

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Virgil C. Summer Nuclear Station

DOCKET NUMBER (2)
0 5 0 0 0 3 1 9 5

PAGE (3)
1 OF 0 1 3

TITLE (4)
PORV Position Indication

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)							
0	9	0	8	4	8	4	0	3	8	0	0	1	0	5	0	0	0
0	9	0	7	8	4	8	4	0	3	8	0	0	1	0	5	8	4

OPERATING MODE (9) 1

POWER LEVEL (10) 0 8 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(a)	<input type="checkbox"/> 80.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 80.38(a)(1)	<input type="checkbox"/> 80.73(a)(2)(v)	<input type="checkbox"/> 73.71(a)
<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 80.38(a)(2)	<input type="checkbox"/> 80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 308A)
<input type="checkbox"/> 20.406(a)(1)(iii)	<input checked="" type="checkbox"/> 80.73(a)(2)(i)	<input type="checkbox"/> 80.73(a)(2)(vii)(A)	
<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 80.73(a)(2)(ii)	<input type="checkbox"/> 80.73(a)(2)(vii)(B)	
<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 80.73(a)(2)(iii)	<input type="checkbox"/> 80.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: A. R. Koon, Jr., Assoc. Mgr., Regulatory Compliance

TELEPHONE NUMBER: 8 0 3 3 4 5 - 5 2 0 9

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
D	I	P		N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 7, 1984, a management review of Technical Specification 3.3.3.6, "Accident Monitoring Instrumentation," identified procedural inadequacies in Surveillance Test Procedure (STP) 135.001 and STP 506.010. The procedures only verified the operability of one (1) Pressurizer Power Operated Relief Valve (PORV) position indication channel versus the two (2) channels required by Technical Specifications. The second position indication channel for the PORV's was declared inoperable until the Licensee could reestablish Technical Specification compliance.

A temperature detector in the downstream piping of the PORV's was determined to be the second position indication channel. The channel was calibrated and channel checked in accordance with Surveillance Requirement 4.3.3.6 prior to declaring the PORV position indication channel operable at 2000 hours on September 26, 1984.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Virgil C. Summer Nuclear Station	DOCKET NUMBER (2) 0 5 0 0 3 9 5 8 4	LER NUMBER (8)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	0 2	OF 0 3
		— 0 3 8	— 0 0			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On September 7, 1984, a management review of Technical Specification 3.3.3.6, "Accident Monitoring Instrumentation," identified procedural inadequacies in Surveillance Test Procedures (STP) 135.001, "Post Accident Monitoring Instruments Channel Check," and STP 506.010, "PORV Position Indicator Calibration." The Technical Specification requires operability verification of two (2) Pressurizer Power Operated Relief Valve (PORV) position indication channels per valve by performance of an 18 month calibration and a 31 day channel check. Contrary to the above, the STP's only verified the operability of one (1) position indication channel per valve. Valve limit switches, which provide main control board (MCB) position indication, comprise the single channel, the operability of which has been verified in accordance with Surveillance Requirement 4.3.3.6.

The Licensee was unable to immediately determine what specific plant equipment should be used to satisfy the Technical Specification requirement for a second channel of PORV position indication. Therefore, a decision was made to voluntarily enter Action Statement "a" of Technical Specification 3.4.4, "Relief Valves," removing the requirement for PORV position indication under Technical Specification 3.3.3.6, until a determination could be made. The PORV's were declared inoperable and the associated block valves either closed or verified to be closed in accordance with the Action Statement at 1639 hours on September 7, 1984.

A subsequent review of system design indicated that the most appropriate instrumentation for use as the second PORV position indication channel is a temperature detector in the piping downstream of the valves. An increase in discharge line temperature would be an indication of leakage through the valves. This issue was discussed with the Resident NRC Inspector and the combination of valve limit switches and the temperature channel was determined to be acceptable in accordance with the requirements of Technical Specification 3.3.3.6 and Regulatory Guide 1.97.

The PORV position indication channel was declared operable per requirements of Technical Specification 3.3.3.6 at 2000 hours on September 26, 1984. The following tests were performed prior to declaring the position indication channel operable.

- 1) A satisfactory channel calibration was performed with Instrument Control Procedure (ICP) 340.012, "Pressurizer Power Relief Line Temperature ITE-463," on September 18, 1984. (The channel had previously been calibrated on August 12, 1982, under the Plant preventive maintenance program.)

NOTE: ICP-340.012 will be incorporated into the Surveillance Testing Program by December 30, 1984.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Virgil C. Summer Nuclear Station	DOCKET NUMBER (2) 0 5 0 0 0 3 9 5 8 4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	- 0 3 8	- 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

- 2) A channel check of the temperature indication was added to STP 135.001 and the Operator at the Controls (Mode 1-4) Technical Specification Log on September 26, 1984.

There were no adverse consequences from this event and the appropriate PORV block valves were opened. Prior to discovery of the procedural errors, the temperature channel had been monitored on a daily basis while in Modes 1-4 by Operations personnel on "Trend Logs." The routine monitoring of the channel provides assurance that PORV leakage would have been detected in a timely manner.

SOUTH CAROLINA ELECTRIC & GAS COMPANY

POST OFFICE 764

COLUMBIA, SOUTH CAROLINA 29218

October 5, 1984

O. W. DIXON, JR.
VICE PRESIDENT
NUCLEAR OPERATIONS

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

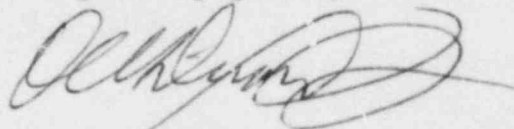
SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
LER 84-038

Dear Sir:

Please find attached Licensee Event Report #84-038 for the Virgil C. Summer Nuclear Station. This Report is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(i).

Should there be any questions, please call us at your convenience.

Very truly yours,



O. W. Dixon, Jr.

CJM:OWD/lcd
Attachment

cc: V. C. Summer	J. F. Heilman
T. C. Nichols, Jr./O. W. Dixon, Jr.	C. L. Ligon (NSRC)
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