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NRC Form 368 (9-83)

NRC Form 366A					U.S. 1	NUCLEAR RE	GULATO	RY COMMI	SSION		
(9.63)	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION						APPROVED OM8 NO. 3150-0104 EXPIRES 8/31/85				
FACILITY NAME (1)		COCKET NUMBER	(2)	LER NUM				PAGE (3)			
Susquehanna Stea Unit 2	m Electric Stat	ion	YEAR	SEQUE	NTIAL BER	RE ISION					

TEXT III more spece is required, use additional NRC Form (364's) (17)

On September 18, 1984, at 1150, the High Pressure Coolant Injection (HPCI) system was removed from service to change its lube oil. Entry into Limiting Condition for Operation (LCO) 3.5.1 was duly noted by Operations personnel. The remaining Emergency Core Cooling Systems (Core Spray, Low Pressure Coolant Injection and Automatic Depressurization) were operable, as was the Reactor Core Isolation Cooling (RCIC) system. Thus, per the LCO Action statement, HPCI could remain inoperable for fourteen (14) days before the LCO would affect plant operation.

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Upon completion of the lube oil changeout on September 19, 1984, the HPCI system was to be run to confirm its proper operation prior to clearing the LCO. Per the procedure being used to run HPCI, the Residual Heat Removal (RHR) system is to be placed in service in its suppression pool cooling mode prior to the HPCI start. (The RHR system can be lined up for any of several modes, one of which is Low Pressure Coolant Injection, an ECCS.) At 1650, the RHR pump 'D' failed to start upon a manual actuation signal. Operations personnel immediately recognized the circumstances as requiring entry into LCO 3.0.3 because the provisions of LCO 3.5.1 could no longer be met with the RHR pump inoperable. LCO 3.0.3 specifies, in effect, that when an LCO is not met except as provided in the ACTION statement, the Unit shall, within one hour, begin shutting down.

Operations personnel racked the pump's switch gear breaker out, then back in. Another attempt to start the pump was made, which was successful. LCO 3.0.3 was cleared at 1705, HPCI was run successfully and LCO 3.5.1 was cleared at 1812. Immediate investigation of the RHR pump 'D' breaker found nothing abnormal.

On October 4, 1984, there was another occurrence of RHR pump 'D' failing to start on a manual actuation signal. This time, an investigation was able to be performed before the breaker was distrubed. It was found that the '52LS' limit switch contacts were functioning intermittently. A spare breaker was functionally tested and installed to replace the breaker with the faulty limit switch. RHR pump 'D' was then test run satisfactorily.



Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

October 18, 1984

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 84-019-00 ER 100450 FILE 841-23 PLA-2337

Docket No. 50-388 License No. NPF-22

Attached is Licensee Event Report 84-019-00. This event was determined reportable per 10CFR50.73(a)(2)(i), in that with the High Pressure Coolant Injection system out of service, one Residual Heat Removal system pump was declared inoperable and the Unit entered Limiting Condition for Operation 3.0.3.

Mionypron for

H.W. Keiser Superintendent of Plant-Susquehanna

LAK/pjg

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Mr. R.H. Jacobs Senior Resident Inspector U.S. Nuclear Regulatory Commission P.O. Box 52 Shickshinny, PA 18655

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