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On 9/25/84 it was determined that the lead-lag controllers in the steamline pressure instrumentation loops had been incorrectly calibrated. The four STEAMLINE LOW PRESSURE-SAFETY INJECTION trips were then declared inoperable per Technical Specification 3.3.2. These trips had been required to be operable since 9/6/84 when RCS pressure went above 1970 psig.

It was found that incorrect nomenclature had been used in a generic calibration procedure, causing setpoints on each lead-lag controller to be incorrectly calibrated.

To comply with Technical Specification 3.3.2, RCS pressure was reduced to 1800 psig. The incorrect procedure was revised and the affected loops recalibrated. Additionally, all instrumentation loops which had been required to be operable were examined for similar errors. No other instrumentation loops were affected by this incident.

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NRC Form 386A 19-831	LICENSEE EVENT REPO	JATION	U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 2150~0104 EXPIRES. 8/31/85				
FACILITY NAME (1)		DOCKST NUMBER (2)	. LER NUMB	ER (6)	PAGE (3)		
	Callaway Plant Unit 1		YEAR SEQUE				
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On 9/25/84 the plant was in Mode 3 with Feactor Coolant System (RCS) temperature and pressure 557°F and 2235 psig, respectively. Surveillance procedure ISL-BB-OT431, "LOOP-TEMP: LOOP 3 AT/TAVG," was being performed as required per Technical Specifications prior to entry into Mode 2 and initial criticality. During the course of this procedure it was discovered that generic calibration procedure ITG-ZZ-WNLL1, "Westinghouse 7300 LEAD/LAG CARD," had a nomenclature error which caused the lead-lag time constants of these circuit cards to be calibrated incorrectly.

A review of instrumentation loops that utilize the lead-lag controller cards was immediately initiated. It was discovered that the twelve STEAMLINE-LOW PRESSURE loops, which had been required to be operable prior to exceeding an RCS pressure of 1970 psig, had been incorrectly calibrated as a result of the procedure error. Therefore, at 0915 CDT the four STEAMLINE LOW PRESSURE-SAFETY INJECTION trips were declared inoperable. The review revealed that no other instrumentation loops that had been required to be operable were affected by this error.

Per Technical Specifications the lead time constant of these lead-lag controllers shall be set to ≥ 50 seconds and the lag time constant set to \$5 seconds. A note in the generic procedure stated that the "TI" time constant setting on the lead-lag card corresponds to the lag time constant. However, for a lead-lag function, "TI" actually sets the lead time constant. As a result of this procedure error the lead time constant had been actually set to 5 seconds and the lag time constant set to 0.5 seconds, which is in violation of Technical Specifications.

To comply with Technical Specifications 3.3.2-1.e and 3.3.2-4.d, RCS pressure was reduced below the P-11 setpoint (1970 psig) and stabilized at 1800 psig.

The generic calibration procedure, ITG-ZZ-WNLL1, has been revised to ensure that the lead-lag time constants are set in accordance with Technical Specifications. The corrected procedure was reissued 9/26/84. The affected steamline pressure loops were recalibrated and restored to service by 9/28/84. Channel calibrations of the RCS Δ T/TAVG loops were satisfactorily completed by 9/30/84 incorporating the revised generic procedure.

The purpose of the STEAMLINE LOW PRESSURE channels is to provide Safety Injection System (SIS) actuation in the event of a main steam system piping failure. Other functions that also provide SIS actuation in the event of a major steamline rupture are:

- 1. Four PRESSURIZER LOW PRESSURE channels
- 2. Three HIGH-1 CONTAINMENT PRESSURE channels

NRC Form 366A (9-83)	LICENSEE EVENT REPO		U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES. 8/31/85				
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As these functions were not affected by the incorrect calibrations and are also required to be operable in conjunction with the STEAMLINE LOW PRESSURE channels, there was no effect on public health and safety.

Previous occurrences: none

UNION ELECTRIC COMPANY CALLAWAY PLANT

MAILING ADDRESS: P.O. BOX 620 FULTON, MO. 65251

October 25, 1984

U. S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

ULNRC-956

CALLAWAY PLANT UNIT 1
FACILITY OPERATING LICENSE NPF-30
LICENSEE EVENT REPORT 84-043-00
STEAMLINE LOW PRESSURE CHANNELS INOPERABLE

Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a)(2)(i) and 50.73(a)(2)(v) concerning incorrect calibration of the Steamline Low Pressure channels.

ften Etheltuberger S. E. Miltenberger

Manager, Callaway Plant

CDN/WRR/JMS/drs Enclosure

cc: Distribution attached

IEZZ

cc distribution for ULNRC-956

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