NAC Form (9-83)	. 366				LIC	ENSE		NT RE	PORT	(LER)	U.S. NU A E	CLEAR REGUL	NO. 3150-0	MISSION	
FACILITY	-										DOCKET NUMBER	(2)	•	GE (3)	
	Duan	e Ar	nold	Energy C	enter						0   5   0   0	0 3 1 3	1 1 0	F 012	
TITLE (4															
EVI	UNDI	anne	a RWC	LER NUMBER	10ns	REP	CAT DAT	E (7)		OTHER	FACILITIES INVO	VED (.)			
MONTH	DAY	YEAR	YEAR	SEQUENTIA	MONTH DAY YEAR FACILITY				FACILITY NA	WES	DOCKET NUMBER(S)				
									None			0   5   0	0 0 1	11	
019	215	814	814	- 01316	- 010	110	215	8 4				0 . 5 . 0 .	0.0.	1.1	
			THIS RE	PORT IS BURNITT	ED PURSUANT	TO THE RE	EQUIREME	NTS OF 1	0 CFR §: /0	heck one or more	of the following) (11	0	-1-1		
POWE LEVE (10)		7 0	20, 20, 20, 20, 20, 20, 20,	402(b) 406(a)(1)(i) 406(a)(1)(ii) 406(a)(1)(iii) 406(a)(1)(iv) 406(a)(1)(v)		20.405(c) 50.36(c) 50.36(c) 50.36(c) 50.73(c) 50.73(c) 50.73(c)	e) (1) (2) (2)(i) (2)(ii) (2)(iii)		X	50.73(a)(2)(iv) 50.73(a)(2)(v) 50.73(a)(2)(vii) 50.73(a)(2)(viii)( 50.73(a)(2)(viii)( 50.73(a)(2)(x)	A) B)	73.71(b) 73.71(c) 0THER below en 366.AJ	Specify in A d in Text, NA	betract RC Form	
						ICENSEE O	CONTACT	FOR THIS	LER (12)		_			_	
NAME	Vana		c p.								AREA CODE	AREA CODE			
	Kenn	eth	5. Pu	tnam, le	chnical	Suppo	ort 1	gine	er		3119	81511	-1714	1516	
CAUSE	SYSTEM	COMP	ONENT	COMPLETE MANUFAC- TURER	REPORTABLE	EACH CO	MPONENT	CAUSE	SYSTEM	COMPONENT	MANUFAC	REPORTABL TO NPRDS	- ·		
Х	CLE	1	1.1		No					111					
		1	1.1						1,1	1.1.1	1111				
				SUPPLEM	ENTAL REPORT	EXPECTS	D (14)				EXPECTE	MON	TH DAY	YEAR	
-	. ///	onniem F	XPECTED	SURMISSION DAT							DATE (1	DN 5)			
ABSTRAC	CT /Limit t	to 1400 m		pproximately fiftee	n single-spece type	ewritten line	es/ (18)							11-	
an is le le	Or od Oct olate ak de ak age	n Sep tober ed as etect e and	20, 20, a re ion 1 ret	er 25, 19 1984 wit esult of logic. I urned to	84 durin ch the ra momenta n each normal	ng noi eactor ry spi case servi	rmal r shu uriou the s ce.	power t dov s sig yster	oper vn, th gnals n was	ation, a e Reacto in the R verified	nd on Sep r Water C eactor Wa to be fr	tember leanup ter Cle ee of a	30, 19 System anup ctual	84	
		94 91 9	41106 DR AI	50219 84	41025 000331 PDR										

NRC Form 386A (9-83) LICENSEE EVE	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION								JLATORY COMMISSION NB NO. 3150-0104 /85			
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)					PAGE (3)					
		YEAR		NUMBER		NUMBER			T			
Duane Arnold Energy Cent	er 0  5  0  0  0  3  3  1	8 4	_	0   3   6	-	010	01	2	OF	0 1 2		

At 0331 hours on September 25. 1984 the Reactor Water Cleanup (RWCU) system isolated upon a signal from its leak detection logic. The reactor was in normal run mode at approximately 70 percent power. A high differential flow alarm was received in the Control Room. The high differential flow logic is designed to initiate when a differential flow condition of greater than 40 gallons per minute exists between the inlet and outlet of the RWCU system. This parameter is a portion of the "B" leak detection isolation logic. No evolutions in the system were in progress at the time of the isolation. Motor operated isolation valves on the inlet line (CE-ISV-2701) and the outlet line (CE-ISV-2740) closed per design. Operators immediately inspected the RWCU system and found that no actual leakage condition existed. The system was unisolated and restored to normal service at 0418 hours without further difficulties.

On September 30, 1984 at 0256 hours with the reactor shutdown for an unrelated maintenance outage, the RWCU system again isolated as a result of a momentary spurious isolation signal in the "B" leakage detection logic. An RWCU high ambient temperature alarm was received Operators confirmed that no actual leak existed and restored the system to operation without further problem.

On October 20, 1984 at 1515 hours an isolation signal from the "B" isolation logic again resulted in isolation of the RWCU system. Operators verified no leakage condition existed and restored the system to normal status at 1545 hours.

The root cause of these spurious isolations is unknown. The RWCU system has experienced numerous spurious isolations in the past (see LER 84-024). Investigation of the source/sources of the spurious isolation signals and possible corrective measures is continuing, however no immediate engineering or maintenance actions have been identified that would eliminate the occasional spurious isolations.

These isolations of the reactor water cleanup system are reportable as actuations of an engineered safety feature (containment isolation). Throughout the events the isolation components of the system performed their safety function as designed.

## Iowa Electric Light and Power Company October 25, 1984 DAEC-84-685

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

> Subject: Duane Arnold Energy Center Docket No. 50-331 Op. License DPR-49 Licensee Event Report No. 84-036

Gentlemen:

In accordance with 10 CFR 50.73 please find attached a copy of the subject Licensee Event Report.

Very truly yours,

Daniel L. Mineck Plant Superintendent - Nuclear Duane Arnold Energy Center

DLM/KSP/kp

attachment

cc: Mr. James G. Keppler Regional Administrator Region III U. S. Nuclear Regulatory Commission 799 Roosevelt Road Glen Ellyn, IL 60137

NRC Resident Inspector - DAEC

File A-118a