

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Browns Ferry - Unit 1						DOCKET NUMBER (2) 0 5 0 0 0 2 5 9			PAGE (3) 1 OF 0 2	
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TITLE (4)
Reactor Manual Scram due to Main Steam Relief Valve Lifting

EVENT DATE (6)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)					
0	6	2	7	8	4	8	4	0	2	7	0	0	0	0		
											0	5	0	0	0	

OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 1		<input type="checkbox"/> 20.402(b)	<input checked="" type="checkbox"/> 20.406(e)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(a)						
		<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
		<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(e)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
		<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
		<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)								

LICENSÉE CONTACT FOR THIS LER (12)						TELEPHONE NUMBER					
NAME D. L. Smith						AREA CODE 2 0 5 7 2 9 - 0 8 6 5					

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR			
<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO					0	9	0	5	8	4

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During normal unit startup following a short unit outage, at approximately 1 percent power and rated pressure, a main steam relief valve began to unseat. It continued to leak causing the torus temperature to approach Technical Specification limits and the unit was manually scrambled at 350 psig. The relief valve subsequently resealed at 100 psig. No unusual occurrences followed.

The valve was replaced and the unit restarted. The failed valve is being disassembled with the results of the investigation to be provided in a followup report.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Browns Ferry - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 5 9 8 4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	- 0 2 7	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A-2 (1/7))

On June 27, 1984, unit 1 was in startup at 1 percent power, unit 2 was at 49 percent power, and unit 3 in a refueling outage. Unit 1 was the only unit affected by this event.

Unit 1 was in startup and approaching 1 percent power at rated pressure, when the pressure control valve 1-4 (RV) unseated. As reactor (RCT) pressure decreased, torus (BT) temperature was approaching its Technical Specification limit; therefore, a reactor manual scram was initiated at 350 psig. The valve subsequently reseated at 100 psig. No unusual events followed, and there was no safety significance to this event.

The failed valve was replaced and taken to Wyle Laboratories for testing and disassembly. The results will be provided as a followup report.

Responsible Plant Section - N/A

Previous Similar Events

- BFRO-50-259/7726
- BFRO-50-259/7713
- BFRO-50-260/7503
- BFRO-50-260/7430
- BFRO-50-260/7429
- BFRO-50-259/7349
- BFRO-50-259/7314

TENNESSEE VALLEY AUTHORITY

CHATTANOOGA, TENNESSEE 37401
Browns Ferry Nuclear Plant
P. O. Box 2000
Decatur, Alabama 35602

JUL 24 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Dear Sir:

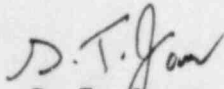
TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET
NO. 50-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE
REPORT BFRO-50-259/84027

The enclosed report provides details concerning reactor manual scram due to
main steam relief valve lifting. This report is submitted in accordance
with 10 CFR 50.73 (a)(2)(iv).

A 10 CFR Part 21 determination will be made following detailed disassembly
report from Wyle Labs.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



G. T. Jones
Plant Manager
Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):
Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement
Region II
101 Marietta Street, Suite 2900
Atlanta, GA 30303

INPO Records Center
Suite 1500
1100 Circle 75 Parkway
Atlanta, GA 30339

NRC Resident Inspector, BFN

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11