LICENSEE EVENT REPORT (LER)									U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85								
PACILIT	Y NAME (1	1)										DOCKET NUMBER	(2)	PAG	E (3)		
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Read	etor	Cor	e I	sol	ation	Cooli	ng C	ontr	oller	Ino	perabl	e					
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	RATING		THIS P	EPORT	IS SUBMITT	ED PURSUANT	TO THE R	EQUIREM	ENTS OF 10	CFR 8: /C	Check one or mor	e of the following) (1		-			
MODE (9)			2	20.402(b)			20.406	20.408(c)			50.73(a)(2)(iv)		73.71(b)				
LEVEL 01814		3	20.405(a)(1)(ii) 20.405(a)(1)(iii) 20.405(a)(1)(iii) 20.405(a)(1)(iv) 20.405(a)(1)(iv)			80.36(a)(1) 80.36(a)(2) 80.73(a)(2)(i) 80.73(a)(2)(ii) 80.73(a)(2)(iii)			X	80.73(a)(2)(v) 80.73(a)(2)(vii 80.73(a)(2)(vii 80.73(a)(2)(vii 80.73(a)(2)(x)	73.71(c) OTHER (Specify in Abstract below and in Text, NRC Form 366.A)						
							LICENSEE	CONTACT	FOR THIS	LER (12)							
NAME Da	avid S	Smith										21 0 15	712191 -		16 15		
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ABSTRACT (Limit to 1400 speces Le soproximetely fifteen single-space typewritten lines) (16)

During performance of surveillance instructions 4.5.F.l.d and e, Reactor Core
Isolation Cooling (RCIC) system failed to properly respond to manual control
signals, therefore, it was declared inoperable. HPCI testing was immediately
initiated. The RCIC ramp generator and signal converter was replaced, RCIC was
successfully retested and declared operable. This failure was random with no
further corrective action required.

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YES (If yes, complete EXPECTED SUBMISSION DATE)

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)		L	ER NUMBER (6	1)		15.	PAC	3E (3	1
		YEAR	AR SEQUENTIAL NUMBER			REVISION NUMBER		T	T	17.4
Browns Ferry - Unit 1	0 5 0 0 0 2 5 9	8 2	-	0 3 6	-	0 0	01	20	FO	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

During normal operation, unit 1 was operating at 85 percent power. Units 2 and 3 were in refueling outages.

As surveillance test 4.5.F.l.d. & e was being run on the reactor core isolation cooling system (BN), manual control signals failed to cause proper turbine (TRB) responses. The reactor core isolation cooling system was declared inoperable. Technical Specification requirements were immediately initiated.

Within 24 hours, the reactor core isolation cooling system was repaired and successfully retested. The ramp generator and signal converter (CNV) was replaced. Its output signals were erratic. This was the only problem found. Since this was a random failure, no further corrective action is necessary.

With the reactor core isolation cooling system inoperable, adequate emergency core cooling was maintained (per the requirements of the final safety analysis report) by the high pressure coolant injection system (BJ).

Responsible Plant Section - N/A

Previous Similar Events - None

October 23, 1984 BCM: DLS: BDL

TENNESSEE VALLEY AUTHORITY

Browns Ferry Nuclear Plant P. O. Box 2000 Decatur, Alabama 35602

October 23, 1984

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 1 - DOCKET NO. 50-259 - FACILITY OPERATING LICENSE DPR-09 - REPORTABLE OCCURRENCE REPORT BFR0-50-259/84036

The enclosed report provides details that concern the reactor core isolation cooling controller inoperability. This report is submitted in accordance with 10 CFR 50.73 (a)(2)(v).

Very truly yours,

TENNESSEE, VALLEY AUTHORITY

G. T. Jones Plant Manager

Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):

Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

INPO Records Center Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339

NRC Resident Inspector, BFN

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