

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Turkey Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 0 2 5 0	PAGE (3) 1 OF 0 2
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TITLE (4)
Technical Specification - Reactor Coolant System Leakage

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)					
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)		
									N/A			0 5 0 0 0 0		
0 6	2 5	8 4	8 4	0 1 9	0 0 0	7 2	5 8	4	N/A			0 5 0 0 0 0		

OPERATING MODE (9) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10) 1 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input checked="" type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)						TELEPHONE NUMBER					
NAME Randall D. Hart, Licensing Engineer						AREA CODE 3 0 5 2 4 5 - 2 9 1 0					

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)											
CAUSE	SYSTEM	COMPONENT	MANUF. TURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUF. TURER	REPORTABLE TO NPRDS		
B	A B I	S V	R 3 4 4	N							

SUPPLEMENTAL REPORT EXPECTED (14)						EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)						<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces; i.e., approximately fifteen single-space typewritten lines) (16)

On June 25, 1984, Unit 3 was shutdown from 100% power due to a Reactor Coolant System (RCS) leak of approximately 10 gpm. The root cause was determined to stem from a packing leak due to a broken gland follower on valve 3-532, the lower isolation valve on the instrument sensing line to LT-3-460. The affected loop bistables were tripped in accordance with Operating Procedure 0208.14. Thus satisfying the Technical Specification requirement for minimum degree of redundancy for reactor trip signals on pressurizer high water level. During a RCS cooldown to affect repairs, valve 3-532 was manually backseated and stopped the leak. Immediate corrective actions included: 1) a manual unit shutdown and subsequent cooldown to repair valve, 2) original valve packing gland flange was replaced with a "strong-back" plate and washer, and 3) an overpressure test and visual leak check of the RCS were performed and satisfactorily completed. The long term corrective action to be taken is to replace the temporary "strong-back" with a permanent engineered device during the next refueling outage. The health and safety of the public were not affected. Similar occurrences: None.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	0 1 9	0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On June 25, 1984, at 6:30 a.m., a reactor shutdown was commenced on Unit 3 due to a Reactor Coolant System (RCS) leak of approximately 10 gpm. The root cause was determined to stem from a packing leak on the lower isolation valve, 3-532, on the instrument sensing line to LT-3-460. The affected loop bistables were tripped in accordance with Operating Procedure 0208.14. Thus satisfying the Technical Specification requirement for minimum degree of redundancy for reactor trip signals on pressurizer high water level.

The first indications of a leak were noted approximately 30 minutes after the pressurizer level transmitter LT-3-460 had failed low when the sump level began to increase rapidly (almost 10 gpm), containment pressure increased, and containment radiation detector readings for R-11 increased. The RCS leak rate, using the volume control tank (VCT) level and containment sump level, was estimated at approximately 9.8 gpm. A manual unit shutdown was commenced and the unit removed from service at 7:13 a.m., and placed in hot shutdown. The leak was identified as a broken gland follower on valve 3-532 during a containment entry. A RCS cooldown was initiated to affect repairs on the valve. A second containment entry resulted in isolating the leak by fully backseating valve 3-532 at approximately 10:43 a.m. The cooldown was terminated and preparations made to repair valve 3-532 at hot shutdown conditions. A plant change modification (PCM 84-115) was prepared, reviewed, and approved to fabricate and install a "strong-back" plate and washer to replace the original valve packing gland flange that was damaged. The replacement parts were fabricated, the valve repacked, the "strong-back" and washer installed, bolted in place, and torqued to the manufacturer's specifications. An overpressure test and visual leak test of the RCS were performed and satisfactorily completed. Based on test results, a determination was made that this event was an isolated incident. The unit was returned to service at 5:00 a.m. on June 26, 1984, and full power operation achieved at 2:10 p.m. of the same day.



July 25, 1984
PNS-LI-84-258

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Gentlemen:

Re: Reportable Event 84-19
Turkey Point Unit 3
Date of Event: June 25, 1984
Technical Specification -
Reactor Coolant System Leakage

The attached Licensee Event Report is being submitted pursuant to the requirements of 10 CFR to provide notification of the subject event.

Very truly yours,

A handwritten signature in cursive script, appearing to read "J. W. Williams, Jr.", is written over the typed name.

J. W. Williams, Jr.
Group Vice President
Nuclear Energy

JWW/PLP/js

Attachment

cc: J. P. O'Reilly, Region II, USNRC
Harold F. Reis, Esquire
File 933.1

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