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On April 13, 1995, it was discovered that the Primary Containment was potentially breached for almost two hours on April 5, 1995, when the plant had been in MODE 1 at 98-percent power. It was determined that two plant operators had drained the containment recirculation suction lines through 3/4-inch drain valves. The opening of a drain valve created a potential flow path from containment to the atmosphere, which was a condition prohibited by Technical Specifications.

The condition was discovered as a result of a questioning attitude by operations personnel during discussions about recent work. The condition had low safety significance. The two operators stationed at the drain valve would have immediately recognized any adverse conditions, and would have immediately closed the drain valve if required. A water seal existed during all draining operations, preventing an open line to containment atmosphere. An accident occurring during the draining operation, would not have resulted in radiological releases exceeding allowable releases.

The contributing causes of the event were personnel error and program weakness. The immediate corrective action was to isolate the drain valves at the time of the event. The actions to prevent recurrence include: personnel counseling and program improvements to label the drain valves with caution tags. Additional actions to prevent recurrence include a new program for labeling all accessable valves outside containment that could affect containment integrity, and a review of the locked valve program will be performed.

PDR

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U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)	PAGE (3)			
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Millstone Nuclear Power Station Unit 3	05000423	95	- 005 -	01	02	OF	04

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Description of Event

On April 13, 1995, it was discovered that the Primary Containment was potentially breached for almost two hours on April 5, 1995, when the plant had been in MODE 1 at 98-percent power. It was determined that plant operators had drained containment recirculation suction lines through 3/4-inch drain valves. The opening of a drain valve created a potential flow path from containment to the atmosphere, which was a condition prohibited by Technical Specifications.

The plant operators drained water from the suction piping of the containment Recirculation Spray System (RSS). The water was drained for approximately two hours, first via a 3/4-inch manual drain valve in an A-Train suction line, then via a 3/4-inch manual drain valve in a B-Train suction line. The drain valve in each RSS suction line is outside the containment isolation valve, in the Engineered Safety Features building. The drain valves also have an end cap to provide a second barrier against any possible leakage. The end caps were removed to perform the draining. A tygon hose was connected to the drain valves in sequence and ran for approximately seven feet to a sump. The hose was secured so that the flow coming out of the end of the hose could always be observed.

The operators were draining the RSS suction lines in order to reduce the amount of time that operators would spend in containment pumping down the containment sump. This action was being done in preparation for a Local Leak Rate Test (Type C test) on the RSS suction line penetrations in the upcoming refueling outage. This operation had been done during previous refueling outages as opposed to draining while the plant was running. This difference was not recognized at the time, but was discovered later through discussions about the work. At the time of the event, the operators had an understanding of the containment boundary requirement, and they had taken precautions to not affect containment vacuum while draining. However, they did not consider the potential impact of opening the drain valves, on the Technical Specification requirement for containment integrity. The operators did not clearly communicate their actions to the Shift Supervisor (SS). As a result, the SS did not log into the required Technical Specification Action Statement.

The draining was done with care being taken to not fully drain the RSS suction lines. One operator was stationed at the manual drain valve, and the other operator remained within talking distance and visual contact so that flow could always be observed. The water seal in the lines was considered to be a boundary, so as to not affect the containment vacuum. The draining of the suction lines in sequence lasted for almost two hours, until the operators observed a decrease in flow rate and isolated the drain valves.

The opening of a drain valve created a condition that was prohibited by Technical Specifications. The event should have resulted in an entry into Technical Specification 3.6.1.1, Primary Containment Integrity. This specification requires that: without primary containment integrity, restore primary containment integrity within one (1) hour or be in at least Hot Standby within the next six (6) hours and in Cold Shutdown within the following thirty (30) hours. Although the water seal maintained containment vacuum, it cannot be relied upon for containment integrity.

The condition was discovered as a result of questions raised by operations personnel during discussions about recent work. The questioning attitude and operators reconstructing earlier events, led to a discovery on April 13, 1995, that a noncompliance with Technical Specifications may have occurred on April 5, 1995.

The condition had low safety significance. The two operators stationed at the drain valve would have immediately recognized any adverse conditions and closed the drain valve if required. At no time was the water drained completely from the RSS suction lines, thereby maintaining a water seal to prevent affecting containment vacuum. A water seal existed during all draining operations, preventing an open line to containment atmosphere. An accident occurring during the draining operation, would not have resulted in radiological releases exceeding allowable releases.

NRC Form 366A (4-;95)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

	TT NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)	PAGE (3)								
			YEAR	YEAR SEQUENTIAL REVISION									
	Millstone Nuclear Power Station Unit 3	05000423	95	- 005 -	01	03	OF	04					
EXT	(If more space is required, use additional copies of NRC Form 366A)	(17)	h										
11.	Cause of Event												
	The event was caused by a combinat were preconditioned by the previous This time the planning did not conside when containment integrity was not re- not being effectively communicated w drain valves while containment integrity.	ion of personnel error an experience of draining th er that the work had prev equired. Also the precon vith the SS. The specifics ity was required, should h	d program e lines du iously be ditioning s of remove have beer	n weakness. Ining previous en done with on this task r ving the end n recognized	The two of s refueling the plant esulted in caps and as a pote	operation outages shut of the w opening ntial lo	tors ges. down ork ng th oss o	e f					
	A program weakness is that the drain valves in this system did not have any caution labels. There was no caution provided, that opening these valves could result in a potential degradation of the containment.												
111.	Analysis of Event												
	The condition had low safety significance. The two operators stationed at the manual drain valve would have immediately recognized any adverse conditions and closed the drain valve if required. A water seal existed in the RSS suction lines during all draining operations, preventing an open line to containment atmosphere. An accident occurring during the draining operation, would not have resulted in a radiological release exceeding allowable releases.												
	The event should have resulted in an entry into Technical Specification 3.6.1.1, Primary Containment Integrity. This specification requires that: without primary containment integrity, restore primary containment integrity within one (1) hour or be in at least Hot Standby within the next six (6) hours and in Cold Shutdown within the following thirty (30) hours. Although the water seal maintained containment vacuum, it cannot be relied upon for containment integrity. The opening of the drain line, although water sealed, created a path to containment, which was a condition prohibited by Technical Specifications.												
	The 3/4-inch drain valve is located or containment sump. There are four RS four RSS pumps, all having a similar of and downstream of the automatic com- isolation valve in these lines. For this the four RSS suction lines to meet the open following a loss of coolant accid containment sump to the RSS pumps 31 days by procedure, to satisfy Tech has an end cap installed to provide a	n the 12-inch RSS suction SS suction lines from the configuration. The drain ntainment isolation valve. system, only one contain General Design Criterion fent, in order to provide a the closed position of nical Specification survei second barrier against a	on line, wi containm valve is lo There is r ment iso n. The RS long – ter the manu llance rec ny possib	hich draws si ent sump, su ocated outsic no inboard au ation valve is S suction line rm recirculati al drain valve guirements. E le drain valve	uction from pplying flue to the con- utomatic con- tornatic con- es are required as a a a a a a a a a a a a a a a a a a	n the ow to tainmi- contair in eac uired to th from ked or n valve	the ent of of to be m the nce p also	t er					
	The event was reviewed for a potentia containment heat removal. Containm Had a Loss of Coolant Accident occur immediately observed a sudden chan The energy observed a sudden chan the energy of the line of	I loss of safety function, s ent integrity was conside rred while draining, the o ige in flow in the tygon dr	specificall ared and v perators a rain tubing o the drai	y: containme vas determin at the valve w g due to cont n valve provi	ent integrit ed to be u ould have ainment p	y and inaffec ressu	cted. re.						

FACILITY	NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6) PAGE					
			YEAR	SEQUENTIAL NUMBER	REVIS ON NUMBER				
	Millstone Nuclear Power Station Unit 3	05000423	95	- 005 -	01	04	OF	04	
EXT (If	more space is required, use additional copies of NRC Form 366A) ((17)							
	Containment heat removal was consid drain line, the operators being at the d operators to act if needed, would have function. The operators were continue were specifically observing for any cha announcements of abnormal plant con simple action that would have been im potential loss of safety function. Reas on safety functions.	lered and was determine train valve to quickly clos prevented any significa ously monitoring the drai ange in flow. They were inditions would be heard. Imediately successful. I sonable operator actions	ed to be use it, and nt diversi n flow thi in an are Closing n summa would hi	unaffected. Ti clear indication on of flow fro rough the sma a where any a a small mani ary, the event ave assured t	he small s ons availa m affectin all tygon t alarms or ual drain did not in here was	size of able to ig this tubing. valve is ivolve is no imp	the the safet The s a a pact	ty Əy	
IV.	Corrective Action								
	Containment integrity was restored an time of the event on April 5, 1995, whe operation.	d full compliance with Te in the drain valves were s	chnical S shut follo	Specifications wing the two-	was achi -hour dra	eved a lining	it the		
	The actions to prevent recurrence inclu operators involved were counseled on SS prior to task performance.	ude personnel counselin the importance of task r	g and pr eview an	ogram improv d effective co	vements. mmunica	The tion wi	ith th	e	
	The generic and programmatic implication improvements include a review of com- positive controls on containment isolat Engineering Group, discussions were decided on additional programmatic in primary containment whose operation label will be installed on these valves be containment will not be labeled becau- required. The caution will indicate that Operating." Also by the end of 1995, identify any formal configuration control	ations of this event have tainment isolation valves tion valves. A review was conducted with other uti mprovements. The acce could affect containmen by November 1, 1995. U se they are not normally t these valves are "Conta a locked valve program of improvements that wo	been inve and a ne s conduc lities, and ssible va t integrity nder this accessib inment E self-ass uld be pr	estigated. As aw program to ted by the Ind d the Operation lives that are I y have been in action, the va- ole when cont Boundary, Con- essment will I rudent in the I	a result, o strength depender ons depar ocated ou dentified. alves loca ainment ii ntact SS/ be compli- ocked val	the pro- the pro- the the the the the the the the the the	bgran ty has the tion side y is rior t bgran	n o n.	
l.	Additional Information								
	The containment integrity program for considerable strengthened to minimize program, there were no other containr	the refueling outage that the potential for outage nent integrity issues.	t was cor problem	nducted in Ap ns. As a resul	ril and Ma ts of this	ay 199 strong	5, wa	1S	
	A review of Licensee Event Reports indicates there have been no similar LERs over the last three years.								
	EIIS Codes System BE – Containment Spray System	Component V – Drain Val	V0						