U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85 LICENSEE EVENT REPORT (LER) DOCKET NUMBER (2) FACILITY NAME (1) 1 OF 0 12 0 |5 | 0 | 0 | 0 | 2 | 8 | 2 Prairie Island Unit 1 TITLE (A) Inadvertent automatic start of Dl Diesel Generator EVENT DATE (5) LER NUMBER (6) REPORT DATE (7) OTHER FACILITIES INVOLVED (8) SEQUENTIAL NUMBER FACILITY NAMES MONTH DAY YEAR YEAR MONTH DAY Prairie Island Unit 2 0 | 5 | 0 | 0 | 0 | 3 | 0 | 6 0 9 0 5 8 4 8 4 00100584 007 0 1510 10 101 THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR S: (Check one or more of the following) (11) OPERATING MODE (9) N 73,71(6) 50,73(a)(2)(iv) 20.406(a)(1)(i) 50.38(e)(1) 50.73(a)(2)(v) 73,71(e) 01916 50.73(a)(2)(vii) 20.406(a)(1)(ii) 50.38(a)(2) OTHER /S 20.405(a)(1)(iii) 50,73(a)(2)(I) 50.73(a)(2)(viii)(A) 20.405(a)(1)(iv) 50.73(a)(2)(viii)(8) 50.73(a)(2)(ii) 20.405(a)(1)(v) 50.73(a)(2)(iii) 50.73(a)(2)(x) LICENSEE CONTACT FOR THIS LER (12) NAME TELEPHONE NUMBER AREA CODE Arne A Hunstad, Staff Engineer 61112 3 | 8 | 8 | - | 1 | 1 | 2 | 1 COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) REPORTABLE TO NPROS MANUFAC-SYSTEM COMPONENT TO NPROS CAUSE CAUSE SYSTEM COMPONENT SUPPLEMENTAL REPORT EXPECTED (14) MONTH DAY YEAR EXPECTED SUBMISSION DATE (15) YES III yes, complete EXPECTED SUBMISSION DATE! X NO

During performance of an isolation procedure for testing, a procedural inadequacy caused an inadvertent automatic start of a diesel generator. The procedure has been revised.

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ABSTRACT (Limit to 1400 spaces, I s., approximatory fifteen single-space typewritten lines (16)

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 3/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)						Г	LER NUMBER (6)									PAGE (3)				
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Prairie Island Unit 1	0	5	10	10	0	2	8	2	8	14	-	01	01	7	_	010	0   2	2	OF	0 12	2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On September 5, 1984 Unit 1 was at 96% power and Unit 2 was shutdown for refueling. At about 2200 hours the Unit 2 Train A DC loads were transferred to their alternate source, the Unit 1 Train A battery (EJ), to prepare for isolation and testing of Unit 2's No. 21 Battery. The specific switches (HS) that were transferred supply DC power to Safeguards Rack (RK) 2ASG1 and 4KV Safeguards Bus (BU) 26. Following this work, at about 2300 hours, a Control Room Operator observed that D1 Diesel Generator (EK)(DG) had started and was running unloaded. A review of the computer logs revealed no entry concerning the diesel generator and no reason for it to start.

The cause of this event was procedural inadequacy. During the transfer of DC power for Bus 26, the voltage restoration scheme sensed an apparent loss of voltage on Bus 26 even though the undervoltage relays (27) were still energized. This was due to drop-out of the undervoltage slave relays (RLY) which are powered by the DC supply to the Bus. This loss of voltage signal did not initiate the voltage restoration scheme nor trip any breakers (BKR) since DC power was not available; however, it did start Dl Diesel Generator without Control Room annunciation.

The procedure for use of the transfer switches was revised to require placing the voltage restoration switch in manual before operating the DC transfer switch.

This event is not repetitive. Public health and safety were unaffected.





## Northern States Power Company

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October 5, 1984

U S Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

## Inadvertent Automatic Start of Dl Diesel Generator

The License Event Report for this occurrence is attached.

This event was reported via Emergency Notification System per 10 CFR Part 72 on September 6, 1984.

for David Musolf
Manager - Nuclear Support Services

DMM/EFE/dab

c: Regional Administrator-III, NRC NRR Project Manager, NRC Resident Inspector, NRC MPCA Attn: J W Ferman

Attachment

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