NRG F RAM 366 **U. S. NUCLEAR REGULATORY COMMISSION** {7.77) LICENSEE EVENT REPORT CONTROL BLOCK: (1)(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) (4) 0 1 OHD Ø Ø Ø (3)4 B Ø 00 LICENSE NUMBER CON'T 14 16 (7) REPORT |L (6) 2 8 0 0 1 015 10 10 10 13 1 1 1 3 2 2 3 SOURCE 68 EVENT DATE DOCKET NUMBER 61 EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) (NP-32-82-01) On 1/13/82 it was determined that the Borated Water Storage Tank (BWST) | 0 2 temperature was higher than the initial conditions assumed in the Davis-Besse Unit 1 03 Final Safety Analysis Report (FSAR). The FSAR assumed an initial condition of 90°F 0 4 injection water during loss of coolant accidents. The slight reduction in cooling 0 5 capacity was compensated by reducing power to approximately 90%. There was no danger 0 6 to the health and safety of the public or station personnel. 0 7 SYSTEM CAUSE CAUSE COMP VALVE CODE COMPONENT CODE SUBCODE Z (13) B D M U Z (15 (16 SEQUENTIAL REPORT NO. OCCURRENCE REPOR REVISION EVENT YEAR CODE TYPE NO LER/RO REPORT 8 21 11 Ø 101 Ø X 2 NUMBER 32 31 NPRD-4 FORM SUB PRIME COMP COMPONENT TAKEN EFFECT ON PLANT SHUTDOWN METHOD SUBMITTED ACTION (22) HOURS Ø Y 11 Ø Ø Ø N (24) 13 A 25 Ø CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Station procedures only provided minimum temperature restrictions and did not specify 1 0 1 1 1 the maximum temperature initial condition listed in Section 6.2.1.3.2 of the FSAR Procedure modifications are completed to provide the high temperature restructions In addition, Toledo Edison analyzed the event and determined that insulation on the 4 BWST would assist in maintaining proper temperatures. Insulation has been added. 80 METHOD OF FACILITY (30) OTHER STATUS % POWER DISCOVERY DESCRIPTION (32) E (28) ØI Ø NA C 5 (31) Operator Observation 9 10 ACTIVITY CONTENT 80 AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36) RELEASED. OF RELEASE Z (33) Z (34) NA NA 44 11 80 PERSONNEL EXPOSURES DESCRIPTION (39) NUMBER TYPE Z (38) 0 0 (37) NA 80 PERSONNEL INJURIES DESCRIPTION (41) NUMBER 00 Ø NA (40)80 LOSS OF OR DAMAGE TO FACILITY (43) TYPE DESCRIPTION Z (42) NA 8410180506 841011 ADOCK 05000346 PUBLICITY PDR PDR NRC USE ONLY DESCRIPTION (45 SUED 5 N (44) NA 68 69 PHONE (419)-259-5000 ext. DVR 82-007 Stan N. BAtch 183 NAME OF PREPARER

TOLEDO EDISON COMPANY DAVIS-BESSE NUCLEAR POWER STATION UNIT ONE SUPPLEMENTAL INFORMATION FOR LER NP-32-82-01

DATE OF EVENT: January 13, 1982

FACILITY: Davis-Besse Unit 1

IDENTITICATION OF OCCURRENCE: Borated Water Storage Tank (BWST) Temperature Elevated

Conditions Prior to Occurrence: The unit was in Mode 1 with Power (MWT) = 2770 and Load (Gross MWE) = 920.

Description of Occurrence: On January 13, 1982 at 1100 hours, it was determined that the BWST water temperature was higher than the initial conditions assumed in the Davis-Besse Unit 1 Final Safety Analysis Report (FSAR). In Section 6.2.1.3.2, "Containment Pressure Transient Analysis Break Spectrum" of the FSAR, an initial condition of 90°F BWST temperature was assumed in the analysis to determine maximum containment pressure over various loss of coolant accident break sizes. Since the BWST temperature was indicating approximately 115°F from an extended operation of a heat exchanger used to maintain minimum temperature, it was possible that operation was less conservative than the accident analyses in the FSAR. This required a prompt notification with written followup per Technical Specification 6.9.1.8.1.

Designation of Apparent Cause of Occurrence: The cause of this occurrence was a lack of administrative restrictions on maximum BWST temperature. Station procedures only provided minimum temperature restrictions and did not specify the maximum temperature initial condition listed in Section 6.2.1.3.2 of the FSAR.

<u>Analysis of Occurrence</u>: There was no danger to the health and safety of the public or station personnel. A safety analysis was performed under Facility Change Request 82-006 which determined the consequences of increasing the BWST temperature from 90°F to 120°F while operating at 90% of design power is less limiting than the normal design conditions utilized.

On February 7, 1982, an Emergency Core Cooling System (ECCS) study on operation at 100% power with a 120°F BWST was completed which determined:

- In light of the low core local operating peaks that were present when the plant was in this configuration, the present large break LOCA analyses would bound the resultant fuel peak cladding temperatures.
- (2) For small break LOCA evaluations, the higher BWST temperature would result in a decreased vessel inventory. However, the resultant peak cladding temperatures would still have substantial margin to the 2200°F peak cladding temperature criteria of 10CFR50.46.

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(3) Mass and energy releases to the containment could increase by 1.7% due to the higher BWST water temperature, but the resultant increase in peak building pressure would be less than 0.6 psi.

<u>Corrective Action</u>: In order to ensure operation in a conservative manner, an estimate of the reduction of the required heat removal capacity for both containment and core loss of coolant accident considerations resulted in a determination that a power reduction to approximately 94% of full power would compensate for the temporarily increased injection water temperature. To add additional margin, it was decided to reduce power to approximately 90% of full power which was completed at approximately 1830 hours on January 13, 1982.

Procedure modifications to ST 5099.02, "Miscellaneous Instrument Daily Check" and SP 1104.66, "BWST Operating Procedure" were completed to provide a high temperature restriction to the BWST in addition to the low temperature restrictions which exist in Technical Specifications 3.1.2.8 and 3.5.4.

At approximately 0230 hours on January 14, 1982, BWST temperature decreased below 90°F. The reactor was restored to full power at approximately 0500 hours on January 14, 1982.

During the review of this incident, it was determined the BWST temperature had been elevated above 90°F when the unit was at full power several times prior to this event. These incidents were also caused by a lack of maximum temperature restrictions and were studied to increase the scope of the available data.

Previously during extreme cold weather conditions, heat loss from the BWST required maximum utilization of the tank steam heating system. Because of stratification it was difficult for the operator to obtain a representative tank temperature when using the BWST heat exchange system. Toledo Edison analyzed the event and determined that insulation added to the BWST would allow a reduced heater return line temperature and reduce temperature stratification internal to the tank. Insulation has now been added to the BWST.

Failure Data: There have been no previously reported similar occurrences since the high temperature restriciton was not recognized.

LER#82-001

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October 11, 1984

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Log No. K84-1285 File: RR 2 (NP-32-82-01) Rev. 2

Docket No. 50-346 License No. NPF-3

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Gentlemen:

LER No. NP-82-01, Rev.2 Davis-Besse Nuclear Power Station Unit 1 Date of Occurrence: January 13, 1982

Enclosed is Licensee Event Report 82-01 Rev. 2, which is being submitted in accordance with 10CFR50.73, to provide 30 day written notification of the subject occurrence.

Yours truly,

Stephen morennos

Stephen M. Quennoz Plant Manager Davis-Besse Nuclear Power Station

SMQ/bec

Enclosure

cc: Mr. James G. Keppler, Regional Administrator, USNRC Region III

> Mr. Walt Rogers DB-1 NRC Resident Inspector

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