Table 3.7-3

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION** AND ELEVATION		ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE**** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
Decay Heat Removal System					
DHH-17	RB	110'-6"	I	No	No
DHH-18	RB	110'-6"	I	No	No
DHH-19	RB	109'-0"	I	No	No
DHH-20	RB2	115'-0"	I	Yes	No
DHH-21	RB2	117'-0"	I	Yes	Yes
DHH-22	RB2	117'-0"	I	Yes	Yes
DHH-23	RB	110'-6"	I	No	Yes
DHH-24	RB	109'-3"	I	No	No
DHH-25	RB	110'-6"	I	No	Yes
DHH-26H	RB	110'-6"	I	No	No
DHH-26V	RB	110'-6"	I	No	No
DHH-27	RB	110'-6"	I	No	Yes
DHH-35	RB2	152'-5"	I	Yes	No
DHH-36	RB2	152'-5"	I	Yes	No
DHH-37	RB2	159'-7"	I	Yes	Yes
DHH-38	RB2	160'-1"	I	Yes	Yes
DHH-39	RB2	165'-9"	I	Yes	No
DHH-661	AB	86'-6"	A	Yes	No
DHR-18	AB	84'-7"	A	Yes	No
DHR-21	AB	103'-6"	A	No	No
DHR-24U	AB	129'-6"	A	No	No
DHR-24L	AB	129'-6"	A	No	No
DHR-28	AB	134'-4"	A	No	No
DHR-31	AB	84'-9"	A	Yes	No
DHR-37	AB	85'-6"	A	Yes	No
DHR-49	AB	85'-6"	A	Yes	No

Table 3.7-3

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION** AND ELEVATION		ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE**** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)	
	Emergency Feedwater System					
EFH-14U	RB	115'-0"	I	No	No	
EFH-14L	RB	115'-0"	I	No	No	
EFH-15U	RB	115'-0"	I	No	No	
EFH-15L	RB	115'-0"	I	No	No	
EFH-27	RB2	145'-9"	I	Yes	Yes	
EFH-28	RB2	145'-9"	I	Yes	Yes	
EFH-92	IB	140'-0"	I	No	No	
EFH-93	IB	131'-8"	I	No	No	
EFH-94	IB	131'-8"	A	No	No	
EFH-95	IB	140'-0"	A	No	No	
EFH-96	IB	141'-3"	A	No	No	
EFH-106	IB	141'-3"	I	No	No	
EFH-107	IB	141'-3"	A	No	No	
EFH-108	IB	141'-3"	A	No	No	
EFH-109	IB	133'-0"	A	No	No	
EFH-110	IB	133'-0"	I	No	No	
EFH-141	IB	126'-6"	A	No	No	
EFH-143	IB	133'-6"	A	No	No	
EFH-144	IB	141'-3"	I	No	No	

Table 3.7-3

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON,	LOCATION**	AND ELEVATION	ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE**** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)	
	Feedwater System (Continued)						
FWH-149	IB		133'-0"	I	No	No	I
FWH-150	IB		133'-0"	A	No	No	
FWH-151	IB		131'-0"	A	No	No	
FWH-152	IB		131'-0"	A	No	No	
FWH-153	IB		131'-0"	I	No	No	
FWH-154	IB		131'-0"	I	No	No	
FWH-155	IB		131'-0"	I	No	No	
FWH-156	IB		131'-0"	I	No	No	
FWH-157	IB		136'-0"	I	No	No	
FWH-158	IB		136'-0"	I	No	No	
FWH-159	IB		136'-0"	A	No	No	
FWH-160	IB		136'-0"	A	No	No	
FWH-161	IB		136'-0"	A	No	No	
FWH-162	IB		142'-0"	I	No	No	I
FWH-163	IB		136'-0"	A	No	No	
FWH-164	IB		136'-0"	A	No	No	
FWH-165	IB		134'-0"	A	No	No	
FWH-166	IB		134'-0"	I	No	No	I
FWH-167	IB		134'-0"	I	No	No	
FWH-168	IB		134'-0"	A	No	No	I
FWH-169	IB		134'-0"	I	No	No	
FWH-170	IB		134'-0"	A	No	No	
FWH-171	IB		119'-0"	A	No	No	

Table 3.7-3

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION** AND ELEVATION		ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE**** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
	Post Accident Venting System				
LRH-59	AB	160'- 10"	A	No	No
LRH-60	AB	160'- 10"	A	No	No
LRH-68	AB	161'- 0"	A	No	No
LRH-69	AB	161'- 0"	A	No	No

Table 3.7-3

SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION** AND ELEVATION		ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE**** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
	Main Steam System				
MSH-117	IB	122'-0"	A	No	No
MSH-118	IB	122'-0"	A	No	No
MSH-119	IB	122'-0"	A	No	No
MSH-120	IB	122'-0"	A	No	No
MSH-121	IB	122'-0"	A	No	No
MSH-122	IB	122'-0"	A	No	No
MSH-123	IB	122'-0"	A	No	No
MSH-124	IB	122'-0"	A	No	No
MSH-125	IB	122'-0"	I	No	No
MSH-126	IB	122'-0"	I	No	No
MSH-128	IB	122'-0"	A	No	No
MSH-139	RB	133'-0"	I	No	No
MSH-147	RB	139'-0"	I	No	Yes
MSH-149	RB	123'-0"	I	No	No
MSH-150	RB	123'-0"	I	No	No
MSH-156	RB	154'-0"	I	No	Yes
MSH-159	RB	148'-0"	I	No	Yes
MSH-160	RB	142'-0"	I	No	No
MSH-161	RB	154'-0"	I	No	No
MSH-162	RB	132'-0"	I	No	Yes
MSH-163	RB	138'-0"	I	No	Yes
MSH-164	RB	123'-0"	I	No	No
MSH-165	RB	123'-0"	I	No	No
MSH-166	RB	148'-0"	I	No	Yes
MSH-167	RB	148'-0"	I	No	Yes

Table 3.7-3

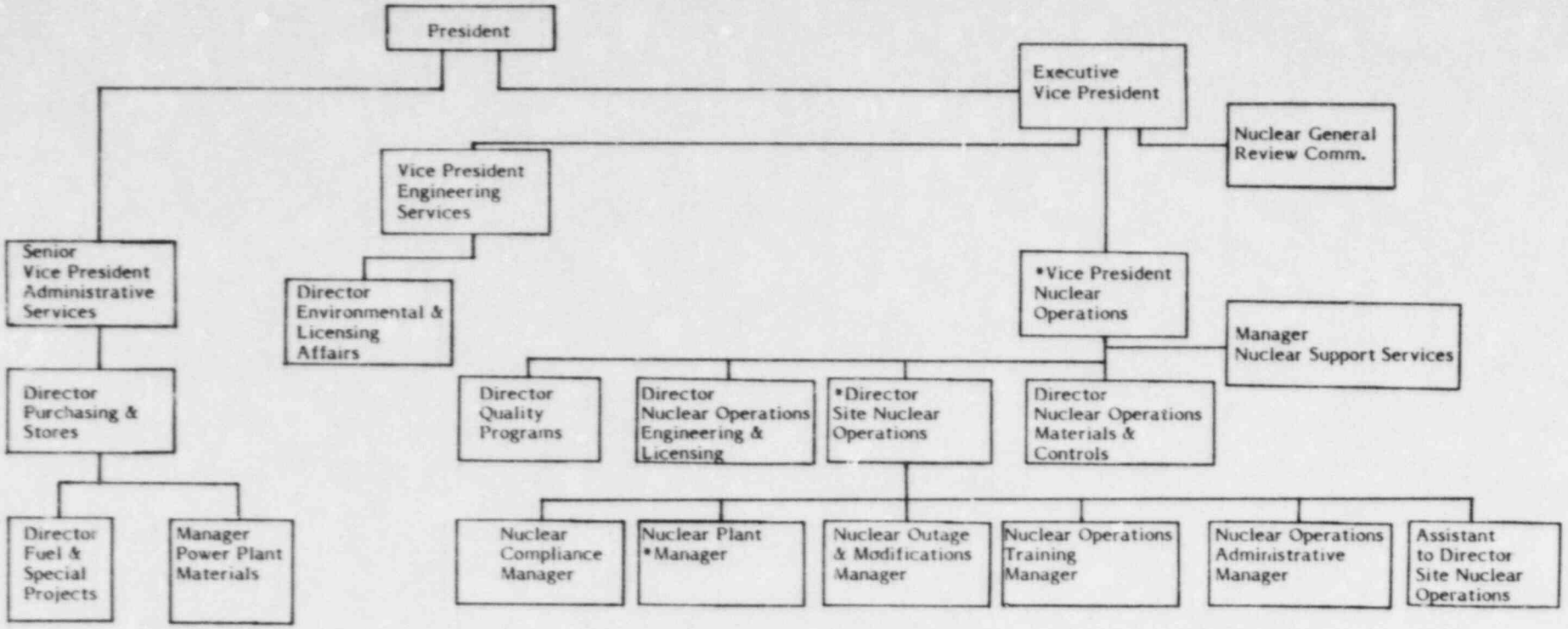
SAFETY RELATED HYDRAULIC SNUBBERS*

SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION** AND ELEVATION		ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE**** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
Main Steam System (Continued)					
MSH-168	RB	142'-0"	I	No	Yes
MSH-169	RB	148'-0"	I	No	Yes
MSH-170	RB	142'-0"	I	No	Yes
MSH-205	IB	132'-0"	I	No	No
MSH-206	IB	136'-0"	I	No	No
MSH-207	IB	136'-0"	I	No	No
MSH-208	IB	130'-0"	A	No	No
MSH-209	IB	130'-0"	A	No	No
MSH-210	IB	122'-0"	A	No	No
MSH-211	IB	109'-0"	A	No	No
MSH-212	IB	134'-0"	I	No	No
MSH-213	IB	130'-0"	I	No	No
MSH-214	IB	115'-0"	A	No	No
MSH-227	IB	122'-0"	I	No	No
MSH-232	IB	120'-0"	I	No	No
MSH-240	IB	122'-0"	I	No	No
MSH-243	RB	147'-0"	I	No	No
MSH-248	IB	136'-0"	A	No	No
MSH-249	IB	136'-0"	A	No	No
MSH-250	IB	111'-0"	A	No	No
MSH-251	IB	109'-0"	A	No	No
MSH-252	IB	136'-0"	A	No	No
MSH-253	IB	104'-0"	A	No	No
MSH-254	IB	104'-0"	A	No	No
MSH-255	IB	105'-0"	A	No	No

Table 3.7-3

SAFETY RELATED HYDRAULIC SNUBBERS*

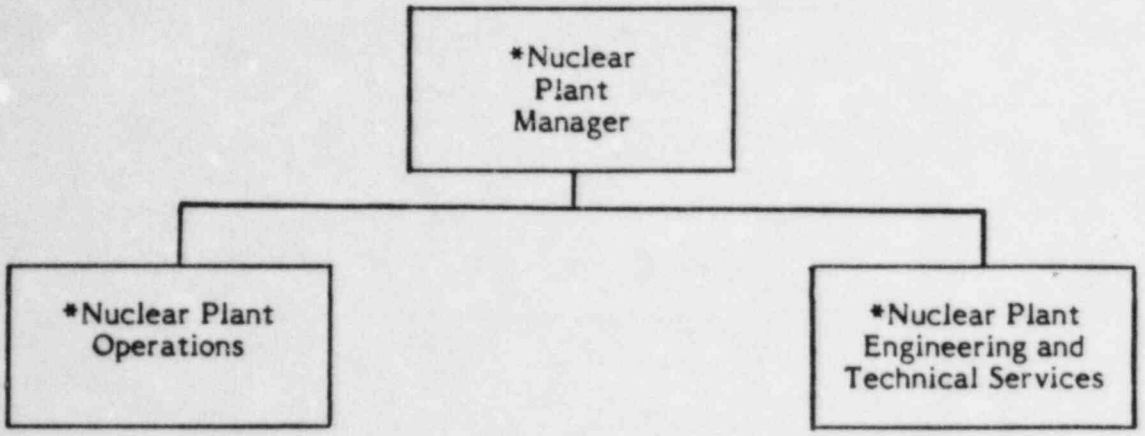
SNUBBER NO.	SYSTEM SNUBBER INSTALLED ON, LOCATION** AND ELEVATION		ACCESSIBLE OR INACCESSIBLE (A or I)	HIGH RADIATION ZONE**** (Yes or No)	ESPECIALLY DIFFICULT TO REMOVE (Yes or No)
	Reactor Coolant System (Cont'd)				
RCH-67	RB2	166'- 0"	I	Yes	No
RCH-68	RB2	167'- 1"	I	Yes	No
RCH-69	RB2	167'- 1"	I	Yes	No
RCH-70	RB2	167'- 1"	I	Yes	No
RCH-71U	RB2	167'- 1"	I	Yes	No
RCH-71L	RB2	167'- 1"	I	Yes	No
RCH-73	RB2	167'- 1"	I	Yes	No
RCH-74	RB2	167'- 1"	I	Yes	No
RCH-76	RB2	139'- 11"	I	Yes	Yes
RCH-77	RB2	131'- 6"	I	Yes	Yes
RCH-78	RB2	150'- 10"	I	Yes	Yes
RCH-79	RB2	156'- 8"	I	Yes	Yes
RCH-80	RB2	168'- 3"	I	Yes	No
RCH-81	RB2	168'- 4"	I	Yes	No
RCH-82E	RB	143'- 9"	I	No	Yes
RCH-82W	RB	143'- 9"	I	No	Yes
RCH-83	RB	145'- 11"	I	No	No
RCH-84	RB	123'- 9"	I	No	No
RCH-85	RB	147'- 0"	I	No	No
RCH-86	RB	144'- 9"	I	No	No
RCH-87	RB	147'- 0"	I	No	No
RCH-88	RB	147'- 0"	I	No	No
RCH-89	RB	124'- 0"	I	No	No
RCH-90	RB	120'- 0"	I	No	No
RCH-530	RB2	170'- 6"	I	Yes	No
RCH-531	RB2	170'- 6"	I	Yes	No



The Corporate Organization may be changed without prior License Amendment to Figure 6.2 -1 provided that a revision to Figure 6.2 -1 is included with the next License Amendment request.

FIGURE 6.2-1
CORPORATE ORGANIZATION

*Indicates position having fire protection responsibility.



- o Operations Staff:
  - Shift Supervisor (SOL)
  - Licensed Operators (OL)
  - Non Licensed Operators
- o Maintenance:
  - Electrical
  - Mechanical
  - Instrumentation & Control
  - Building Services
- o Health Physics
- o Chemical Control
- o Waste Management
- o Compliance Reviews

- o Inservice Inspection
- o Reliability & Performance Reviews
- o Fire Protection
- o Technical Specification Coordination
- o Reactor Physics
- o Compliance Reviews
- o Nuclear Security
- o Safety Reviews
- o Plant Engineering:
  - Electrical
  - Mechanical
  - Instrumentation & Control
  - Structural
  - Computer

Legend: SOL - Senior Operator License
OL - Operator License
* - Fire Protection Responsibilities

The Facility Organization may be changed without prior License Amendment to Figure 6.2.2 provided that a revision to Figure 6.2.2 is included with the next License Amendment request.

Figure 6.2-2 FACILITY ORGANIZATION

6.3 FACILITY STAFF QUALIFICATIONS

6.3.1 Each member of the facility staff shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions, except for the Chemistry and Radiation Protection Superintendent who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975, and the Operations Technical Advisor, who shall have a Bachelor's degree, or the equivalent, in a scientific or engineering discipline with specific training in plant design and response and analysis of the plant for transients and accidents.

6.4 TRAINING

6.4.1 A retraining and replacement training program for the facility staff shall be maintained under the direction of the Director, Site Nuclear Operations and shall meet or exceed the requirements and recommendations of Section 5.5 of ANSI N18.1-1971 and Appendix "A" of 10 CFR Part 55.

6.4.2 A training program for the Fire Brigade shall be maintained under the direction of the Director, Site Nuclear Operations and shall meet or exceed the requirements of Section 27 of the NFPA Code-1976, except for Fire Brigade training sessions which shall be held at least quarterly.

6.5 REVIEW AND AUDIT

6.5.1 PLANT REVIEW COMMITTEE (PRC)

FUNCTION

6.5.1.1 The Plant Review Committee shall function to advise the Nuclear Plant Manager on all matters related to nuclear safety.

COMPOSITION

6.5.1.2 The Plant Review Committee shall be composed of the:

Chairman: Nuclear Operations Superintendent
Member: Assistant Nuclear Plant Engineer & Technical Services Manager
Member: Nuclear Maintenance Superintendent
Member: Nuclear Security and Special Projects Superintendent
Member: Nuclear Compliance Supervisor
Member: Nuclear Chem/Rad Protection Superintendent
Member: Nuclear Plant Engineering Superintendent
Member: Nuclear Safety & Reliability Superintendent
Member: At Large (Designated by Chairman)
Member: At Large (Designated by Chairman)

ALTERNATES

6.5.1.3 All alternate members shall be appointed in writing by the PRC Chairman to serve on a temporary basis; no more than two alternates shall participate as voting members in PRC activities at any one time.

MEETING FREQUENCY

6.5.1.4 The PRC shall meet at least once per calendar month and as convened by the PRC Chairman or his designated alternate.

QUORUM

6.5.1.5 A quorum of the PRC shall consist of the Chairman or his designated alternate and five members including alternates.

RESPONSIBILITIES

6.5.1.6 The Plant Review Committee shall be responsible for:

a. Review of 1) all procedures and changes thereto as required by Specification 6.8.2, 2) any other proposed procedures or changes thereto as determined by the Nuclear Plant Manager to affect nuclear safety.

b. Review of all proposed tests and experiments that affect nuclear safety.

c. Review of all proposed changes to the Appendix "A" Technical Specifications.

d. Review of all proposed changes or modifications to plant systems or equipment that affect nuclear safety, and changes to Radwaste Systems which could significantly alter their ability to meet Appendix I.

e. Investigation of all violations of the Technical Specifications including the review of reports covering evaluation and recommendations to prevent recurrence to the Vice President, Nuclear Operations and to the Chairman of the Nuclear General Review Committee.

f. Review of events requiring 24-hour written notification to the Commission.

g. Review of facility operations to detect potential nuclear safety hazards.

h. Performance of special reviews, investigations or analyses and reports thereon as requested by the Chairman of the Nuclear General Review Committee.

i. Review of the Plant Security Plan and implementing procedures.

j. Review of the Emergency Plan and implementing procedures.

COMPOSITION

6.5.2.2 The NGRC shall be composed of the Chairman, Vice Chairman, and at least 5 members. No more than a minority of the members shall have line responsibility for operation of the facility. The committee shall collectively have the experience and competence required to review problems in the following areas:

a. Nuclear power plant operations
b. Nuclear engineering
c. Chemistry and radiochemistry
d. Metallurgy
e. Nondestructive testing
f. Instrumentation and control
g. Radiological safety
h. Mechanical and electrical engineering
i. Administrative controls
j. Environmental
k. Quality assurance practices

QUALIFICATIONS

6.5.2.3 The following minimum experience requirements shall be established for those persons involved in the independent off-site safety review and audit programs:

a. Chairman and Vice Chairman-Bachelor of Science in engineering or related field or the equivalent and ten years related experience including five years involvement with operation and/or design of nuclear power plants.

b. Member-Bachelor of Science in engineering or related field or the equivalent and five years related experience including three years involvement with operation and/or design of nuclear power plants.

ALTERNATES

6.5.2.4 All alternate members shall be appointed in writing by the NGRC Chairman to serve on a temporary basis; however, no more than two alternates shall participate as voting members in NGRC activities at any one time.

CONSULTANTS

6.5.2.5 Consultants shall be utilized as determined by the NGRC Chairman to provide expert advice to the NGRC.

MEETING FREQUENCY

6.5.2.6 The NGRC shall meet at least once per calendar quarter during the initial year of facility operation following fuel loading and at least once per six months thereafter.

QUORUM

6.5.2.7 A quorum of NGRC shall consist of the Chairman or his designated alternate and five additional NGRC members, including alternates. No more than a minority of the quorum shall have line responsibility for operation of the facility.

REVIEW

6.5.2.8 The NGRC shall review:

a. Safety evaluations of changes to the facility, changes in procedures and tests and experiments not described in the Final Safety Analysis Report which were completed without prior NRC approval.

b. Proposed changes in the facility, changes in procedure or proposed tests or experiments which may involve an unreviewed safety question.

c. Proposed changes in the Technical Specifications or license amendments relating to nuclear safety prior to implementation, except in those cases where the change is identical to a previously reviewed proposed change.

d. Violations, deviations and reportable events, which require reporting to the Nuclear Regulatory Commission in writing within 24 hours. This review should include results of investigations and recommendations resulting from such investigations.

e. Any other matter involving safe operation of the plant which an independent reviewer deems appropriate for consideration, or which is referred to the reviewers by members of Nuclear Operations staff or by other functional organizational units within Florida Power Corporation.

f. Reports and meeting minutes of the Plant Review Committee.

g. Changes to the "Process Control Program" and to the Offsite Dose Calculation Manual.

AUDITS

6.5.2.9 Audits of facility activities shall be performed under the cognizance of the NGRC. These audits shall encompass:

a. The conformance of facility operation to provisions contained within the Technical Specifications and applicable license conditions at least once per 12 months.

b. The performance, training and qualifications of the entire facility staff at least once per 12 months.

c. The results of actions taken to correct deficiencies occurring in facility equipment, structures, systems or method of operation that affect nuclear safety at least once per 6 months.

d. The performance of activities required by the Operational Quality Assurance Program to meet the criteria of Appendix "B", 10 CFR 50, at least once per 24 months.

e. The Facility Emergency Plan and implementing procedures at least once per 12 months.

f. The Facility Security Plan and implementing procedures at least once per 12 months.

g. The Facility Fire Protection Program and implementing procedures at least once per 12 months.

h. The Radiological Environmental Monitoring Program and the results thereof at least once per 12 months.

i. The Offsite Dose Calculation Manual and implementing procedures at least once per 24 months.

j. The Process Control Program and implementing procedures for solidification of radioactive wastes at least once per 24 months.

k. The performance of activities required by the Quality Assurance Program for effluent and environmental monitoring at least once per 12 months.

l. Any other area of facility operation considered appropriate by the NGRC or the Executive Vice President.

AUTHORITY

6.5.2.10 The NGRC shall report to and advise the Executive Vice President on those areas of responsibility specified in Sections 6.5.2.8 and 6.5.2.9.

RECORDS

6.5.2.11 Records of NGRC activities shall be prepared, approved and distributed as indicated below:

a. Minutes of each NGRC meeting shall be prepared, approved and forwarded to the Executive Vice President within 14 days following each meeting. |

b. Reports of reviews encompassed by Section 6.5.2.8 above, shall be prepared, approved and forwarded to the Executive Vice President within 14 days following completion of the review. |

c. Audit reports encompassed by Section 6.5.2.9 above, shall be forwarded to the Executive Vice President and to the management positions responsible for the areas audited within 30 days after completion of the audit. |

6.6 REPORTABLE OCCURRENCE ACTION

6.6.1 The following actions shall be taken for REPORTABLE OCCURRENCES:

a. The Commission shall be notified and/or a report submitted pursuant to the requirements of Specification 6.9.

b. Each REPORTABLE OCCURRENCE requiring 24-hour notification to the Commission shall be reviewed by the PRC and submitted to the NGRC and the Vice President, Nuclear Operations.

6.7 SAFETY LIMIT VIOLATION

6.7.1 The following actions shall be taken in the event a Safety Limit is violated:

a. The facility shall be placed in at least HOT STANDBY within one hour.

b. The Safety Limit violation shall be reported to the Commission, the Vice President, Nuclear Operations and to the NGRC within 24 hours.

c. A Safety Limit Violation Report shall be prepared. The report shall be reviewed by the PRC. This report shall describe (1) applicable circumstances preceding the violation, (2) effects of the violation upon facility components, systems or structures and (3) corrective action taken to prevent recurrence.

d. The Safety Limit Violation Report shall be submitted to the Commission, the NGRC and the Vice President, Nuclear Operations within 14 days of the violation.

6.8 PROCEDURES

6.8.1 SCOPE

Written procedures shall be established, implemented and maintained covering the activities referenced below:

a. The applicable procedures recommended in Appendix "A" of Regulatory Guide 1.33, November, 1972.

b. Refueling operations.

c. Surveillance and test activities of safety related equipment.

d. Security Plan implementation.

e. Emergency Plan implementation.

f. Fire Protection Program implementation.

g. Systems Integrity Program implementation.

h. Iodine Monitoring Program implementation.

i. PROCESS CONTROL PROGRAM implementation.

j. OFF-SITE DOSE CALCULATION MANUAL implementation.

k. Quality Assurance Program for effluent and environmental monitoring.

6.8.2 REVIEW PROCESS

6.8.2.1 Each procedure and administrative policy of 6.8.1 above, and changes thereto, shall be reviewed and approved prior to implementation as follows:

a. The Emergency Plan, Security Plan, Fire Protection Plan and implementing procedures, Administrative Instructions, and those test procedures associated with safety related plant modifications shall be reviewed by the PRC and approved by the Nuclear Plant Manager prior to implementation.

b. For all other procedures, the review cycle shall consist of: an intradepartmental review by a Qualified Reviewer, an interdisciplinary review by Qualified Reviewer(s) in interfacing departments, as specified in Administrative Procedures, and approval by the responsible Superintendent or Manager, as specified by Administrative Procedures. The PRC shall then review the 10 CFR 50.59 evaluation within 14 days of approval.

6.8.2.2 The training and qualification of Qualified Reviewers shall be governed by Administrative Procedures, with final certification by the Nuclear Plant Manager. Recertification will be required on a periodic basis and upon transfer between departments. As a minimum, all Qualified Reviewers shall meet the requirements of ANSI N18.1-1971, Sections 4.2, 4.3, 4.4, or 4.6, or the equivalent.

6.8.2.3 Each procedure and administrative policy of 6.8.1 shall be reviewed on a periodic basis as set forth in Administrative Procedures.

MONTHLY OPERATING REPORT

6.9.1.6 Routine reports of operating statistics and shutdown experience shall be submitted on a monthly basis to the Director, Office of Inspection and Enforcement, U.S. Nuclear Regulatory Commission, Washington, D.C. 20555, with a copy to the Regional Office, no later than the 15th of each month following the calendar month covered by the report.

REPORTABLE OCCURRENCES

6.9.1.7 The REPORTABLE OCCURRENCES of Specifications 6.9.1.8 and 6.9.1.9 below, including corrective actions and measures to prevent recurrence, shall be reported to the NRC. Supplemental reports may be required to fully describe final resolution of occurrence. In case of corrected or supplemental reports, a licensee event report shall be completed and reference shall be made to the original report date.

PROMPT NOTIFICATION WITH WRITTEN FOLLOW-UP

6.9.1.8 The types of events listed below shall be reported within 24 hours by telephone and confirmed by telegraph, mailgram, or facsimile transmission to the Director of the Regional Office, or his designate, no later than the first working day following the event, with a written follow-up report within 14 days. The written follow-up report shall include, as a minimum, a completed copy of a licensee event report form. Information provided on the licensee event report form shall be supplemented, as needed, by additional narrative material to provide complete explanation of the circumstances surrounding the event.

a. Failure of the reactor protection system or other systems subject to limiting safety-system settings to initiate the required protective function by the time a monitored parameter reaches the setpoint specified as the limiting safety-system setting in the technical specifications or failure to complete the required protective function.

b. Operation of the unit or affected systems when any parameter or operation subject to a limiting condition for operation is less conservative than the least conservative aspect of the limiting condition for operation established in the techical specifications.

c. Abnormal degradation discovered in fuel cladding, reactor coolant pressure boundary, or primary containment.