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LICENSEE EVENT REP									ORT (LER)									APPROVED OMBINO 31500104 EXPIRES 4/30/92 STIMATED BURDEN PER RESPONSE TO COMPLY WTH TH FORMATION COLLECTION REQUEST 50.0 HRS FORWAR OMMENTS REGARDING BURDEN ESTIMATE TO THE HECORI NO REPORTS MANAGEMENT BRANCH (P.530) U.S. NUCLEA EQULATORY COMMISSION W 14HIN' TON DC 20565 AND T HE PAPERWORK REDICTION PROVING 1050010 OFFIC F MANAGEMENT AND BUDGE' WAC INGTON DC 20503									
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On December 7, 1990, at 0055, it was concluded that improper servicing resulted in as many as 54 Control Rod Scram Accumulator level switches potentially being inoperable throughout the second fuel cycle in violation of Technical Specification 3.1.3.3. On November 9, 1990, surveillance testing revealed the failure of 54 (out of 177) Control Rod Scram Accumulator level switches. Additional testing was performed, due to the unexpectedly high failure rate, to determine a cause of the failure. On December 7, 1990, the Instrumentation and Controls (I&C) Engineer determined that the methodology for charging the Control Rod Scram Accumulators was causing the level switches to become inoperable and that firm evidence existed that the 54 failed level switches were potentially inoperable prior to or during the second fuel cycle.

The cause of this event was a previously unidentified system interaction. It was not recognized that using the Nitrogen Supply system to service accumulators could result in level switch damage. Excessive flow, while servicing the accumulator caused the level switch float, to rotate which caused the bias spring to unravel, tangle and thus, prevent operation of the level switch.

To prevent recurrence, a System Operating Instruction is being revised to ensure proper level switch operation after accumulator servicing. All level switches have been retested to ensure operability after accumulator servicing. A surveillance instruction is being revised to ensure that each level switch is tested only after the accumulator is serviced. Additionally, a design change is being considered to make an improved servicing rig a permanent part of the Nitrogen Supply system.

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NRC FORM 396A U.	S. NUCLEAR REQULATORY COMMISSION	APPROVED OMB ND. 3160.0104												
LICENSEE EVENT REPORT TEXT CONTINUATION		EXPIRES 4/30/92 ESTIMATED RUHDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST \$0.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P430) U.S. NUCLEAR REQULATORY COMMISSION WASHINGTON DC 20568 AND TO THE PAPERWORK REDUCTION PROJECT (J190-J104). DFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503												
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On December 7, 1990, at 0055, it w in as many as 54 Control Rod [AA] potentially being inoperable throu Technical Specification 3.1.3.3. Operational Condition 4 (Cold Shut approximately 140 degrees Fahrenhe psig.	Scram Accumulator [# ghout the second fue At the time of disco down), Reactor cool	ACC] level switches [el cycle in violation overy, the plant was lant temperature was	LS] of in											
On November 9, 1990 Condition Report 90-374 documented the failure of 54 (out of 177) Control Rod Scram Accumulator level switches during the performance of Surveillance Instruction (SVI-C11-T0009), "Control Rod Scram Accumulator Pressure/Leak Detection Functional/Calibration for 1C11-R(XX-YY), 1C11-N(XX-YY)B and 1C11-N(XX-YY)A (XX-YY represents HCU coordinates for 177 accumulators)." It was assumed that the failures occurred at the time of testing.														
Additional testing was performed, determine a cause of the failure. Control Rod Scram accumulator leve all been recently charged in accor level switches had been proven ope November, 1990. Three of the twen required replacement. On December (I&C) Engineer determined that the Accumulators was causing the level evidence existed that as many as 5 inoperable prior to or during the	On December 6, 1990 1 switches were test dance with the appro- rable by the perform ty switches failed t 7, 1990, the Instru- methodology for cha- switches to become 4 failed level switches), a random sample of ted. These accumulat oved procedure, and a mance of SVI-C11-T000 the functional test a imentation and Contro arging the Control Ro inoperable and that	twenty ors had 11 9 in nd 1s d Scram firm											
The cause of this event was a previously unidentified system interaction. It was not recognized that using the Nitrogen Supply system [:K] to service Control Rod Scram Accumulators would cause a problem with the level switches; however, the pressure/volume capabilities of the Nitrogen Supply system are much greater than those of the alternate method of using a nitrogen bottle for servicing. This greater pressure/volume capability allowed excessive flow through the instrument block to the accumulator which in turn rotated the level switch float, unraveled and tangled the bias spring, and prevented operation of the level switch.														
The unraveling and tangling of the switches were replaced. Because t servicing after the SVI-Cl1-T0009 no way to identify that the switch	he switches were dam performance of May 1	naged by accumulator 1989, there would hav	e been											

failed and the absence of the alarm had been noticed. However, no failures of this type occurred during the second fuel cycle. Because these accumulators are subject to servicing and draining within the fuel cycle and because such servicing does not always result in a failed level switch, it is not possible to determine precisely when any of the level switches failed.

NRC FORM 366A U. (68-9)	S. NUCLEAR REQULATORY COMMISSION	APPROVED ONE NO. 3150-0104
LICENSEE EVENT REPORT TEXT CONTINUATION		EXPIRES 4/30/82 ESTIMATED BURDEN PER RESPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REDUEST BOD HRS FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P 530) U.S. NUCLEAR REGULATORY COMMISSION WASHINGTON DC 2055 AND TO THE PAPERWORK REDUCTION PROJECT (3)50-0104) DFFICE OF MANAGEMENT AND BUDGET WASHINGTON, DC 20503
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)
		YEAR SEQUENTIAL REVISION NUMBER NUMBER
Perry Nuclear Power Plant, Unit 1	0 5 0 0 0 4 4 0	9 0 - 0 3 6 - D 0 3 0F 0 4
TEXT // more space a required, use additional NRC form 2004 2/117 Each Control Rod Scram accumulato control rod at any vessel pressure a free-floating piston. The pisto below. A check valve in the accum pressure in the event supply press accumulator piston is seated at the decreases the nitrogen pressure, we alarm in the control room. To ensist scram, it is continuously monitored actuates an alarm if water leaks y accumulator instrumentation block. In this event, as miny as 54 Contropotentially been inoperable during internal accumulator water leakage control rod scram accumulators in and the requirement to declare the accordance with Technical Specific	e. The accumulator on separates the wat mulator charging lin sure is lost. Durin he bottom of its cyl which actuates a pre sure the accumulator ed for water leakage past the piston barr rol Rod Scram Accumu g power operation. e resulted in the in accordance with Tec e 54 associated cont	is a hydraulic cylinder with er on top from the nitrogen e prevents loss of water g normal operation, the inder. Loss of nitrogen ssure switch and sounds an is always able to produce a . A float type level switch ier and collects in the lator Level Switches have This inability to monitor operability of the 54 hnical Specification 4.1.3.3 rol rods inoperable in
Because it is not possible to dete switch, the conservative approach switches inoperable throughout the accumulator/control rod inoperabil accumulator leakage that would have their intended function. Addition of the 54 control rods in power of operating. Based on the availabil availability of the scram accumula remaining Technical Specification considered to be safety significant No other previous events involving due to Control Rod Scram Accumulator reported. From 1985 to the present	is being taken to d e last fuel cycle. lity due to this eve ve rendered the accu nally during the ent peration, a control lity of control rod ators if needed, and action requirements nt. g scram accumulator/ tor Level Switch mal	eclare the failed level During the time of the nt, there was no internal mulators unable to perform ire period of inoperability rod drive pump was hydraulic pressure, satisfaction of the , this event is not control rod inoperability function have been
nitrogen flow have been experience Procedural precautions and develop responses to the previous switch i the time of testing. These previo the December 7, 1990 event, and pa brought into question.	ed while performing pment of a nitrogen failures which were ous switch failures	SVI-C11-T0009 testing. throttling rig were the believed to have occurred at were lesser in number than

To prevent recurrence, System Operating Instruction (SOI-Cl1 (CRDH)) "Control Rod Drive Hydraulic System (Unit 1)" is being revised to specifically require the use of an improved nitrogen servicing rig that utilizes a high pressure regulator in series with a metering valve to allow better control of the nitrogen flow when servicing Control Rod Scram Accumulators. SOI-Cl1 (CRDH) is also being revised to require testing of the level switches after accumulator servicing until

NRC FORM 266A (6-89)	U.S. NU	U.S. NUCLEAR REDULATORY COMMISSION										APPROVED OME NO. 3150-0104 EXPIRES: 4/30/92											
LICENSEE EVENT REPORT (LER) TEXT CONTINUATION											ESTIMATED BURDEN PER RESPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REQUEST 500 HRS FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REQULATORY COMMISSION WASHINGTON DC 20565 AND TO THE PAPEHWORK REDUCTION PROJECT (3150-0104), OFFICE DF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.												
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sufficient data has been evaluated to ensure that the new method of accumulator servicing does not damage the level switches. All Control Rod Scram Accumulator level switches were retested after accumulator servicing to ensure operability. SVI-C11-T0009 is being revised to ensure that testing of each level switch is performed only after the accumulator is serviced. Additionally, a design change is being considered to make the charging rig a permanent part of the Nitrogen Supply system. As part of the established requalification training program, all plant licensed and non-licensed operators will be instructed on the lessons learned from this event.

Energy Industry Identification System Codes are identified in the text as [XX].