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JUL 27 1984

50-352

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Mr. A. Schwencer, Chief Docket Nos.: Licensing Branch No. 2 U. S. Nuclear Regulatory Commission Washington, D. C. 20555

Subject: Limerick Generating Station, Units 1 and 2 Request for Additional Information NUREG-0737 Items II.F.1 Attachments 1 and 2, and Item III.D.1.1.

Reference: Letter, A. Schwencer to E. G. Bauer, Jr. dated May 9, 1984.

File: GOVT 1-1 (NRC)

Dear Mr. Schwencer:

The reference letter requested additional information on the subject NUREG-0737 items. Information pertaining to Item II.F.1 Attachments 1 and 2 is provided in the attached draft FSAR pages that will be incorporated into the FSAR via Revision 35 which will be submitted in August, 1984. Information pertaining to Item III.D.1.1 will be furnished in September, 1984.

Sincerely,

Ju Bellyton

DFC/gra/07198402

cc: See Attached Service List

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- cc: Judge Lawrence Brenner Judge Richard F. Cole Troy B. Conner, Jr., Esq. Ann P. Hodgdon, Esq. Mr. Frank R. Romano Mr. Robert L. Anthony Charles W. Elliot, Esq. Zori G. Ferkin, Esg. Mr. Thomas Gerusky Director, Penna. Emergency Management Agency Angus R. Love, Esq. David Wersan, Esq. Robert J. Sugarman, Esq. Spence W. Perry, Esq. Jay M. Gutierrez, Esq. Atomic Safety & Licensing Appeal Board Atomic Safety & Licensing Board Panel Docket & Service Section Martha W. Bush, Esq. Mr. James Wiggins Mr. Timothy R. S. Campbell Ms. Phyllis Zitzer Judge Peter A. Morris
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- (i) procedures for minimizing occupational exposures;
- (ii) calculational methods for converting instrument readings to release rates based on exhaust air flow and taking into consideration radionuclide spectrum distribution as function of time after shutdown;
- (iii) procedures for dissemination of information; and

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(iv) procedures for calibration.

### TABLE II.F.1-2

### INTERIM PROCEDURES FOR QUANTIFYI G HIGH-LEVEL ACCIDENTAL RADIOACTIVITY RI LEASES

Applicants are to implement procedures for estimating noble gas and radioiodine release rates if the existing 'ffluent instrumentation goes off-scale.

Examples of major elements of a highly radioactive effluent release special procedures (noble gas).

- Preselected location to measure radiation from the exhaust air, e.g., exhaust duct or sample line.
- Provide shielding to minimize background interference.
- Use of an installed monitor (preferable) or dedicated portable monitoring (acceptable) to measure the radiation.
- Predetermined calculational method to convert the radiation level to radioactive effluent release rate.

	[11.5.2.2.1, and its piping and instrumentation	diagram)
Response	L is provided in Figure 11.5-1.	L

All reactor enclosure stack releases following an accident will be through the north stack. The wide range accident monitoring subsystem of the north stack effluent monitoring system provides continuous monitoring of post-accident releases of noble gases in accordance with the requirements of Table II.F.1-1. The system is described in Sections 7.6 and -Control room displays provided for this system meet the requirements of Regulatory Guide 1.97, Revision 2, and are described in Section 7.5. Table II.F.1-2 outlines the requirements for an interim method for quantifying releases to be used by operating reactors and therefore is not applicable to Limerick. Human factors aspects of TMI Item II.F.1 are considered part of the control room design review required by Item I.D.1. DRAFT

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Two	array i pressur for pro indepen 64 unif	is provided, re sensors an oviding an in ident samplin formly-spaced	sampling can b consisting of d 32 uniformly stantaneous tr g arrays, each isokinetic no les at the sta	128 uniformly -spaced stati averse across consisting o zzles are pro	y-spaced ic press s the st of a set ovided f	of extractin
		au providor	-	ha	- 1	INSERT (2)
	radiati	on monitorin	a sample for t g subsystem G	ne normal pla	int oper	ation
	particu the cor effluen a noble	responding in t, an iodine gas monitor	paths. Each provided with ntegrated meas filter provid ing chamber. ovide the foll	a radiation d urement of th ed with an in Thus, each of	ed throu detector ne parti n-place the tw	indicating culate detector, an
2	•	Sampling f	low rates	D	RA	FT
DASOLE	•	Particulate	e radioactivit			****
DISPLAY CONSOLES	•	Iodine radi	ioactivity, in	tegrated		
IN SOMA	•	Noble gas n	radioactive co	ncentration		
The The	adioact detector outputs with men	tive effluent rs are fed in to in the cor	the stack flow t may readily h nto microproces ntrol room. Th on capability f	be evaluated. ssors, which he microproce	Readou in turn ssors an	uts from the provide re provided
1	ivenc of	t a power rar	Iure.			COMMON
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(1	igh and		EITHER HALLS			
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Sufficient redundancy is provided to allow release rates. maintenance and checking of one channel without losing monitoring Sent WIDE RANGE ACCIDENT MONITOR capability. The wide-range accident monitoring subsystem is independent of the normal plant operation monitoring subsystem and operates COMO sample, drawn from continuously. the second 64-nozzle array described above, is passed through a particulate filter, iodine filter, and noble gas detector DISPLAY B assembly This provides redundancy to the normal plant operation monitoring subsystem. from a separate comb-type RMDS probe located downstream of the isokinetic nozzle arrays. This (c)sample is passed through shielded particulate and iodine filters range noble gas detector assembly. Detector and, THE D outputs are fed into microprocessor E Outputs of the microprocessors are transmitted to readout module recorders in the control room. The microprocessors have memory G retention in event of loss of power. One Downscale AND TWO UPSCALE ALARMS ANNUNCIATE ABNORMAL MONITOR CONDITIONS TO THE CONTROL ROOM. THE WRAMALSO PAO-PVIDES A MICH RADIATION TRIP TO THE CONTAINMENT PURGE VALVES. The particulate and iodine filters of the wide-range accident monitoring subsystem are used as grab sample modules to provide the capability of collecting representative samples of iodines and particulates for onsite analysis during and following an accident. The sample lines are heat traced to preclude entrained moisture in the effluent stream that could degrade the filters. Three removable filter modules are provided in both sample flow paths to This also allows the control room allow continuous collection. operator to select a clean set of filters in order to prevent appreciable concentrations of noble gases produced by iodine decay in a loaded filter, which could be falsely interpreted by the noble gas detectors as high activity in the effluent stream. V Filters on the high activity sample flow path are shielded to keep personnel exposure in sample handling and transport below the General Design J Criteria 19 limits of 5 rem whole-body exposure and 75 rem to the extremities during the duration of the accident. 1 K 11.5.2.2.2 South Stack Effluent Radiation Monitors

The objectives and functions of the south stack monitoring system are the same as those of the north stack normal plant operation monitoring subsystem. A system for post-accident monitoring is not provided because any HVAC exhaust to this stack containing accident effluents is automatically isolated.

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INSERT (2) p. 11-5-13

A DIFFERENTIAL PRESSURE SENSOR CONNECTED TO THIS SENSING ARRAY PROVIDES THE STACK FLOW RATE SIGNAL. TO ALLOW MAINTENANCE WITHOUT LOSING MONITORING CAPABILITY A REDUNDANT STACK FLOW RATE SIGNAL IS PROVIDED BY WINE THERMAL-TYPE VELOCITY JENSORS CONNECTED TO AN AVERAGING NETWORK. A LOCAL SWITCH ALLOWS SELECTION OF ONE OF THE TWO STACK FLOW SIGNALS TO BE INPUT TO THE MICROPROCESSOR-BASED NSE AND WRAM MONITORS FOR ISORIMETIC CONTROL OF THE SAMPLE FLOW RATES. THE STACK FLOW RATE AND SAMPLING FLOW RATES ARE INDICATED ON DEMAND IN THE CONTROL ROOM VIA THE WRAM READOUT MODULE OR THE RMDS DISPLAY CONSOLES (SECTION 11.5.6).

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... AND THE OTHER FOR THE WRAM. SAMPLE BYPASS WITH ASSOCIATED PUMPS AND FLOW CONTROLS ARE PROVI-DED TO ALLOW THE SAMPLING ARRAY FLOW TO FOLLOW THE STACK FLOW RATE NOKINETICALLY WITHOUT EXCLLEING

THE MAXIMUM SAMPLE FLOW RANGE OF THE MONITORY. THE SYSTEM HAS THE CAPABILITY TO MAINTAIN ISO KINEFIC CONDITIONS WITH VARIATIONS IN STACK FLOW RATE OF ± 25 %.

INSERT A p. 11.5-14

EFFLUENT SAMPLES ARE DRAWN VIA TWO SAMPLE FLOW PATHS. DURING NORMAL PLANT OPERATION ONE ...

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INSENT B

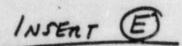
IOW RANGE

INSERT C

IF THE LOW RANGE DETECTOR APPADACHES ITS UPSCALE LIMIT, THE SYSTEM AUTO MATICALLY STARTS PUMPING EFFLUENT JAMPLE ...

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A MID- AND HIGH -...



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WHEN THE MID-RANGE DETECTOR REACHES A PRE-SELECTED POINT, THE LOW RANGE DETECTOR IS AUTO-MATICALLY PURGED AND FLOW THROUGH IT IS STOPPED. PURGING ASSURES THAT THE LOW RANGE DETECTOR CAN RESUME MONITORING WHEN ACTIVITY AGAIN DECREASES. A SIMILAR AUTOMATIC PURGE/ SHUT DOWN CYCLE IS PERFORMED ON THE MID-AND HIGH-RANGE DETECTORS WHEN ACTIVITY DE-CREASES. P (CONTINUES)

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INSENT E P. 11.5-14 (CONTINUED)

THE LOW RANGE DEFECTOR ASSEMBLY CONSISTS OF A SHIELDED CHAMBER AND A BETA-JENSITIVE PLASTIC SCINTILLATION DEFECTOR. THE MID- AND HIGH-RANGE DEFECTORS USE CADMIUM TELLURIDE SOLID STATE SENSORS HOUSED IN A SHIELDED CHAMBER.

INSENT (F)

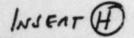
MATA APPLIES CONVERSION FACTORS, DETERMINES IF ALARM SETPOINTS HAVE EVEN EXCEEDED AND RETAINS DATA FUR EACH DETECTOR CHANNEL IN HISTORY FILES. IN ADDITION, THE MICAOPALO CESSOR AND MATICALLY CALCULATES ETFLUENT RELEASE RATE PER UNIT TIME BAJED ON DETECTOR MEASUREMENT: AND STACK FLOW RATE. BACKGROUND JUBTRACTION IS PROVIDED BASED ON MANUALLY ENTERED BACKGROUND VALUES. DRAFT

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ONE THREE-PEN REIONDEN PROVIDES INDICATION OF RADIOACTIVITY CONCENTRATION IN THE LOW, MID- AND HIGH-RANGE CHANNELS, WHILE EFFLUENT RELEASE RATE IS RECORDED IN A SINGLE PEN RECORDER. THE RMDS DISPLAY CONSOLES CAN ALSO PROVIDE TREND PRINTOUTS ON DEMAND OF THE MICROPRIS-CESSOR HISTORY FILES.

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THE MONITON IS INITIALLY CALIBRATED BY THE MANUFACTUREN USING GASEOUS AND SOLID SOURCES INCLUDING SR-90. TRANSFER SOURCES AND THEN USED TO TRANSFER THIS CALIBRATION DATA BASE TO THE PLANT BY REALIGNING EACH DETECTOR TO THE CONDITIONS ESTABLISHED DURING PRIMARY CALIBRATISN. THIS TRANSFER CALIBRATION IS PERFORMED PLAID DICALLY IN THE PLANT AT DESIGNATED INTERVALS ACCORDING TO THE PLANT TECHNICAL SPECIFICATIONS

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DISSEMINATION OF INFORMATION FROM THIS MONITOR VIA THE RMMS DATA LINKS TO THE CONTROL ROSM, TECHNICAL SUPPORT CENTER AND EMERGENCY OPENATIONS FACILITY, AS WELL AS CONSIDERATION OF RADIONUCLIDE DISTRIBUTION AS A FUNCTION OF TIME AFTER SHUT-DOWN IN DOSE ASSESSMENT IS DISCUSSED IN SECTION 11.5.6.



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CONTROLS ARE PROVIDED IN THE CONTROL ROOM TO SELECT FILTERS, )GARE SAMPLING LOCALIT OR REMOTELY, AND TO INITIATE AUTOMATICALLY TIMED GRAE SAMPLING. DRAFT INSERT (R)

... ENCLOSURES DESIGNED FOR EASE OF REMOVAL AND REPLACEMENT OF FILTER MEDIA. FILTER REMOVAL IS PROVIDED BY MEANS OF QUICK DISCONNECT COUPLINGS. AFTER REMOVAL, THE FILTER IS PLACED IN A SHIELDED CASK FOR TRANSPORT TO THE ONSITE ANALYSIS FACILITY. THE FILTER ENCLOSURE AND TRANSPORT CASK HAVE BEEN DESIGNED, AND THE ACCESS ROUTES SELECTED. in accordance with the requirements & NOREG-0737...

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FOR IDDINE AND FOR 0.3 MILLION PARTICLES.

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TECHNICAL SUPPORT CENTER (TSC) AND EMERGENCY OPERATIONS FACILITY (EOF)

In the RMDS dual computer configuration each computer is equipped with fast access mass storage devices to store executable programs, application data base and a dynamically created data storage. RMDS display consoles and terminals located in the are interfaced to the RMDS computers to control room. allow the access of information from the computers and PRMs and to display PRMs status. The RMDS has seven data links from each computer. Five data links communicate each RMDS computer with the PRMs located throughout the plant. A sixth link communicates each RMDS computer with the MMDRS. The seventh link provides communications between each RMDS computer.

The RMDS provides PRMs status and RMDS status through annunciation of alarm conditions at the RMDS display consoles to supplement the PRM alarms at the plant annunciator panel. The operator is alerted of abnormal conditions through Visva/ alarms located at each RMDS display console. Alarm events are stored on the RMDS mass storage devices and logged on the RMDS terminals. Thend DISPLAYS OF THE PRM MICHOPAGESSONS HISTORICAL FILES ANE AVAILABLE FOR DISPLAY ON HANDLOPY AT THE RMDS DISPLAY CONSOLES.

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11. 5.6.2.1

5.2.2 Meteorological Monitoring Display and Reporting Subsystem: (MMDRS)

The MMDRS is a computer system provided with increased memory capabilities, fast access mass storage devices, high speed line printing devices, magnetic tape storage facilities, and remote display console devices with hardcopy outputs.

The MMDRS computer is equipped with remote consoles and hardcopy units located in the control room, TSC. Counting laboratory, and the EDF to allow access of information from the computer, display FRMs information obtained from the RMDS, display meteorological data obtained from the meteorological instrumentation, and to allow for manual input of multichannel analyzer data, radiation levels, meteorological data and process data.

The MMDRS has three data links. Two data links communicate data from each RMDS computer to the MMDRS. The third data link communicates data from the meteorological instrumentation rescribed in section 2.3 to the MMDRS computer.



## 11.5.6.3.2

The MMDRS computer can also do calculations of atmospheric dispersion and dose for accidental release of gaseous effluent. To do calculations of atmospheric dispersion (X/Q values), the MMDRS computer uses data obtained from the meteorological towers or prompts the operator to enter the desired data. These data are used to compute the most recent dispersion conditions for a given release point. Dose calculations can be done using radiological release data obtained automatically from the RMDS or entered manually by the operator. These calculations can be performed using data obtained over a 1-minute, 15-minute, or 1 hour time interval.

The model used for the dispersion and dose calculations corresponds to the Class A model outlined in Appendix 2 of NUREG-0654.

An enhanced Class A model is provided which is also used for dispersion and dose calculations. This model uses on-site and forecast meteorological data to plot the time dependent movement of the plume.

The results of the dose calculations, in addition to being available at the control room, TSC counting laboratory , and EOF are available to local, state and federal emergency officials through an interrogation/broadcast system.

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THE SOFTWARE INCLUDES RADIONUCLIDE SPECTRUM DISTRIBUTION AS A FUNCTION OF TIME FOR DIFFERENT TYPES OF ACCIDENTS. THE JOPTWARE PROVIDES DECAY OF THE SPECTRA FROM THE TIME ACCIDENTS BEGIN TO THE TIME THE CONCENTRATIONS ARE CALCULATED. MANUAL ENTRY ALLOWS USE OF ACTUAL SPECTRA THAT IS OBTAINED ONCE GRAB SAMPLES CAN RE RETRIEVED. (5) License applicants should have available for review the final design description of the as-built system, including piping and instrument diagrams together with either (a) a description of procedures for system operation and calibration, or (b) copies of procedures for system operation and calibration. Changes to technical specifications will be required. Applicants will submit the above details in accordance with proposed review schedule, but in no case less than 4 months prior to the issuance of an operating license. A postimplementation review will be performed.

#### Response

Sampling of plant gaseous effluents for post-accident releases of iodines and particulates is provided as part of the wide range accident monitoring subsystem of the north stack effluent radiation monitoring system described in Sections 7.6 and 1552 1990 The design of onsite laboratory facilities for analysis of these samples is described in Chapter 12. The design of the sampling media and sampling considerations are in conformance with Table II.F. 1-3 of NUREG-0737. Human factors aspects of TMI Them II.F. 1 are considered Part of the control room design review required by ATTACHMENT 3, Containment High-Range Radiation Monitor Item I.D.1.

#### Position

In containment radiation-level monitors with a maximum range of 10° rad/hr shall be installed. A minimum of two such monitors that are physically separated shall be provided. Monitors shall be developed and gualified to function in an accident environment.

#### Clarification

- Provide two radiation monitor systems in containment which are documented to meet the requirements of Table II.F.1-4.
- (2) The specification of 10<sup>8</sup> rad/hr in the above position was based on a calculation of postaccident containment radiation levels that include both particulate (beta) and photon (gamma) radiation. A radiation detector that responds to both beta and gamma radiation cannot be qualified to post-LOCA (loss-of-coolant accident) containment environments but gama-sensitive instruments can be so qualified. In order to follow the course of an accident, a containment monitor that measures only gamma radiation is adequate. The requirement was revised in the October 30, 1979 letter to provide for a photon-only measurement with an upper range of 107 R/hr.
- (3) The monitors shall be located in containment(s) in a manner as to provide a reasonable assessment of area radiation conditions inside containment. The monitors shall be widely separated so as to provide independent measurements