

NRC Form 386
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Grand Gulf Nuclear Station - Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 4 1 6	PAGE (3) 1 OF 0 2
--	--------------------------------------	----------------------

TITLE (4) Vessel Level Transmitters Exceed Technical Specification Limits
--

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																														
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																																												
0 6	2 8	8 4	8 4	0 3 2	0 0	0 7	2 7	8 4	NA		0 5 0 0 0																																																												
<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td colspan="2">OPERATING MODE (9)</td> <td colspan="10">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)</td> </tr> <tr> <td colspan="2">4</td> <td><input type="checkbox"/> 20.402(b)</td> <td><input type="checkbox"/> 20.405(v)</td> <td colspan="2"><input type="checkbox"/> 50.73(a)(2)(iv)</td> <td colspan="2"><input type="checkbox"/> 73.71(b)</td> </tr> <tr> <td colspan="2">POWER LEVEL (10)</td> <td><input type="checkbox"/> 20.405(a)(1)(i)</td> <td><input type="checkbox"/> 50.36(c)(1)</td> <td colspan="2"><input type="checkbox"/> 50.73(a)(2)(v)</td> <td colspan="2"><input type="checkbox"/> 73.71(c)</td> </tr> <tr> <td colspan="2">0 0 0</td> <td><input type="checkbox"/> 20.405(a)(1)(ii)</td> <td><input type="checkbox"/> 50.36(c)(2)</td> <td colspan="2"><input checked="" type="checkbox"/> 50.73(a)(2)(vii)</td> <td colspan="2">OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td> </tr> <tr> <td colspan="2"></td> <td><input type="checkbox"/> 20.405(a)(1)(iii)</td> <td><input type="checkbox"/> 50.73(a)(2)(i)</td> <td colspan="2"><input type="checkbox"/> 50.73(a)(2)(viii)(A)</td> <td colspan="2"></td> </tr> <tr> <td colspan="2"></td> <td><input type="checkbox"/> 20.405(a)(1)(iv)</td> <td><input type="checkbox"/> 50.73(a)(2)(ii)</td> <td colspan="2"><input type="checkbox"/> 50.73(a)(2)(viii)(B)</td> <td colspan="2"></td> </tr> <tr> <td colspan="2"></td> <td><input type="checkbox"/> 20.405(a)(1)(v)</td> <td><input type="checkbox"/> 50.73(a)(2)(iii)</td> <td colspan="2"><input type="checkbox"/> 50.73(a)(2)(ix)</td> <td colspan="2"></td> </tr> </table>												OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)										4		<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(v)	<input type="checkbox"/> 50.73(a)(2)(iv)		<input type="checkbox"/> 73.71(b)		POWER LEVEL (10)		<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)		<input type="checkbox"/> 73.71(c)		0 0 0		<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input checked="" type="checkbox"/> 50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)				<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)						<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)						<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)																																																																					
4		<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.405(v)	<input type="checkbox"/> 50.73(a)(2)(iv)		<input type="checkbox"/> 73.71(b)																																																																	
POWER LEVEL (10)		<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)		<input type="checkbox"/> 73.71(c)																																																																	
0 0 0		<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input checked="" type="checkbox"/> 50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)																																																																	
		<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)																																																																			
		<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)																																																																			
		<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)																																																																			

LICENSEE CONTACT FOR THIS LER (12)											
NAME								TELEPHONE NUMBER			
Ronald W. Byrd/Licensing Engineer								6 0 1 4 3 7 - 2 1 4 9			

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		
D	J C	L T R	3 6 9								

SUPPLEMENTAL REPORT EXPECTED (14)								EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO												

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

Reactor vessel level transmitter calibration procedures were recently revised to compensate for the effect of condensate pot movement during vessel heatup to operating temperature and to include more accurate level elevation values. When the revised calibration procedures were performed, three transmitters were found to have been outside Technical Specification limits due to the values used in the previous calibration.

8408060030 840727
PDR ADOCK 05000416
S PDR

NRC Form 366A
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Grand Gulf Nuclear Station - Unit 1	0500041684	-	032	-	002	OF 02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During a reactor vessel nuclear heatup, data was taken to determine the effect of condensate pot movement on vessel level instrumentation. As a result, vessel level instrument calibration procedures were revised using new level elevation values to calibrate transmitters for a more accurate reading at vessel operating temperatures. When the revised procedures were performed, three of the transmitters were found to be outside of Technical Specification limits due to values used in the previous calibration.

The RPS level 8 trip, Channel D, exceeded the maximum 54.1 inch limit with an as-found value of 54.8 inches. The RPS level 3 trip, Channel C, exceeded the minimum 10.8 inches with an as-found value of 9.8 inches. The level 1 trip Channel C, would have occurred at -152.6 inches rather than at the Technical Specification maximum limit of -152.5 inches.

One function of the level 1 trip is to close the Main Steam Line drain valves. The inboard drain valve isolation would have occurred 0.1 inch lower than the Technical Specification limit since both B and C Channels are required for the trip function. The outboard isolation valve would have tripped within Technical Specification limits.

All other trip functions required by these systems would have occurred within the Technical Specification limits as other channels were available to complete the logic. In addition, the Level 8 trip is not required until Operational Condition 1.

This is reported pursuant to 10CFR 50.73 (a)(2)(vii) due to the D and C channels of independent systems becoming inoperable as a result of a common cause. The procedure changes and recalibrations have been completed.



MISSISSIPPI POWER & LIGHT COMPANY

Helping Build Mississippi

P. O. BOX 1640, JACKSON, MISSISSIPPI 39205

July 27, 1984

NUCLEAR LICENSING & SAFETY DEPARTMENT

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

SUBJECT: Grand Gulf Nuclear Station
Unit 1
Docket No. 50-416
License No. NPF-13
File: 0260/L-835.0
Vessel Level Transmitters Exceed
Technical Specification Limits
LER 84-032-0
AECM-84/0380

Attached is Licensee Event Report (LER) 84-032-0 which is a final report.

Yours truly,

L. F. Dale
Director

EBS/SHH:rg
Attachment

cc: Mr. J. B. Richard (w/a)
Mr. R. B. McGehee (w/o)
Mr. N. S. Reynolds (w/o)
Mr. G. B. Taylor (w/o)

Mr. Richard C. DeYoung, Director (w/a)
Office of Inspection & Enforcement
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Mr. J. P. O'Reilly, Regional Administrator (w/a)
U. S. Nuclear Regulatory Commission
Region II
101 Marietta St., N.W., Suite 2900
Atlanta, Georgia 30323