



**Duquesne Light**

Nuclear Group  
P.O. Box 4  
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Telephone (412) 393-6000

January 2, 1991  
ND3MNO:3082

Beaver Valley Power Station, Unit No. 2  
Docket No. 50-412, License No. NPF-73  
LER 90-026-00

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Gentlemen:

In accordance with Appendix A, Beaver Valley Technical Specifications, the following Licensee Event Report is submitted:

LER 90-026-00, 10 CFR 50.73.a.2.iv, "ESF Actuation - Containment Radiation Monitor Inlet Isolation Valve Closure".

Very truly yours,

T. P. Noonan  
General Manager  
Nuclear Operations

JGT/sl

Attachment

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PDR ADOCK 05000412  
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cc: Mr. T. T. Martin, Regional Administrator  
United States Nuclear Regulatory Commission  
Region 1  
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King of Prussia, PA 19406

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J. M. Riddle  
NUS Operating Service Corporation  
Park West II  
Cliff Mine Road  
Pittsburgh, PA 15275

LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 60.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P.530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  
Beaver Valley Power Station Unit 2

DOCKET NUMBER (2)  
0 5 0 0 0 4 1 2 1

PAGE (3)  
1 OF 0 3

TITLE (4)  
ESF Actuation - Containment Radiation Monitor Inlet Isolation Valve Closure

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	N/A	FACILITY NAMES	DOCKET NUMBER(S)														
1	2	0	2	9	0	9	0	0	2	6	0	0	0	1	0	2	9	1			0	5	0	0	0

OPERATING MODE (9) 1

POWER LEVEL (10) 1 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

20.402(b)		20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)		73.71(b)
20.406(a)(1)(i)		50.36(a)(1)		50.73(a)(2)(v)		73.71(c)
20.406(a)(1)(ii)		50.36(a)(2)		50.73(a)(2)(vi)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.406(a)(1)(iii)		50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		
20.406(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		
20.406(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)		

LICENSEE CONTACT FOR THIS LER (12)

NAME: T.P. Noonan, General Manager Nuclear Operations

TELEPHONE NUMBER: 4 1 2 6 4 3 - 1 2 5 8

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NFRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NFRDS	
X	I	L	I	S	V	X	X	X	X	N

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewriter lines) (16)

On 12/2/90 at 0020 hours, a loss of flow alarm was received on the Containment Airborne Radiation Monitor Skid. The monitor inlet isolation valve (2CVS\*SOV153A) was found shut. The valve was reopened and flow was restored at 0030 hours. This valve failed closed three additional times over the next 9 hours. In each case, flow was restored after opening the valve. At 1050 hours, the radiation monitor skid was removed from service to allow maintenance activities on the inlet isolation valve. Limit switch adjustments were performed and the valve was returned to service at 1445 hours. On 12/3/90 at 0900 hours, it was identified that the closure of this valve constitutes an Engineered Safety Features (ESF) actuation, as this valve receives a Containment Isolation Phase "A" Signal. The Nuclear Regulatory Commission was notified of these actuations at 0950 hours on 12/3/90. There were no safety implications as a result of this event. The Containment Airborne Radiation Monitor remained operable throughout this event. No abnormal radioactivity was present in Containment. No radiation releases to the public occurred.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATIONESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS  
INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD  
COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS  
AND REPORTS MANAGEMENT BRANCH (F-630), U.S. NUCLEAR  
REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO  
THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE  
OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503

FACILITY NAME (1)  Beaver Valley Power Station Unit 2	DOCKET NUMBER (2)  0 5   0   0   0   4   1   2 9 0	LER NUMBER (6)		PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		0	026	0	0	02 OF 03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT

On 12/2/90 at 0020 hours, a loss of flow alarm was received on the Containment Airborne Radiation Monitor Skid. The monitor inlet isolation valve (2CVS\*SOV153A) was found shut. The valve was reopened and flow was restored at 0030 hours. This valve failed closed three additional times over the next 9 hours. In each case, flow was restored after opening the valve. At 1050 hours, the radiation monitor skid was removed from service to allow maintenance activities on the inlet isolation valve. Technical Specification 3.4.6.1 was entered to allow troubleshooting of the limit switch for the inlet isolation valve. Limit switch adjustments were performed and the valve was returned to service at 1445 hours, exiting Technical Specification 3.4.6.1. On 12/3/90 at 0900 hours, it was identified that the closure of this valve constitutes an Engineered Safety Features (ESF) actuation, as this valve receives a Containment Isolation Phase "A" Signal. The Nuclear Regulatory Commission was notified of these actuations at 0950 hours on 12/3/90.

CAUSE OF THE EVENT

The cause for this event was the inlet isolation valve, 2CVS\*SOV153A, limit switch being out of adjustment.

CORRECTIVE ACTIONS

The following corrective actions have been taken as a result of this event:

1. Operations personnel restored radiation monitor flow by reopening the inlet isolation valve following each closure.
2. Instrument and Control personnel performed limit switch adjustments on the inlet isolation valve. Post maintenance testing was performed to ensure proper limit switch operation.

LICENSEE EVENT REPORT (LER)  
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 600 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)  Beaver Valley Power Station Unit 2	DOCKET NUMBER (2)  0 5 0 0 0 4 1 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 0	-- 0 2 6	-- 0 0	0 3	OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

PREVIOUS OCCURRENCES

There are no previously reported events of this type involving the Containment Airborne Radiation Monitor.

SAFETY IMPLICATIONS

There were no safety implications as a result of this event. The Containment Airborne Radiation Monitor remained operable throughout this event. No abnormal radioactivity was present in Containment. No radiation releases to the public occurred.

REFORTABILITY

The Containment Airborne Radiation Monitor inlet isolation valves (2CVS-SOV153A,B) are Engineered Safety Features (ESF) components, as they receive a close signal on a Containment Isolation Phase "A" Signal. This unexpected actuation was reported at 0950 hours in accordance with 10CFR50.72.b.2.ii. This written report is being submitted in accordance with 10CFR50.73.a.2.iv.