



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0701

PHILADELPHIA ELECTRIC COMPANY

DOCKET NO. 50-352

LIMERICK GENERATING STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 103  
License No. NPF-39

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company (the licensee) dated January 27, 1995, as supplemented October 10, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-39 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 103, are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment: Changes to the  
Technical Specifications

Date of Issuance: October 30, 1995

ATTACHMENT TO LICENSE AMENDMENT NO. 103

FACILITY OPERATING LICENSE NO. NPF-39

DOCKET NO. 50-352

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 6-24

3/4 6-25

3/4 6-43

Insert

3/4 6-24

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TABLE 3.6.3-1 (Continued)

## PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC) (26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
040G-1	ILRT DATA ACQUISITION	60-1057	60-1058	NA NA		11 11	60
040G-2	ILRT DATA ACQUISITION	60-1071	60-1070	NA NA		11 11	60
040H-1	CONTAINMENT INSTRUMENT GAS SUPPLY - HEADER 'A'	59-1005A(CK)	HV59-129A	NA 7	C,H,S		59
042	STANDBY LIQUID CONTROL	48-1F007(CK) (X-116)	HV48-1F006A	NA 60		29	48
043B	MAIN STEAM SAMPLE	HV41-1F084	HV41-1F085	10 10	B B		41
044	RWCU ALTERNATE RETURN	41-1017	41-1016(X-9A, X-9B) PSV41-112	NA NA NA		5,31	41
045A(B,C,D)	LPCI INJECTION 'A' (B,C,D)	HV51-1F041A(B,C, D)(CK) HV51-142A(B,C, D)	HV51-1F017A (B,C,D)	NA 7 38		9,22 9,22	51
050A-1	DRYWELL PRESSURE INSTRUMENTATION		HV42-147B	45		10	42
053	DRYWELL CHILLED WATER SUPPLY - LOOP 'A'	HV87-128	HV87-120A HV87-125A	60 60 NA	C,H C,H	11 11 34	87

LIMERICK - UNIT 1

3/4 6-24

Amendment No. 13, 18, 23, 89, 103

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME. IF APP. (SEC) (26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
054	DRYWELL CHILLED WATER RETURN - LOOP 'A'	HV87-129		60	C,H	11	87
			HV87-121A	60	C,H	11	
			HV87-124A	NA		34	
055	DRYWELL CHILLED WATER SUPPLY - LOOP 'B'	HV87-122		60	C,H	11	87
			HV87-120B	60	C,H	11	
			HV87-125B	NA		34	
056	DRYWELL CHILLED WATER RETURN - LOOP 'B'	HV87-123		60	C,H	11	87
			HV87-121B	60	C,H	11	
			HV87-124B	NA		34	
061-1	RECIRC PUMP 'A' SEAL PURGE	43-1004A(CK)		NA		15	43
			(XV43-103A - SEE PART B, THIS TABLE)	NA		1	
061-2	RECIRC PUMP 'B' SEAL PURGE	43-1004B(CK)		NA		15	43
			(XV43-103B - SEE PART B, THIS TABLE)	NA		1	
062	DRYWELL H2/O2 SAMPLE RETURN, N2 MAKE-UP	SV57-150(X-220A)		5	B,H,R,S	11	57
			SV57-159 (X-220A)	5	B,H,R,S	11	
			HV57-116 (X-220A)	30**	B,H,R,S	11	
			SV57-190 (X-220A)	5	B,H,R,S	11	

LIMERICK - UNIT 1

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Amendment No. 2, 12, 17, 103

TABLE 3.6.3-1  
PRIMARY CONTAINMENT ISOLATION VALVES  
NOTATION

NOTES

(Continued)

21. Automatic isolation signal causes TIP to retract; ball valve closes when probe is fully retracted.
22. Isolation barrier remains water filled or a water seal remains in the line post-LOCA. Isolation valve may be tested with water. Isolation valve leakage is not included in 0.60 La total Type B & C tests.
23. Valve does not receive an isolation signal. Valves will be open during Type A test. Type C test not required.
24. Both isolation signals required for valve closure.
25. Deleted
26. Valve stroke times listed are maximum times verified by testing per Specification 4.0.5 acceptance criteria. The closure times for isolation valves in lines in which high-energy line breaks could occur are identified with a single asterisk. The closure times for isolation valves in lines which provide an open path from the containment to the environs are identified with a double asterisk.
27. The reactor vessel head seal leak detection line (penetration 29A) excess flow check valve is not subject to OPERABILITY testing. This valve will not be exposed to primary system pressure except under the unlikely conditions of a seal failure where it could be partially pressurized to reactor pressure. Any leakage path is restricted at the source; therefore, this valve need not be OPERABILITY tested.
28. (DELETED)
29. Valve may be open during normal operation; capable of manual isolation from control room. Position will be controlled procedurally.
30. Valve normally open, closes on scram signal.
31. Valve 41-1016 is an outboard isolation barrier for penetrations X-9A, B and X-44. Leakage through valve 41-1016 is included in the total for penetration X-44 only.
32. Feedwater long-path recirculation valves are sealed closed whenever the reactor is critical and reactor pressure is greater than 600 psig. The valves are expected to be opened only in the following instances:
- a. Flushing of the condensate and feedwater systems during plant startup.
  - b. Reactor pressure vessel hydrostatic testing, which is conducted following each refueling outage prior to commencing plant startup.
- Therefore, valve stroke timing in accordance with Specification 4.0.5 is not required.
33. Valve also constitutes a Unit 2 Reactor Enclosure Secondary Containment Automatic Isolation Valve and a Refueling Area Secondary Containment Automatic Isolation Valve as shown in Table 3.6.5.2.1-1 and Table 3.6.5.2.2-1 respectively.
34. Auto isolation signals have been removed from HV-087-124 A/B and 125 A/B. Valves to be closed with associated circuit breakers locked open during OPCONS 1, 2, and 3.



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

PHILADELPHIA ELECTRIC COMPANY

DOCKET NO. 50-353

LIMERICK GENERATING STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 67  
License No. NPF-85

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company (the licensee) dated January 27, 1995, as supplemented October 10, 1995, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

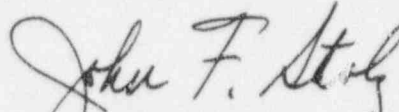
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-85 is hereby amended to read as follows:

Technical Specifications

The Technical Specifications contained in Appendix A and the Environmental Protection Plan contained in Appendix B, as revised through Amendment No. 67, are hereby incorporated into this license. Philadelphia Electric Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance, to be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



John F. Stolz, Director  
Project Directorate I-2  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Attachment: Changes to the  
Technical Specifications

Date of Issuance: October 30, 1995



ATTACHMENT TO LICENSE AMENDMENT NO. 67

FACILITY OPERATING LICENSE NO. NPF-85

DOCKET NO. 50-353

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by Amendment number and contain vertical lines indicating the area of change.

Remove

3/4 6-24

3/4 6-25

3/4 6-43

Insert

3/4 6-24

3/4 6-25

3/4 6-43

TABLE 3.6.3-1 (Continued)

## PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME, IF APP. (SEC) (26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
040G-1	ILRT DATA ACQUISITION	60-2057	60-2058	NA NA		11 11	60
040G-2	ILRT DATA ACQUISITION	60-2071	60-2070	NA NA		11 11	60
040H-1	CONTAINMENT INSTRUMENT GAS SUPPLY - HEADER 'A'	59-2005A(CK)	HV59-229A	NA 7	C, H, S		59
042	STANDBY LIQUID CONTROL	48-2F007(CK) (X-116)	HV48-2F006A	NA 60		29	48
043B	MAIN STEAM SAMPLE	HV41-2F084	HV41-2F085	10 10	B B		41
044	RWCU ALTERNATE RETURN	41-2017	41-2016(X-9A, X-9B) PSV41-212	NA NA NA		5,31	41
045A(B,C,D)	LPCI INJECTION 'A' (B,C,D)	HV51-2F041A(B,C, D)(CK) HV51-242A(B,C, D)	HV51-2F017A (B,C,D)	NA 7 38		9,22 9,22	51
050A-1	DRYWELL PRESSURE INSTRUMENTATION		HV42-247B	45		10	42
053	DRYWELL CHILLED WATER SUPPLY - LOOP 'A'	HV87-228	HV87-220A HV87-225A	60 60 NA	C, H C, H	11 11 35	87

LIMERICK - UNIT 2

3/4 6-24

Amendment No. 82,67

TABLE 3.6.3-1 (Continued)

PART A - PRIMARY CONTAINMENT ISOLATION VALVES

PENETRATION NUMBER	FUNCTION	INBOARD ISOLATION BARRIER	OUTBOARD ISOLATION BARRIER	MAX. ISOL. TIME IF APP. (SEC)(26)	ISOL. SIGNAL(S), IF APP. (20)	NOTES	P&ID
054	DRYWELL CHILLED WATER RETURN - LOOP 'A'	HV87-229		60	C,H	11	87
			HV87-221A	60	C,H	11	
			HV87-224A	NA		35	
055	DRYWELL CHILLED WATER SUPPLY - LOOP 'B'	HV87-222		60	C,H	11	87
			HV87-220B	60	C,H	11	
			HV87-225B	NA		35	
056	DRYWELL CHILLED WATER RETURN - LOOP 'B'	HV87-223		60	C,H	11	87
			HV87-221B	60	C,H	11	
			HV87-224B	NA		35	
061-1	RECIRC PUMP 'A' SEAL PURGE	43-2004A(CK)		NA		15	43
			(XV43-203A - SEE PART B, THIS TABLE)	NA		1	
061-2	RECIRC PUMP 'B' SEAL PURGE	43-2004B(CK)		NA		15	43
			(XV43-203B - SEE PART B, THIS TABLE)	NA		1	
062	DRYWELL H2/O2 SAMPLE RETURN, N2 MAKE-UP	SV57-250(X-220A)		5	B,H,R,S	11	57
			SV57-259 (X-220A)	5	B,H,R,S	11	
			HV57-216 (X-220A)	30**	B,H,R,S	11	
			SV57-290 (X-220A)	5	B,H,R,S	11	

LIMERICK - UNIT 2

3/4 6-25

Amendment No. 67

TABLE 3.6.3-1  
PRIMARY CONTAINMENT ISOLATION VALVES  
NOTATION

NOTES

(Continued)

21. Automatic isolation signal causes TIP to retract; ball valve closes when probe is fully retracted.
22. Isolation barrier remains water filled or a water seal remains in the line post-LOCA. Isolation valve may be tested with water. Isolation valve leakage is not included in 0.60 La total Type B & C tests.
23. Valve does not receive an isolation signal. Valves will be open during Type A test. Type C test not required.
24. Both isolation signals required for valve closure.
25. Deleted
26. Valve stroke times listed are maximum times verified by testing per Specification 4.0.5 acceptance criteria. The closure times for isolation valves in lines in which high-energy line breaks could occur are identified with a single asterisk. The closure times for isolation valves in lines which provide an open path from the containment to the environs are identified with a double asterisk.
27. The reactor vessel head seal leak detection line (penetration 29A) excess flow check valve is not subject to OPERABILITY testing. This valve will not be exposed to primary system pressure except under the unlikely conditions of a seal failure where it could be partially pressurized to reactor pressure. Any leakage path is restricted at the source, therefore, this valve need not be OPERABILITY tested.
28. (DELETED)
29. Valve may be open during normal operation; capable of manual isolation from control room. Position will be controlled procedurally.
30. Valve normally open, closes on scram signal.
31. Valve 41-2016 is an outboard isolation barrier for penetrations X-9A, B and X-44. Leakage through valve 41-2016 is included in the total for penetration X-44 only.
32. Feedwater long-path recirculation valves are sealed closed whenever the reactor is critical and reactor pressure is greater than 600 psig. The valves are expected to be opened only in the following instances:
  - a. Flushing of the condensate and feedwater systems during plant startup.
  - b. Reactor pressure vessel hydrostatic testing, which is conducted following each refueling outage prior to commencing plant startup.Therefore, valve stroke timing in accordance with Specification 4.0.5 is not required.
33. Valve also constitutes a Unit 1 Reactor Enclosure Secondary Containment Automatic Isolation Valve and a Refueling Area Secondary Containment Automatic Isolation Valve as shown in Table 3.6.5.2.1-1 and Table 3.6.5.2.2-1, respectively.
34. Isolation signal causes recombiner to trip; valve closes when recombiner is not operating.
35. Auto isolation signals have been removed from HV-087-224 A/B and 225 A/B. Valves to be closed with associated circuit breakers locked open during OPCONs 1, 2, and 3.