		1823244
NRC'FORM 366 (12 81) 10 (FR 50	LICENSEE EVENT REPORT	APPROVED BY OMI
CONTROL BLOCK:	1 PLEASE PHINE OR TYPE A	ALL REQUIRED INFORMATION)
OT LA L BR F 1 O O	0 [- 10 10 10 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0	1 1 1 1 1 1 1 1 0 L 1 1 0
CONT I REPORT LOG 0 5 0	0 0 2 5 9 0 1 2 1 0 8 3	(0 1 5 2 5 8 4 0
event description and probable [0]2 On 12/10/83, with unit	econstauences (0) I in a refueling outage, on opera	nter observed RHR
o 3 pump 10 motor was spar	king and smoking. The motor was a	that down and subsequently
[0 4] restarted. After rest	arting, the motor tripped on over	urrent. With RHR Loop II
[0 5] Lout-of-service, only R	HR pump 1A, Loop I, was openable,	This exceeded limit
of LCO, Tech Spec 3.5.	B.9. Limits were exceeded for two	hours. There was no
	h and safety. Reactor vessel temp	erature remained within
o 8 limits required by Tec	h Specs 3.6.A.3 and .5.	
CODE CODE		COMP. VALVE UBCODE SUBCODE
CF WX	(12) Z (13) MOTORX	Z 1 (15) Z (16)
(1) ASTORT 8 3		X 2
ACTION PUTURE EFFECT SH	UTDOWN ATTACHMENT NERD	PRIME COMP. COMPONENT (26
	Z 1 0 0 0 0 0 Y 1 3 N	18. SUPPLIER MANUFACTURER
CAUSE DESCRIPTION AND CORRECT	IVE ACTIONS (27)	., ., .,
[1]0] [GE model No. 5K6348XC2]	3A, 2000 HP, 3 phase motor tripped	due to bearing and
	lure. RHR Loop II was returned to	
	is considered a random failure sine	
	fe nor is this a generic type fail	
consists of periodic of	il analyses to detect potential bea	aring failure.
1 5 H 28 0 0 0 9	NA A Operator obse	ervation (32)
ACTIVITY CONTENT	0	TION OF HELEASE (36)
16 2 3 2 0	NA J NA	
NUMBER TYPE PERCHIPTION	NA NA	
NUMBER DESCRIPTION (1)	NA	**
TYPE DESCRIPTION (43)	NA 8408030196 840525 PDR ADOCK 05000259 S PDR	
PUBLICITY DESCRIPTION (43)	NΛ	NRC USE ONLY
1 10		62 11 11 11 11 11
NAME OF PREPARER W.	alter T. Christopher PHON	NE 205-729-0800
	The state of the s	1 400 / 3

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Tennessee Valley Authority Browns Ferry Nuclear Plant

LER SUPPLEMENTAL INFORMATION

BRF0-50-	259/33068R2	Technical Sp	pecification to	volved	9
Reported	Under Technical	Specification	5.7.2.a(2)	Date	 7/16/84

Unit I was in a refueling outage with irradiated seel in the vessel and the vessel at atmospheric pressure; unit 2 was operating normally at 100 percent power; unit 3 was in a refueling outage. Unit I was the only unit affected by this event. On December 10, 1983, an operator observed that Risk pump 10 motor was sparking and smoking. The motor was shut down and subsequently restarted. After being restarted and running for a few minutes the motor tripped on overcutrent at 2045 hours. A and C phase and ground instantaneous overcurrent relays actuated. RHR Loop II was out-of-service. Therefore, RHR pump 1A, Loop 1, was the only RHR pump operable. This exceeded the limiting condition of operation as defined by Technical Specification (T.S.) 3.5.8.9. All unit 1 and 2 diesel generators were operable. RHR Loop II was returned to service at 2243 hours. T.S. limits 3.5.B.9 were therefore exceeded for 2 hours. RHR pump 1C motor replacement was completed on 12/16/83. The GE model number 5K6348XC23A, 2000 HP, 3 phase motor tripped due to lower guide bearing and subsequent winding failure. There was no effect on public health and safety. Reactor ve lel temperature was maintained within limits as required by T.S. 3.6.A.3 and .5 during this event. Evidence indicates that the lower guide bearing (GE model No. 2892334P26) failed due to excessive or continuous upward thrust being applied to the bearing. The estimated accumulated operating time does not approach the rated end-of-life for these bearings, nor is this believed to be a generic type failure. Therefore, this is considered a random failure. Recurrence control will consist of taking periodic oil samples for analysis to provide early notification of potential bearing failure.

Previous Similar Events

BFRO-50-296/81034

TENNESSEE VALLEY AUTHORITY

P. O. Box 2000 Decatur, Alabama 35602

JUL 2 4 1984

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Dear Cir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT UNIT 1 - DOCKET NO. 56-259 - FACILITY OPERATING LICENSE DPR-33 - REPORTABLE OCCURRENCE REPORT BFRO-50-259/83068 R2

Reference: H. J. Green's memorandum to you dated December 20, 1983.

The enclosed report provides followup information concerning failure of residual heat removal system pump motor. This report is submitted in accordance with Browns Ferry Unit 1 Technical Specification 6.7.2.a.(2).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

G. T. Jones Plant Manager

Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):
Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement

Region II 101 Marietta Street, Suite 2900

Atlanta, GA 30303

INPO Records Center Suite 1500 1100 Circle 75 Parkway Atlanta, GA 30339

NRC Resident Inspector, BFN

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