NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Pilgrim Nuclear Power Station - Unit #1	0 5 0 0 0 2 9 3	1 OF 0 2

TITLE (4)
Unplanned Actuation of Engineered Safeguards Feature (Reactor Proection System)

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
										0 5 0 0 0
0 9	2 8	8 4	8 4	— 0 1 4	— 0 0	1 0	2 6	8 4		0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 0 0		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Richard M. Schifone - Plant Engineer	AREA CODE 6 1 7 7 4 6 - 7 9 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	
D	E D	Z Z Z Z	Z Z Z Z	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 9/28/84, while shut down for refueling, a reactor scram signal was generated through the Reactor Protection System. The signal was initiated when 480V Bus B-4 was deenergized by Maintenance electricians. The deenergizing occurred as a result of an attempt to cross-tie Bus B-4 with 480V Bus B-2 without defeating the interlock trip mechanisms.

This event did not impact the health and safety of the public.

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1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Pilgrim Nuclear Power Station - Unit 1	0 5 0 0 0 2 9 3	8 4	— 0 1 4	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 9/28/84, while shut down for refueling with the reactor vessel defueled, an unplanned reactor scram signal was generated through the Reactor Protection System (RPS). Prior to the initiation, maintenance was being performed on the Scram Discharge Volume (SDV), which resulted in SDV level trips being present in the RPS. A reactor scram signal was received when 480V Bus B-4 was deenergized.

The deenergizing occurred while Maintenance electricians were attempting to remove from service the feeder breaker to 480V Bus B-4. This bus was to remain energized by cross-tying it with emergency Bus B-2. The attempt was made without defeating the interlock trip mechanism. As a result, the feeder breaker (52-401) and the crosstie breaker (52-410) both opened and deenergized Bus B-4. The loss of power to Bus B-4 removed power from the "B" RPS Motor Generator Set which, when coupled with the SDV trips, completed the RPS logic for a full reactor scram signal.

The cause of the event has been determined to be a procedural inadequacy. To preclude recurrence of similar events, a new procedure is being drafted which will provide additional guidance for cross-tying buses.

This event did not impact the health and safety of the public.

BOSTON EDISON COMPANY
800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

WILLIAM D. HARRINGTON
SENIOR VICE PRESIDENT
NUCLEAR

October 26, 1984

BECo Ltr. #84-183

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Docket Number 50-293
License DPR-35

Dear Sir:

The attached Licensee Event Report 84-014-00, "Unplanned Actuation of Engineered Safeguards Feature (Reactor Protection System)," is hereby submitted in accordance with the requirements of 10CFR50.73.

If there are any questions on this subject, please do not hesitate to contact me.

Respectfully submitted,

W D Harrington

W. D. Harrington

RS:caw

Enclosure: LER 84-014-00

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Standard BECo LER Distribution

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