NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
North Anna Power Station, Unit 1	0 5 0 0 0 3 3 8	1 OF 0 2

TITLE (4)
Pressurizer PORV Opening While In Cold Shutdown

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 9	1 4	8 4	8 4	0 1 1	0 0	1 0	1 1	8 4		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
5	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 1 0 0	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	Special Report
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
E. Wayne Harrell	AREA CODE 7 0 3 8 9 4 1 5 1 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

On September 14, 1984, a Unit 1 pressurizer Power Operated Relief Valve (PORV) automatically opened three times during a pressure transient. Preparations were being made to return the unit to operation at the end of a refueling outage. The Reactor Coolant System (RCS) was solid. The overpressurization occurred when attempts were made to stabilize RCS pressure after a reactor coolant pump was started. The PORV opened at its low pressure, low temperature setpoint as required by Tech. Specs. 3.4.9.3. The PORV opening and operator action decreased pressure immediately after the valve opened. This event is reportable pursuant to Tech. Spec. 6.9.2.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

On September 14, 1984, Unit 1 was in cold shutdown, Mode 5, and final filling and venting of the reactor coolant system (RCS) was being performed as preparations were made to return to power after a refueling outage. RCS temperature was 88°F and pressure was 350 psig. The pressurizer was solid (100% level) and both PORV's were in automatic control. One residual heat removal pump and no reactor coolant pumps (RCP) were running. When loop "A" reactor coolant pump (RCP) was started, air trapped in the RCS caused pressure to rapidly decrease. Subsequent pressure adjustments made by the Control Room operator caused a pressurizer PORV, PCV-1455C, to open momentarily three times during the resultant pressure transient. The operator was attempting to maintain pressure high enough to have a sufficient pressure drop across the RCP seals, but low enough to prevent PORV actuation.

The PORV opening and operator action immediately decreased the RCS pressure each time the overpressurization occurred. The PORV was open for a total of less than thirty seconds during the five minute pressure transient which took place before pressure was stabilized at 366 psig. The maximum pressure during this event was approximately 410 psig and the minimum was approximately 317 psig.

The low temperature, low pressure setpoint for PCV-1455C is 410 psig when RCS temperature is less than 140 °F. The redundant PORV, PCV-1456, will open at 425 psig; however, pressure was never high enough for it to open. These setpoints are required by Technical Specification 3.4.9.3 and are within 10CFR50 Appendix G guidelines; therefore, the health and safety of the general public were not affected.

In order to minimize the probability of reoccurrence, a discussion of this event will be included as part of the licensed operator retraining. Unit 2 LER 82-024/03 is similar. This event is reportable pursuant to Technical Specification 6.9.2.

Vepco

VIRGINIA ELECTRIC AND POWER COMPANY
NORTH ANNA POWER STATION
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October 11, 1984

U. S. Nuclear Regulatory Commission
Document Control Desk
016 Phillips Building
Washington, D.C. 20555

Serial No. N-84-019
NO/RST: 11
Docket No. 50-338

License No. NPF-4

Dear Sirs:

The Virginia Electric and Power Company hereby submits the following License Event Report applicable to North Anna Unit No. 1.

Report No. LER 84-011/03L-0

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to Safety Evaluation and Control for their review.

Very Truly Yours,

E. Wayne Harrell
Station Manager

Enclosures (3 copies)

cc: Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
Region II
101 Marietta Street, Suite 2900
Atlanta, Georgia 30303

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