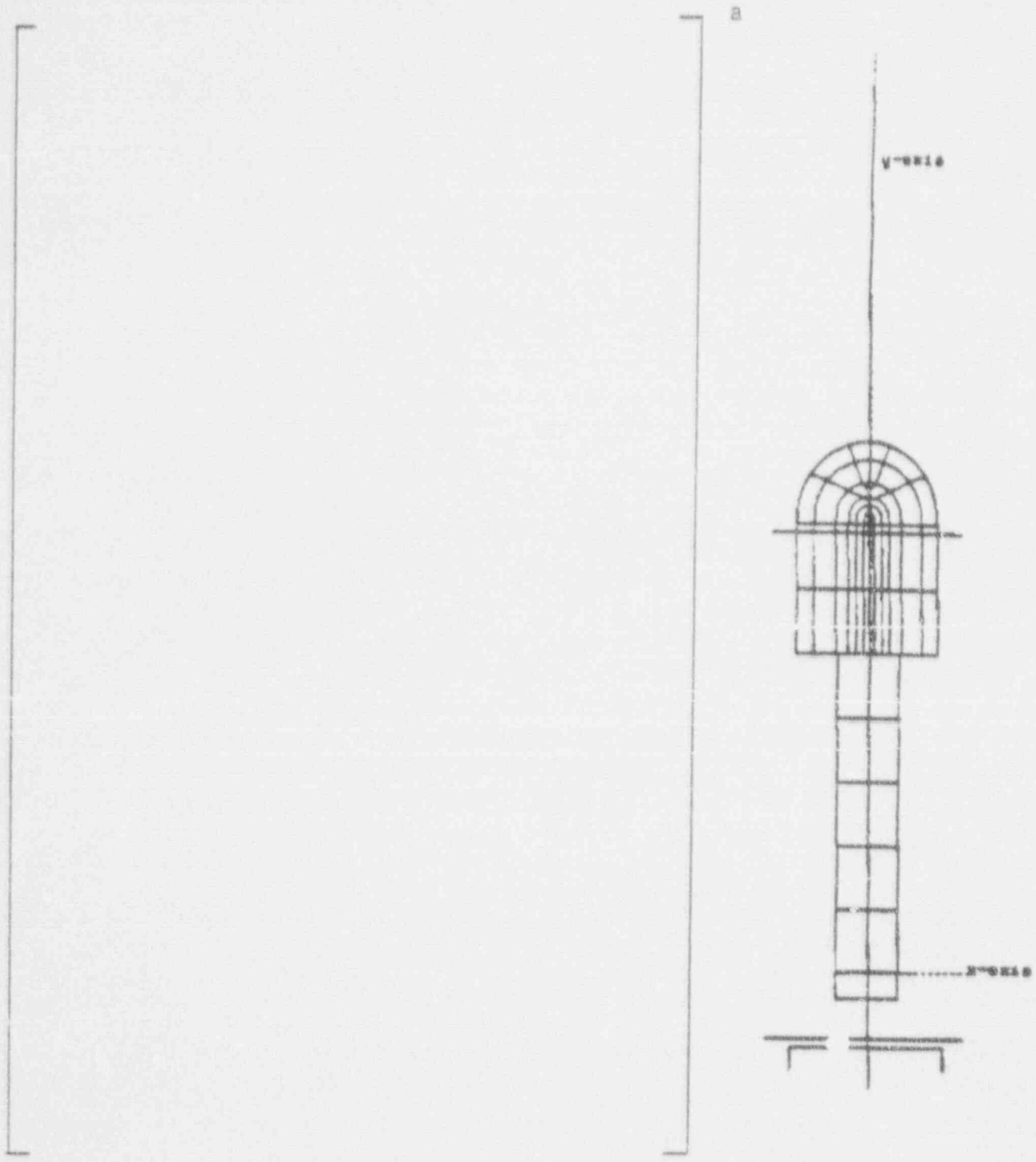
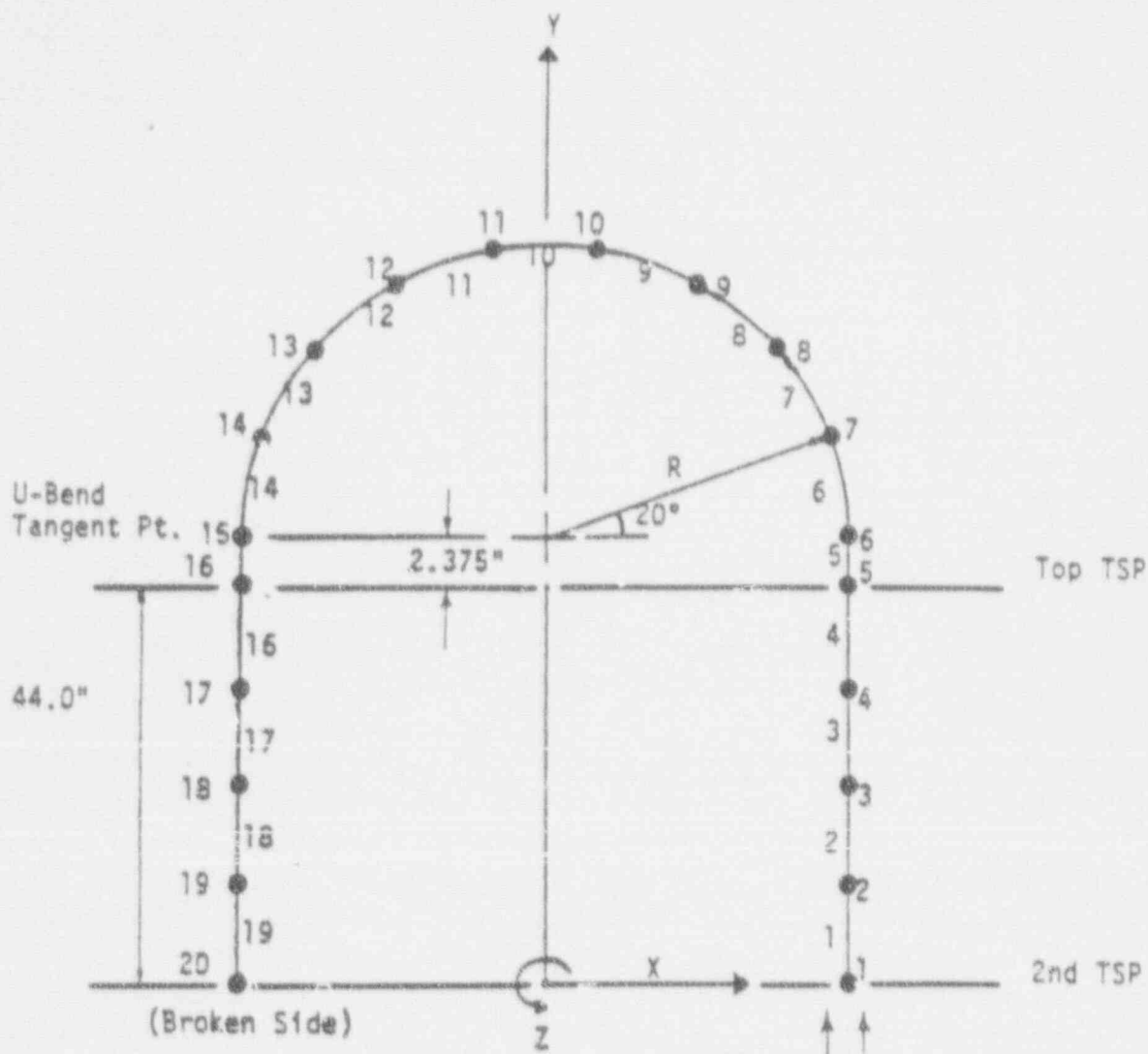


FIGURE 11-8
SERIES 51 SEISMIC FINITE ELEMENT MODEL GEOMETRY



Element Numbers Node Numbers

$R_1 = 53.25$ inch (largest)

$R_2 = 24.83$ inch (average)

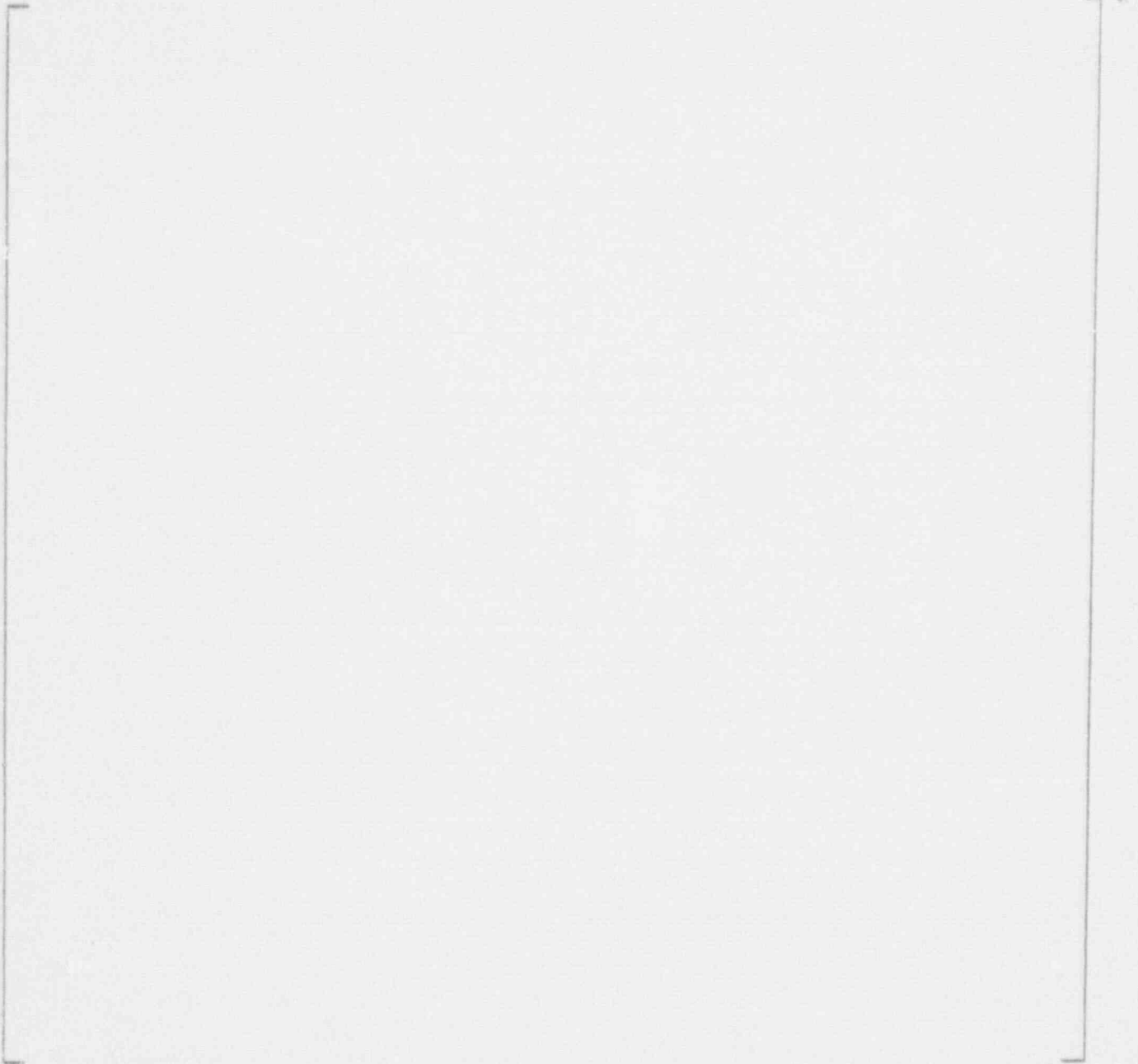
FIGURE 11-9
T/H TUBE MODEL FOR LOCA RAREFACTION WAVE ANALYSIS

FIGURE 11-10

a



FIGURE 11-11



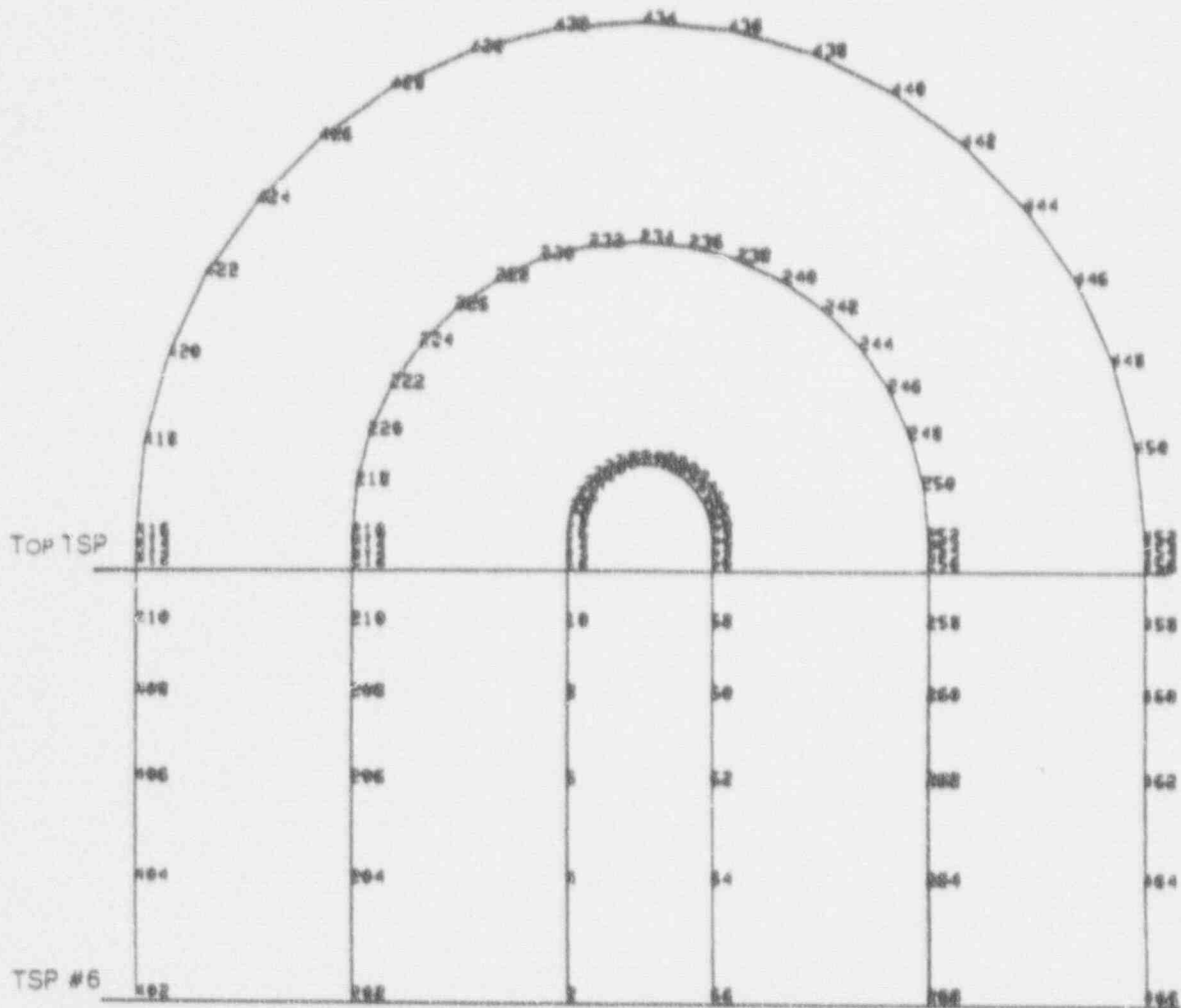
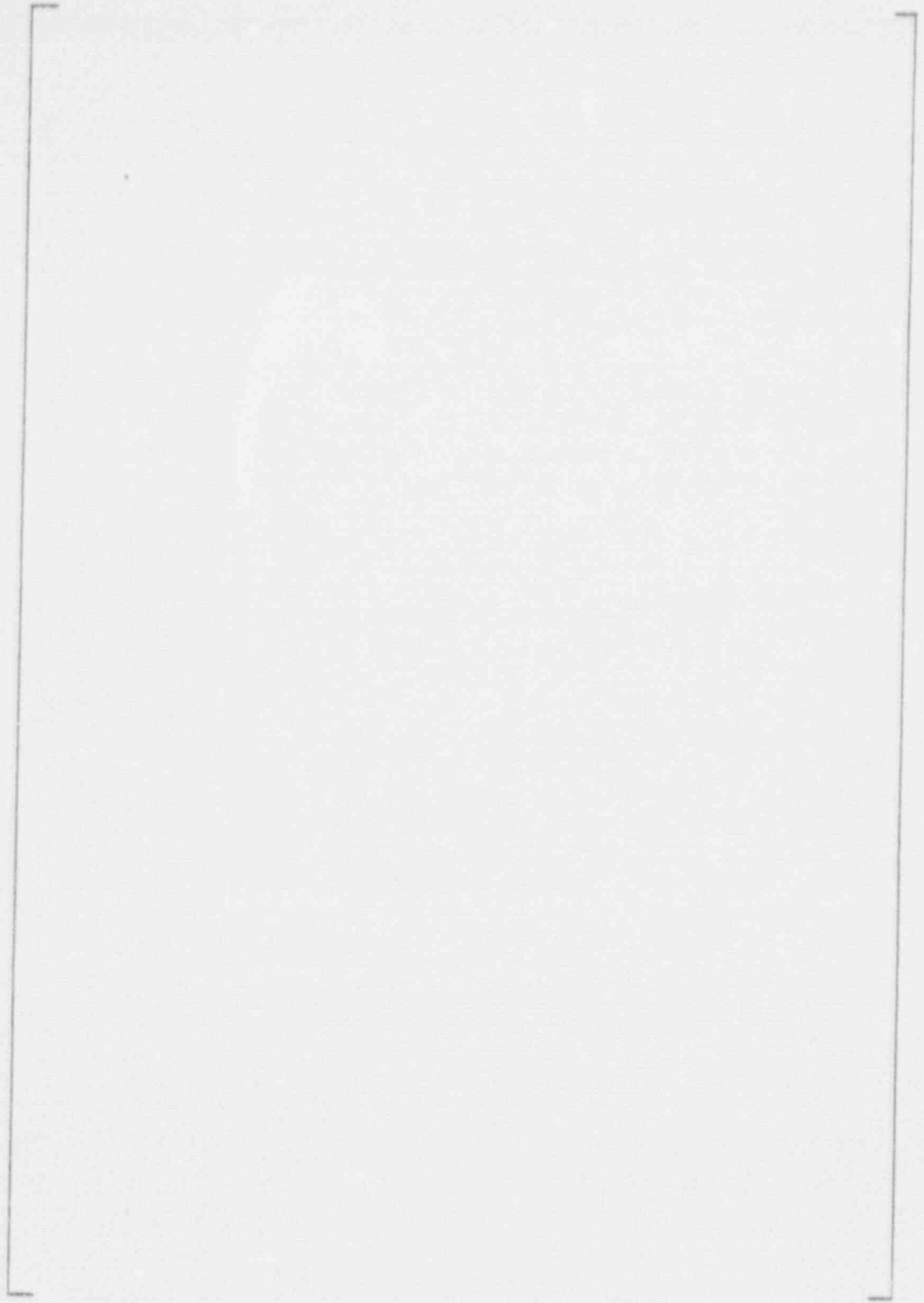


FIGURE 11-12
 FINITE ELEMENT MODELS FOR
 STRUCTURAL LOCA TIME HISTORY ANALYSIS



8

FIGURE 11-13
PLOT OF LOCA RAREFACTION FORCE DISTRIBUTION
STEAM GENERATOR INLET BREAK

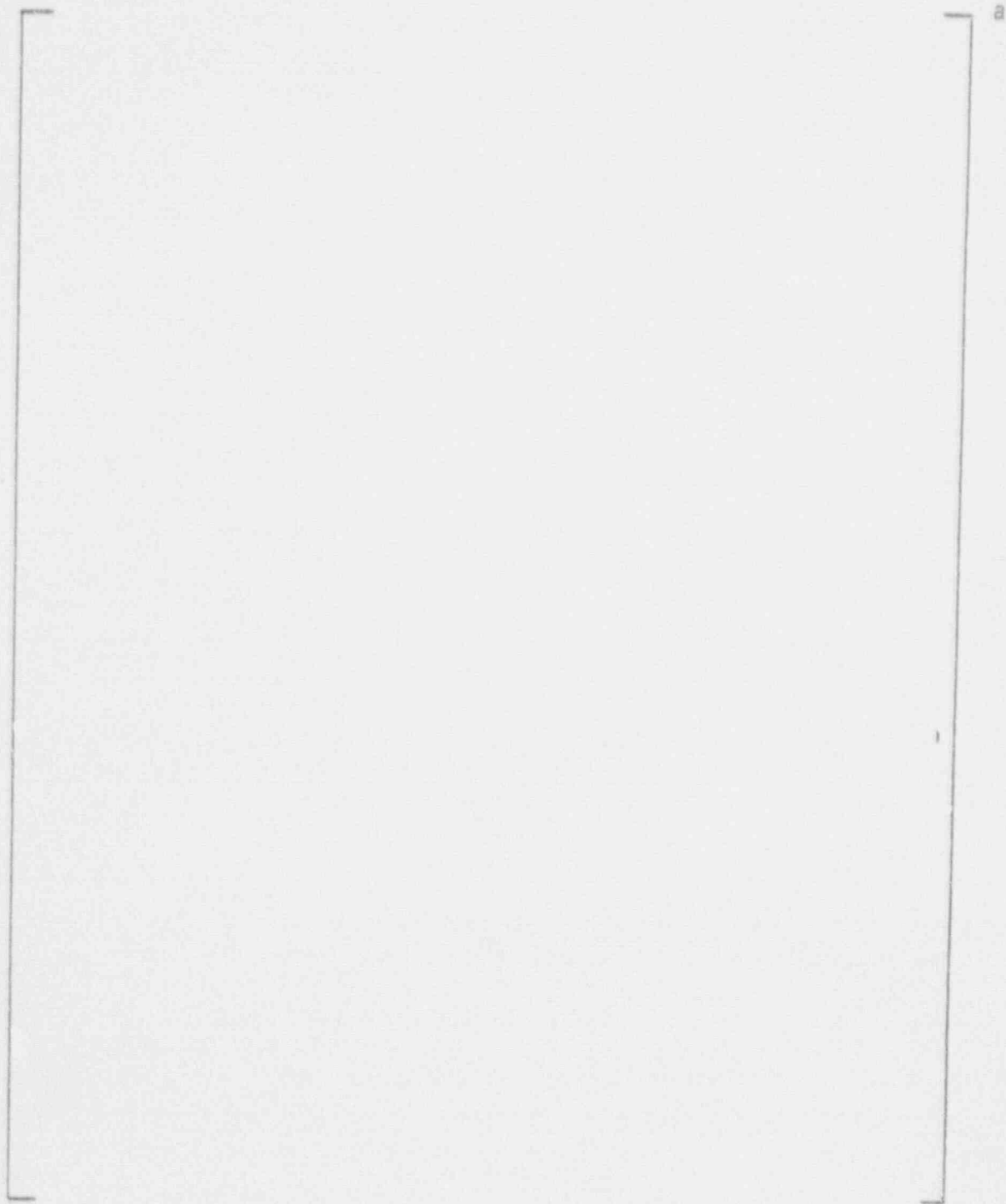


FIGURE 11-14
WEDGE GROUP ORIENTATION
LOOKING DOWN ON TSP

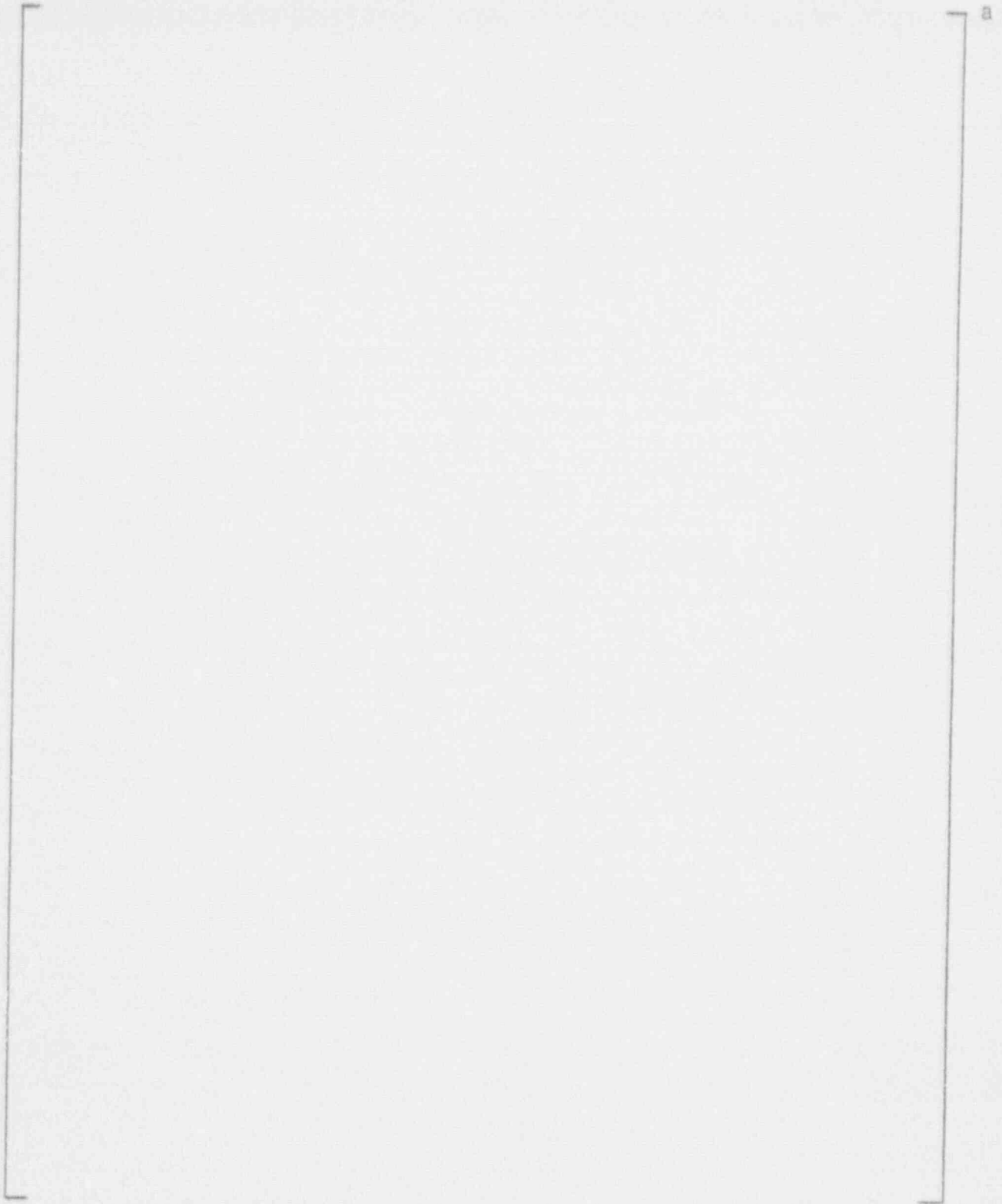


FIGURE 11-15
SUMMARY OF WEDGE LOAD DISTRIBUTION
TSP 2-7

FIGURE 11-16

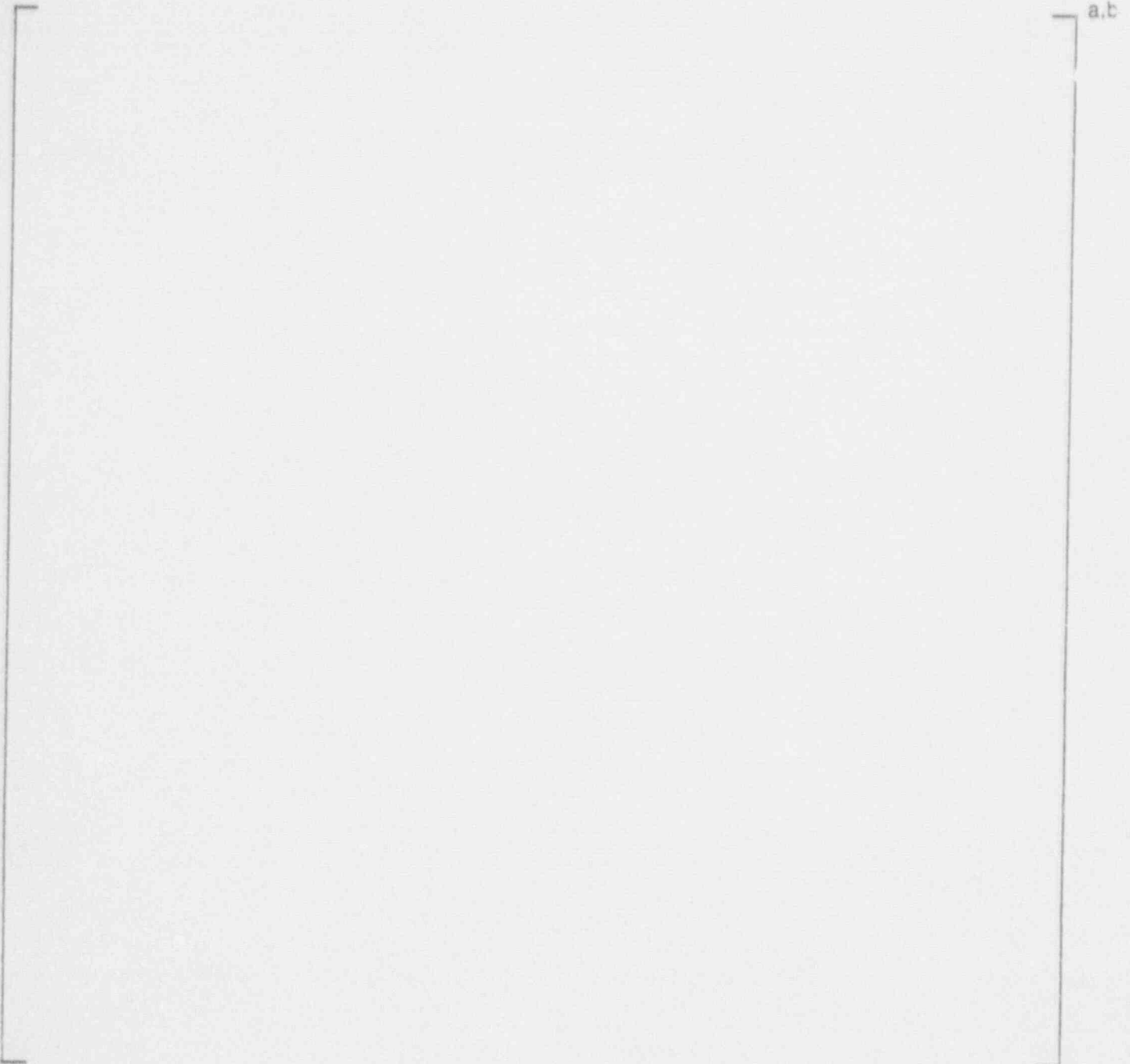
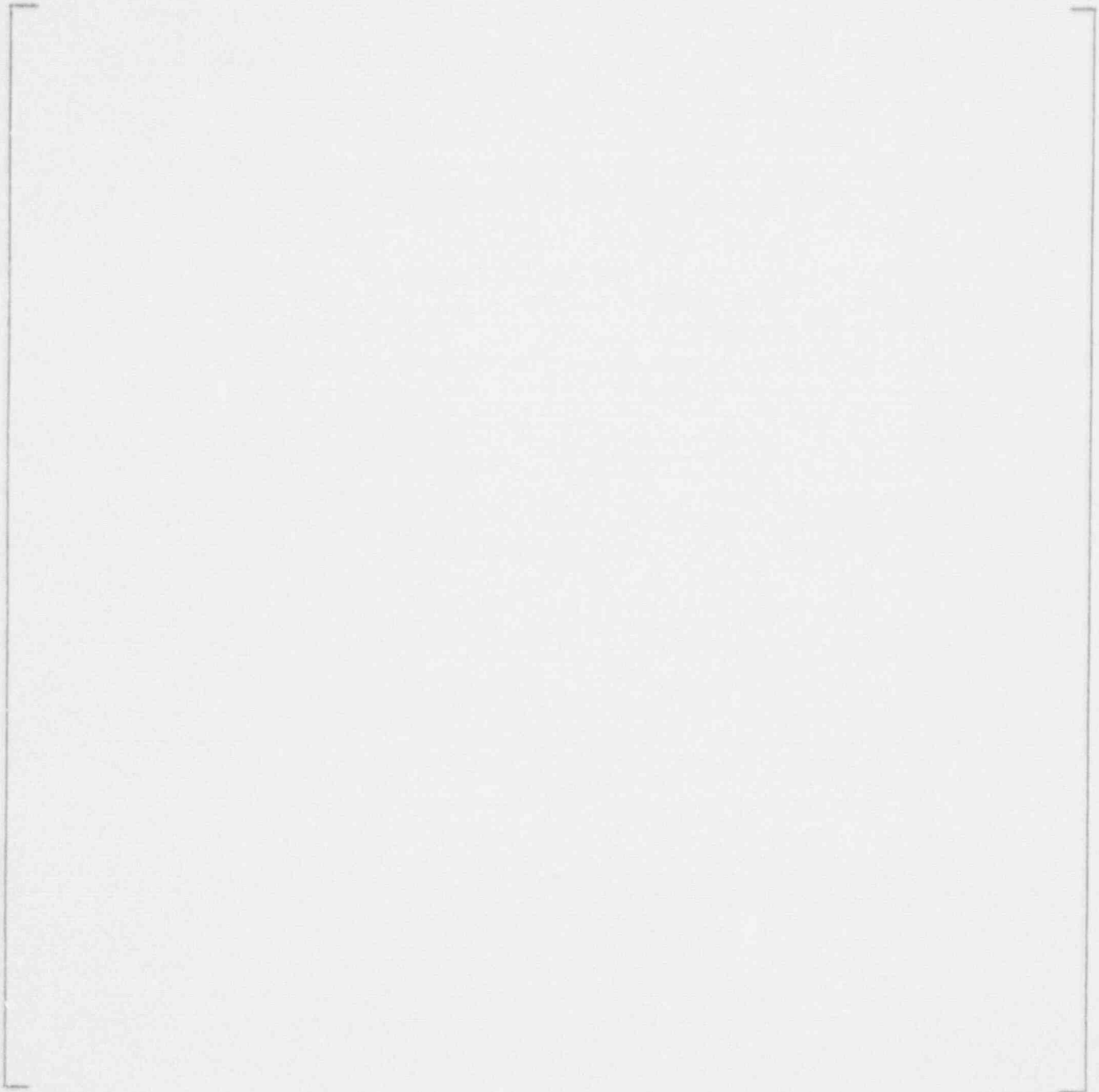


FIGURE 11-17

a,b



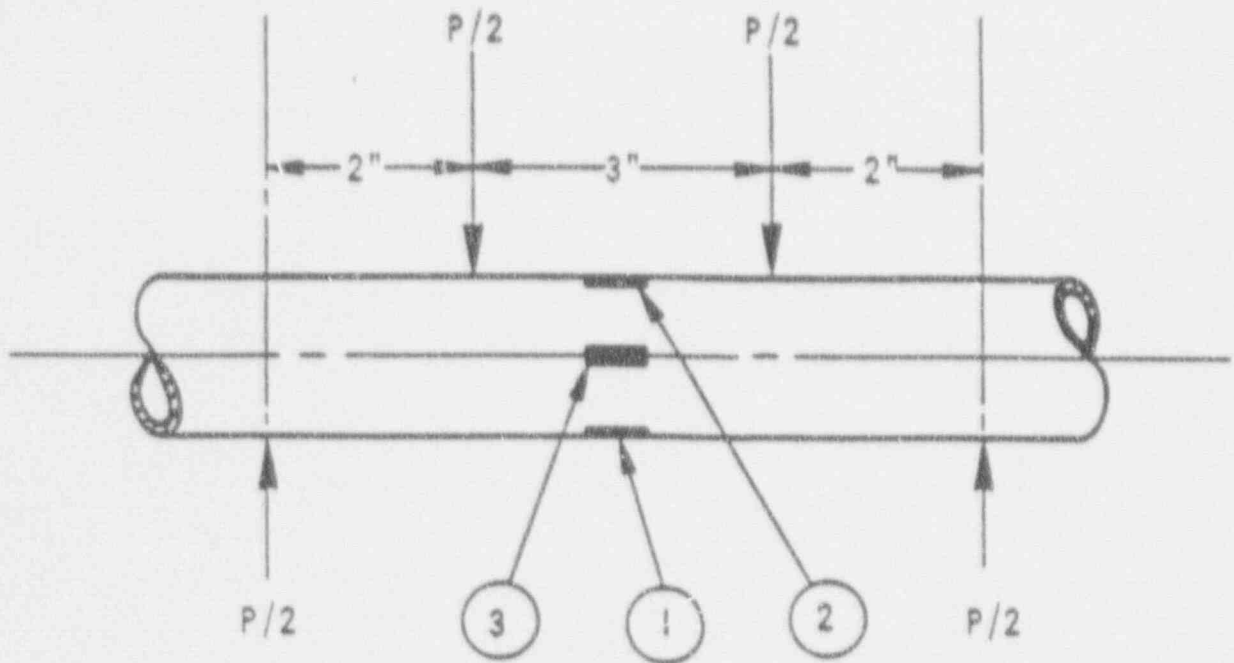
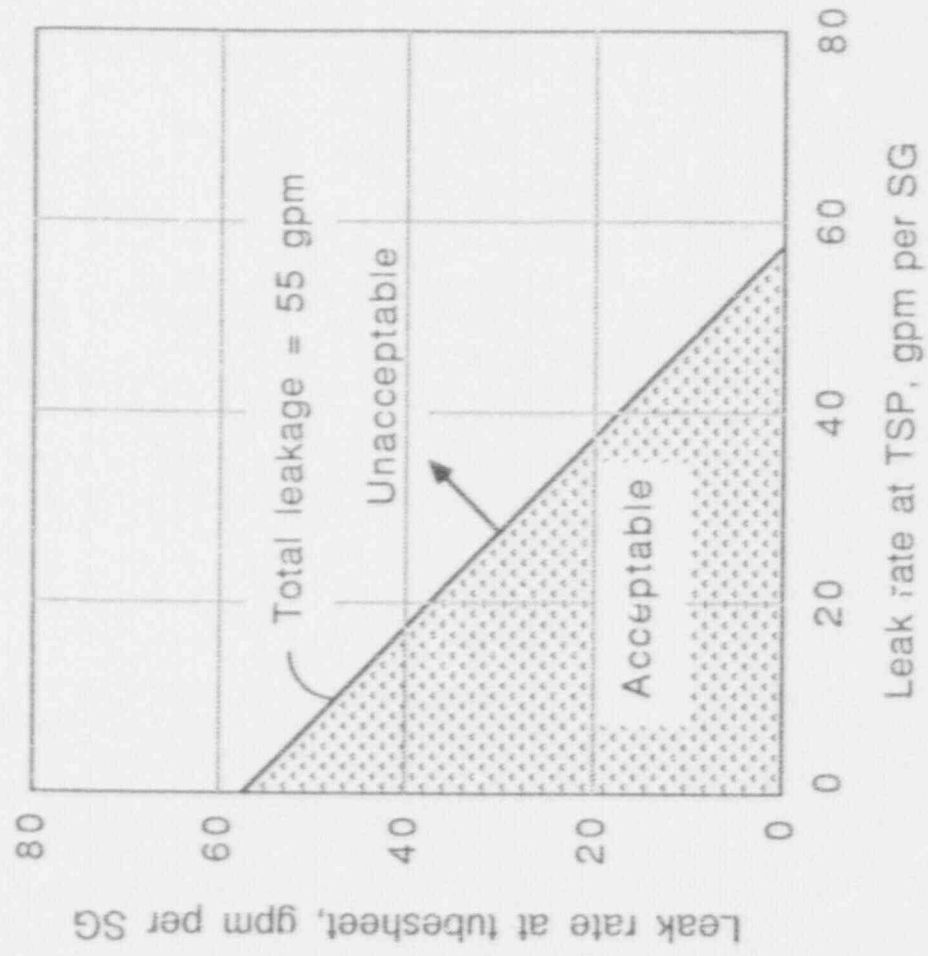


FIGURE 11-18
EXTERNALLY APPLIED BENDING LOAD AND
LOCATIONS OF THROUGH WALL SLOTS

FIGURE 11-19

Allowable Primary to Secondary Leakage During SLB



12.0 TUBE PLUGGING CRITERIA FOR ODSCC AT TSPS

This section integrates the results obtained from the prior sections to develop the technical basis for tube plugging criteria for ODSCC at TSPs.

12.1 General Approach to Plugging Criteria

The general approach taken to develop the tube plugging criteria for the Farley S/Gs includes:

- 1) Specifying conservative burst correlations based on free (uncovered) span ODSCC under accident conditions to demonstrate structural integrity.
- 2) Conservatively assuming open crevice conditions to maximize leakage potential.
- 3) Satisfying the R.G. 1.121 structural guidelines for tube burst margins by establishing a conservative structural limit on voltage amplitude that assures 3 times normal operating pressure differential for tube burst capability.
- 4) Satisfying the FSAR requirements for allowable leakage under accident conditions by demonstrating that the dose rate associated with potential leakage from tubes remaining in service is a small fraction of 10 CFR 100 limits.
- 5) Including considerations for crack growth and NDE uncertainties in both the structural assessment and leakage analysis.
- 6) Specifying a requirement to perform 100% BC inspection for all hot leg TSP intersections and all cold leg intersections down to the lowest cold leg TSP where ODSCC indications have been identified.

12.2 Test and Field Data Summary

The model boiler test data supporting the plugging criteria were discussed in Sections 8 and 9. To clarify the application of the laboratory data, the laboratory burst pressure and leak rate data used for the criteria development are given in Table 12.1. For 7 of the first 13 specimens of Table 12.1, leak rates were measured twice. After the initial measurements at normal operating conditions for these specimens, the generally tight model boiler TSP simulants were removed for supplemental testing. The subsequent eddy current measurements generally showed increased values and preceded the SLB leak rate and burst measurements and thus are the appropriate values for the burst and leakage correlations. Only the second test results are included in the data base of Table 12.1.

Field data supporting the criteria are given in Table 6.2 for pulled tube burst pressure/leak rate data, Table 6.3 for occurrences of operating leakage, Figure 6-1 for pulled tube bobbin coil voltage and depth and Figure 6-4 for typical field inspection data for tubes without operating leakage. The field data of Tables 6.2/6.3 and Figure 6-1 are combined in Figure 12-1 with the model boiler data of Table 12.1 to show an integrated

data base. Also shown in Figure 12-1 are the data points with leakage under SLB conditions. Only the data of Tables 6.2 and 12.1 are used in the voltage versus burst pressure and SLB leak rate correlations. The voltage/burst correlation is given in Section 9.6. The SLB leakage correlation and analysis model are developed in Sections 9.7 and 11.4. These data together with the Farley growth rates given in Section 5.3 and the NDE uncertainties developed in Section 8.8 define the data base used to develop the Farley tube plugging criteria for ODSCC at TSPs.

The actual in-service leakage from ODSCC at TSP intersections has been very limited on a world wide level. No leakage from ODSCC has been reported for U.S. plants with nondented tubes. This experience is consistent with a 40% depth repair policy, even if deeper defects cannot be precluded and may exist. Other countries following a policy similar to that currently used in the United States have reported only one leakage event. This event occurred at a Spanish plant and resulted from a pluggable indication that was missed during the previous outage.

In European countries with no repair criteria to prevent through-wall defects at TSP intersections, reported leakage events are low. In Belgium, no leakage has been observed at Plant K-1 (where all 3 steam generators are known to have been affected by ODSCC for a number of years). A 1990 leakage event reported for plant E-4 cannot be quantitatively correlated with the two detected leakers at the TSP level because of three leaking tubes attributed to primary water stress corrosion cracking (PWSCC) in the expansion zones (EZ) of roll transitions. In France, 11 units with ODSCC at TSP intersections have been operating for a significant period (more than five years for at least two units) without detectable leakage.

This insignificant in-service leakage from TSP ODSCC, even when no criteria are set to prevent through-wall defects, is likely to result from a combination of the following factors:

- o Crack morphology is such that wall penetration is not readily achieved (relatively long cracks are prevented from leaking by a thin ligament on the ID side and, even after penetration, the ID length remains substantially less than the OD length). Also, unbroken ligaments between the crack faces often tend to restrict the leakage path.
- o The small opening areas of through-wall cracks can get clogged easily by circulating corrosion products, impurities, or precipitates.
- o The crevice chemistry may block the leak path, either by corrosion product accumulation (leading to "packed crevices"), or by tube denting from the corroded TSP.

While this experience indicates that leakage from TSP-ODSCC is not an operational concern, consideration is given in Section 12.5 to tube leak-before-break (LBB) to deal with possible unanticipated leaks or cracks that might grow at a greater than expected rate and thus challenge the adequacy of the structural repair limit. Using the LBB methodology to reduce the probability of tube break to a negligible level also addresses the issue of a single large leaker (outside the predicated range) during postulated faulted loads.

12.3 Tube Plugging Criterion for Margins Against Tube Burst

The tube plugging criteria are developed to preclude free span tube burst if it is postulated that TSP displacement would occur under accident conditions. The plugging limits to provide R.G. 1.121 tube burst margins are developed in this section.

12.3.1 Voltage for Structural Requirement

Tube burst test results are given for pulled tubes in Table 6.2 and for model boiler specimens in Tables 9.1 and 12.1. The combined field and laboratory burst test results are evaluated in Section 9.6 to develop a correlation between bobbin coil voltage and burst pressure. This correlation was adjusted to account for operating versus test temperatures and for minimum material properties to obtain a reference burst pressure correlation at the lower 95% confidence level. The results given in Section 9.6 show that a bobbin voltage amplitude of []9 volts satisfies the R.G. 1.121 guidelines that burst pressure capability exceeds 3 times the normal operating pressure differential across the tube. Thus the structural requirement for the tube plugging criteria is a limit of []9 volts for the bobbin coil amplitude.

12.3.2 Allowance for NDE Uncertainty

The allowance for NDE uncertainties was developed in Section 8.8 as a 16% uncertainty. The 16% is applied at the voltage limit requiring tube plugging. To support this uncertainty, voltage calibration standards normalized to the reference laboratory standard are utilized for Farley S/G inspections applying the plugging criteria of this report. In addition, Appendix A provides data collection and analysis guidelines to be implemented upon application of the plugging criteria.

The probe wear standard of Appendix A is used to maintain probe centering capability within a 10% probe wear limit as described in Sections 8.4 and 8.8. Preliminary field experience has indicated some difficulties in applying the wear standard within the physical constraints of the channel head with the associated limited vertical height where bending of probe leads can influence probe centering. Pending further field experience with the probe wear standard, the probe wear limit has been increased to 15% for the EC data acquisition guidelines of Appendix A. The total NDE uncertainty is conservatively increased to 20% for development of the plugging limits in Section 12.3.4.

12.3.3 Allowance for Crack Growth

Voltage growth rates for the Farley S/Gs are developed in Section 5.3 and compared with other plant data in Figure 6-12. Growth rates are defined in terms of percent growth per cycle and are applied at the voltage amplitude requiring tube plugging. The Farley percent growth rates from historical data are shown to decrease with increasing BOC (beginning of cycle) amplitude while data from a European plant indicates percent growth may be essentially independent of amplitude. It is thus conservative to assume percentage growth is independent of BCC amplitude and to use overall average growth from Farley operating experience for the growth rate allowance in the plugging limit

Table 5.4 summarizes voltage growth rates. For additional margin, the growth rates obtained by setting negative changes in measured voltages between cycles to zero is applied to obtain plugging limits. This process conservatively ignores negative fluctuations in voltage growth while keeping positive fluctuations in the average growth rate calculation. Overall, the more random positive and negative fluctuations would be expected to average out to approximately zero in the average growth.

It is seen from Table 5.4 that zeroing negative growth tends to increase implied growth rates by 5 to 13%. Farley growth rates have decreased since 1987 with averages over the last 2 cycles of 40% and 37% for Units 1 and 2. Growth rates over the last operating cycle were 37% and 29%.

To provide margin for variations in future cycles, an average growth allowance of 50% per cycle is conservatively applied to establish tube plugging limits. Average growth rates are considered to provide an adequate allowance for growth in satisfying R.G. 1.121 structural guidelines for burst pressure capability of three times normal operating pressure differential. Per R.G. 1.121, an allowance for NDE uncertainties is included in developing the tube plugging limit from the structural requirement for tube burst as shown below in Section 12.3.4. Allowances for growth uncertainties are addressed by demonstrating large margins against burst at SLB conditions as described in Section 12.4.

12.3.4 Tube Plugging Criterion

The structural voltage limit must be reduced by the allowances for crack growth and NDE uncertainty to obtain the voltage limit for tube plugging or repair consistent with R.G. 1.121 guidelines for margins against tube rupture. This can be expressed as:

$$V_{RL} + V_{NDE} + V_{CG} = V_{SL} \quad (12-1)$$

where: V_{RL} = voltage limit for tube repair or plugging,
 V_{NDE} = NDE voltage measurement uncertainty,
 V_{CG} = voltage growth rate per cycle, and
 V_{SL} = voltage structural limit from the burst pressure versus BC voltage correlation.

The NDE voltage uncertainty and voltage growth rate terms are provided as a percentage of measured BC voltage (% V_{NDE} and % V_{CG}). Using V_{RL} as the maximum measured BC voltage to be left in service, V_{NDE} and V_{CG} are:

$$V_{NDE} = V_{RL} \times \%V_{NDE}/100, \text{ and}$$

$$V_{CG} = V_{RL} \times \%V_{CG}/100.$$

Using these expressions for V_{NDE} and V_{CG} , Eq. (12-1) can be rewritten as:

$$V_{RL} = V_{SL}/(1 + \%V_{NDE}/100 + \%V_{CG}/100). \quad (12-2)$$

Values for VSL, %VNDE and %VCG are given in Sections 12.3.1 to 12.3.3 above and are VSL = []⁰ volts, %VNDE = 20 and %VCG = 50. Substituting these values into Equation 12-2 gives

$$VRL = VSL/1.70 = 3.6 \text{ volts} \quad (12-3)$$

An alternate summary of the development of the tube plugging or repair voltage limit is given in Table 12.2. The criterion for tube plugging is that the measured bobbin coil voltage exceed 3.6 volts. This criterion is applied independent of the bobbin coil indicated crack depth.

12.4 SLB Tube Burst and Leakage Evaluation

This section describes analysis requirements for both margins against tube burst at SLB conditions and for demonstrating acceptable SLB leakage. In Section 11.3, it was shown that the bounding accident condition for primary to secondary leakage is the SLB event and that the allowable SLB leak limit is 55 gpm.

12.4.1 Margins Against Burst at SLB Conditions

Margins against burst at SLB conditions are demonstrated by considering the through wall crack length required for tube burst and by a Monte Carlo analysis considering cracks left in service, growth and the voltage burst correlation. The tube repair limits developed above and shown in Table 12.2 are based upon currently available voltage growth rates at low voltage levels compared to the tube repair limits. To minimize the potential future impact of new growth rates on the repair limits, percentage growth rates have conservatively been applied at the higher voltage levels even though current data show a decreasing percentage growth at higher voltage amplitudes. As new growth rate data is obtained upon implementation of the repair limits of this report, the potential implications of these growth data will be assessed following each outage. The development of repair limits in Section 12.3 and Table 12.2 will be reviewed for acceptability of the repair limits. In addition, margins against burst at SLB conditions will be evaluated following each outage for the distribution of indications left in service. The burst analyses will apply Monte Carlo analyses as described below to calculate the probability of tube burst at SLB conditions. If the new growth data indicates that either the repair limit of 3.6 volts should be revised or that the SLB burst probability approaches or exceeds a guideline value of 2.5×10^{-2} for a single tube burst, these results will be reported to the NRC together with an assessment of potential revised tube repair limits.

Burst pressure versus axial crack length data from multiple sources are shown in Figure 12-2 as taken from EPRI Report NP-6864-L. The methods to perform the burst tests for the upper figure curves differ in the techniques for applying the required pressure for tube burst. In testing of tubes with through-wall slits [

] a.c

ja.c. However, recent tests in Belgium have demonstrated that for axial cracks, foil reinforcement of seals leads to the same burst pressures as in tests with very high capacity pumps and no seal whatsoever. This is illustrated in Figure 12-2. The Westinghouse and British curves do not use any seal reinforcement and lie below the Belgian burst curve. The burst data without any sealant system falls along the burst curves where seal reinforcement was used. Therefore it is appropriate to remove the conservatism of the Westinghouse axial crack burst curve and use the Belgian burst curve. The Belgian burst curve for 7/8 inch diameter tubing is shown in the upper plot of Figure 12-2. It is seen that a throughwall crack length of 0.84 inch is required for burst at SLB conditions. The TSP height is 0.75 inch so that even a uniformly throughwall crack equal to the length of the TSP, which is extremely unlikely, would not burst under accident conditions.

The probability of tube burst at SLB conditions can be calculated by Monte Carlo methods for the distribution of bobbin voltage indications left in service. The Monte Carlo methods are described in Sections 9.6 and 12.4.2. The BOC indications left in service are projected to the EOC utilizing distributions for NDE uncertainties and voltage growth per cycle. For each EOC Monte Carlo sample of bobbin voltage, the voltage/burst correlation is randomly sampled to obtain a burst pressure. The probability of tube burst at SLB is obtained as the sum of samples resulting in burst pressures less than the SLB pressure differential of 2650 psi divided by the number of times the distribution of indications left in service is sampled.

The probability of tube burst was evaluated by Monte Carlo analyses for a BOC 3.6 volt indication at the tube repair limit. Based on 100,000 Monte Carlo samples for the EOC voltage, the EOC SLB burst probability per indication was obtained as 3×10^{-5} . Similarly, 100,000 Monte Carlo samples at 2.0 and 2.5 BOC volts resulted in no occurrences of burst at SLB conditions. The results indicate that the probability of burst at SLB for a distribution of indications left in service would be primarily influenced by indications of about 3.0 to 3.6 volts. The Farley-2 distribution of indications from the last outage (Figure 5-5), if a 3.6 volt plugging limit were applied, would yield a probability of burst at SLB of about 3×10^{-5} . As noted above, the probability of burst at SLB conditions will be evaluated following each outage for indications left in service to confirm the adequacy of the tube repair limits as new growth rate data are developed.

12.4.2 SLB Leakage Analysis

The SLB leakage limit of 55 gpm per S/G was developed in Section 11.3. It is required that SLB leakage analysis be performed following each inspection to demonstrate that the potential SLB leakage for tubes left in service is less than the 55 gpm limit. If combined with alternate plugging criteria for tubesheet region indications, the total SLB leakage must be evaluated against the requirements developed in Section 11.3.

Laboratory measurements on cracked tubes indicate no significant leakage for tubes with through wall crack lengths up to about 0.2 inch at normal operating conditions and about 0.15 inch for SLB conditions. Typical examples with SLB leakage <1 gpd include Farley Tubes R4C73 (0.18" TW), and R21C22 (0.15" TW) and Model Boiler Specimens

509-3 (0.16" TW) and 535-1 (0.11" TW). With OD to ID crack length ratios > 2 found for OD3CC, tubes with through wall cracks and measurable leakage even in the laboratory can be expected to have significant voltage amplitudes. Thus a threshold voltage value can be expected for measurable leakage.

For normal operating conditions, the data show: a 6.5 volt model boiler specimen with operating leakage; a field leaker at 7.7 volts; no reported field leakage below this 7.7 volt leaker including a substantial number of field indications below about 4 volts; and no measurable leakage for the Farley-2 pulled tube at 2.8 volts. Thus a reasonable judgment is that the voltage amplitude for the leakage threshold at operating conditions is between 2.8 and 6.5 volts. Furthermore, there is significant likelihood that the voltage threshold for operating plant leakage detectability would exceed 6 volts.

For SLB conditions Farley-2 tube R4C73 at 2.8 volts had a very small leak rate of 0.17 l/hr (< 0.001 gpm) while pulled tubes at 1.44, 1.82, 2.3, 2.7 and 3.83 volts, but not through wall penetration, had no leakage. The lowest voltage found for a pulled tube with a through wall crack (0.01 inch long) was 1.9 volts (3/4 inch diameter tube) and this tube had no detectable leakage in the laboratory. Figure 12-3 shows the pulled tube and model boiler data for SLB leakage for various bobbin voltage intervals. Data obtained from 3/4 inch diameter and 7/8 inch diameter model boiler specimens are included in Figure 12-3. From this figure, it is seen that none of 32 specimens below 2.0 volts have shown SLB leakage while only 1 of 8 specimens between 2.0 and 3.2 volts had any leakage (a negligible leak rate of < 0.001 gpm). Thus a voltage amplitude of 3.2 volts appears to be a reasonable lower voltage threshold for significant SLB leakage. This would correspond to a BOC cutoff voltage of about 2.0 volts based upon allowances at 95% cumulative probability for NDE uncertainty (0.4 volt) and growth (0.8 volt, Figure 5-15). Monte Carlo analyses using 100,000 samples were run for a BOC indication of 2 volts. The EOC results showed no samples with burst pressures below the SLB pressure differential of 2650, which implies a burst probability at least less than 10^{-5} per BOC 2 volt indication. Similarly, Monte Carlo analyses indicate that 4000 indications at 2.0 volts would be required to obtain a leak rate of 1 gpm (much less than the allowable limit of 55 gpm) at 90% cumulative probability of the Monte Carlo results. Thus the indications of 2.0 volts or less can be omitted from the Monte Carlo analysis and the burst probabilities and leak rates would be dominated by the larger indications left in service that approach the tube plugging limit. The 2.0 volt cutoff for SLB analyses is provided to facilitate completion of the SLB analyses during an inspection outage to expedite finalization of tube plugging lists. Preparation and checking of the voltage indications and completion of the analyses can be expedited by not including the potentially large number of indications below the 2.0 volt cutoff value.

Guidelines for performing the Monte Carlo analysis include a minimum of 100,000 samples for each indication left in service above the voltage threshold value of 2.0 volts and voltage growth rates developed from the prior operating cycle. Voltage growth rates are to be developed as a cumulative probability of the change in voltage per cycle or per EFPY. Figure 5-15 shows examples of voltage growth distributions. The growth rate distribution should be based on the largest BOC voltage amplitudes approaching the tube plugging limit to be representative of growth of the largest indications left in service. For example, the largest 200 voltage EOC indications or all indications above about 2.0 volts (1.6 volts below the tube plugging limit) for tubes left in service at the beginning

of the prior cycle can be used to develop the cumulative distribution of growth rates. The NDE uncertainty distribution for the Monte Carlo analysis is developed in Section 8.8.

For each EOC Monte Carlo sample voltage, the SLB leak rate vs. voltage correlation is randomly sampled to obtain a sample leak rate. The leak rates from each Monte Carlo sample are summed to obtain a distribution of total leak rate per SG for the indications left in service. The leak rate obtained at 90% cumulative probability from the Monte Carlo generated distribution will be utilized to compare with the 55 gpm allowable leak limit. A 90% cumulative probability provides adequate conservatism based on the following considerations: the SLB leak rate correlation is based on model boiler specimens which show higher leak rates than available pulled tube data; 4 of 13 model boiler specimens between 5 and 9 volts (bounding EOC voltages) show no leakage; and margins/conservatisms are included in developing the allowable SLB leak limit of 55 gpm as described in Section 11.3.

If it is found that the potential SLB leakage for degraded intersections planned to be left in service exceeds 55 gpm per S/G, then additional tubes would be plugged to reduce SLB leakage potential to below 55 gpm. As also noted in Section 11.4 applying the current leakage model to the distribution of indications found at the last Farley outages would yield a maximum of 0.4 gpm per S/G at SLB conditions (at the 90% cumulative probability level) which is well below the 55 gpm limit per S/G.

12.5 Operating Leakage Limit

R.G. 1.121 acceptance criteria for establishing operating leakage limits are based on leak before break (LBB) consideration such that plant shutdown is initiated if the leakage associated with the longest permissible crack is exceeded. The longest permissible crack is the length that provides a factor of safety of 3 against bursting at normal operating pressure differential. As noted above, a voltage amplitude of []⁹ volts for typical ODSCC cracks corresponds to meeting this tube burst requirement at the lower 95% confidence level on the burst correlation. Alternate crack morphologies could correspond to []⁹ volts so that a unique crack length is not defined by the burst pressure to voltage correlation. Consequently, typical burst pressure versus through wall crack length correlations are used below to define the "longest permissible crack" for evaluating operating leakage limits.

The CRACKFLO leakage model has been developed for single axial cracks and compared with leak rate test results from pulled tube and laboratory specimens. Fatigue crack and SCC leakage data have been used to compare predicted and measured leak rates as shown in Figure 12-4. Generally good agreement is obtained between calculation and measurement with the spread of the data being somewhat greater for SCC cracks than for fatigue cracks. Figure 12-5 shows normal operation leak rates including uncertainties as a function of crack length.

The through wall crack lengths resulting in tube burst at 3 times normal operating pressure differentials (4380 psi) and SLB conditions (2650 psi) are about []^a, respectively, as shown in Figure 12-2. Nominal leakage at normal operating conditions for these crack lengths would range from about []^a

[]^a would cause undue restrictions on plant operation and result in unnecessary plant outages, radiation exposure and cost of repair. In addition, it is not feasible to ensure LBB for all tubes by reducing the leak rate limit. Crevice deposits, presence of small ligaments and irregular fracture faces can, in some cases, reduce leak rates such that LBB cannot be ensured for all tubes by lowering leak rate limits.

An operating leak rate of 150 gpd (~0.1 gpm) will be implemented in conjunction with application of the tube plugging criteria. As shown in Figure 12-5, this leakage limit provides for detection of []

[]^a. Thus, the 150 gpd limit provides for plant shutdown prior to reaching critical crack lengths for SLB conditions at leak rates less than a -95% confidence level and for 3 times normal operating pressure differentials at less than nominal leak rates.

The tube plugging limits coupled with 100% inspection at affected TSP locations provide the principal protection against tube rupture. The 150 gpd leakage limit provides further protection against tube rupture. In addition, the 150 gpd limit provides the capability for detecting a rogue crack that might grow at much greater than expected rates and thus provides additional protection against exceeding SLB leakage limits.

12.6 Supplemental Requirements for Implementation of the Plugging Criteria

Upon implementation of the tube plugging criterion and associated limits on operating SLB leakage, additional requirements as noted in this section are to be implemented.

A 100% bobbin coil inspection is required for all hot leg TSP intersections and for cold leg intersections down to at least the lowest TSP with ODSCC indications. This requirement provides confidence that all ODSCC indications which could contribute to SLB leakage are identified for the leak rate evaluation. If <100% inspection is performed below cold leg TSP intersections with previously identified ODSCC indications, and indications at TSPs are identified, the inspection must be extended to 100% of the affected TSP or the 40% depth limit for tube plugging must be applied for all indications.

An RPC inspection is required to establish that the principal indications can be characterized as ODSCC. For Farley, the RPC inspection will be performed for all ODSCC indications left in service that exceed a 1.5 volt bobbin coil amplitude. The RPC results are to be evaluated to establish that the principal indications can be characterized as ODSCC. As discussed in Section 8.1, bobbin coil indications <1.5 volt for degradation other than ODSCC are not expected to impact tube integrity, for potential tube degradation at Farley. Thus, characterization of the type of degradation above 1.5 volts by RPC is adequate to determine appropriate plugging limits. If indications other than ODSCC are identified, these indications should be evaluated against a 40% depth requirement for tube plugging. The >1.5 volt bobbin coil amplitude for RPC inspection also is below the 3.2 volt threshold below which SLB leakage is expected to be negligible as noted in Section 12.4.2. Once an indication at a TSP is confirmed to be ODSCC, reconfirmation by RPC inspection at alternate refueling outages is acceptable.

Branching of cracks in the circumferential direction is acceptable within the tube plugging criterion. Circumferential branching has been found in pulled tubes and model boiler specimens. Examples are summarized in Table 12.3. These data are included in the voltage/burst correlation. The branching includes some IGA effects as well as SCC. At the higher voltages (>20 volts) summarized in Table 12.3, the effects of branching and tearing of ligaments between axial microcracks can be seen in the orientation of the burst crack opening. Even for these tubes, the burst pressures are comparable to burst pressures for indications without circumferential branching. The burst tests tend to indicate that at very high voltage levels (model boiler specimens 543-1, 543-2 with voltages >100 volts), the branching effects may result in reduced burst capability compared to uniquely axial cracks. Thus voltage levels much higher than the plugging limits of this report are required before the circumferential branching effects influence tube integrity for ODSCC at TSPs. Thus no special inspection techniques are required to inspect for indications with circumferential branching.

Circumferential cracks at TSPs have only been found in one Westinghouse plant that has significantly more and larger dents than found in the Farley S/Gs. No circumferential cracks have been found at the TSPs in the Farley SGs by either RPC inspection (RPC) or by pulled tube examination and none are expected based on operating experience in other plants. The RPC inspections required for indications >1.5 volt are adequate to continue to monitor for potential occurrence of circumferential cracks. RPC resolution is considered adequate to define separation between circumferential and axial cracks. It is not necessary to resolve potential circumferential cracks between closely spaced axial cracks since circumferential cracking is not anticipated at Farley. Any initial identification of circumferential cracks by RPC should be based upon a well defined circumferential indication as contrasted to inadequate RPC resolution. If a well defined circumferential indication should be identified at the TSPs in the Farley S/Gs, guidelines for RPC interpretation would be reviewed at that outage and consideration given to supplemental UT inspection for resolution of the degradation mode. Tubes with identified circumferential indications would be plugged or repaired.

12.7 Summary of Tube Plugging Criteria

As developed in the sections above, the plugging criteria for ODSCC at TSPs can be summarized as follows:

Tube Plugging Criterion

Tubes with bobbin coil indications exceeding 3.6 volts will be plugged or repaired.

SLB Leakage Criterion

Predicted SLB leak rates from tubes left in service must be less than 55 gpm for each S/G, including considerations for NDE uncertainties and ODSCC growth rates.

Inspection Requirements

A 100% bobbin coil inspection shall be performed for all hot leg TSP intersections and all cold leg intersections down to the lowest cold leg TSP with ODSCC indications.

All tubes with bobbin coil indications >1.5 volts at TSP intersections shall be inspected using RPC probes. The RPC results shall be evaluated to support ODSCC as the dominant degradation mechanism. Indications at TSPs confirmed to be ODSCC shall be reinspected by RPC at alternate refueling outages (once in two outages) for reconfirmation as ODSCC.

Operating Leakage Limits

Plant shutdown will be implemented if normal operating leakage exceeds 150 gpd per S/G.

Exclusions from Tube Plugging Criterion

Tubes with RPC indications not attributable to ODSCC and circumferential indications shall be evaluated for tube plugging based on a 40% depth limit.

Table 12.1

7/8-Inch Diameter Model Boiler Specimens: Test Data Summary

No.	Model Boiler Spec. #	Bobbin Coll.		RPC		Leak Rate (l/hr)		Burst Press. psi	Destructive Exam.	
		Volts	% Depth	Volts	# Cracks	N. Op. ΔP	SLB ΔP		Length - inch Max.	Thruwall
1	500-1									
2	509-2									
3	509-3									
4	510-1									
5	525-1									
6	528-1									
7	528-2									
8	532-1									
9	532-2									
10	533-4									
11	535-1									
12	536-1									
13	542-4									
14	543-1									
15	543-2									
16	543-4									
17	555-3									
18	557-1									
19	557-2									
20	557-4									
21	558-1									
22	568-1									
23	568-2									
24	568-4									
25	568-6									
26	571-1									
27	574-4									
28	576-2									
29	576-4									
30	605-2									
31	605-3									
32	607-3									
33	607-4									

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- * For specimens without throughwall penetration, maximum depth of penetration is listed.
- ** Destructive examination and review of RPC data shows that only 1 crack has a significant response that contributes to the bobbin signal.
- *** Tube not burst tested due to physical limitation of specimen.

Table 12.2

Tube Plugging Limits to Satisfy Structural Requirements

<u>Item</u>	<u>Volts</u>	<u>Basis</u>
Maximum Voltage Limit to Satisfy Tube Burst Structural Requirement	3.6	Section 9.6, Figure 9-2, Burst Pressure vs. Voltage Correlation at -95% confidence level.
Allowance for NDE Uncertainty	-0.8 (20%)(1)	Section 8.8 shows development of 16% uncertainty at 90% cumulative probability. Conservatively increased to 20% to establish plugging limits
Allowance for Crack Growth Between Inspections	-1.8 (50%)(1)	Section 5.3, Table 5.4 shows average growth/cycle of 37% and 29% for Units 1 and 2. Allowance increased to 50% of Tube Plugging Limit to provide conservative margin for variations in future cycles.
Tube Plugging Voltage Limit	3.6	
o Acceptable Limit to Meet Structural Requirement		

Note

1. Voltage percentage allowances for NDE and growth/cycle applied to Tube Plugging Voltage Limit of 3.6 volts.

Table 12.3

Examples of Circumferential Branching for ODSCC at TSPs

<u>Plant/Tube</u>	<u>R.C. Voltage</u>	<u>Burst Pressure</u> <u>Pulled Tubes</u>	<u>Destructive</u> <u>Exam. Figures</u>	<u>Circumferential Branching</u> <u>Description</u>
Pulled Tubes				
A-2:R38C46	[]	4-11 to 4-13	Numerous microcracks of axial and circumferential orientation
A-2:R31C46			4-1 to 4-2	Minor circumferential branching
B-1:R4C61			4-3 to 4-4	Short circumferential cracks with IGA patches
Model Boiler Specimens				
528-2	[]	10-18 to 10-21	Burst opening includes circumferentially oriented ligaments
532-1			10-22 to 10-25	Burst opening includes minor circumferential orientation
532-2			10-26 to 10-29	Irregular burst opening involving tearing of interconnecting ligaments
535-1			10-30 to 10-31	Example of minor branching within tube wall
555-3			10-32 to 10-34	Burst involves irregular pattern with turn connecting ledges between cracks

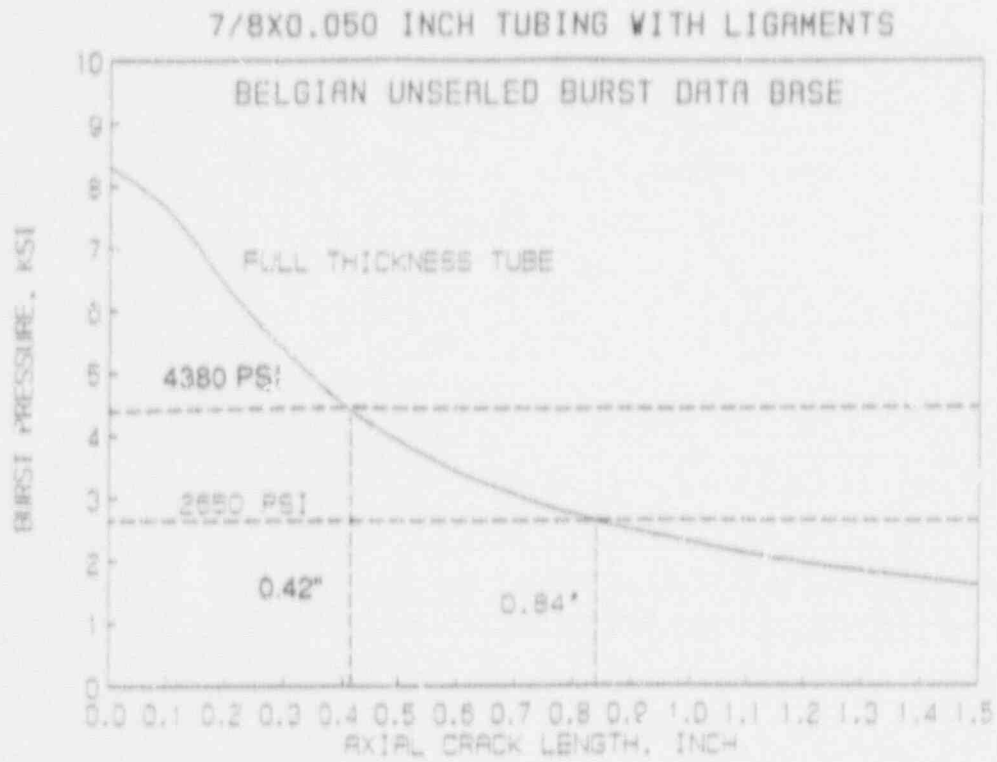
Figure 12-1
Field and Model Boiler Data Base: Leakage Under SLB
Conditions



Chart Figure 12-1 2/11/92

Figure 12-2

Burst Pressure vs. Crack Length



b.c

Figure 12-3. Field and Model Boiler Data for Steam Line Break Leakage

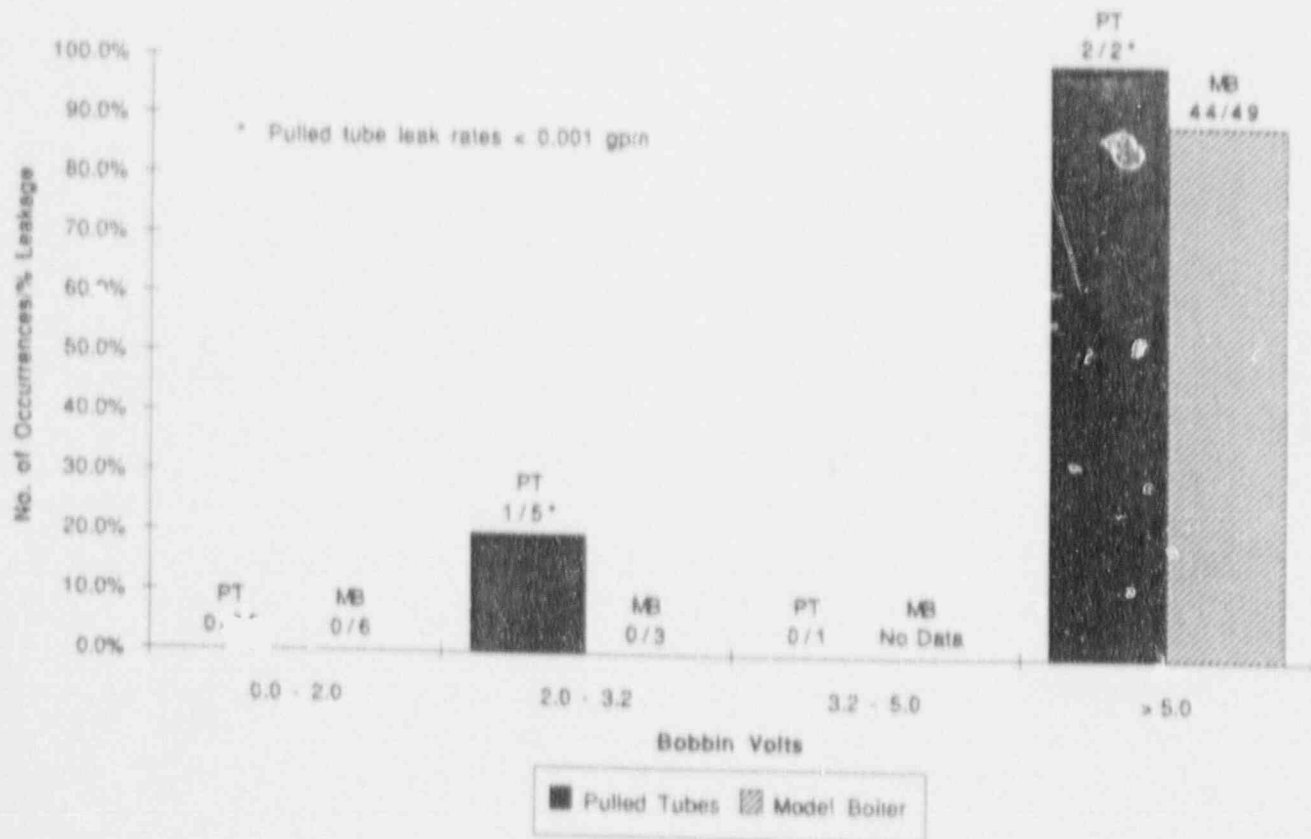


Chart Volt vs. SLB Leakage 2/11/92

Figure 12-4

80

Comparison Between Predicted and Measured Leak Rates


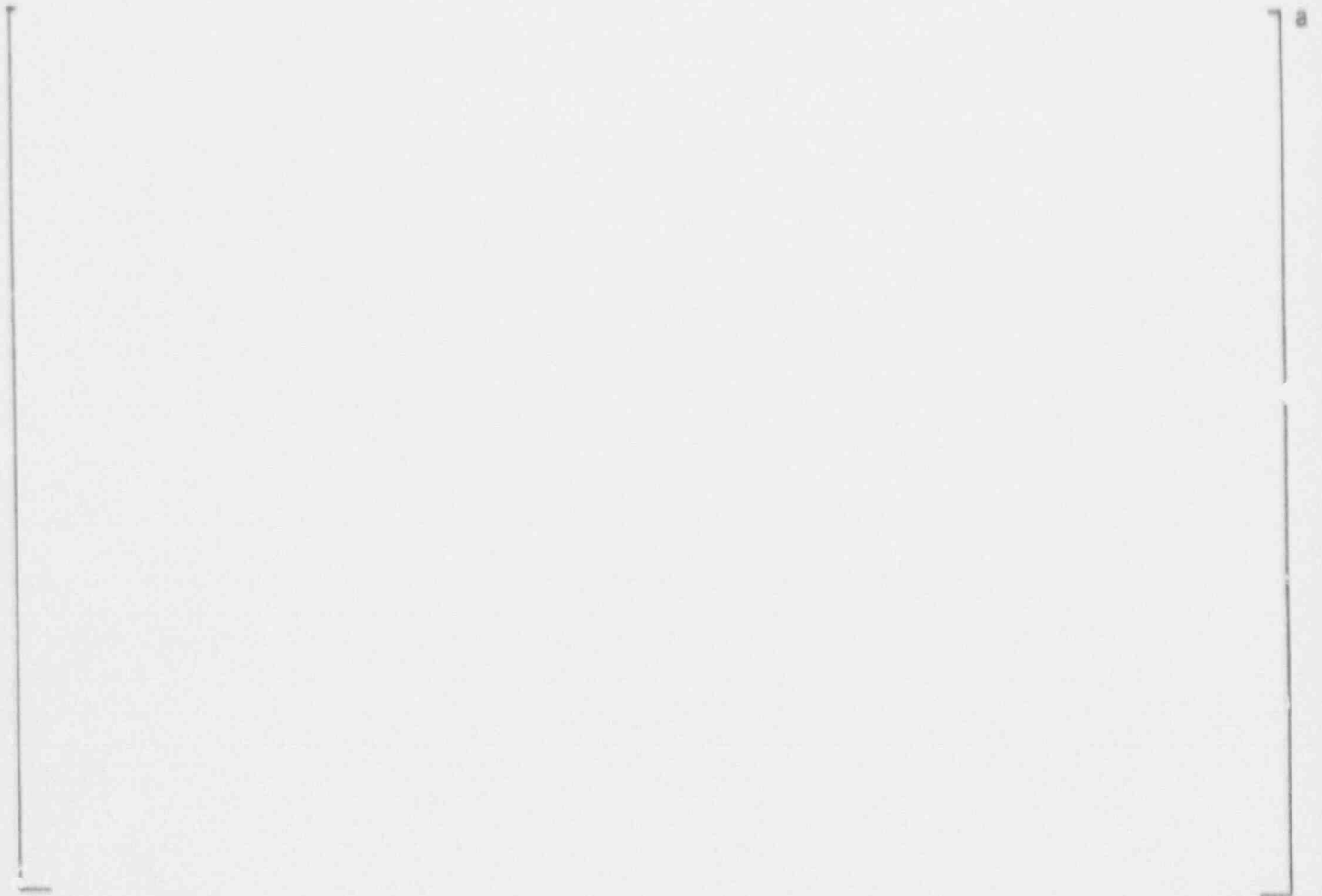


Figure 12-5

Normal Operating Condition Leak Rate vs Axial Crack Length



Appendix A

NDE Data Acquisition and Analysis Guidelines

A.1 INTRODUCTION

This appendix documents techniques for inspection of the Farley 1 and 2 S/G tubes related to identification of ODSCC at the tube support plate regions. ODSCC at the TSP intersections was first observed in 1986, when tubes were pulled at Farley Unit 2. No tube leakage has been attributed to this degradation mode in either Farley Unit 1 or 2.

This appendix contains guidelines which provide direction in applying the ODSCC alternate repair limits described in this report. The procedures for eddy current testing using bobbin coil and rotating pancake coil (RPC) technique, are summarized. The procedures given apply to the bobbin coil inspection, except as explicitly noted for RPC inspection. The methods and techniques detailed in this appendix are requisite for implementation of the alternate repair limit and are to be incorporated in the applicable inspection and analysis procedures. The following sections define specific acquisition and analysis parameters and methods to be used for the inspection of the steam generator tubing.

A.2 DATA ACQUISITION

The Farley Units 1 and 2 steam generators utilize 7/8" OD x 0.050" wall, Alloy 600 mill-annealed tubing. The carbon steel support plates are designed with drilled holes. The following guidelines are specified for non-destructive examination of the tubes within the TSPs at Farley 1 and 2.

A.2.1 Probes

Bobbin Coil Probes

Eddy current equipment used shall be the ERDAU (Echoram Tester), Zetec MIZ-18 or other equipment with similar specifications. To maximize consistency with laboratory data, differential bobbin probes with the following parameters shall be used:

- 0.720" outer diameter
- two bobbin coils, each 60 mils long, with 6.0 mils between coils (coil centers separated by 120 mils)
- non-magnetic bias.

In addition, the probe design incorporates centering features that provide for minimum probe wobble and offset.

Rotating Pancake Coil Probes

The coil diameter shall be $\leq 0.125"$. The maximum probe pulling speed shall be ≈ 0.2 in./sec (or 0.4 in./sec. for the case of 2 x RPC probe). The maximum rotation speed shall be ≈ 300 rpm. This would result in a pitch of ≈ 40 mils.

A.2.2 Calibration Standards

Bobbin Coil Standards

The bobbin coil calibration standards shall contain:

- Four 0.033" diameter through wall holes, 90° apart in a single plane around the tube circumference; the hole diameter tolerance shall be ± 0.001 ".
- One 0.109" diameter flat bottom hole, 60% through from OD
- One 0.187" diameter flat bottom hole, 40% through from the OD
- Four 0.187" diameter flat bottom holes, 20% through from the OD, spaced 90° apart in a single plane around the tube circumference.
- A simulated support ring, 0.75" long, comprised of SA-285 Grade C carbon steel or equivalent.
- This calibration standard will need to be calibrated against the reference standard used for the APC laboratory work.
- A probe wear standard for monitoring the degradation of probe centering devices leading to off-center coil positioning and potential variations in flaw amplitude responses. This standard shall include four through wall holes, 0.067" in diameter, spaced 90° apart around the tube circumference with an axial spacing such that signals can be clearly distinguished from one another (see Section A.2.3).

RPC Standard

The RPC standard shall contain:

- Two axial EDM notches, located at the same axial position but 180° apart circumferentially, each 0.006" wide and 0.5" long, one 80% and one 100% through wall from the OD.
- Two axial EDM notches, located at the same axial position but 180° apart circumferentially, each 0.006" wide and 0.5" long, one 60% and one 40% through wall from the OD.
- A simulated support segment, 270° in circumferential extent, 0.75" thick, comprised of SA-285 Grade C carbon steel or equivalent.

The center to center distance between the support plate simulation and the nearest slot shall be at least 1.25". The center to center distance between the EDM notches shall be at least 1.0". The tolerance for the widths and depths of the notches shall be ± 0.001 ". The tolerance for the slot lengths shall be ± 0.010 ".

A.2.3 Application of Bobbin Coil Probe Wear Standard

A calibration standard has been designed to monitor bobbin coil probe wear. During steam generator examination, the bobbin coil probe is inserted into the wear monitoring standard; the initial amplitude response from each of the four holes is determined and compared on an individual basis with subsequent measurements. Signal amplitudes or voltages from the individual holes - compared with their previous amplitudes - must remain within 15% of each other for an acceptable probe wear condition. If this condition is not satisfied, then the probe must be replaced. If any of the last probe wear

standard signal amplitudes prior to probe replacement exceed the 15% limit, say by a variable value, x%, then indications measured since the last acceptable probe wear measurement that are within x% of the plugging limit must be re-inspected with the new probe.

A.2.4 Acquisition Parameters

The following parameters apply to bobbin coil data acquisition and should be incorporated in the applicable inspection procedures to supplement (not necessarily replace) the parameters normally used.

Test Frequencies

This technique requires the use of 400 kHz and 100 kHz test frequencies in the differential mode. It is recommended that the absolute mode also be used, at test frequencies of 100 kHz and 10 kHz. The low frequency (10 kHz) channel should be recorded to provide a positive means of verifying tube support plate edge detection for flaw location purposes.

RPC frequencies should also include 400 kHz and 100 kHz.

Digitizing Rate

A minimum bobbin coil digitizing rate of 30 samples per inch should be used. Combinations of probe speeds and instrument sample rates should be chosen such that:

$$\frac{\text{Sample Rate (samples/sec.)}}{\text{Probe Speed (in./sec.)}} \geq 30 \text{ (samples/in.)}$$

Spans and Rotations

Spans and rotations can be set at the discretion of the user and/or in accordance with applicable procedures.

Mixes

A bobbin coil differential mix is established with 400 kHz as the primary frequency and 100 kHz as the secondary frequency, and suppression of the tube support plate simulation should be performed. This channel is also used to assess changes in signal amplitude for the probe wear standard.

A.2.5 Analysis Parameters

This section discusses the methodology for establishing bobbin coil data analysis variables such as spans, rotations, mixes, voltage scales, and calibration curves. Although indicated depth measurement may not be required to support an alternative repair limit, the methodology for establishing the calibration curves is presented. The use of these curves is recommended for consistency in reporting and to provide

compatibility of results with subsequent inspections of the same steam generator and for comparison with other steam generators and/or plants.

400 kHz Differential Channel

Rotation: The signal from the 100% through wall hole should be set to 40° ($\pm 1^\circ$) with the initial signal excursion down and to the right during probe withdrawal.

Voltage Scale: The peak-to-peak signal amplitude of the signal from the four 20% OD flaws should be set to 4.0 volts.

Calibration Curve: Establish a curve using measured signal phase angles in combination with the "as-built" flaw depths for the 100%, 60%, and 20% flaws on the calibration standard.

400/100 kHz Differential Mix Channel

Rotation: Set the signal from the probe motion to be horizontal with the initial excursion of the 100% through wall hole signal going down and to the right during probe withdrawal.

Voltage Scale: The peak-to-peak signal amplitude of the signal from the four 100% through wall flaws shall be set to 6.4 volts. The RPC amplitude shall be set to 20 volts for the 0.5 inch through wall notch.

Calibration Curve: Establish a curve using measured signal phase angles in combination with the "as-built" flaw depths for the 100%, 60%, and 20% flaws on the calibration standard.

A.2.6 Analysis Methodology

Bobbin coil indications at support plates attributable to ODSCC are quantified using the Mix 1 (400 kHz/100kHz) data channel. This is illustrated with the example shown in Figure A-1. The 400 kHz primary analysis channel (Channel 1), along with other appropriate frequency channels, can be used to locate the indication of interest within the support plate signal. Locate largest amplitude signal representing indication in Channel 1 or any other frequency. One then switches to Mix 1 (shown in the lower part of the figure), establishes the peak-to-peak voltage and records the relative Mix 1 channel signal amplitude. This voltage is then entered as the analysis of record for comparison with the repair limit voltage.

A.2.7 Reporting Guidelines

The reporting requirements identified below are in addition to any other reporting requirements specified by the user.

Minimum Requirements

At a minimum, flaw signals in the 400/100 mix channel at the tube support plate intersections whose peak-to-peak signal amplitude exceeds

1.5 volts, requiring RPC inspection, must be reported. Signals, however small, should also be reported for historical purposes and to provide an assessment of the overall condition of the steam generator(s).

Additional Requirements

For each reported indication, the following information should be recorded:

Tube identification	(row, column)
Signal amplitude	(volts)
Signal phase angle	(degrees)
Indicated depth	(%) [†]
Test channel	(ch#)
Axial position in tube	(location)
Extent of test	(extent)

[†] It is recommended that an indicated depth be reported as much as possible rather than some letter code. While this measurement is not required to meet the alternate repair limit, this information might be required at a later date and/or otherwise be used to develop enhanced analysis techniques.

RPC reporting requirements should include a minimum of: type of degradation (axial, circumferential or other), voltage, phase angle, crack lengths, and location of center of crack within the TSP.

A.3 DATA EVALUATION

A.3.1 Use of 400/100 Differential Mix for Extracting the Flaw Signal

In order to positively identify a discontinuity in the composite signal as an indication of a flaw in the tube wall, a simple signal processing procedure of mixing the data from the two test frequencies is used which reduces the interference from the support plate signal by about an order of magnitude. The test frequencies most often used for this signal processing are 400 kHz and 100 kHz where 50 mil wall Inconel-600 tubing is involved. The processed data is referred to as 400/100 mix channel data. This procedure may also eliminate the interference from magnetite accumulated in the crevices. Any of the differential data channels including the mix channel may be used for flaw detection (though the 100 kHz channel is subject to influence from many different effects), but the final evaluation of the signal detection, amplitude and phase (although a depth estimate is not required in the alternate plugging limit) will be made from the 400/100 differential mix channel. Upon detection of a flaw signal in the differential mix channels, confirmation from other channels is not required. The voltage scale will be set from the 400/100 differential channel on the basis of 6.4 volts for the 4 holes, 100% ASME standard.

The present evaluation procedure requires that there is no minimum voltage for flaw detection purposes and that all flaw signals however small be identified. The intersections with flaw signals ≥ 1.5 volt will be inspected with RPC in order to

confirm the presence of ODSCC. Although the signal voltage is not a measure of the flaw depth, it is an indicator of the tube burst pressure when the flaw is identified as axial ODSCC with or without minor IGA.

The procedure using the 400/100 mix for reducing the influence of support plate and magnetite does not eliminate the interference from copper, alloy property change or dents. These are discussed below.

A.3.2 Amplitude Variability

It has been observed that voltage measurements taken off the same data by different analysts even when using identical guidelines may differ. By observing the graphical output produced by each analyst, it can be seen that the dots which mark the lissajous traces for computer amplitude calculations are sometimes placed at different locations; this reflects a degree of analyst judgment within the specified guidelines for peak-to-peak voltage measurements. Figures A-2 to A-5 illustrate such differences. This source of error becomes more noticeable when the data involves complicating factors or interferences which make the process of flaw identification more difficult; the contrast between Farley 1 which exhibits signs of minor denting in the TSPs and Farley 2 which is essentially free from corrosion-induced denting presents such circumstances.

In order to minimize the chance that bobbin flaw voltages near setpoints will not be underestimated, e.g. the plugging limit or the RPC-testing threshold, it is recommended that the placement of dots be done in such a manner as to obtain the maximum voltage consistent with the lissajous figure portion which represents the flaw. Placement of the dots to effect maximum amplitude measurement is illustrated in Figure A-6. The Mix 1 lissajous trace shows the dots placed at the extremes of the vertical displacement to be consistent with the apparent flaw segment in the 200 kHz. It may be necessary to iterate the position of the dots between the identifying frequency data (e.g. 200 kHz) and the Mix 1 data to assure proper placement of the dots. Figure A-7 shows that initial placement in the 200 kHz channel identified only part of the flaw segment, while Figure A-8 shows that a small change in the 200 kHz channel dot placement identified more completely the flaw extent in the Mix 1 data, with ~50% change in amplitude.

It may be difficult to determine the correct portion of the lissajous trace corresponding to the possible flaw in the TSP. Figure A-9 presents a situation in which identification of the correct portion of the Mix 1 lissajous as the flaw segment is difficult because the flaw features in the 200 kHz channel or 400 kHz channel are overwritten. By expansion of the viewing window (200 kHz, for example), the region of overwriting is expanded and the identification of the flaw segment is made easier; Figure A-10 confirms that the 64° portion of the trace represents the flaw segment.

It is noted that by employing these techniques, identification of flaws is improved and that conservative amplitude measurements are promoted. The Mix 1 traces which result from this approach conform to the model of TSP ODSCC which represents the degradation as a series of microcrack segments axially integrated by the bobbin coil; i.e., short segments of changing phase direction represent changes in average depth with changing axial position. This procedure is to be followed for reporting voltages

for the plugging criteria of this report. This procedure may not yield the maximum bobbin depth call. If maximum depth is desired for information purposes, shorter segments of the overall crack may have to be evaluated to obtain the maximum depth estimate. However, the peak to peak voltages as described herein, must be reported even if a different segment is used for the depth call.

A.3.3 Copper Interference

In situations where significant copper interference in the eddy current data is noted, the eddy current technique basically becomes unreliable. This results from the unpredictability of the amount and morphology of copper deposit on the tubes which may be found in operating steam generators. The above observation is true both for bobbin and RPC or any other eddy current probe. Fortunately, significant copper interference does not occur in the support plate crevice regions of Farley 1 and 2. This is confirmed by destructive examination of the support plate intersections pulled from Farley 1 and 2. No plated copper was found on the tube OD within the support plate crevice although some minor copper patches outside the crevice region were sometimes observed. Copper is not a significant issue for evaluating the support plate intersections of the tubing of Farley #1 and #2 steam generators. If copper interference is observed at Farley Units 1 and 2, the existing rules and procedures for complying with the technical specification plugging limit based on depth of wall penetration will apply.

A.3.4 Alloy Property Changes

This signal manifests itself as part of the support plate "mix residual" in the 400/100 differential channel and as an offset in the 400/100 mix absolute channel. It has often been confused with copper deposit as the cause. Such signals are found often at support plate intersections of operating plants, as well as in the model boiler test samples, and are not necessarily indicative of tube wall degradation. Six support plate intersections of the Farley 2 plant judged as free of tube wall degradation on the basis of the 400/100 differential channel using the guidelines given in Section A.2.5 and A.2.6 of this document were pulled in 1989. Examples of the bobbin coil field data are shown in Figure A-11 to A-13. The mix residuals for these examples are between 2 and 3 volts in the 400/100 differential channel and no discontinuity suggestive of a flaw can be found in this channel. All of them have an offset in the 400/100 absolute channel which could be confused as a possible indication. These signals persisted without any significant change even after chemically cleaning the OD and the ID of these tubes. The destructive examination of these intersections showed very minor or no tube wall degradation. Thus, the overall mix residual of the 400/100 differential channel and the offset in the 400/100 absolute channel themselves are not an indication of tube wall degradation. One needs to examine the detailed structure of the "mix residual" (as outlined in Sections A.2.5 and A.2.6) in order to assess the possibility that a flaw signal is present in the residual composite. Such offsets in the absolute channels have been observed at the top of the tubesheet in plants with partial length roll expansions; in such cases, destructive examination of sections pulled from operating plants have shown no indication of tube wall degradation.

A.3.5 Dent Interference

There are essentially no corrosion induced dent signals of any significance at the support plate intersections of Farley Unit 2. A small population of original condition dents, mainly at the upper support plates, with voltages up to about 10-15 volts, are present; this is typical of the as-built condition of a typical steam generator, which may have random local dents ("dings"), some at support plate elevations. Farley Unit 2 has a small population of corrosion induced dents, Figure A-14, at the support plate intersections. These locations, when tested with bobbin probes, produce signals which are a composite of the dent signal plus other contributing effects such as packed magnetite, conductive deposits, alloy property change (artifacts) plus flaw signals if present and the support plate itself.

The 400/100 kHz (differential) support plate suppression mix eliminates the support plate and the magnetite signals, but the resulting processed signal may still be a composite of the dent, artifact, and the flaw signals. These composite signals represent vectorial combinations of the constituent effects, and as such they may not conform to the behavior expected from simple flaw simulations as a function of test frequency.

The average signal amplitude of the dent population in the most affected steam generator at Farley (S/G C in Unit 1) is 3.2 volts peak-to-peak; very few of the dents observed exceed 10 volts measured at the 400 kHz calibration settings. It can be seen that only a few hundred intersections (<1% of the tubes) exhibit TSP dents greater than 2 volts.

The effect of the dent on the detection and evaluation of a flaw signal depends on both the relative amplitudes of the flaw and dent signals and the relative spatial relationship between them. If the flaw is located near the center of the dent signal, interference with flaw detection may become insignificant, even for relatively large dent to flaw signal amplitude ratios. The flaw signal in a typical support plate dent in this event occurs mid-plane--away from the support plate edges where the dent signal has maximum voltage; thus the flaw in the middle section of the support plate shows up as a discontinuity in the middle of the composite signal. Some examples of such cases in the field data are shown in Figures A-15 to A-20 for dents with peak-to-peak amplitudes ranging from ~4 to 10 volts. The top pictures in these figures show the composite signal voltages; the pictures in the bottom half give the flaw voltages. For example, in Figure A-15, the dent voltage at 400 kHz is ~10.3 volts and the flaw signal voltage in the 400/100 kHz mix channel is ~1.3 volts. It can be observed from these figures that one can readily extract a flaw signal when the signal to noise (S/N) ratio is less than unity. The question of S/N ratio requirements for the detection and evaluation of the flaw signal is answered by examination of Figures A-15 to A-20. In all cases shown, S/N is less than 1, and the flaw signal can be detected and evaluated.

The greatest challenge to flaw detection due to dent interference occurs when the flaw occurs at the peak of the dent signal. Detection of flaw signals of amplitude equal to or greater than 1.5 volts--the criterion associated with confirmatory RPC testing--in the presence of peak dent voltages can be understood by vectorial combination of 1.5 volt flaw signals across the range of phase angles associated with 40% (110°) to 100% (40°) throughwall penetrations with dent signals of various amplitudes. It is

easily shown that 1.5 volt flaw signals combined with dent signals up to ~13 volts (1 mil equivalent) peak-to-peak will yield resultant signals with phase angles less than 170° -- within the Farley flaw reporting range; in all cases the amplitude will exceed 1.5 volts. All such signals will be subjected to RPC testing under the Farley guidelines in the normal course of events. To demonstrate this one-half, the dent peak-to-peak voltage (entrance or exit lobe) at 180° is combined with the 1.5 volts flaw signal at the desired phase angle. The Farley 1 inspection data is shown in Figures A-15 to A-20 to permit flaw detection and evaluation for flaws situated away from the peak dent voltages. The vector combination analysis shows that for moderate dent voltages where flaws occur coincident with dent entrance or exit locations, flaw detection at the 1.5 volts amplitude level is successful via phase discrimination of combined flaw/dent signals from dent only signals. Supplemental information to reinforce this phase discrimination basis for flaw identification can be obtained by examination of the 200/100 kHz mix channel; dent response would be lessened while the flaw response is increased relative to the 400/100 kHz mix. RPC testing of indications identified in this fashion will confirm the dependability of flaw signal voltages identified in this fashion.

A.3.6 Flaw Characterization

The RPC inspection of the intersections with bobbin coil flaw indications >1.5 volts is recommended in order to verify the applicability of alternate repair limit; this is based on establishing the presence of ODSCC with minor IGA as the cause of the bobbin indications. Once an indication is characterized by RPC as ODSCC, inspection of the indication at alternate refueling outages is acceptable.

The signal voltage for the RPC data evaluation will be based on 20 volts for the 100% deep, 0.5" long EDM notch at all test frequencies. The nature of degradation will be determined from careful examination of isometric plots of the RPC data. The presence of axial ODSCC at the support plate intersections has been well documented, but the presence of circumferential ODSCC at the support plate intersections has been established by tube pull in only one plant. Thus, the examples shown in Figures A-21 to A-23 show examples of multiple/single axial ODSCC. Figure A-24 gives an example of circumferential ODSCC from another utility at tube support plates. The examples of circumferential ODSCC given here show that if the circumferential involvement is 60° to 90° , one can unambiguously establish the existence of circumferential ODSCC although it may be possible to establish this in cases of arc lengths as small as 45° .

RPC resolution is considered adequate for separation between circumferential and axial cracks. Circumferential cracking is not expected in the Farley SGs, since denting is small and affecting a limited number of tubes in Farley 1. Figure A-25 shows an example of an axial indication call for closely spaced axial cracks. Also shown is a picture of the cracks from the destructive examination for these tubesheet crevice indications. The axial RPC call was based on the detected length of the indications. If a well defined circumferential indication is identified at the TSPs in the Farley SGs ($>60\%$ circumferential extent), guidelines for RPC interpretation will be reviewed at that outage and consideration given to a supplemental UT inspection if necessary for resolution of the degradation mode.

A.3.7 Confinement of ODSCC/IGA Within the Support Plate Region

In order to establish that a bobbin indication is within the support plate, the displacement of each end of the signal is measured relative to the support plate center. If this distance exceeds half the support plate axial length (0.375"), the crack will be considered to have progressed outside the support plate.

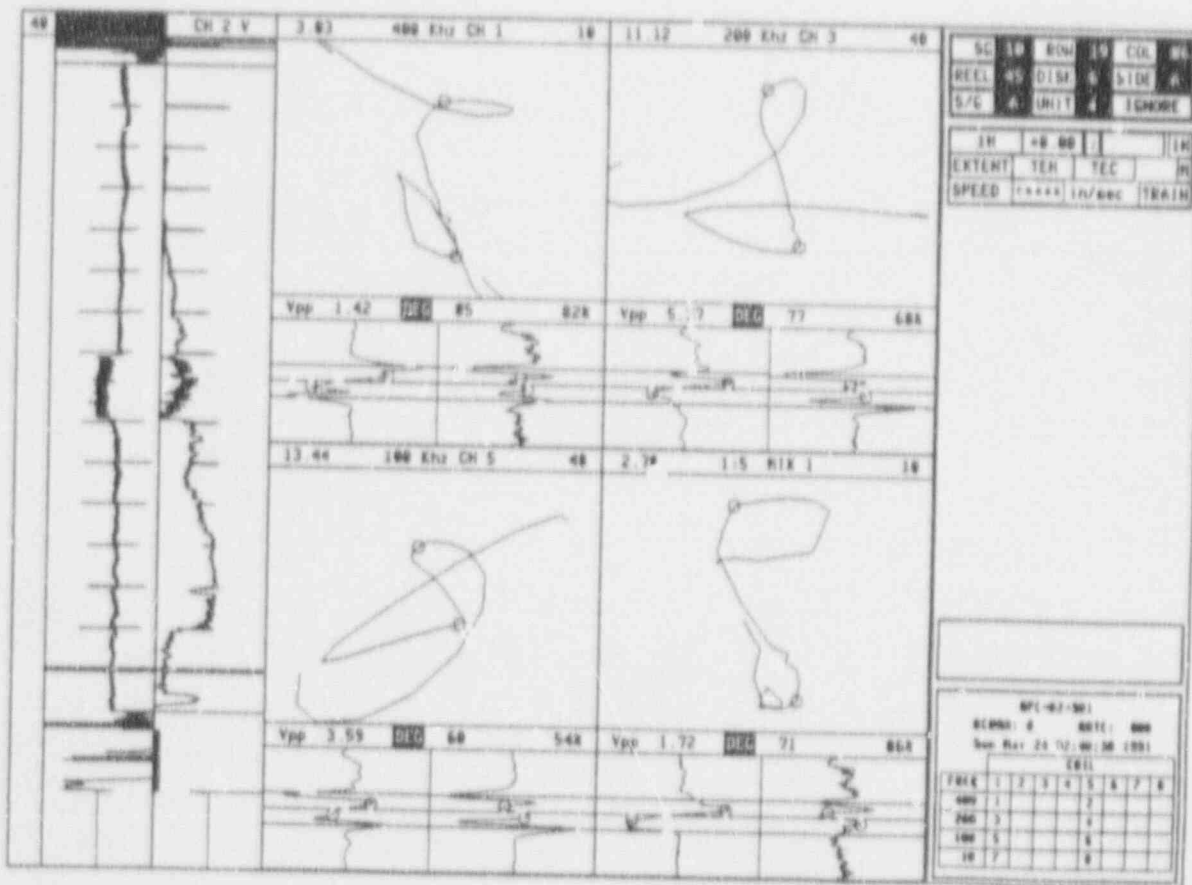


Figure A-2. Placement of Dots Marking Lissajous Traces for R19C86 - Analyst 1

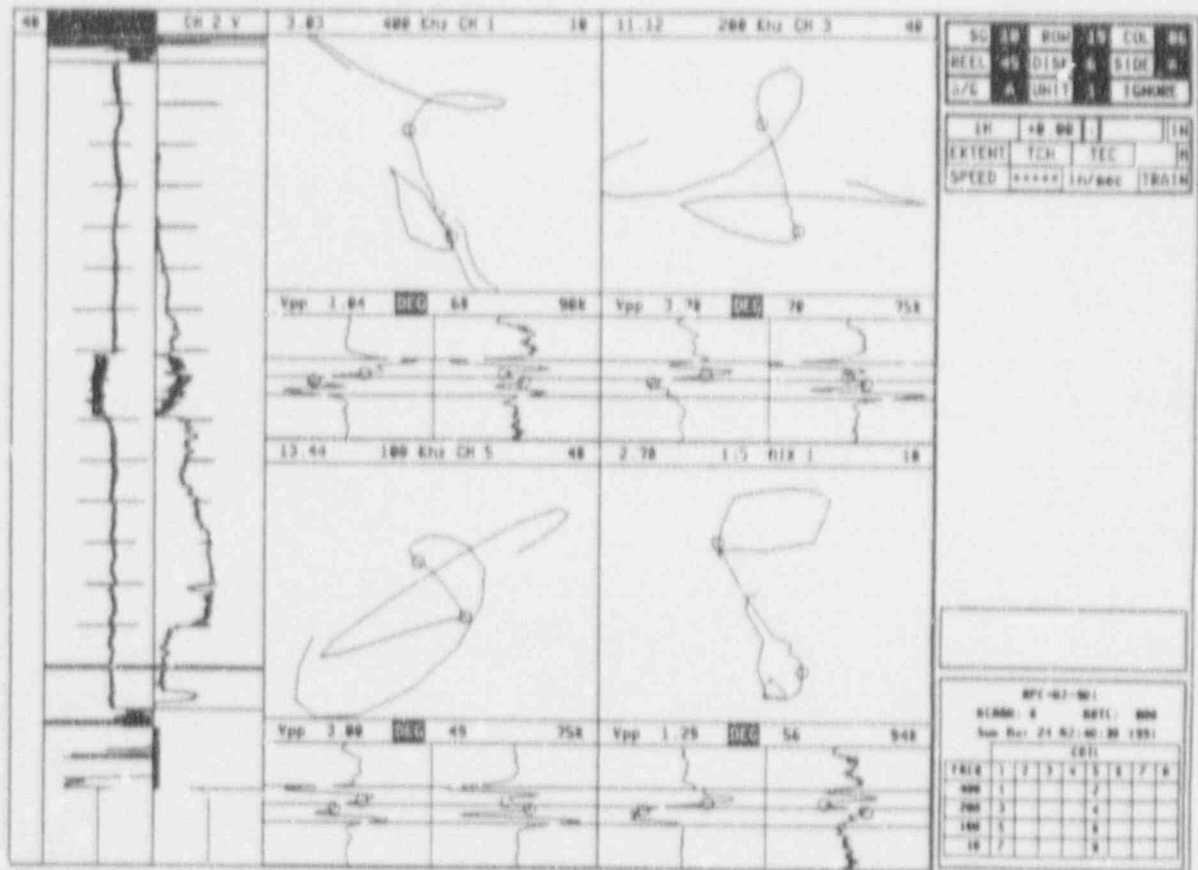


Figure A-3. Placement of Dots Marking Lissajous Traces for R19C86 - Analyst 2

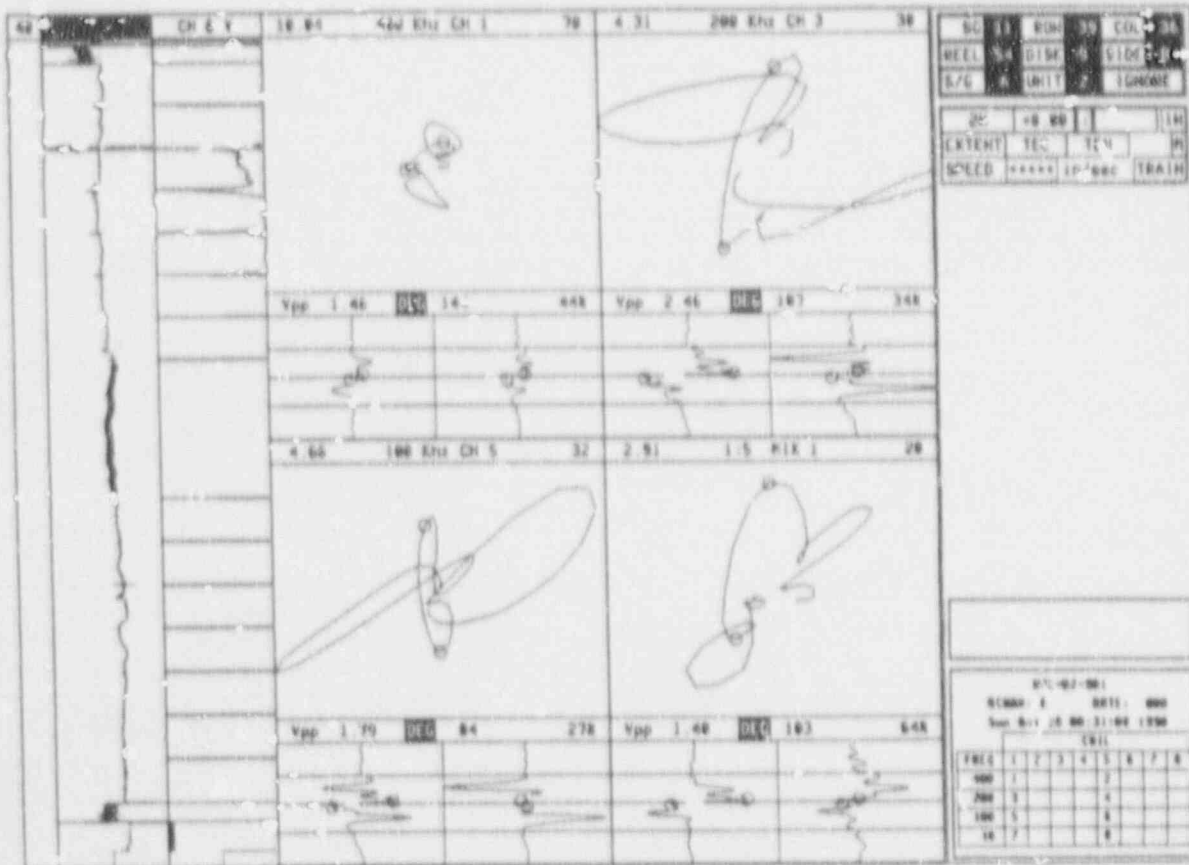


Figure A-4. Placement of Dots Marking Lissajous Traces for R39C36 - Analyst 1

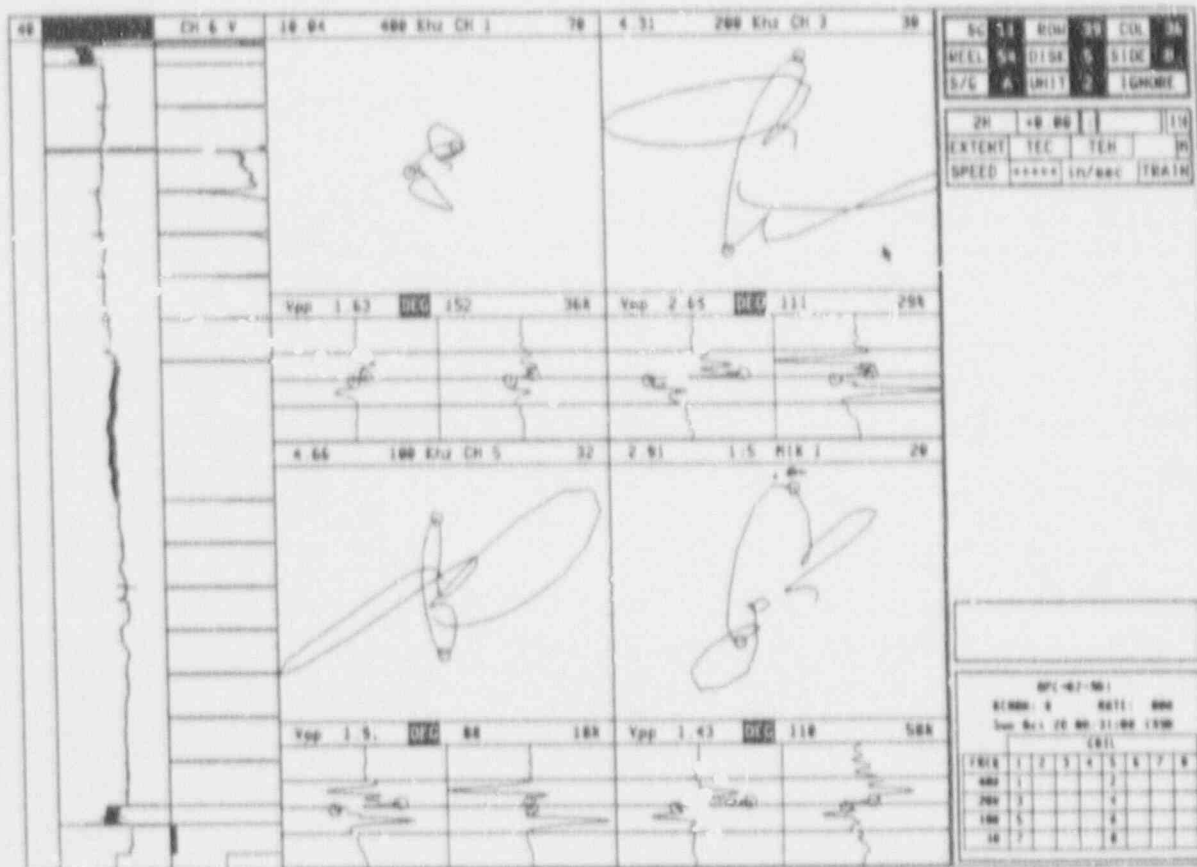


Figure A-5. Placement of Dots Marking Lissajous Traces for R39C36 - Analyst 2

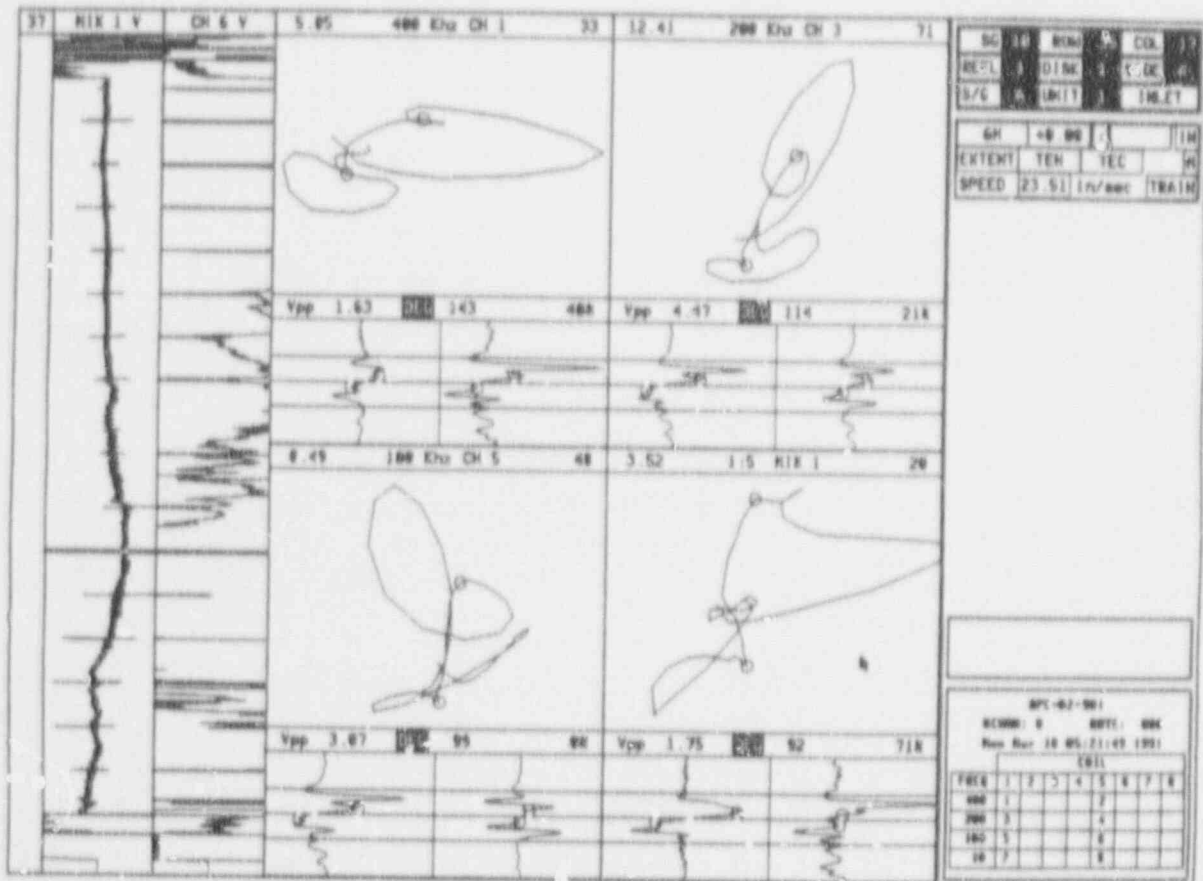


Figure A-6. Placement of Dots to Effect Maximum Voltage Amplitude

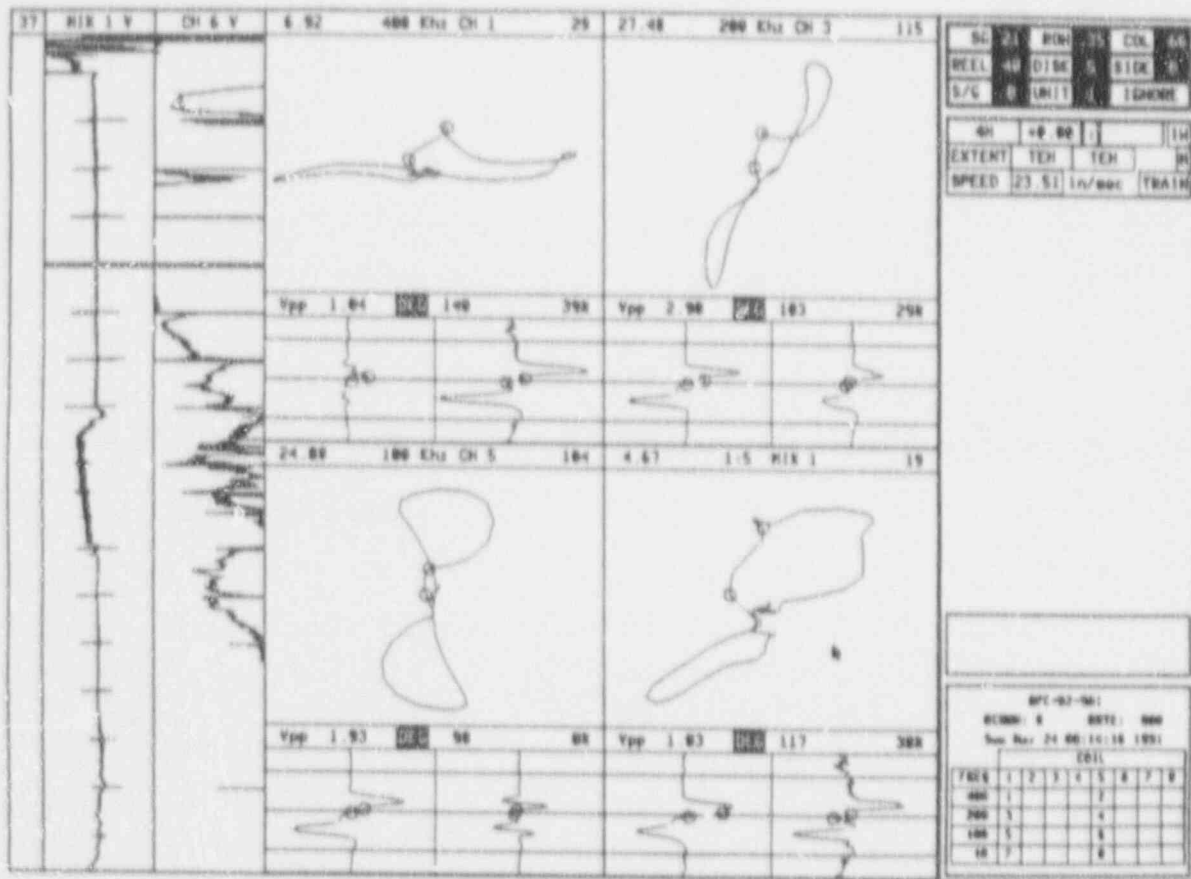


Figure A-7. Initial Iteration of Dot Placement to Effect Max. Amplitude

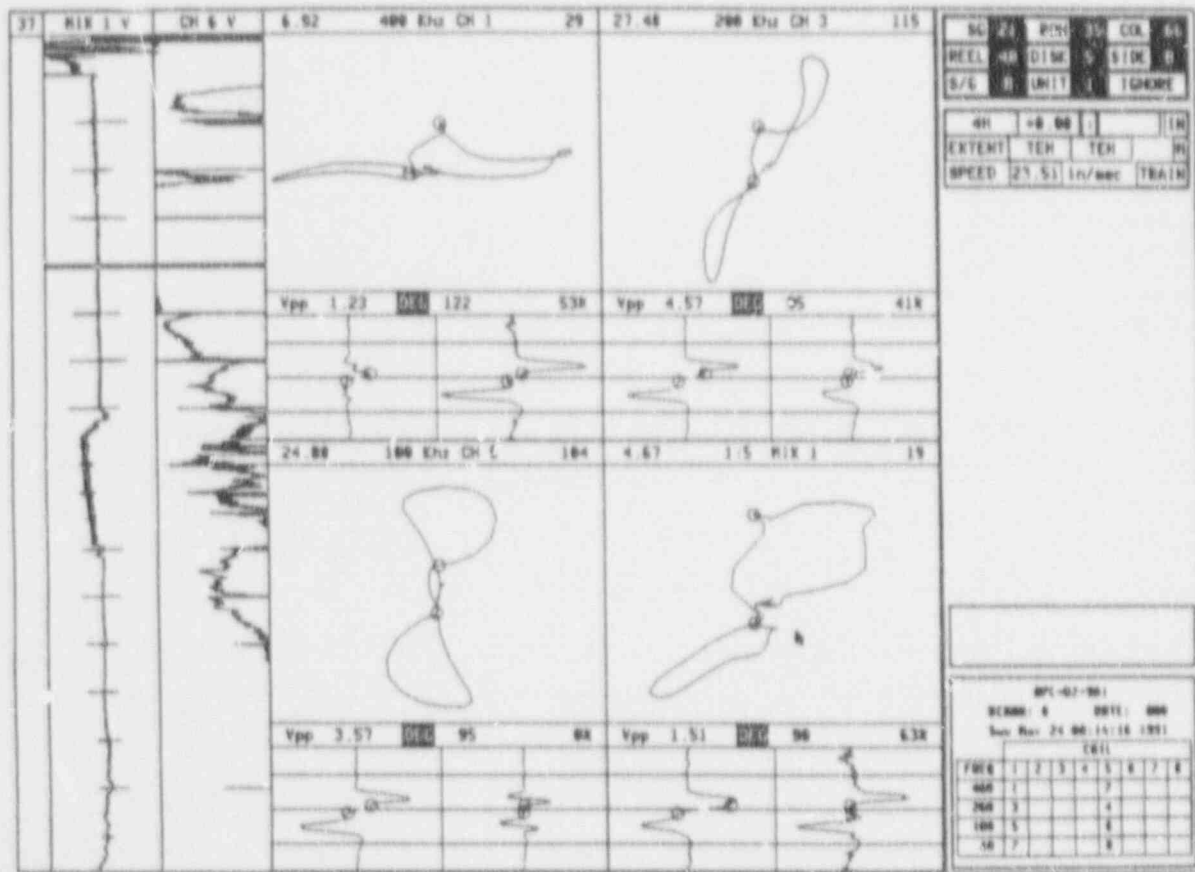


Figure A-8. Second Iteration of Dot Placement to Effect Maximum Amplitude

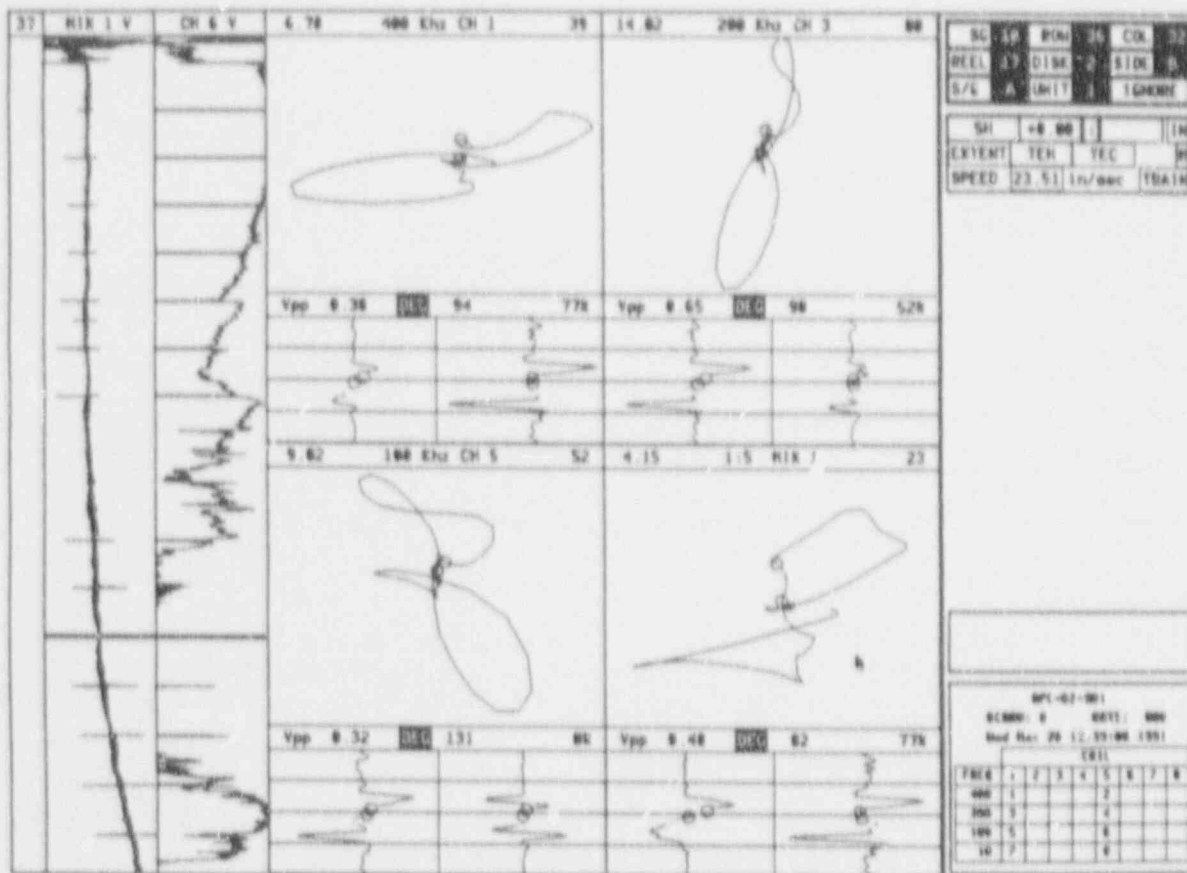


Figure A-9. Example of Overwritten Flaw Features Which Can Affect Identification of Correct Portion of Mix 1 Lissajous as the Flaw Segment

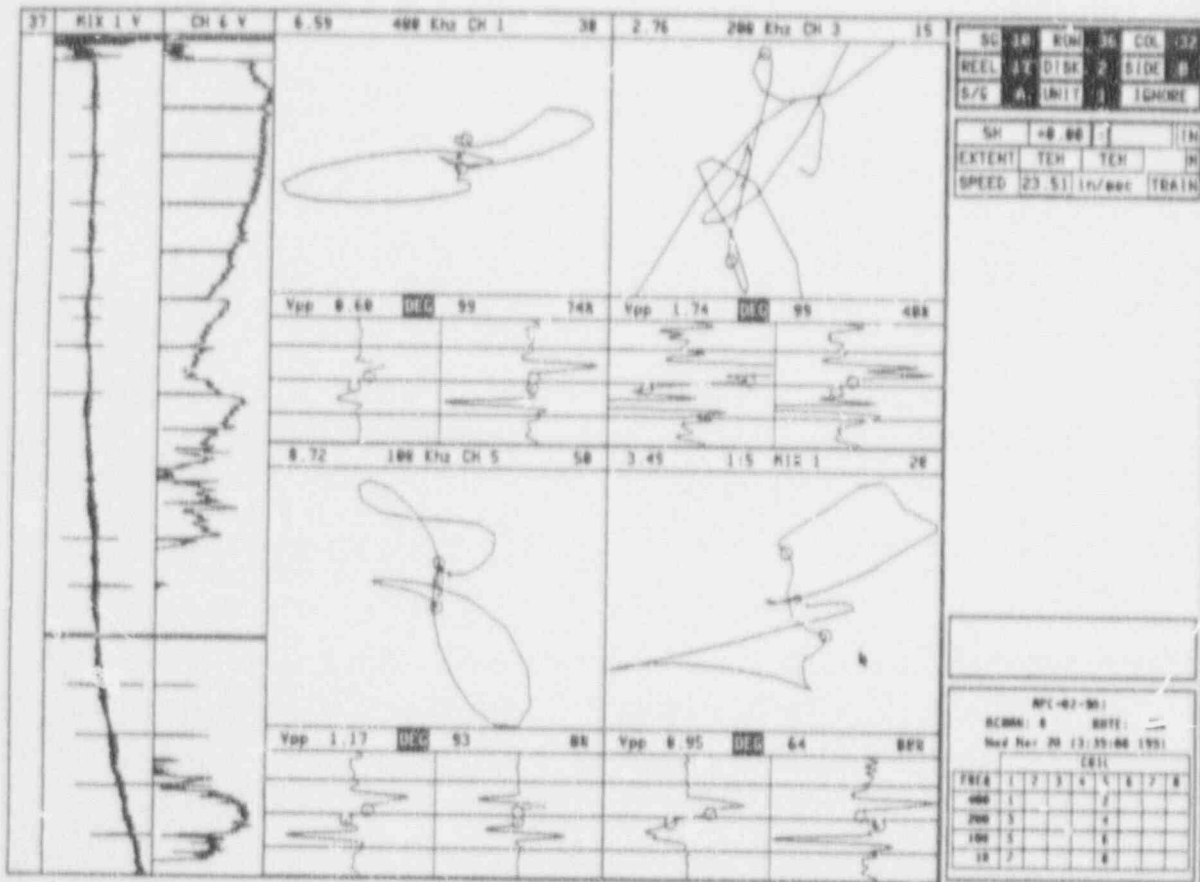


Figure A-10. Identification of 64° Portion of Trace as Flaw Segment

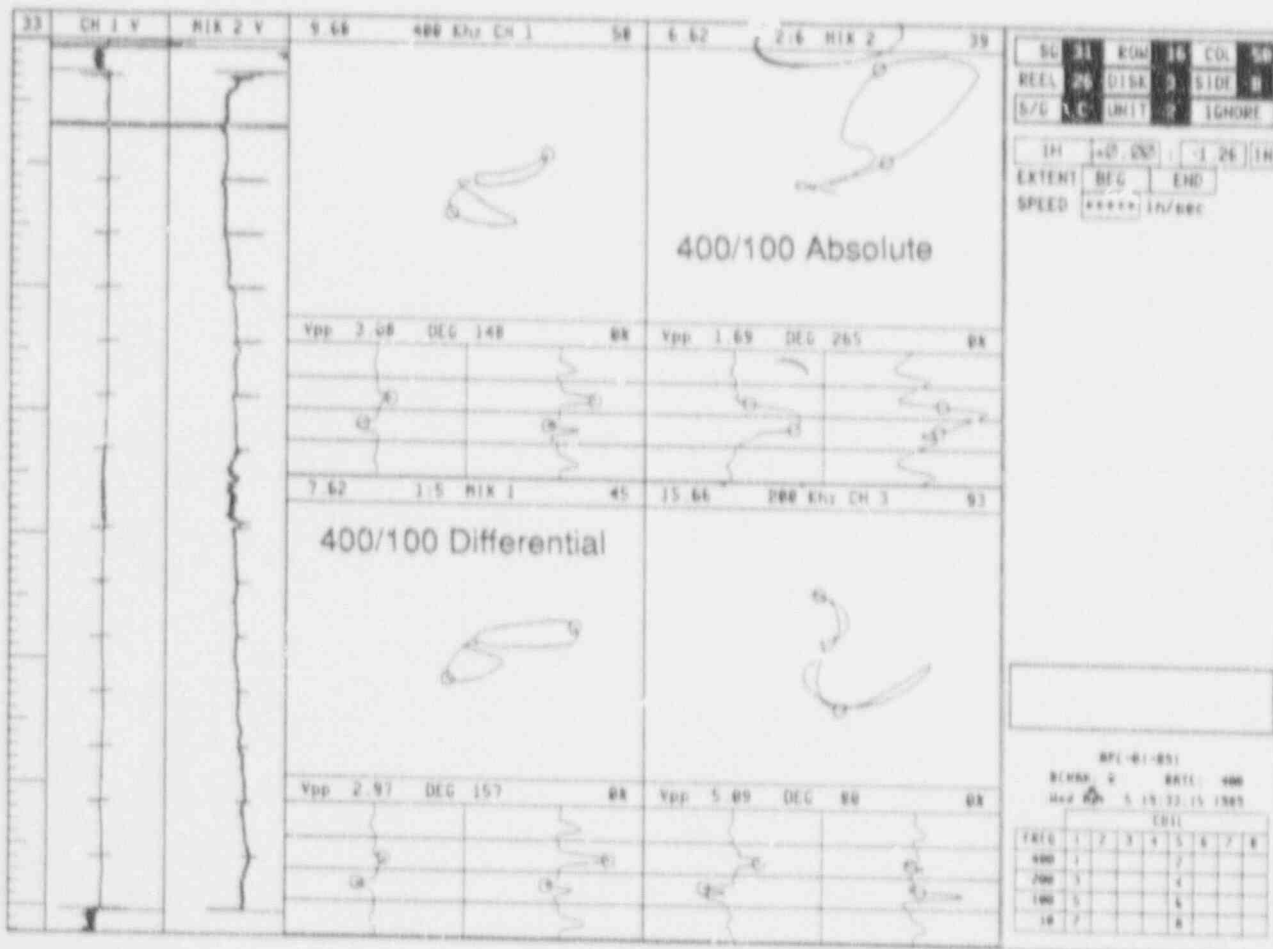


Figure A-11. Example of Bobbin Coil Field Data - Absolute Mix With No ODSCC

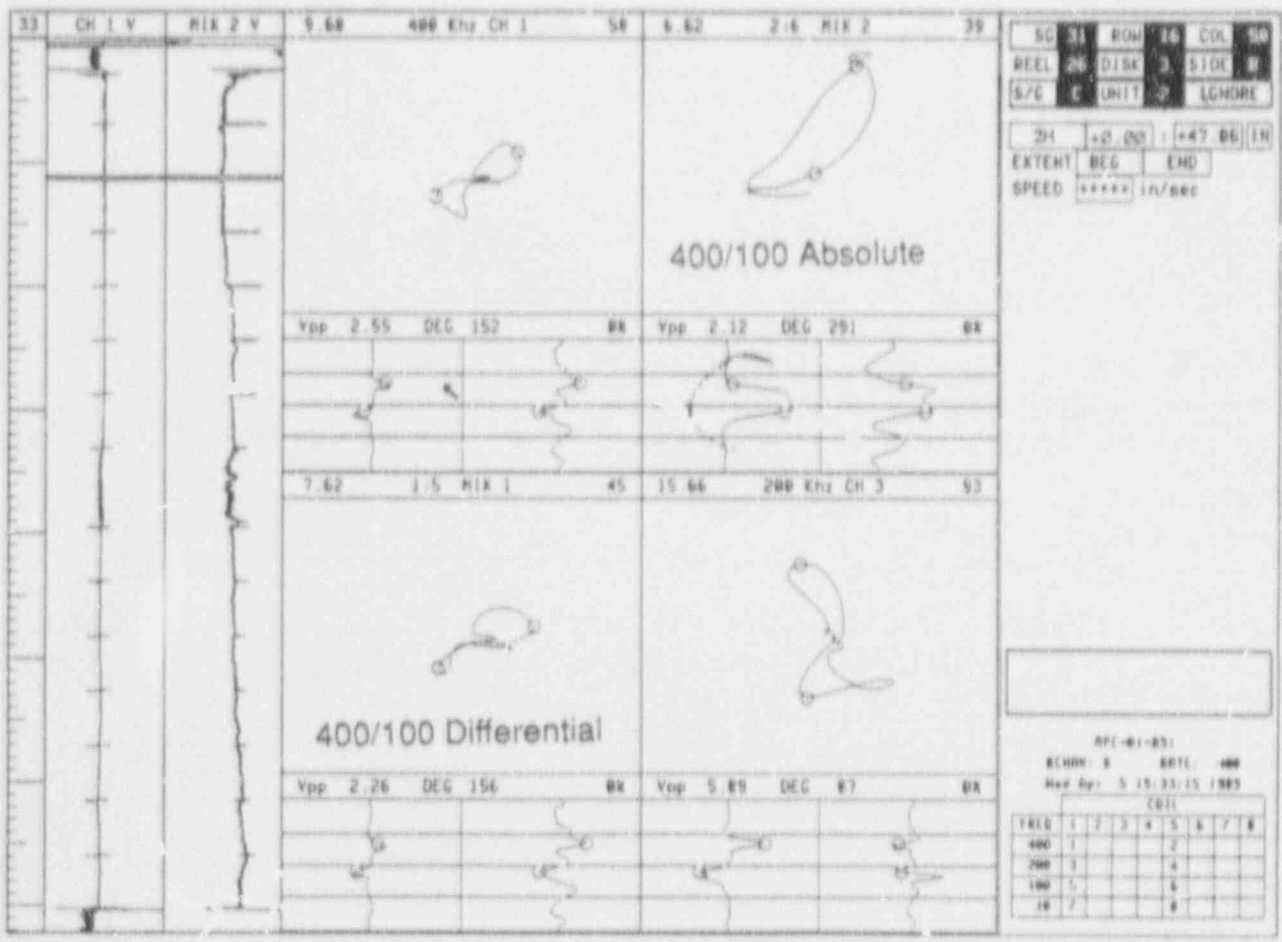


Figure A-12. Example of Bobbin Coil Field Data - Absolute Mix With No ODSCC

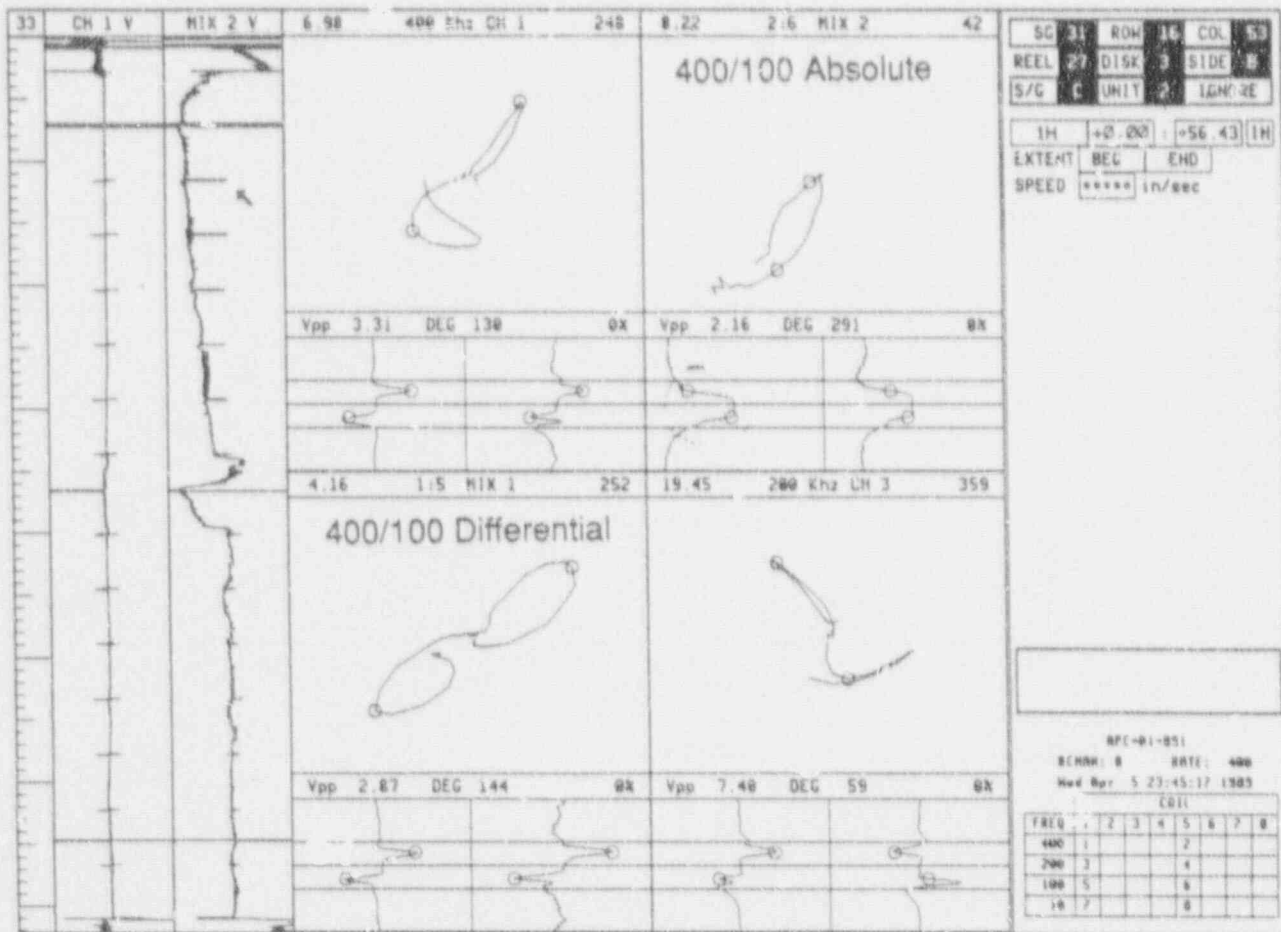


Figure A-13. Example of Bobbin Coil Field Data - Absolute Mix With No ODSCC

FARLEY SUPPORT PLATE DENT ANALYSIS IEDA CALLS (2 VOLT THRESHOLD)

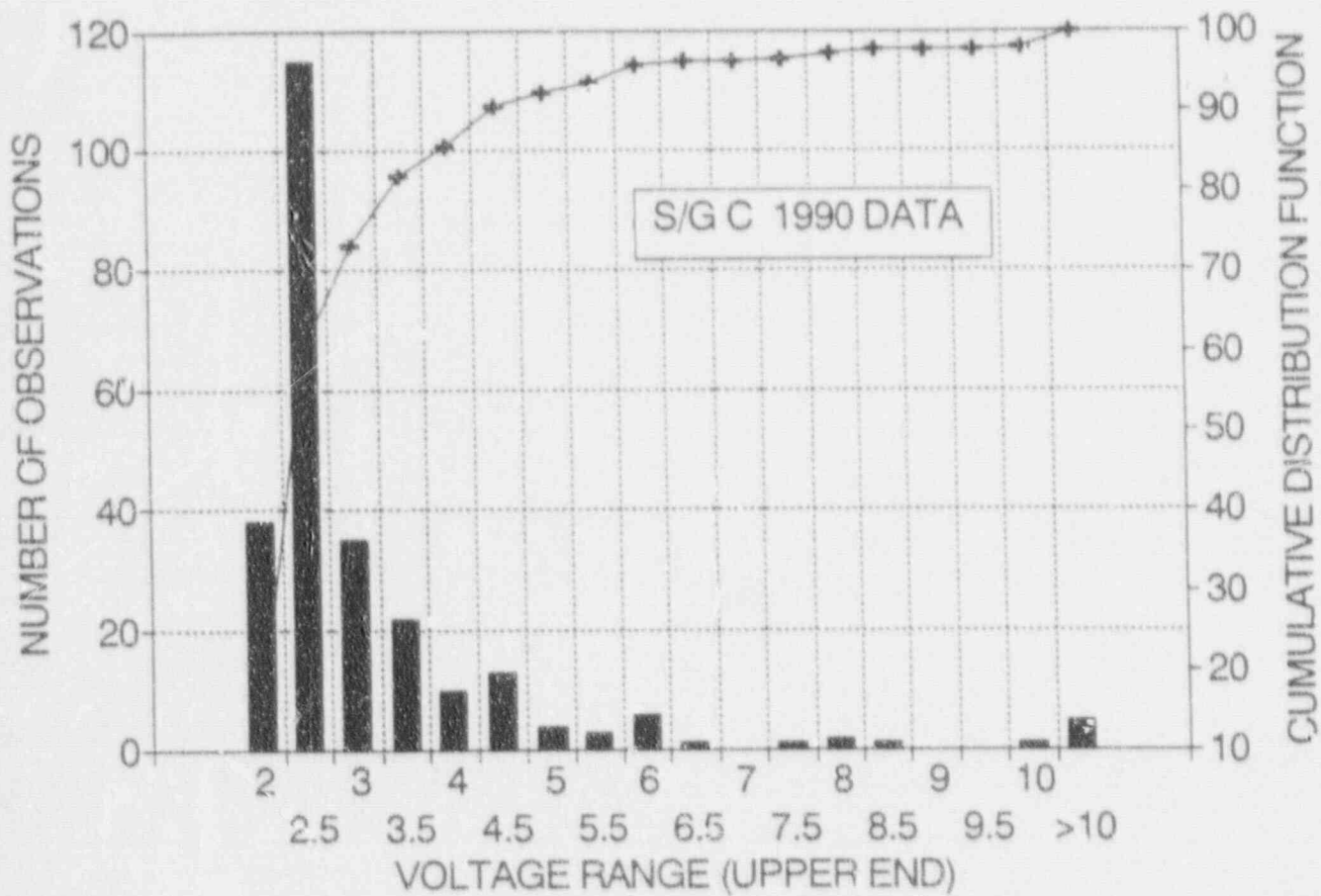


Figure A-14. Farley Unit 1, SG "C" Tube Support Plate Dent Size Distribution.

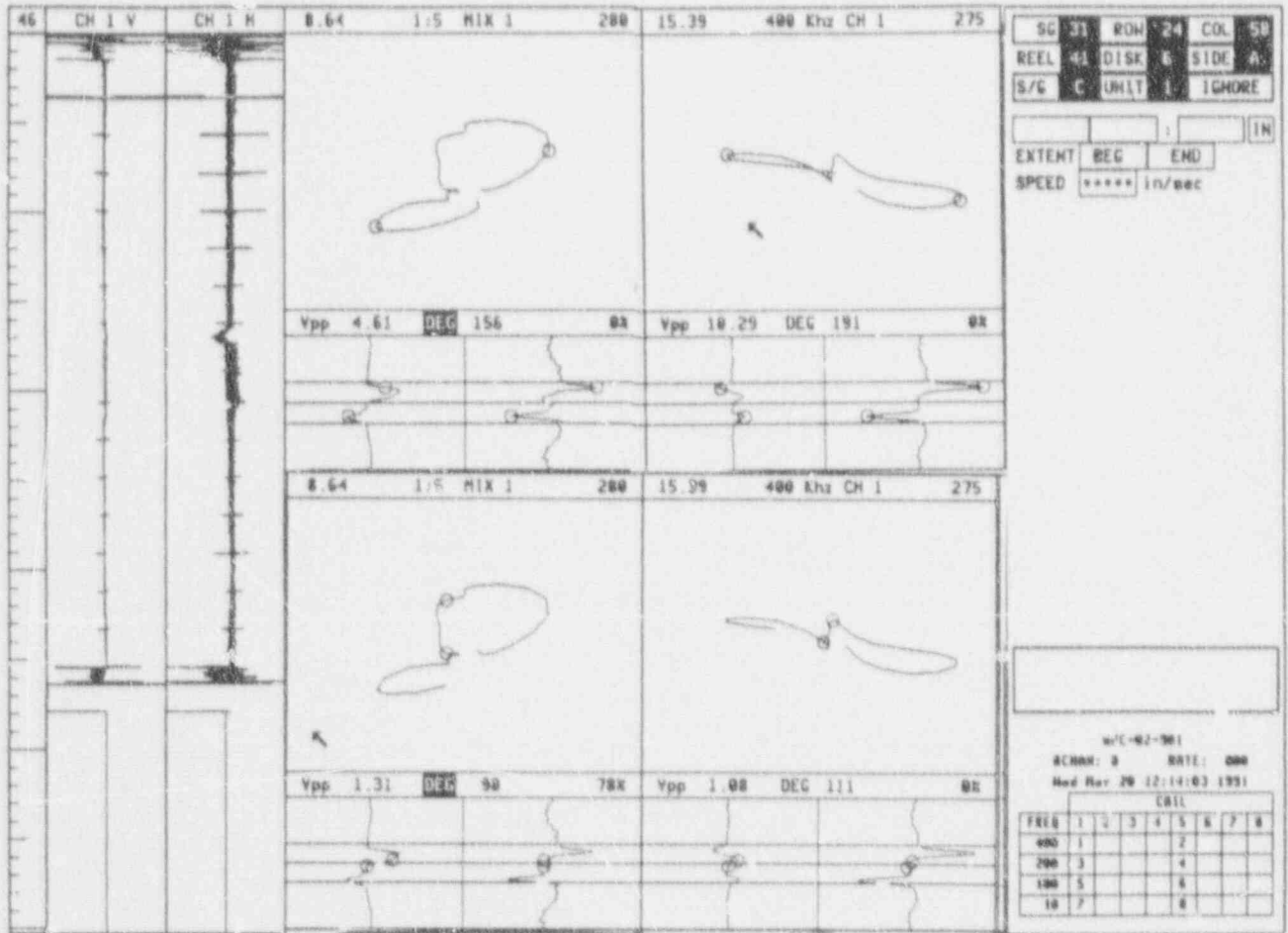


Figure A-15. Example of Bobbin Coil Field Data - Flaw Signals for ODSCG at Dented TSP Intersection

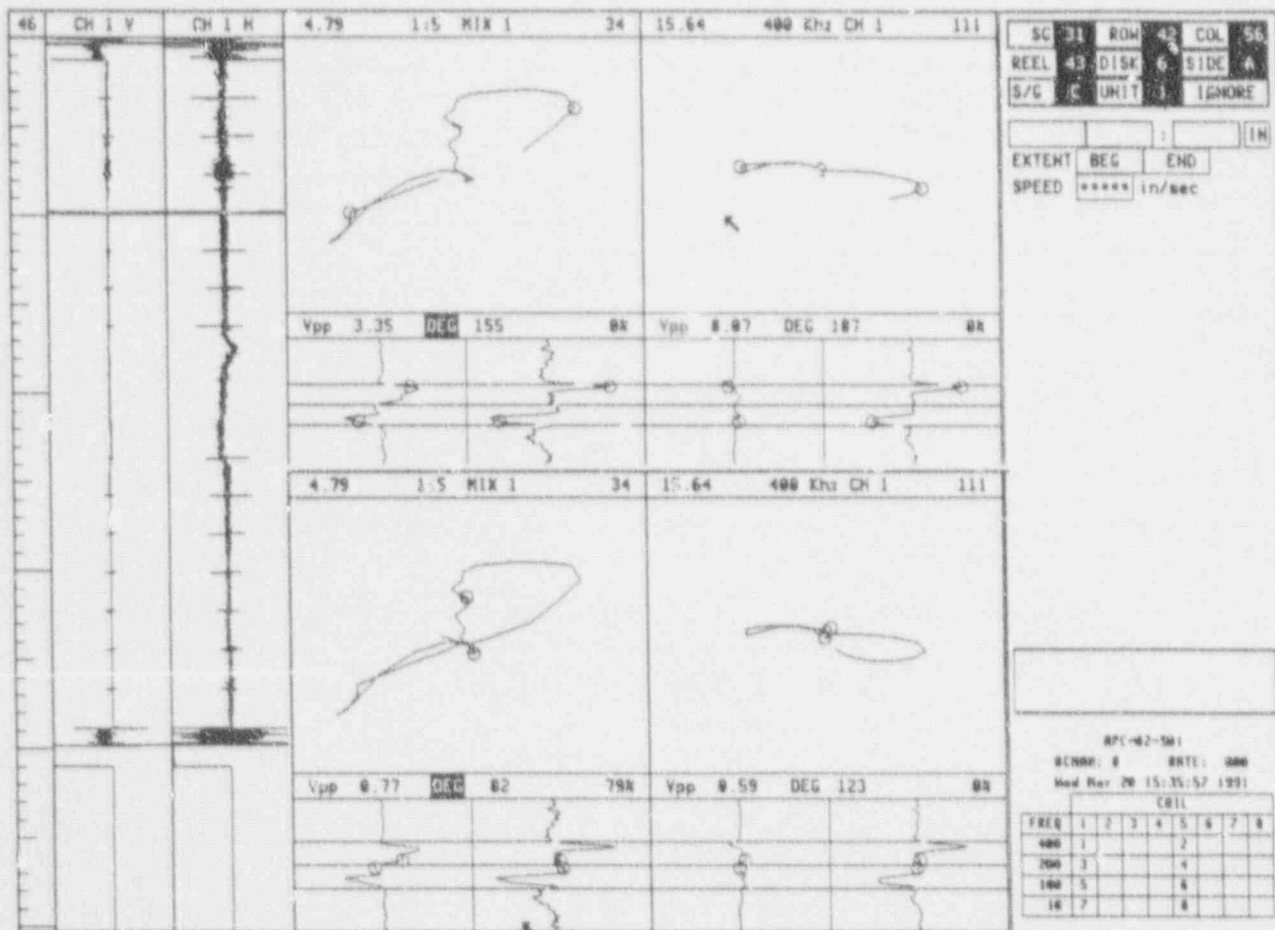


Figure A-16. Example of Bobbin Coil Field Data - Flaw Signals for ODSCC at Dented TSP Intersection

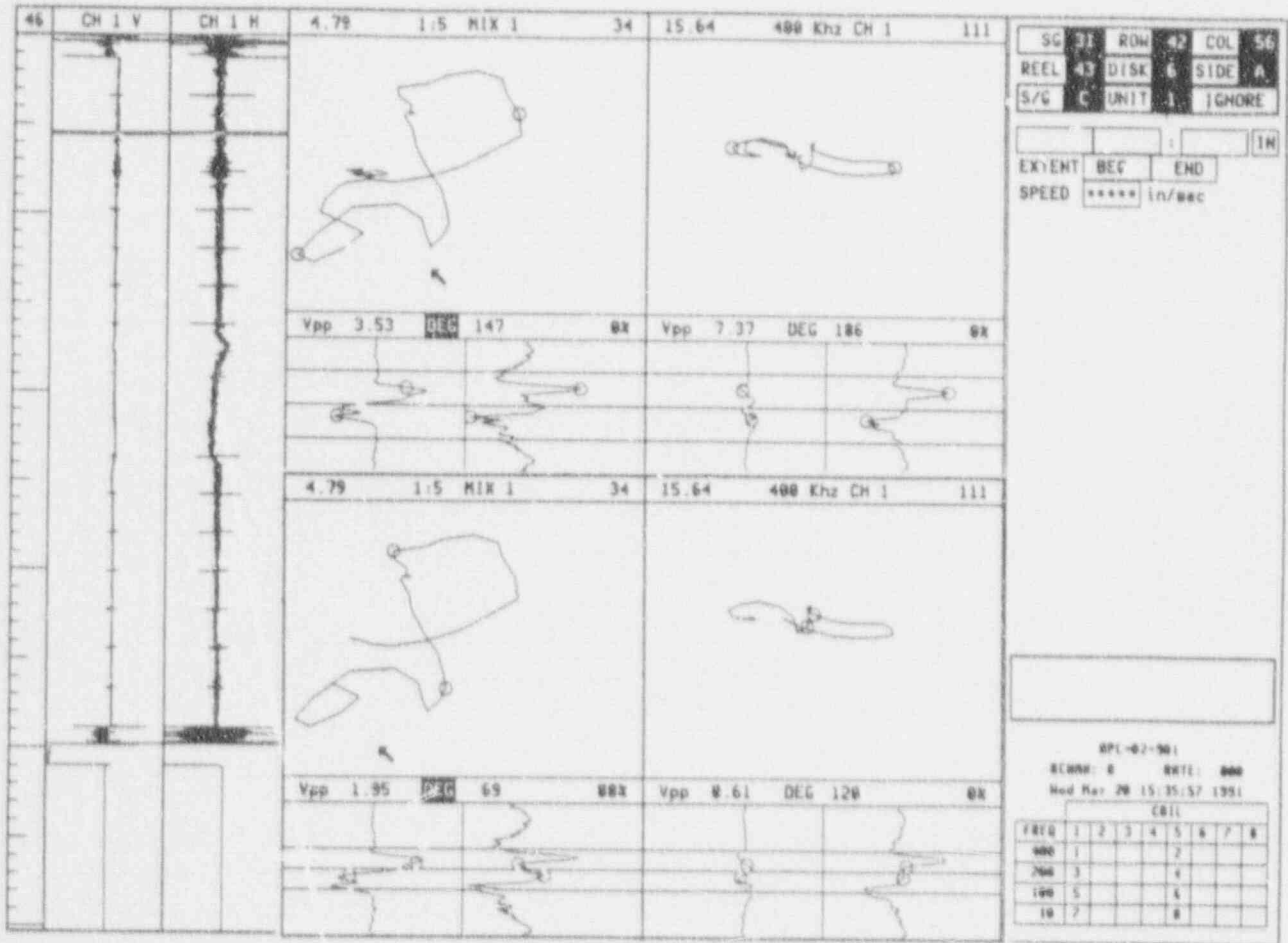


Figure A-17. Example of Bobbin Coil Field Data - Flaw Signals for ODSCC at Dented TSP Intersection

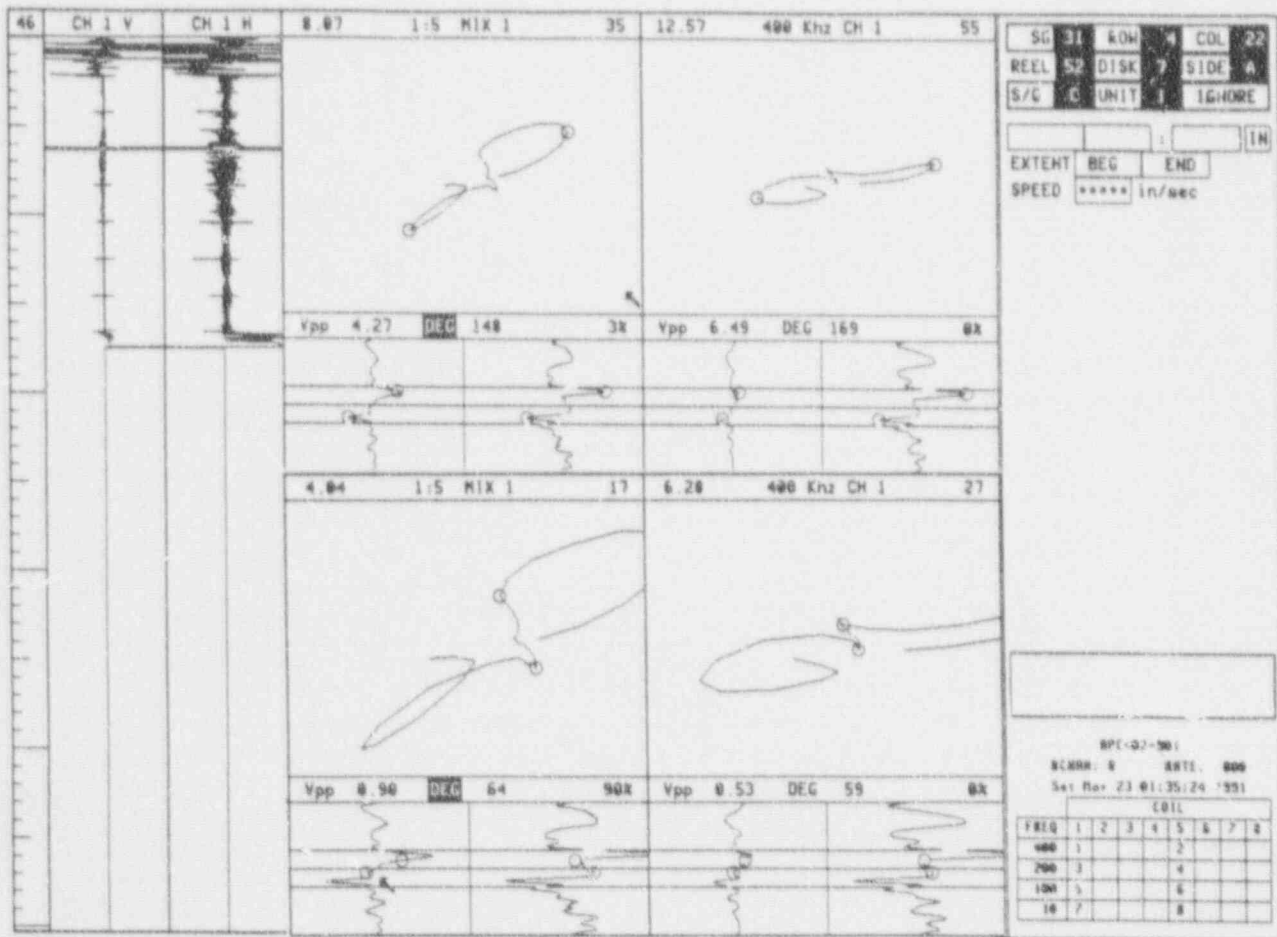


Figure A-18. Example of Bobbin Coil Field Data - Flaw Signals for ODSCC at Dented TSP Intersection

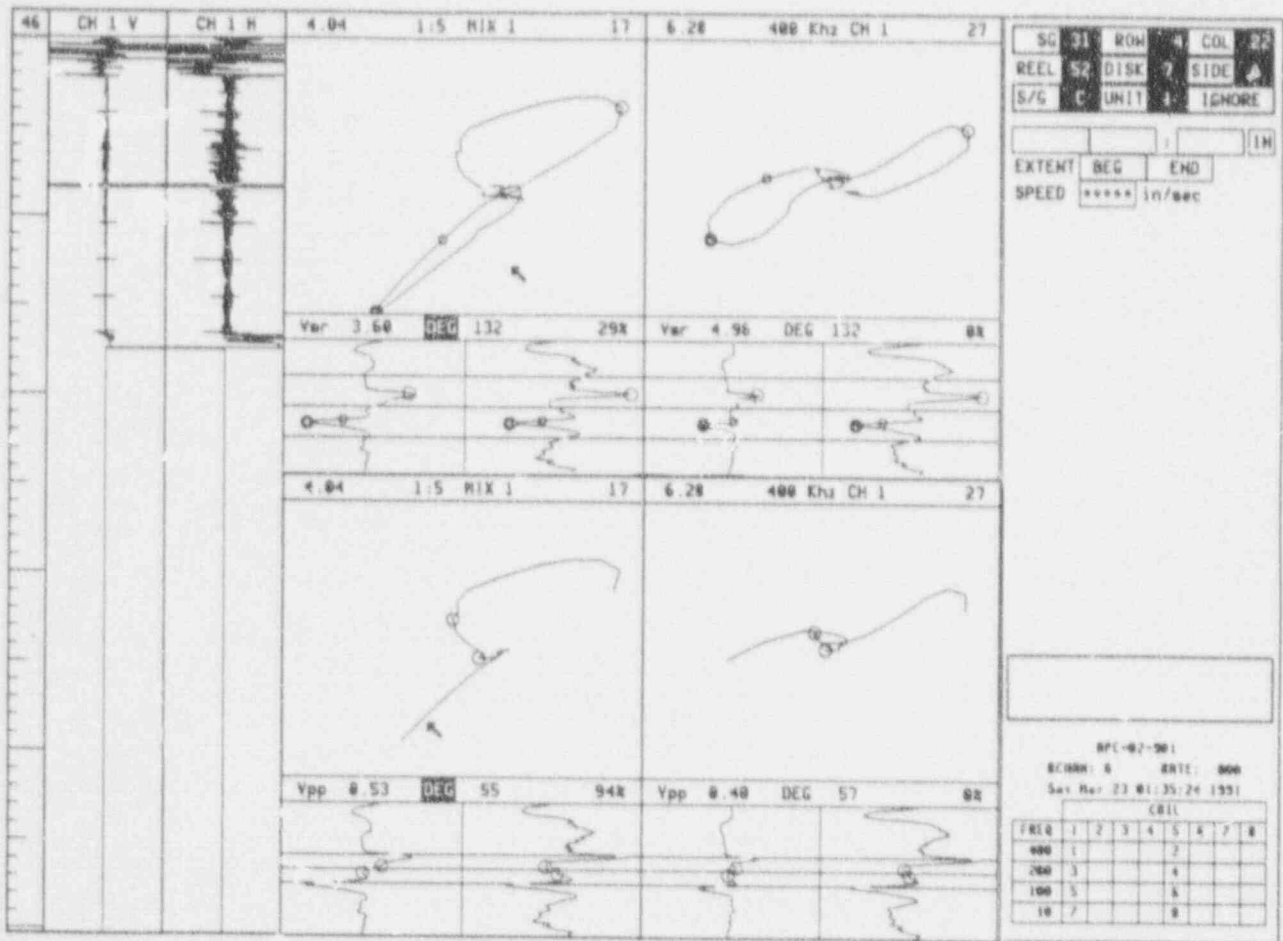


Figure A-19. Example of Bobbin Coil Field Data - Flaw Signals for ODSCC at Dented TSP Intersection

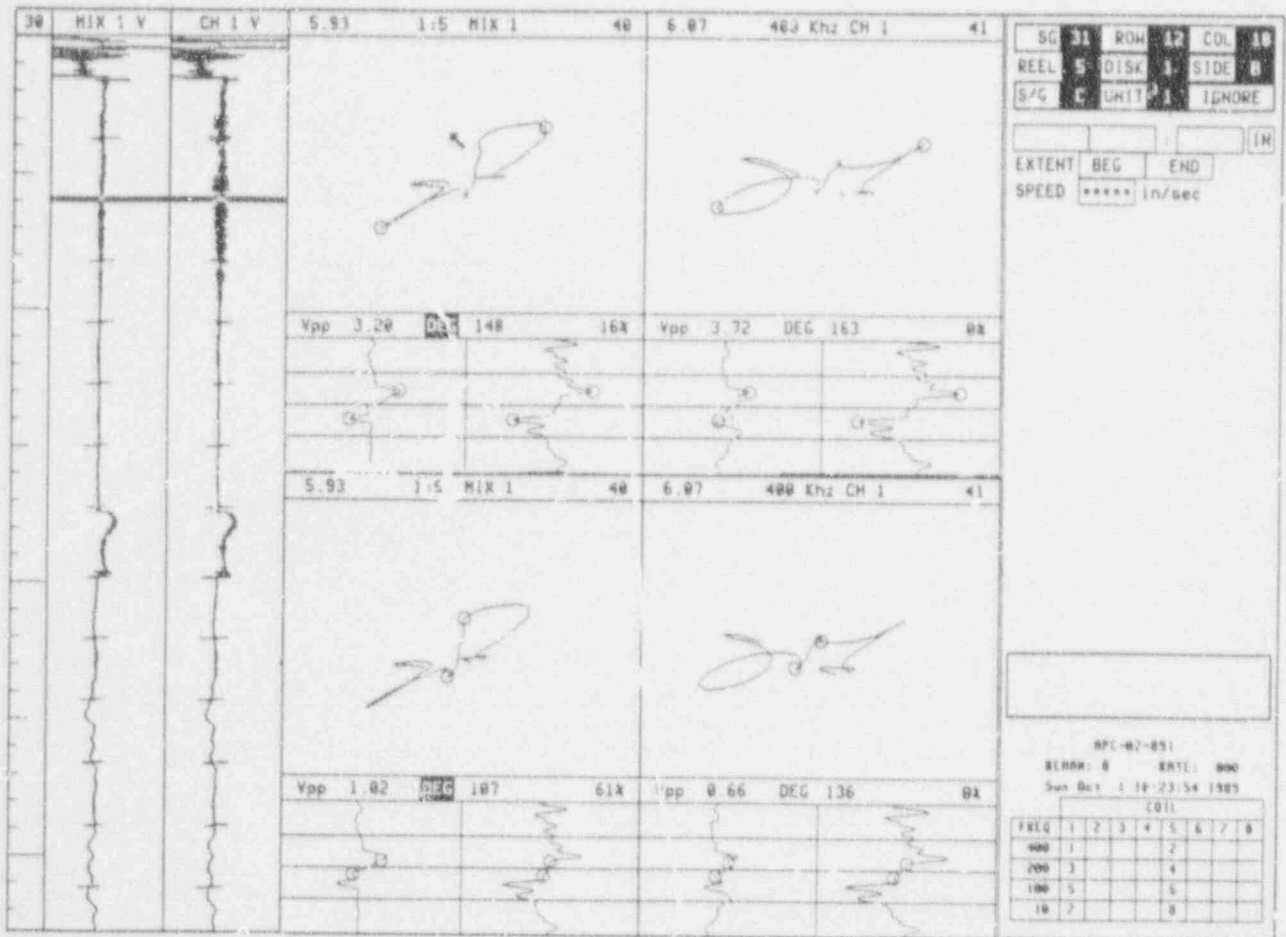
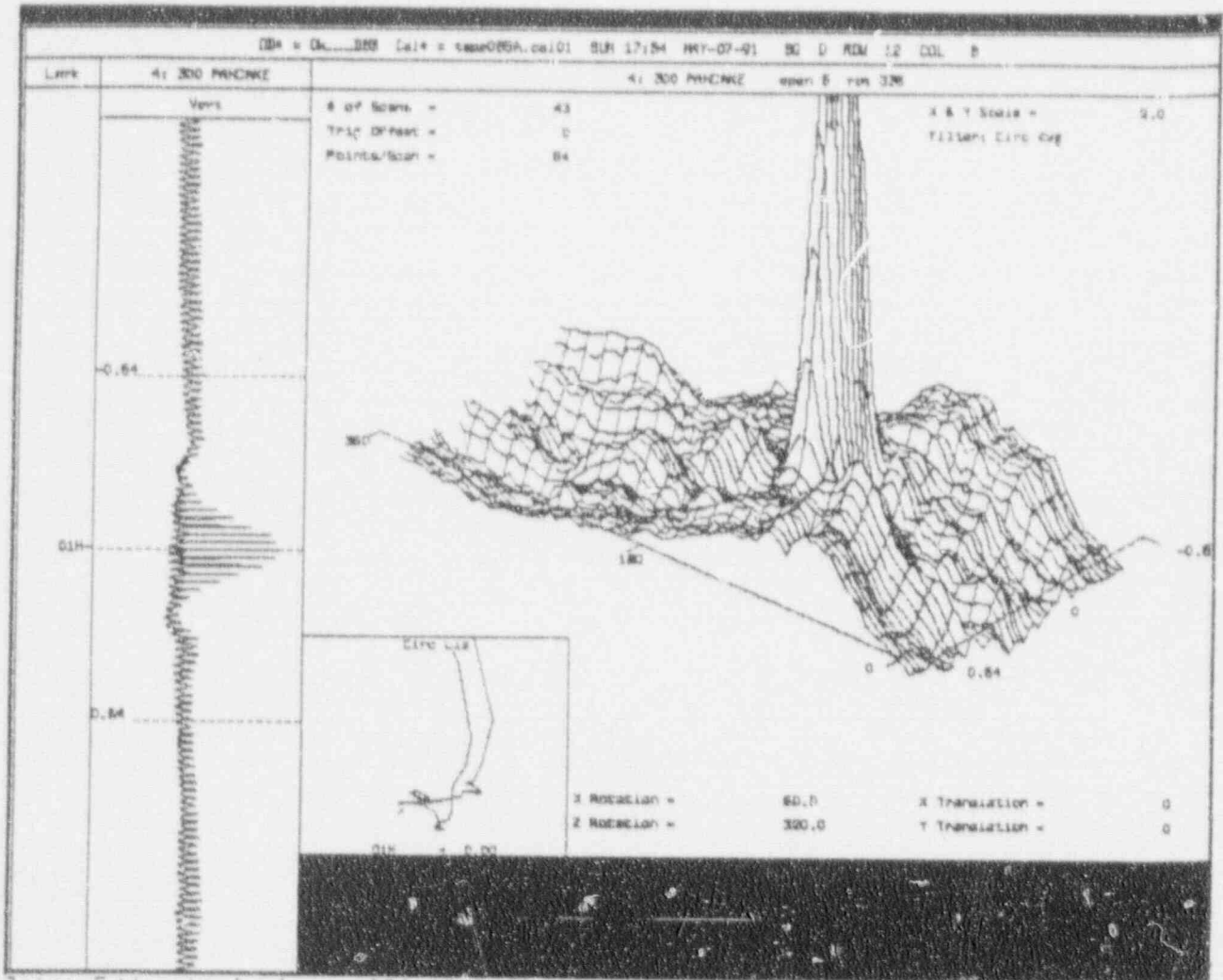


Figure A-20. Example of Bobbin Coil Field Data - Flaw Signals for ODSCC at Dented TSP Intersection



Zetec-Eddynet: Analysis [C]-1989,90 [user] as pri] at ND13f546

Figure A-21. Example of Axial ODSOC Indication at TSP 1

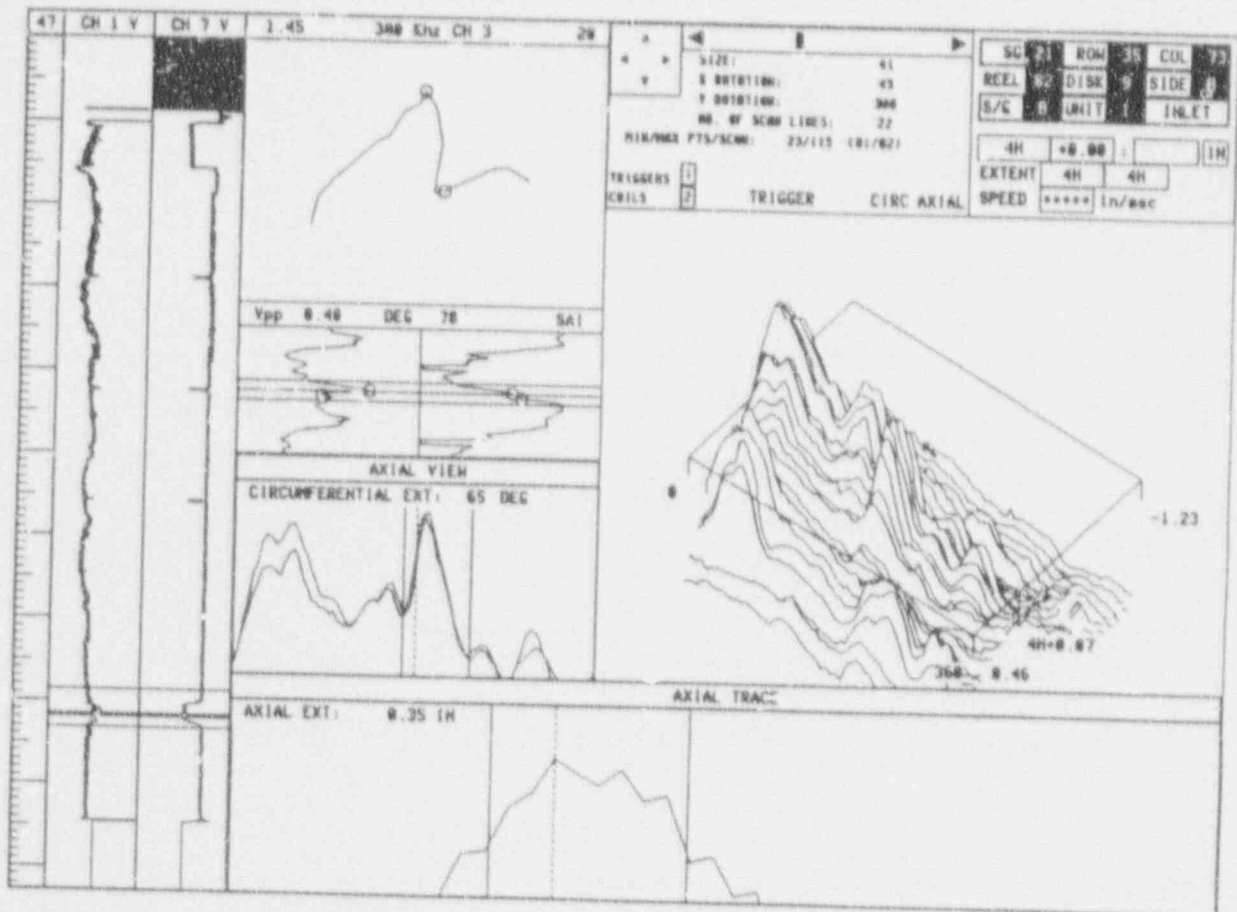


Figure A-23. Axial ODS (On-Demand Signal Control) Indications (MAI) at TSP - Farley Unit 1

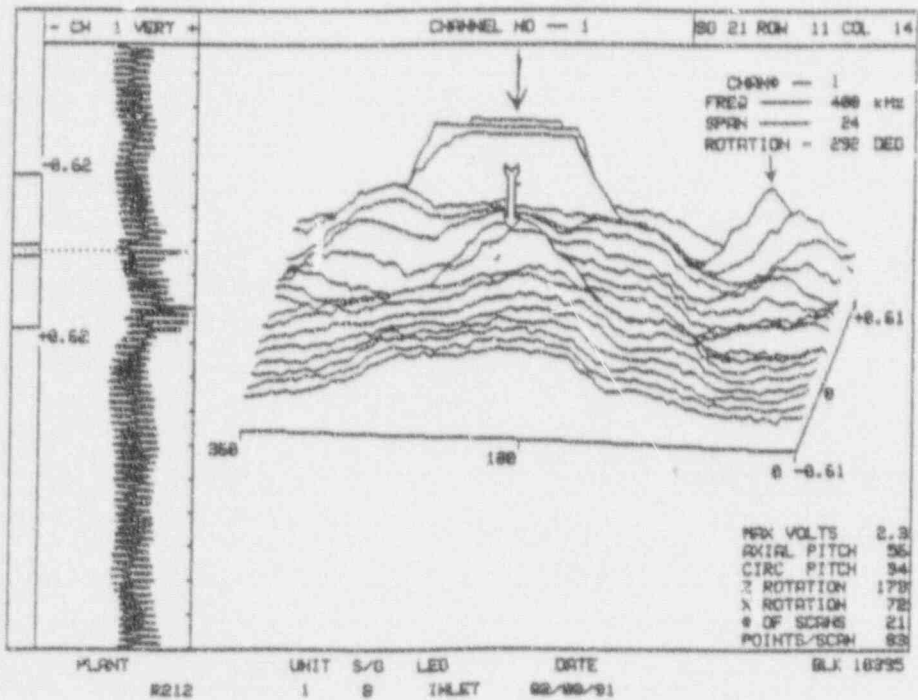
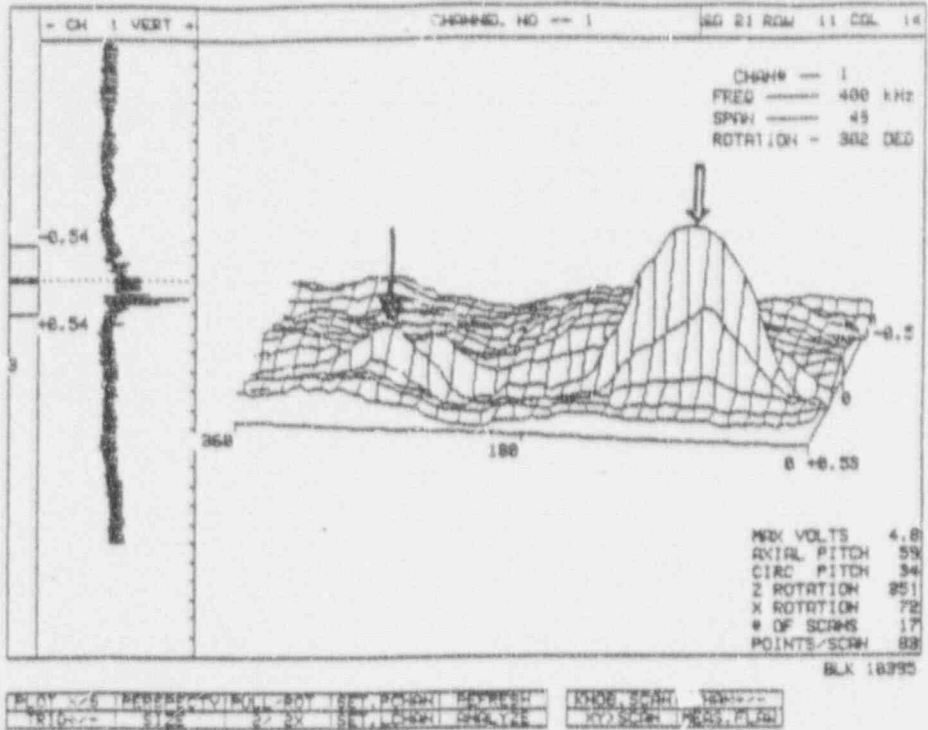
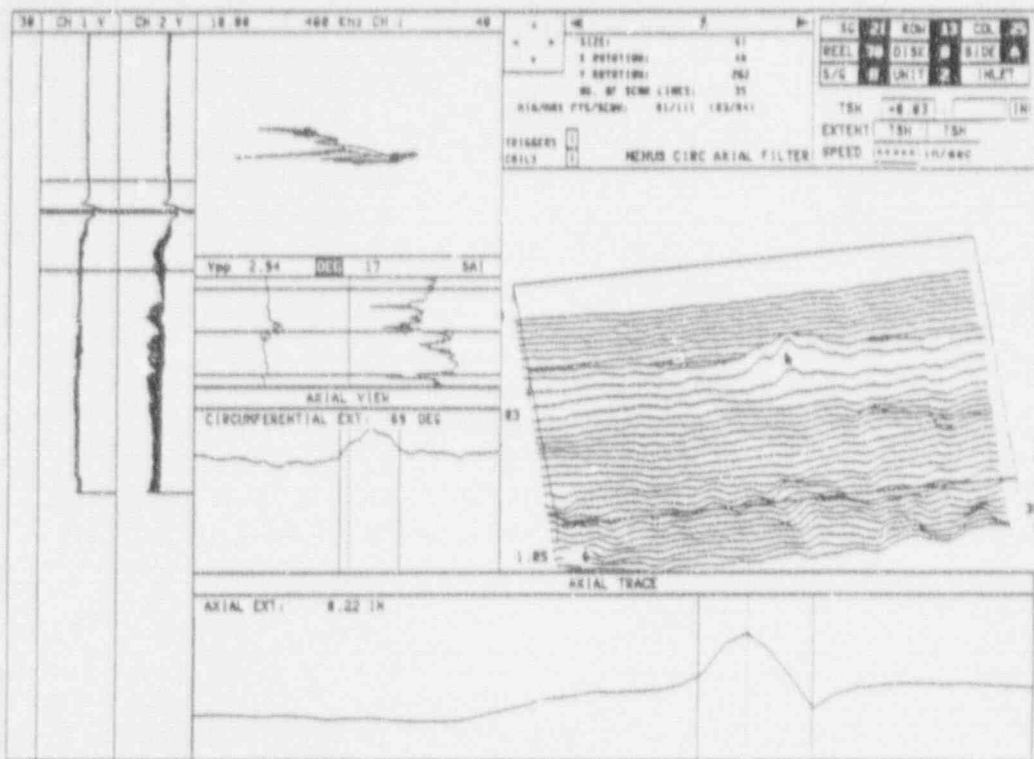
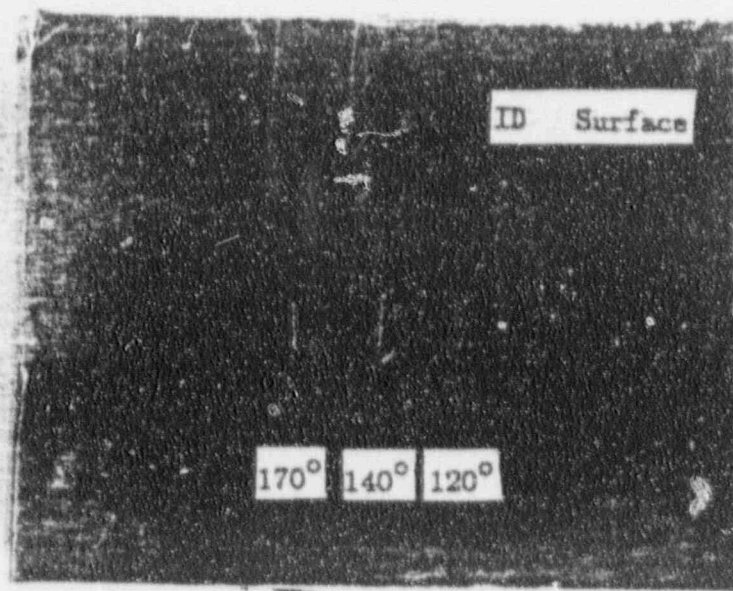


Figure A-24. Circumferential ODSCC Indications at Support Plates



Top



Mag. 3X

Figure A-25. Farley-2 RPC and Destructive Exam Results for Closely Spaced Axial Cracks

Appendix B

Evaluation of Pulled Tubes from Plant L

Data from Plant L provide the most extensive pulled tube support for the voltage/burst correlation of this report. The Farley pulled tubes include higher amplitude indications than Plant L. Thus the data from Farley and Plant L are complementary. The eight pulled tubes from Plant L include 24 intersections of which 23 were destructively examined and 21 were burst. Due to the importance of the Plant L data to the voltage/burst correlation, the field eddy current data tapes were reevaluated by applying the analysis guidelines of Appendix A. In addition to reanalysis of the bobbin and RPC voltages, this reevaluation included adjustments for differences between ASME calibration standards for Plant L and the reference laboratory standard. These results as well as general considerations of crack morphology are described in this report.

B.1 Summary of Plant L Data

Table B.1 provides a summary of the field eddy current inspection results as well as a summary of the destructive examination results. From the pre-pull bobbin coil data in Table B.1, it is seen that all 24 intersections were detected by the bobbin coil inspection of Plant L. The bobbin results represent a Phase 2 inspection that used analysis guidelines for calling indications that are essentially the same as Appendix A. Voltage amplitude analyses in the field inspection did not have the emphasis required by Appendix A for application to the repair criteria of this report. For this reason, the voltage amplitudes are reevaluated from the field data tapes in Section B.3.

As noted in Table B.1, the RPC analysis guidelines were progressively refined as the inspection progressed with Revision 3 as the final guidelines. The differences between Revision 2 and Revision 3 are that signals with a high noise to signal ratio were called 3:1 indications in Phase 2 while calls of SAI, MAI or NDD were made for the 3:1 indications in Phase 3. The RPC Phase 3 guidelines are generally consistent with Appendix A in that specific calls other than 3:1 are required in Appendix A. Interpretation of the noisy signals for indications near the threshold of detection can, however, be subjective calls. It can be noted that the average crack depths are < 40% for the RPC 3:1 and NDD indications and thus these indications do not represent a challenge to tube integrity. In addition, the bobbin voltages for the 3:1 and RPC NDD indications are about 1 volt or less and would be acceptable for continued service by the plugging limits of this report.

It is noted that two indications on tube R16C74 were NDD by RPC while all were detected by the bobbin inspection. These results support the use of the bobbin coil as the principal inspection for the repair limits of this report. These pulled tube results as well as other historical results show that, for indications near the detection threshold, some indications may be detected by either the bobbin or RPC probes but not both probes. As the indications become more significant and become closer to the repair limits of this report, the indications are detectable by both the bobbin and RPC probes.

B.2 Voltage Corrections for Calibration Standards

The field voltages for Plant L differ from the Appendix A guidelines both by differences in voltage normalization to the ASME standard and by differences between the as manufactured field and laboratory reference ASME standards.

The Plant L field voltages were normalized to 5.0 volts at 400 KHz for the 20% ASME holes. Appendix A requires this calibration at 4.0 volts. Thus the Plant L field voltages need to be multiplied by a factor of 0.8 for comparisons with the amplitudes of this report. This difference is further addressed in the reevaluation of the field data as described in Section B.3.

The influence of the as manufactured tolerances of the ASME standards on voltage amplitudes was evaluated in two ways. First, a transfer standard calibrated against the reference laboratory standard was sent to Plant L and the Plant L ASME standards were cross calibrated against the transfer standard and thus cross calibrated against the reference standard. Secondly, the Plant L ASME standards were sent to the laboratory and directly calibrated against the reference standard. The resulting calibration ratios of the Plant L ASME standards to the reference standard provide an assessment of the effectiveness of the transfer standard process.

The calibration ratios obtained in the field using the transfer standard are given as the field data in Table B.2. The calibration ratios represent the average of five measurements with the variability between measurements limited to < 4%. At 400 KHz, the calibration ratios for both Plant L standards were obtained as 0.86. However, at the 400/100 KHz mix, standard Z-7981 showed a calibration ratio of 1.12 compared to an average of about 0.86 for standard Z-7980. The calibration ratios would be expected to be similar for the two frequencies. This difference was further evaluated by shipping the field standards to the laboratory for further evaluation. Standard Z-7981 has a dent at the TSP simulation which could have some influence on the evaluation.

In the laboratory, calibration ratios were developed through application of the transfer standard and by direct comparison with the reference standard. In addition, both Westinghouse and Zetec probes were used. The results for the laboratory developed calibration ratios are given in Table B.2. The laboratory results show calibration ratios of essentially 0.86 for Z-7980 and 0.96 for Z-7981 at 400 KHz and 0.76 for Z-7980 and 0.88 for Z-7981 at the 400/100 KHz mix. No significant differences were found between use of the transfer or laboratory standard or between the Westinghouse and Zetec probes. The laboratory and field results show good agreement only for Z-7980 at 400 KHz.

To assess the differences between standards, the 20% ASME hole diameters and depths for the laboratory, Z-7980 and Z-7981 standards were accurately measured for the 20% holes. The laboratory standard had all hole diameters and depths within 0.5 mil of the ASME dimensions (0.187" diameter, 20% deep) and less than 1 mil variation in wall thickness. Standard Z-7981 also had all dimensions within 0.5 mil of the ASME dimensions, although on the average slightly smaller than the laboratory standard. These dimensions are consistent with the calibration ratio near unity (0.96). Standard Z-7980 had holes and depths undersized with up to 2 mils undersize for one hole. These dimensions are consistent with the lower calibration ratio for the standard. The

tolerances on all standards were considerably better than the three mils permitted by the ASME Code. Both Z-7980 and Z-7981 had wall thickness variations up to about 3 mils.

Based on consistency of the laboratory calibration ratios with the measured hole dimensions, the laboratory calibration ratios are applied in this report. However, as noted in the footnote of Table 7, three additional standards were used in the pre-pull inspections. Direct calibration ratios were not obtained for these standards. Bobbin data for tube R8C66 was available for both the Z-7980 and Z-6743. Bobbin voltages evaluated for the two data sets showed an average ratio of 1.03 for Z-7980/Z-6743. This ratio was applied to obtain an estimated calibration ratio of 0.9 for Z-6743. For standards Z-6738 and Z-7979, the calibration ratio is assumed to be 0.95.

B.3 Voltage Evaluation

The field bobbin coil data tapes were reevaluated using the analysis guidelines of Appendix A. For the reevaluation, the voltage was normalized to 4.0 volts in the 400 KHz mix for the 20% ASME flaw, which is consistent with the pulled tube and model boiler data normalization of this report. This is essentially equivalent to 4.0 volts at 400 KHz. The results of this evaluation are given as the Reevaluation-Evaluated column of Table B.3. The laboratory evaluation results of the field bobbin tapes are shown in Figures B-1 to B-8 for each intersection of the 8 pulled tubes. The field columns of Table B.3 show the as reported field values from Table B.1 and as renormalized to 4.0 volts at 400 KHz for the 20% ASME flaw. For voltages less than about 1 volt, the reevaluated voltages tend to be somewhat higher due to the guidelines emphasis on obtaining peak to peak voltages. For the 7 indications above 1 volt by both evaluations, the differences in voltage amplitudes are small.

The reevaluated voltages were then multiplied by the calibration ratios developed above in Section B.2 to correct for differences in ASME calibration standards. The corrected voltages are given in the Calibration Corrected (Cal.Corr.) column of Table B.3. These values are used for the Plant L data in the voltage/burst correlation and given in the pulled tube data summary of Table 6.2.

The Plant L RPC voltages were also reevaluated to the calibration standards of Appendix A. The field RPC voltages for the axial coil of the 3 coil probe were normalized to 20 volts for the 100% ASME hole at 300 KHz. The Appendix A calibration is based on the pancake coil normalized to 20 volts for a 0.5 inch long slot, 100% deep at 400 KHz. To obtain a renormalization factor for the field RPC voltages, field RPC data for 13 indications were reevaluated using the Appendix A calibration. The results of this reevaluation are given in the last column of Table B.3 for tube R16C74 and lower in the Table. The reevaluation indicates about a factor of 3 reduction in the field RPC voltages for the Appendix A normalization. This factor of 3 reduction was then applied to estimate the renormalized RPC voltages for the remaining tubes in Table B.3.

B.4 Post-Pull Eddy Current Considerations

To assess the potential influence of the tube pull loads on the crack morphology, post-pull bobbin coil data tapes were reevaluated for selected tubes. The indications were reevaluated by locating the position of the TSP based on identifying bobbin signals typical of the "Alloy Property Change" as discussed in Section A.3.3. Once the TSP location is identified, the bobbin data were evaluated for flaw voltages.

The results of the pre- and post-pull evaluations are given in Table B.4. The data show that, except for a few intersections, the post-pull bobbin voltages are much larger than the pre-pull voltages. As noted in Table B.4, the post-pull bobbin data commonly show clear evidence of tube deformation signals, which occurred during the tube removal process. The deformation signals are not present in the pre-pull data while the post-pull flaw signals are greatly distorted (where noted in Table B.4) by the deformation signals. Examples of the post-pull bobbin data are shown in Figures B-9 and B-10. Figure B-9 shows examples of indications influenced by tube deformation resulting from the tube pulling process. The large horizontal excursions of the Figure B-9 data are typical of dent-like deformation and are present in almost all post-pull data (Table B.4) and are totally absent in the pre-pull data (Figures B-1 to B-8). The horizontal distortion of the bobbin signal is most noticeable in the high frequency channel (400 or 600 kHz when available). In the extreme cases, the small flaw indications in the post-pull data are difficult to identify because of the contribution of the dent-like signals (TSP 2H of R29C70, for example). In a few cases, the post-pull data show almost no evidence of tube degradation on the bobbin signal as shown for the second TSP of R8C66 in Figure B-10. It should be noted that the presence of dent-like signals results in conservative voltage calls compared to the pre-pull data. It is generally more difficult to identify the pure flaw signal from the larger, more complex dent plus flow signal.

Where noted in Table B.4, the deformation has led to conservative increases in the flaw signal. In these cases, the bobbin data cannot be confidently used to imply evidence of crack opening as the cause of the increased voltages. Only the R8C69 indications and the second TSP of R8C66 appear to have post-pull indications useful for comparing with pre-pull bobbin. The R8C69 first TSP indication shows an increased voltage while the R8C66 indication is unchanged. The RPC data of Table B.1 and Table B.4 should be more useful than the bobbin data for assessing the effects of tube pulling forces as the RPC is less sensitive to tube deformation than the bobbin probe. Except for R20C66 and to a lesser extent R12C70, the RPC post-pull voltages of Table B.1 show significant voltage increases compared to pre-pull data for at least one intersection per tube. The increases in RPC voltages may be attributable to changes in crack morphology such as opening of cracks or tearing of ligaments.

B.5 Crack Morphology Considerations

The Plant L pulled tubes cover a range of crack morphologies as described in Section 4. Table B.5 provides a summary of the crack morphologies for the Plant L pulled tube intersections for which burst pressures were obtained. For comparisons, the burst Farley pulled tube morphologies are also summarized. While ODSCC is the principal

crack morphology for all the indications in Table B.5, the indications involve varying degrees of volumetric IGA and cellular SCC.

The influence of the combined crack morphologies on the burst capability of the tubes can be assessed by comparing the measured burst pressures with the predicted burst pressures obtained from burst models for single cracks simulated by EDM slots. If the combined crack morphologies are influential on the burst pressure, the measured burst pressures would be lower than obtained for the single crack model. For these comparisons, the crack lengths and average crack depths as determined from destructive examination are used for the predicted burst pressures. The lengths and average depths for the Plant L tubes are given in Table B.1.

Cracked tubes have been evaluated based on tubes with EDM slots to various depths (WCA P-8429). The burst pressure of partial depth slots has been verified to be predictable using the equation,

$$P = \frac{2\sigma_y t}{\sqrt{a}} \quad \text{ja,b}$$
$$P = \frac{2\sigma_y t}{\sqrt{a}} \quad \text{ja,b}$$

Figure B-11 shows the comparison between the measured burst pressures for Farley, Plant L, and Plant P, and the calculated burst pressures using the single crack model. The circles in the figure represent the burst test data on EDM slots used to formulate the burst model. The above slot equation was applied using the nominal tube dimensions and lower tolerance limit material properties as the actual material properties are not available for all the pulled tubes.

The calculated burst pressures were obtained at room temperature for direct comparison with the measured burst pressures. From Figure B-11, it is seen that the measured burst pressures exceed the predicted burst pressures for a single crack equal to the length and average depth of the pulled tube intersections. The higher measured burst pressures are likely due to material properties exceeding lower tolerance values (judged to be about 5-10% increase in burst pressure) and to presence of ligaments in the corrosion cracks which are not accounted for in the model.

Overall, it is concluded from Figure B-11 that the combined crack morphologies of the Farley, Plant L and Plant P tubes result in burst capability associated with the limiting single crack in an otherwise undegraded tube. That is, the combined crack morphologies do not significantly reduce the burst pressure of the limiting ODSCC crack.

B.6 Assessment of Tube R8C69

The burst pressure of 5900 psi for the 1st TSP of tube R8C69 is lower than the general trend of data for the voltage/burst correlation. Additional destruction examinations were performed on this tube to look for potential contributors to either a low burst pressure or a low bobbin voltage as described in Section 4.

From Figure B-9, the 5900 psi burst pressure is consistent with the crack length and

average depth. Tensile tests performed on pieces of the pulled tube show material properties above nominal such that the material condition would not lead to a reduced burst pressure. In addition, uncorroded ligaments were found in the burst crack which would tend to increase rather than reduce the burst pressure. Thus, the burst pressure is in the expected range for the crack size found by destructive examination. Thus, a lower than expected bobbin voltage for the crack size is the most likely contributor to the lower probability position of this indication on the voltage/burst correlation.

Potential contributors to lower than expected voltage for this indication could include: eddy current analysis variability in defining the voltage from the bobbin data; influence of deposits in the crevice or on the tube surface or crack face; or small uncorroded ligaments which could conceptually reduce the voltage amplitude but not significantly increase burst pressure. As shown in Table B.3 for R8C69, 1st TSP, the voltage difference between the field and Westinghouse analysis is only 0.17 volts which would not significantly influence the voltage/burst correlation position for this point. The EC signal is not particularly difficult to evaluate and the 0.17 volt difference between analyses is primarily due to the emphasis on calling peak to peak voltages in the Westinghouse analysis.

Energy dispersive spectroscopy (EDS) analyses were performed on the crack surface and crack face to examine for surface deposits such as copper plating which might influence the eddy current response. No copper was present on the crack face and only discrete spots of copper indicating a particulate form were found on the tube surface. The particulate form of copper would not significantly influence the eddy current response. No deposits were identified that would be expected to significantly influence the voltage response. Similarly, deposits in the tube to TSP crevice, which cannot be specifically evaluated for the pulled tube, are not expected to substantially influence the eddy current response. Thus there is not data to indicate that deposits influenced the measured bobbin voltage.

SEM fractography was performed on all three intersections of tubes R8C69, R12C70 and R30C64 to quantify microcrack sizes (depth/length) and the size of uncorroded ligaments between microcracks. Evaluations of these data were made with regard to average microcrack length, average length between uncorroded ligaments, and uncorroded ligament widths. Average microcrack lengths were within a factor of 2 for all nine indications with no substantial difference between the 1st TSP of R8C69 and the other indications. Similarly, the sum of uncorroded ligament widths for this indication is toward the middle of the range found for the 9 indications. Overall, no particular features of the crack morphology were identified that could explain the low voltage amplitude.

The post-pull bobbin and RPC data of Tables B.1 and B.4 show voltage increases of about a factor of two for the first TSP of R8C69 compared to pre-pull data. These results would indicate that some change to crack morphology occurred as a consequence of the tube pulling forces, such as tearing of ligaments at the openings of tight cracks. However, there is no way to quantify this effect. For conservatism, only pre-pull bobbin voltages are used in the voltage/burst correlation of this report.

In summary, the burst pressure for the first TSP of R8C69 is consistent with the crack

size while the bobbin voltage is lower than normally found for the associated crack size. No specific cause for the lower voltage amplitude has been identified other than potential changes to crack morphology during the tube pull, such as loss of ligaments.

Table B.1. Summary of Plant L Pulled Tubes

10 Jan 92

ESTIMATED CRACK DEPTH MEASUREMENTS BASED ON FAX FROM ABB-CE DATED 1-10-92

TUBE	TSP S/G	Year	Tube	Max (I) Depth	Avg Crack	Number of cracks on transverse section	Leak/Duct Pressure in PSI	R/S/E/C Burst Crack Length	Number of Microcracks in Burst Crack	Max Length At Max Depth	Average Crack Length	Pre Pull 400/100		Pre Pull Bobbin Coil Phase Angle	Pre Pull 100 kHz Axial Coil MRPC Voltage	Original MRPC Signal Call	(Rev 2) Pre Pull MRPC Signal Call	(Rev 3) Pre Pull MRPC Signal Call	Post Pull 400/100		Post Pull Bobbin Coil Phase Angle	Post Pull 100 kHz Axial Coil MRPC Voltage	Post Pull MRPC Signal Call
												Bobbin Coil Voltage	Bobbin Coil Signal						Bobbin Coil Voltage	Bobbin Coil Signal			
R12C8	1	D	1989	92%	38%	83*	3300	Circum.	N/A	N/A	5.17	82%	64	4.35	MAI	MAI	MAI	12.51	69%	78	7.26	MA	
				N.B.T.	N.B.T.	50	N.B.T.	N/A	N/A	N/A	N/A	1.72	55%	96	0.50	N.T.	MAI	MAI	3.29	67%	80	1.16	SA
				55%	42%	85	10500	0.75	22	0.08	0.93	1.14	55%	96	0.52	N.T.	MAI	MAI	7.75	18%	127	5.87	SA
R29C70	1	C	1991	74%	54%	3	10400		1	0.17	0.29	0.31	42%	81	0.16	SAI	SAI	MAI	1.74	3%	139	1.05	SA
				74%	49%	5	9000	0.62	12	0.23	0.05	0.37	64%	85	0.34	MAI	MAI	SAI	6.07	0%	171	0.79	MA
				70%	34%	4	10400	0.45	6	0.16	0.08	0.34	0%	357	0.25	N.T.	3:1	SAI	7.00	0%	187	0.49	SA
R30C64	1	C	1991	55%	37%	29	10500	0.53	12	0.18	0.04	0.62	43%	105	0.15	N.T.	3:1	SAI	3.35	92%	41	1.30	MA
				62%	50%	85	8800	0.61	8	0.28	0.08	1.13	93%	55	0.57	MAI	MAI	MAI	9.42	0%	158	2.74	MA
				49%	33%	30	10200	0.45	6	0.12	0.08	1.12	0%	139	0.59	N.T.	3:1	MAI	4.86	0%	147	1.37	MA
R16C74	1	D	1991	69%	48%	44	8600	0.56	14	4.35	0.04	1.12	85%	57	0.66	MAI	MAI	MAI	7.16	64%	87	0.98	MA
				60%	27%	62	9500	0.30	9	0.25	0.03	0.87	17%	4	N/A	N.T.	NDD	NDD	8.60	49%	100	0.31	MA
				49%	33%	43	10400	0.25	6	0.13	0.04	0.99	35%	14	N/A	N.T.	NDD	NDD	4.53	42%	24	0.49	SA
R20C66	1	D	1990	76%	37%	60	8150	0.60	22	0.21	0.03	2.58	66%	78	1.18	MAI	MAI	MAI	7.98	28%	114	0.97	MA
				58%	45%	27	8750	0.48	12	0.17	0.06	2.25	64%	80	1.51	SAI	MAI	MAI	8.65	1%	156	1.10	MA
				56%	41%	34	9300	0.40	8	0.33	0.08	0.99	92%	44	0.80	N.T.	MAI	MAI	2.81	35%	109	0.76	MA
R8C66	1	D	1989	96%	54%	41	7500 / 7500	< 0.75	10	0.66	0.08	3.12	67%	77	2.16	MAI	MAI	MAI	6.00	40%	110	1.60	MA
				72%	40%	8	8750	< 0.75	16	0.68	0.06	1.64	73%	29	0.54	N.T.	MAI	MAI	2.11	43%	134	0.79	MA
				52%	31%	2	10600	0.50	10	0.11	0.05	0.5	48%	19	0.18	N.T.	SAI	SAI	0.49	63%	86	N/A	NDD
R8C69	1	C	1991	98%	59%	46	5500 / 5900	0.86	16	0.27	0.05	1.08	94%	45	0.94	MAI	MAI	MAI	4.38	56%	98	1.91	MA
				81%	55%	47	7700	< 1.00**	17	0.06	0.04	0.89	91%	36	0.30	N.T.	MAI	MAI	1.92	52%	119	0.28	MA
				69%	45%	60	9700	0.49	11	0.06	0.04	1.13	94%	45	0.25	SAI	MAI	MAI	1.55	84%	63	0.33	MA
R12C70	1	C	1991	97%	60%	44	7100	0.78	16	0.87	0.05	1.34	97%	35	0.40	MAI	MAI	MAI	4.82	72%	83	0.63	MA
				N/A	N/A	24	N.B.T.	N/A	N/A	N/A	N/A	0.73	84%	32	0.27	N.T.	SAI	SAI	1.84	90%	56	0.53	MA
				64%	44%	38	9500	< 0.75	14	0.69	0.05	0.59	87%	52	0.20	N.T.	SAI	SAI	1.04	61%	90	0.47	MA

N.B.T. Not burst tested

SAI = Single Axial Indication

MAI = Multiple Axial Indications

NDD = No detectable defects

*Approximately 400 cracks are noted from the S.E.M. mount

R12/C8 1st TSP Post pull ECT was not done using a standard TSP ring.

Burst Crack Lengths shown not corrected for elongation during burst test

MRPC 3 coil probe effective sensing element size (axial) 0.040"

** Very conservative estimate, includes burst deformation

(1) Per ABB-CE measurements

N.T. Not Tested (No bobbin coil PI per Spring 1991 analysis criteria)

Revision 0 to the MRPC data analysis guidelines April 4, 1991

Revision 1 to the MRPC data analysis guidelines August 1991

Revision 2 to the MRPC data analysis guidelines August 1991(revised for clarification)

Revision 3 to the MRPC data analysis guidelines October 1991

Bobbin coil voltages set to 5.0 V for the 20% TWI on the ASME cal. std.

FILE: 1992 EXCEL BA TPREV1.XLS

Table B.2. Calibration Ratios of Plant L ASME Standards to Reference Standard(1)

Trojan Std.	Plant L Field Data ⁽²⁾		Laboratory Measurements			
	400 KHz	400/100 KHz	Transfer Standard ⁽²⁾		Laboratory Standard	
	400 KHz	400/100 KHz	400 KHz	400/100 KHz	400 KHz	400/100 KHz
Z-7980	0.85	0.84	0.87 ⁽⁵⁾	0.76 ⁽⁵⁾	0.88 ⁽⁵⁾	0.78 ⁽⁵⁾
	0.86	0.87 ⁽³⁾	0.85 ⁽⁶⁾	0.76 ⁽⁶⁾	0.84 ⁽⁶⁾	0.75 ⁽⁶⁾
Z-7981 ⁽⁴⁾	0.80	1.12	0.96 ⁽⁵⁾	0.86 ⁽⁵⁾	0.96 ⁽⁵⁾	0.88 ⁽⁵⁾
			0.96 ⁽⁶⁾	0.89 ⁽⁶⁾	0.95 ⁽⁶⁾	0.88 ⁽⁶⁾

Notes:

1. Calibration ratios are ratios of voltage for Trojan ASME standards to reference laboratory standard for 20% ASME hole set to 4.0 volts at 400 KHz and 2.75 volts at 400/100 KHz mix.
2. Plant L calibration ratios relative to laboratory standard as obtained using transfer standard and guidelines applied for field data.
3. Repeat measurement.
4. Values obtained on 1/2 TSP ring due to small dent on standard TSP.
5. Values using Zetec probe (Z-720 SFRM/HF).
6. Values using Westinghouse probe (EP-720-BBM).
7. Application of Plant L standards to pulled tubes:
 - o Z-7980: R8C66
 - o Z-7981: R8C69, R12C70, R29C70, R30C64 (1H,3H)
 - o Z-6738: Not cross calibrated to lab. std.: R30C64 (2H)
 - o Z-6743: Not cross calibrated to lab. std.: R16C74, R20C60, R8C66
 - o Z-7979: Not cross calibrated to lab. std.: R12C8

Table B.3. Voltage Amplitudes for Plant L Pulled Tubes

Tube	ISP	Field - Bobbin		Lab. Reevaluation - Bobbin		RPC Voltages	
		Reported	Renorm. ⁽¹⁾	Evaluated	Cal.Corr. ⁽²⁾	Field	Renorm. ⁽³⁾
R12C8	1	5.17	4.14	4.03	3.83	4.35	1.45(4)
	2	1.72	1.38	1.45	1.38	0.50	0.17(4)
	3	1.14	0.91	0.90	0.85	0.52	0.17(4)
R29C70	1	0.31	0.25	0.67	0.64	0.16	0.05(4)
	2	0.37	0.30	0.29	0.28	0.34	0.11(4)
	3	0.34	0.27	0.61	0.59	0.25	0.08(4)
R30C64	1	0.62	0.50	0.70	0.67	0.15	0.05(4)
	2	1.13	0.90	1.33	1.26	0.57	0.19(4)
	3	1.12	0.90	1.13	1.08	0.59	0.20(4)
R16C74	1	1.12	0.90	0.96	0.86	0.66	0.24
	2	0.87	0.70	0.67	0.60	NDD	< 0.1 (NDD)
	3	0.99	0.80	1.08	0.97	NDD	< 0.1 (NDD)
R20C66	1	2.58	2.06	2.02	1.82	1.10	0.32
	2	2.25	1.80	1.74	1.57	1.51	0.45
	3	0.99	0.80	0.69	0.62	0.80	0.36
R8C66	1	3.12	2.50	2.66	2.29	2.16	0.75
	2	1.66	1.33	1.49	1.28	0.56	0.2
	3	0.5	0.40	1.03	0.88	0.18	0.07
R8C69	1	1.08	0.86	1.03	0.99	0.96	0.46
	2	0.89	0.71	0.97	0.93	0.30	< 0.1(5)
	3	1.13	0.90	1.08	1.04	0.25	< 0.1(5)
R12C70	1	1.34	1.07	1.24	1.19	0.40	0.33
	2	0.73	0.58	0.7	0.67	0.27	< 0.1(5)
	3	0.59	0.47	0.56	0.54	0.20	< 0.1(5)

Notes:

1. Reported field voltages renormalized from 5.0 volts to 4.0 volts (normalization of this report) for 20% ASME hole at 400 KHz.
2. Voltages corrected for calibration standard differences between field and reference standard (See Table B.3)
3. Reported field RPC voltages reevaluated and renormalized from 300 KHz pancake coil amplitude of 20 volts for 100% ASME hole to 20 volts for 0.5 inch, 100% slot at 400 KHz.
4. Voltages reduced by factor of 3 based on other reevaluated results given in this column.
5. Noisy signals preclude further refinement of voltages. Indications are present.

Table B.4. Comparison of Pre- and Post-Pull Voltages

Tube	ISP	Pre-pull Voltage	Post-pull Voltage	Comments on Post-pull Data
<u>Robbin Voltages</u>				
R12C8	1	4.03	12.5	Dent influence on flaw voltage*
	2	1.45	3.3	Dent influence on flaw voltage
	3	0.90	7.8	Dent influence on flaw voltage
R29C70	1	0.67	1.39	Dent influence on flaw voltage
	2	0.29	4.84	Dent influence on flaw voltage
	3	0.61	5.80	Not sure of signal
R30C64	1	0.7	2.7	Dent influence on flaw voltage
	2	1.33	7.44	Dent influence on flaw voltage
	3	1.13	4.5	Dent influence on flaw voltage
R16C74	1	0.96	1.6	Dent influence on flaw voltage
	2	0.67	1.45	Dent influence on flaw voltage
	3	1.08	4.3	Not sure of signal
R20C66	1	2.02	7.1	Dent influence on flaw voltage
	2	1.74	7.7	Dent influence on flaw voltage
	3	0.69	2.35	Dent influence on flaw voltage
R8C66	1	2.66	3.4	Dent influence on flaw voltage
	2	1.49	1.39	No dent influence
	3	1.0	0.6	Not sure of signal
R8C69	1	1.03	2.0	Dent influence on flaw voltage
	2	0.97	0.9	Minor dent influence
	3	1.08	1.53	Minor dent influence
R12C70	1	1.24	3.39	Dent influence on flaw voltage
	2	0.7	1.37	Dent influence on flaw voltage
	3	0.56	0.91	Dent influence on flaw voltage
<u>RPC Voltages</u>				
R8C66	1	0.75	1.3	
	2	0.2	0.8	
	3	0.07	0.5	
R8C69	1	0.46	1.6	
	2	<0.1	0.35	
	3	<0.1	0.46	

* Dent influences on signals are result of tube deformation during the tube pulling process such as bending, indentations, etc.

Table B.5. Summary of Crack Morphologies for Burst Farley and Plant L Tubes

Tube	TSP	General Description	Azimuthal Crack Density	Additional Observations on Corrosion
Farley Pulled Tubes				
R4C73	1	SCC	Low*	Minor IGA (~12 mils) at crack face
R21C22	1	SCC	Low	Minor IGA (~30 mils) at crack face
R38C46	1	SCC	Low	Cellular or spider shaped crack distribution in one local area
Plant L Pulled Tubes				
R12C8	3	IGA/SCC	Moderate*	Intermittent pitch IGA present (max. 21% deep)
R29C70	1	SCC	Low	
	2	SCC	Low	
	3	SCC	Low	
R30C64	1	SCC	Moderate	Local cellular IGA/SCC possibly present
	2	SCC	Moderate	
	3	SCC	Moderate	
R16C74	1	SCC	Moderate	A few patches of IGA present (19% max depth) cellular IGA/SCC locally present
	2	SCC	Moderate	A few patches IGA (17% max)
	3	SCC	Moderate	
R20C66	1	SCC	Moderate	Cellular IGA/SCC probably locally present
	2	SCC	Moderate	
	3	SCC	Moderate	Cellular IGA/SCC possibly locally present
R8C66	1	SCC	Moderate	Cellular IGA/SCC probably locally present
	2	SCC	Moderate	Cellular IGA/SCC possibly locally present
	3	SCC	Moderate	
R8C69	1	SCC	Moderate	Cellular IGA/SCC probably locally present
	2	SCC	Moderate	Intermittent patch IGA (<10% max) Cellular IGA/SCC possibly locally present
	3	SCC	Moderate	Cellular IGA/SCC probably locally present
R12C70	1	SCC	Moderate	
	3	SCC	Moderate	Cellular IGA/SCC possibly locally present

*Low <25 cracks, moderate 25-100 cracks

Figure B-3

Laboratory Evaluation of Bobbin Data from Tube R30 C64 at TSPs 3H, 2H and 1H

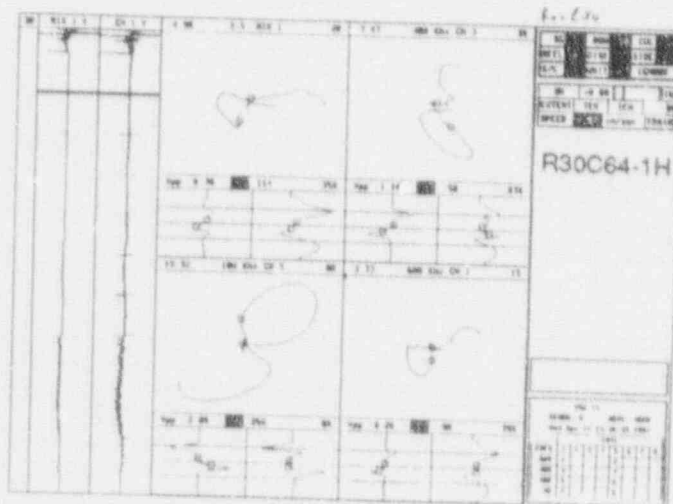
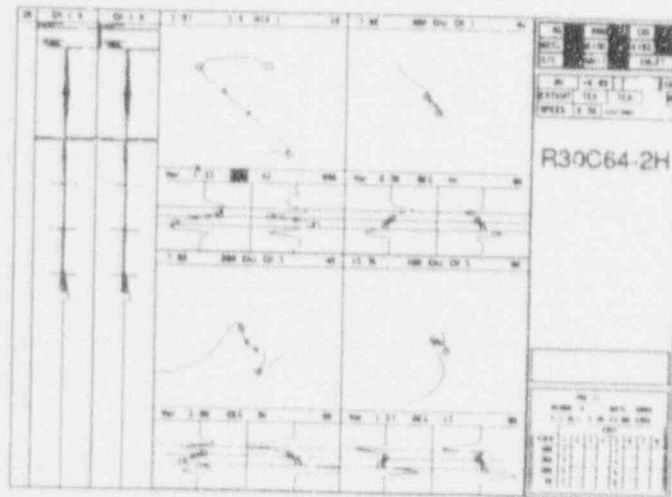
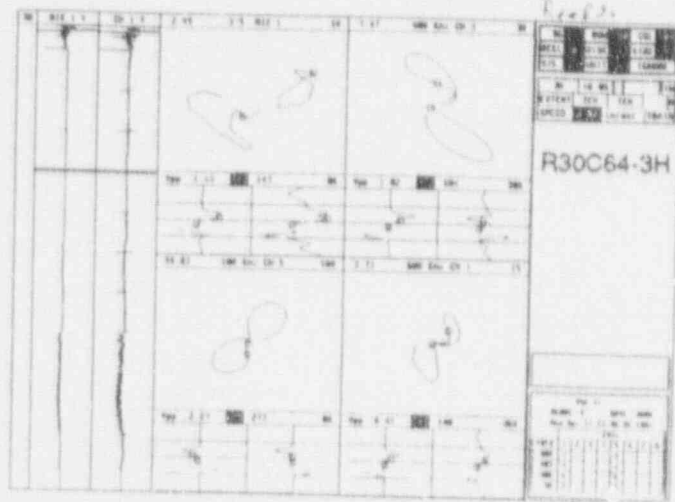


Figure B-4

Laboratory Evaluation of Bobbin Data from Tube R16 C74 at TSPs 3H, 2H and 1H

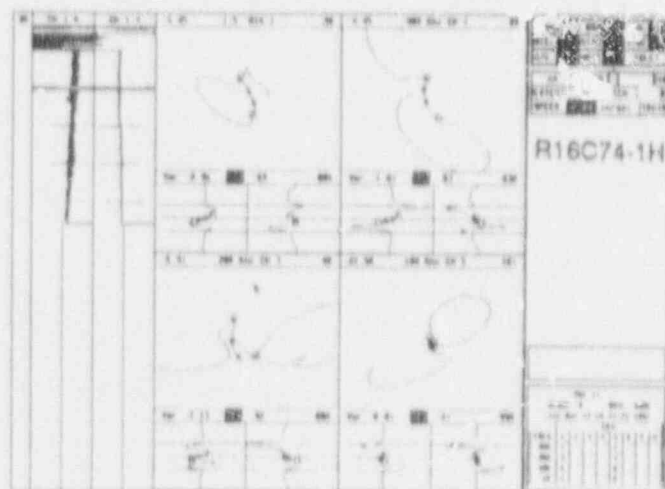
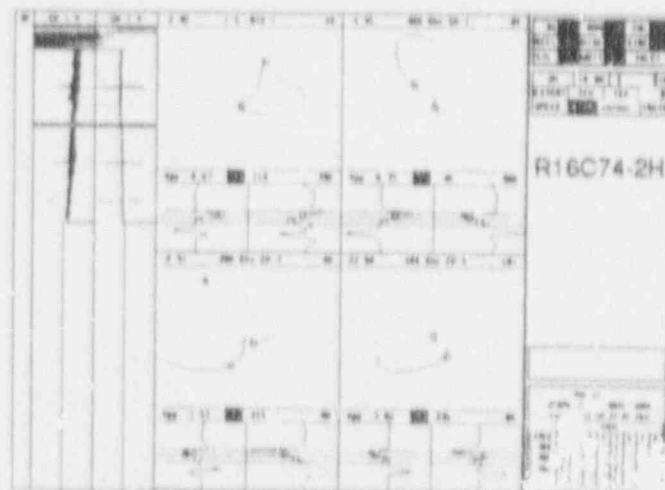
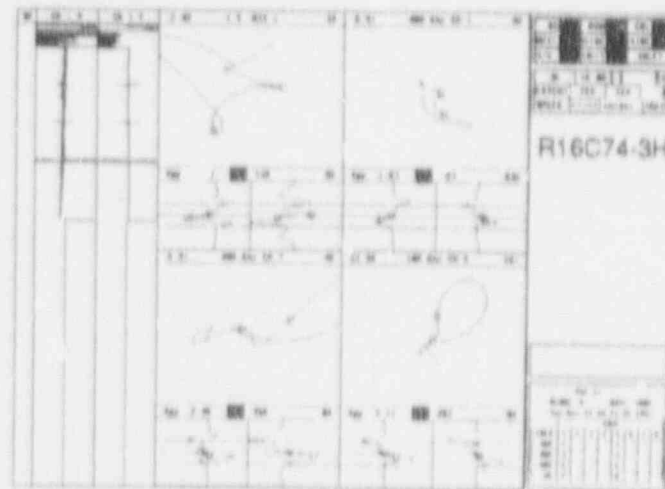


Figure B-5

Laboratory Evaluation of Bobbin Data from Tube R20 C66 at TSPs 3H, 2H and 1H

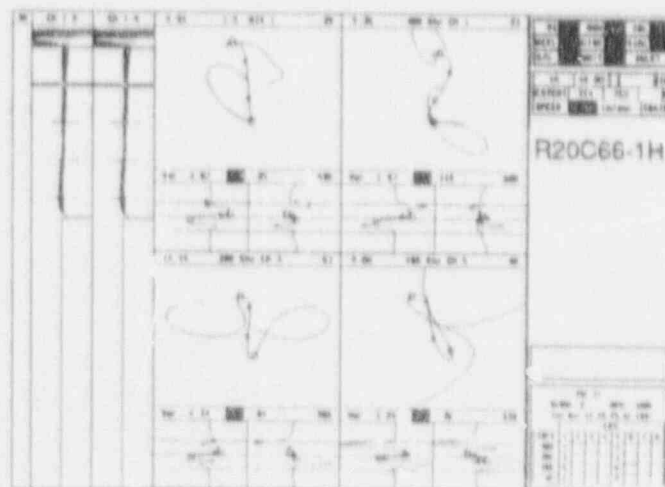


Figure B-6

Laboratory Evaluation of Bobbin Data from Tube R8 C66 at TSPs 3H, 2H and 1H

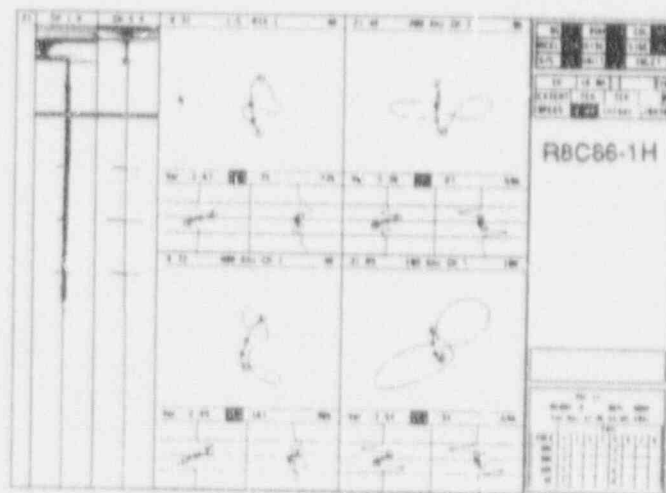
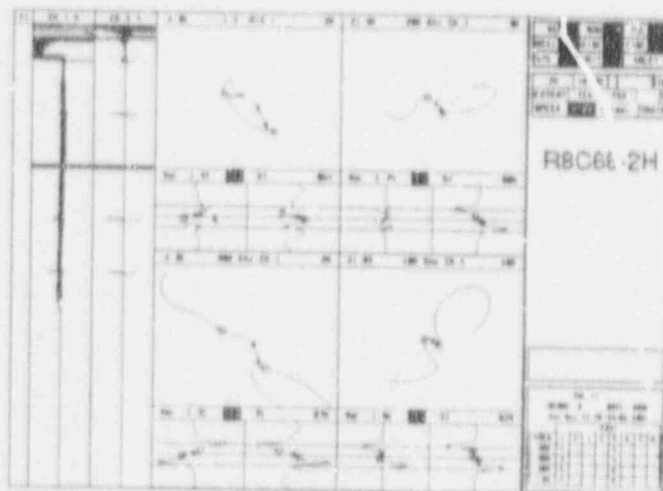
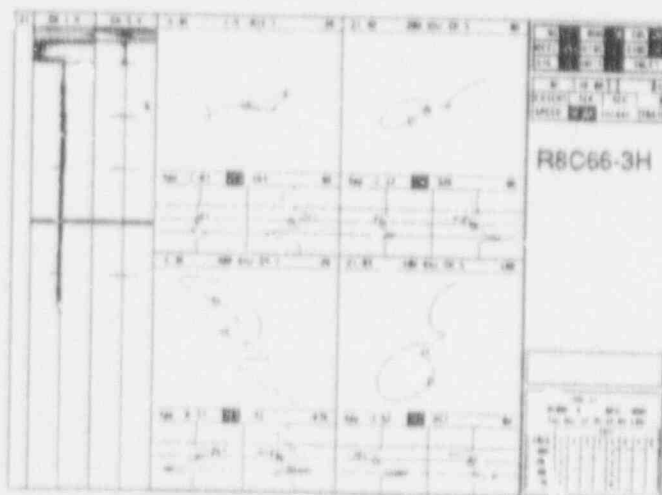


Figure B-7

Laboratory Evaluation of Bobbin Data from Tube R8 C69 at TSPs 3H, 2H and 1H

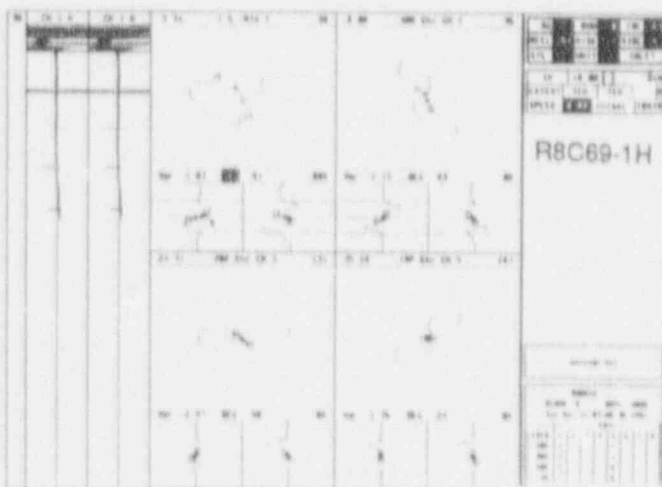
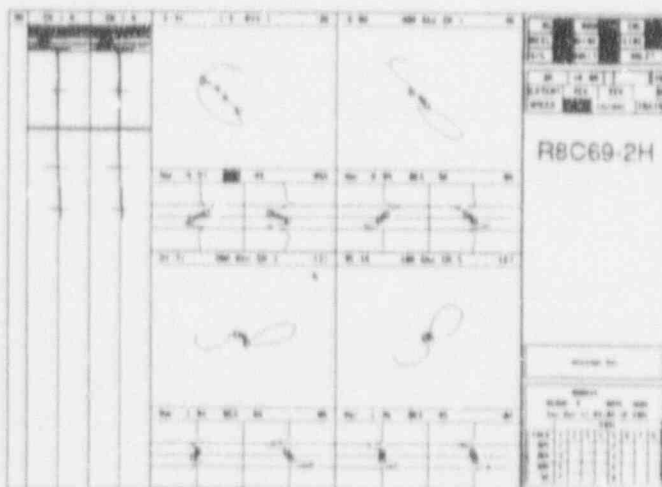


Figure B-8

Laboratory Evaluation of Bobbin Data from Tube R12 C70 at TSPs 3H, 2H and 1H

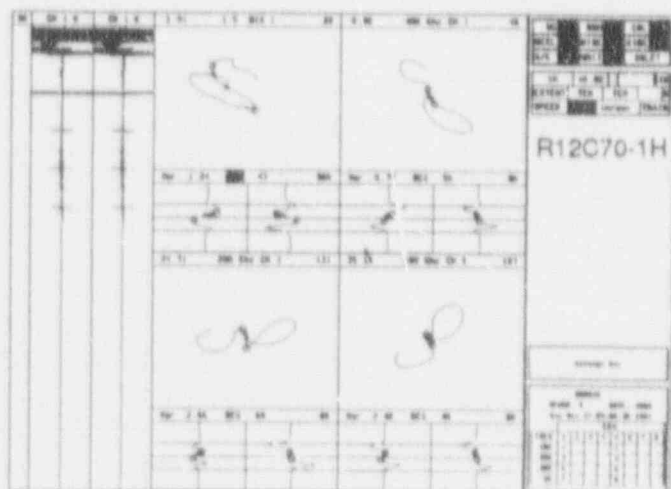
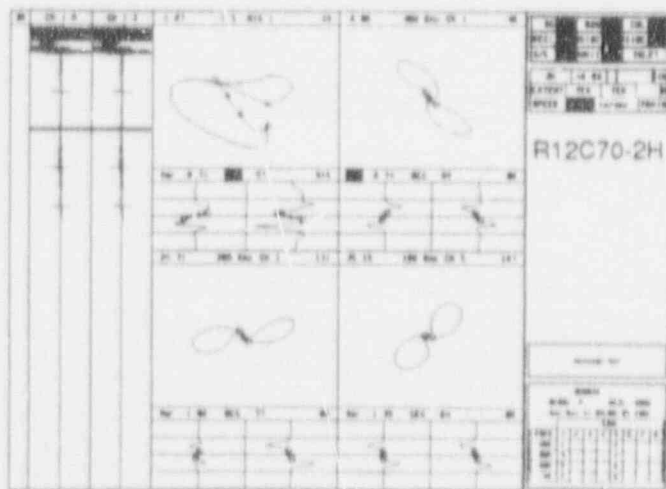
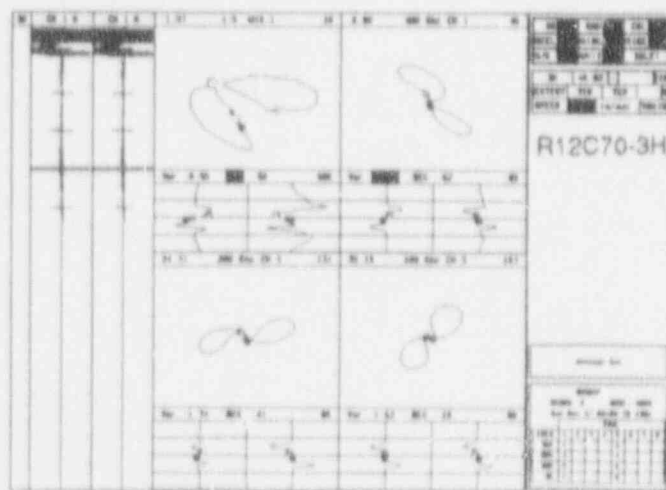


Figure B-9

Examples of Post-Pull Bobbin Indications Influenced by Tube Deformation

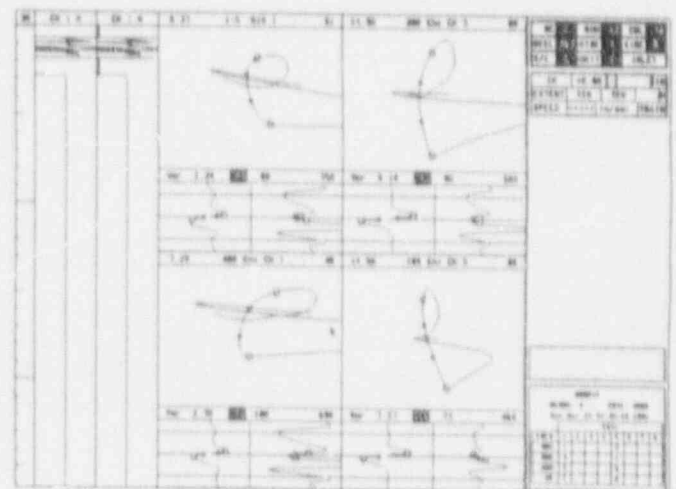
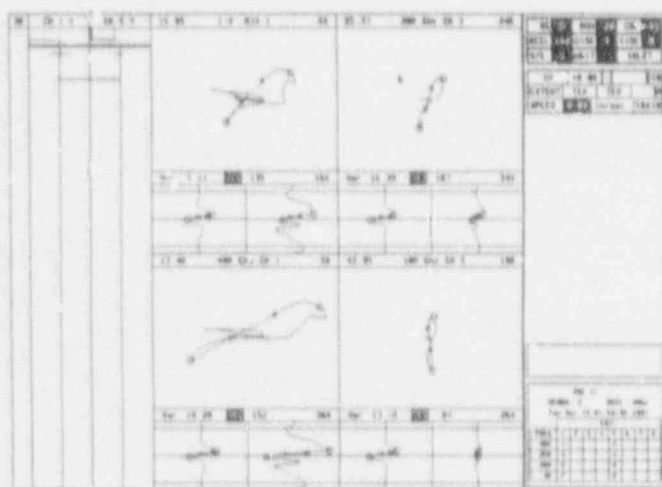
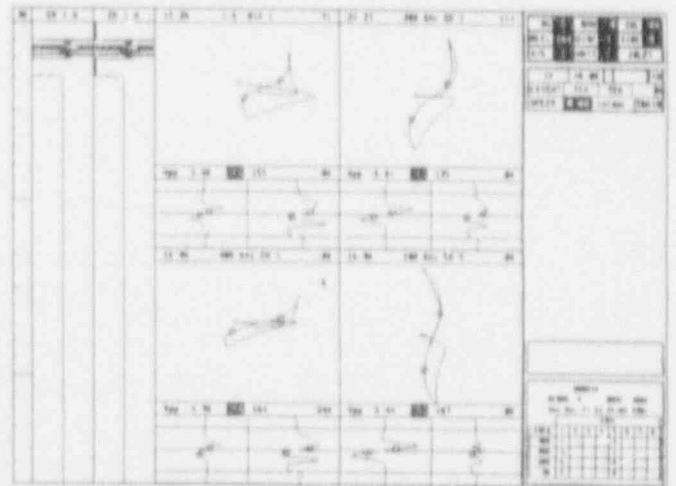
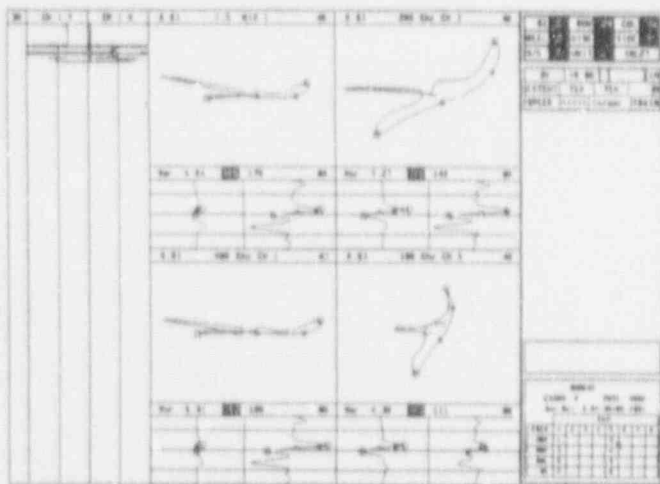
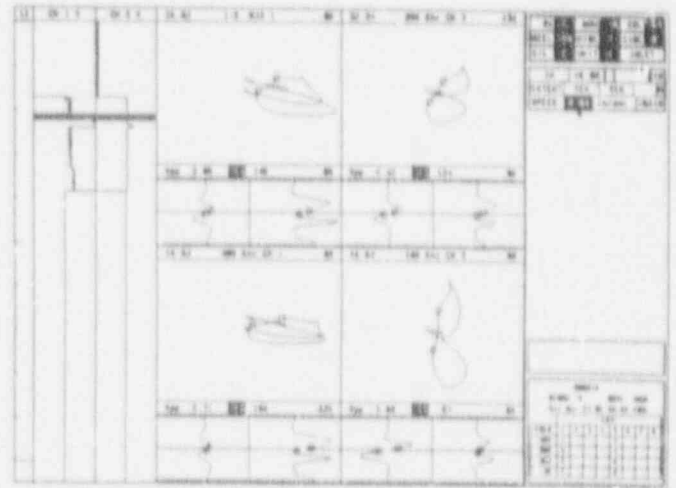
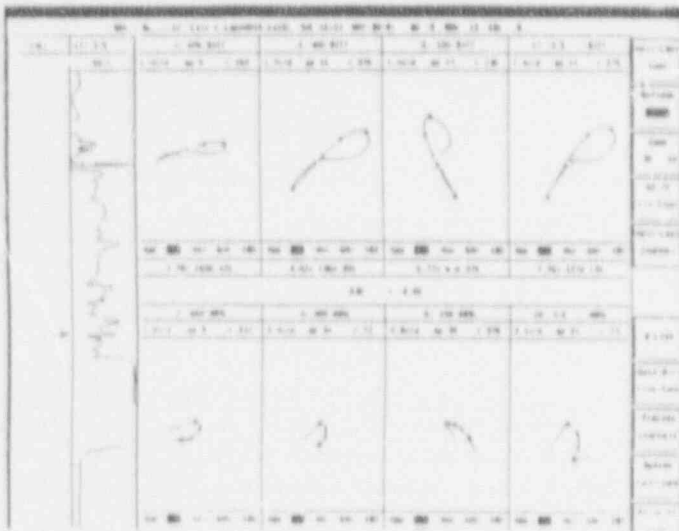


Figure B-10

Examples of Post-Pull Bobbin Indications With Minimal Tube Deformation Influence

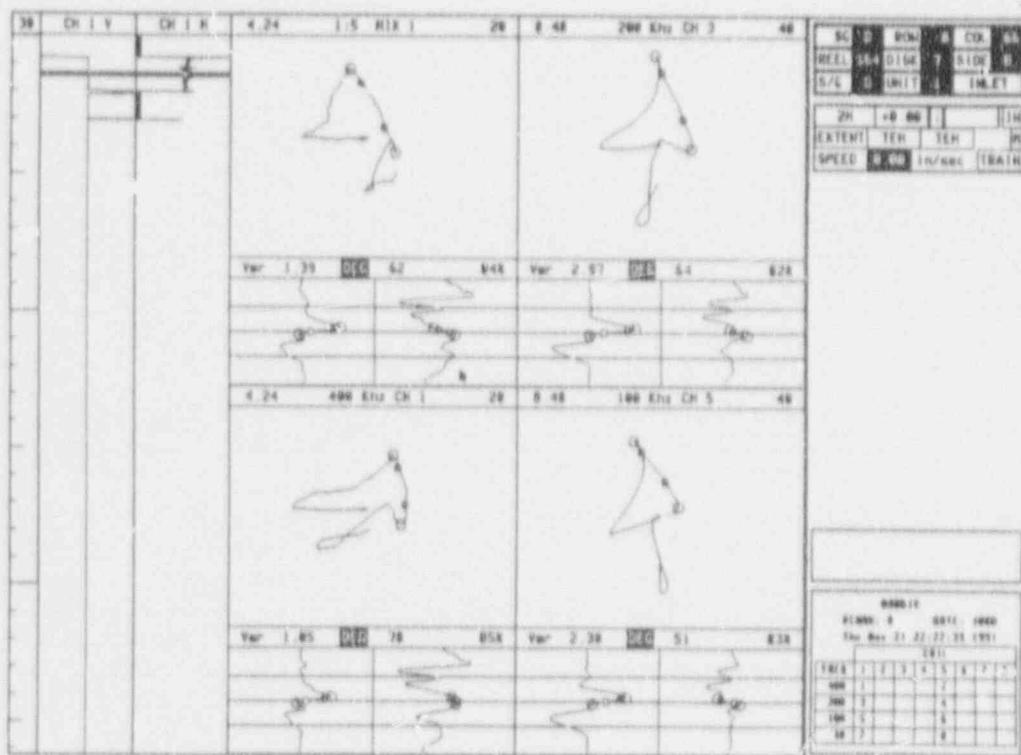
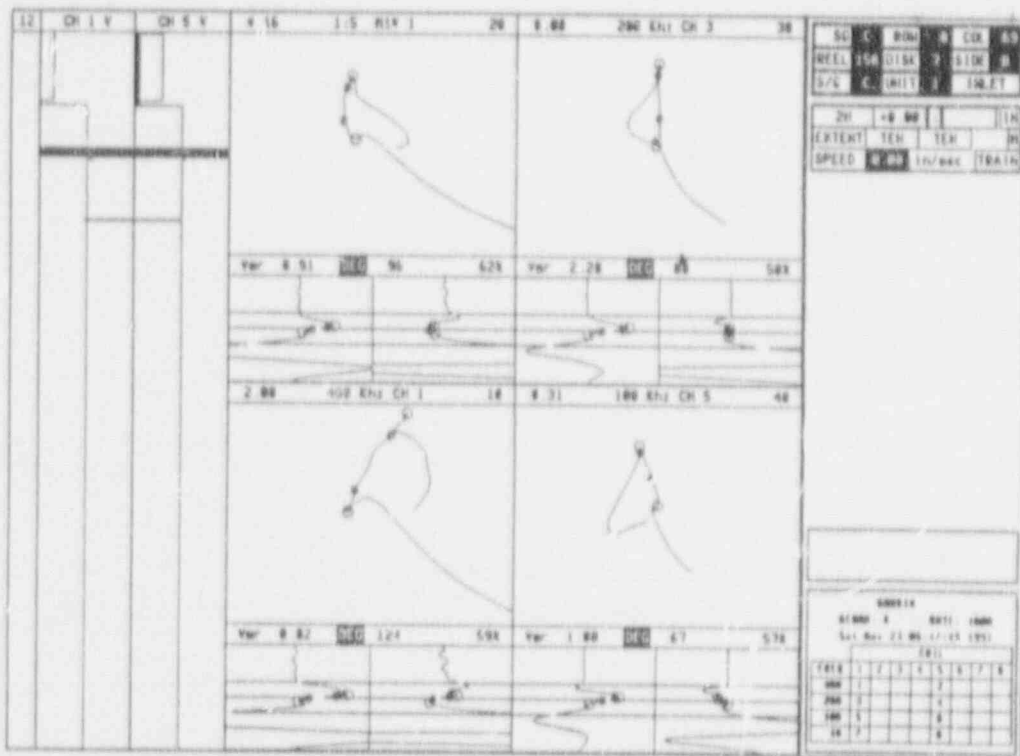


Figure B-11

Comparison of Measured Burst Pressures for Farley, Plant L and Plant P
With Calculated Burst Pressures Using Single Crack Model

