NRC Form 366 (9-63)											LICENSEE EVENT REPORT (LER)									U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/86						
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During normal operation, LPCI/CCSW heat exchanger delta P valve MO 3-1501-3A was taken out-of-service for limit switch adjustments, rendering 'A' LPCI/CCSW loop inoperable. No ECCS surveillances were completed prior to rendering the 'A' loop inoperable (Tech. Spec. 4.5.8.3). DOS 1500-3 (Containment Cooling Pump Test), was immediately initiated to verify the operability of the 'B' LPCI/CCSW loop. The valve was returned to service after the limit switches were adjusted. Safety significance was minimal since the 'B' LPCI/CCSW loop was found to be operable and capable of providing core cooling. Cause of the event was due to personnel error. Operator personnel was instructed in the prudent policy of performing required surveillances before taking safety equipment out-of-service.

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YES (If yes, complete EXPECTED SUBMISSION DATE)
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SUPPLEMENTAL REPORT EXPECTED (14)

MONTH

EXPECTED SUBMISSION DATE (15) DAY

YEAR

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85

PACILITY NAME (1)	DOCKET NUMBER (2)			PAGE (3)					
Dresden Nuclear Power Station		YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	7	T		
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TEXT (If more space is required, use additional NRC Form 395A's) (17)

During normal operation, LPCI/CCSW heat exchanger delta P valve MO 3-1501-3A was taken out-of-service for limit switch adjustments, rendering the 'A' LPCI/CCSW loop inoperable. No ECCS surveillances were completed prior to rendering the 'A' loop inoperable, violating Technical Specification 4.5.8.3. DOS 1500-3 (Containment Cooling Pump Test) was immediately initiated on the 'B' LPCI/CCSW loop. The '3A' valve limits were adjusted and the valve was returned to service. The total time the valve was out-of-service was an hour and a half. Safety significance was minimal since the LPCI/CCSW 'B' loop was found to be operable and capable of providing core cooling. Cause of the event was personnel error. The Operating personnel was instructed in the prudent policy of performing required surveillances before taking safety equipment out-of-service. First reported occurrence of this type.



October 4, 1984

DJS Ltr # 84-1111

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Licensee Event Report #84-014-00, Docket # 050249, is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73(a)(2)(i)(B).

D.J. Scott

Station Superintendent Dresden Nuclear Power Station

DJS/aw

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical