

Virginia Electric and Power Company
North Anna Power Station
P. O. Box 402
Mineral, Virginia 23117

October 12, 1995

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

NAPS: GSS
Docket No. 50-339
License No. NPF-7

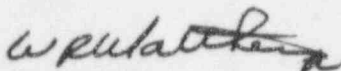
Dear Sirs:

Pursuant to North Anna Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following supplemental Licensee Event Report applicable to North Anna Unit 2.

Report No. 50-339/95-001-01

This Report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to the Management Safety Review Committee for its review.

Very truly yours,



J. A. Stall
Station Manager

Enclosure:

cc: U.S. Nuclear Regulatory Commission
101 Marietta Street, N.W.
Suite 2900
Atlanta, Georgia 30323

R. D. McWhorter
NRC Senior Resident Inspector
North Anna Power Station

JE221

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HOURS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, DC 20503.

FACILITY NAME (1) North Anna Power Station Unit 2	DOCKET NUMBER (2) 05000339	PAGE (3) 1 OF 4
--	-------------------------------	--------------------

TITLE (4)
MAIN STEAM AND PRESSURIZER SAFETY VALVE SETPOINTS OUT OF TOLERANCE DUE TO SETPOINT DRIFT

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
04	03	95	95	001	01	10	12	95	FACILITY NAME	05000
									FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9) 6	POWER LEVEL (10) 0	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)								
		20.402(b)			20.405(c)			50.73(a)(2)(iv)		73.71(B)
		20.405(a)(1)(i)			50.36(c)(1)			50.73(a)(2)(v)		73.71(C)
		20.405(a)(1)(ii)			50.36(c)(2)			50.73(a)(2)(vii)		OTHER
		20.405(a)(1)(iii)			X 50.73(a)(2)(i) (B)			50.73(a)(2)(viii)(A)		(Specify in Abstract below and in Text, NRC Form 166A)
20.405(a)(1)(iv)			50.73(a)(2)(ii)			50.73(a)(2)(viii)(B)				
20.405(a)(1)(v)			50.73(a)(2)(iii)			50.73(a)(2)(x)				

LICENSEE CONTACT FOR THIS LER (12)

NAME Mr. J. A. Stall	TELEPHONE NUMBER (Include Area Code) (540) 894-2101
-------------------------	--

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	AB	RV	D243	Y					
X	SB	RV	C710	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, completed EXPECTED SUBMISSION DATE)	X	NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
---	---	----	-------------------------------	-------	-----	------

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On April 3, 1995, with Unit 2 in Mode 6 (Refueling), the "as found" set pressure for one Main Steam Safety Valve was found to be outside the setpoint tolerance allowed by Technical Specifications (TS). During subsequent testing, the "as found" set pressure for another Main Steam Safety Valve and two Pressurizer Safety Valves were also found to be outside the TS setpoint tolerance. These events are reportable pursuant to 10 CFR 50.73 (a) (2) (i) (B) for conditions prohibited by Technical Specifications.

The safety valves had been sent to Wyle Labs for testing to ensure conformance to Technical Specifications requirements. The "as found" set pressures for two Pressurizer Safety Valves and two Main Steam Safety Valves were found to be outside Technical Specifications tolerances. The Main Steam Safety Valves were repaired, as necessary, and retested to be within the setpoint limit tolerances allowed by Technical Specifications. The Pressurizer Safety Valves were subsequently stroked within Technical Specifications limits without any repairs being performed.

This event posed no significant safety implications because the safety valves would have performed its safety function in the event of an overpressure condition. The health and safety of the public were not affected by this event.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HOURS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
North Anna Power Station Unit 2	05000339	YEAR 95	SEQUENTIAL NUMBER 001	REVISION NUMBER 01	2 OF 4

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

1.0 Description of the Event

Fifteen Main Steam Safety Valves (EISS System SB, Component RV, Vendor Identifier C710) and three Pressurizer Safety Valves (EISS System AB, Component RV, Vendor Identifier D243) were sent to Wyle Laboratories for the performance of periodic test procedures 2-PT-70, "Main Steam Code Safety Valve Setpoint Verification" and 2-PT-50, "Pressurizer Code Safety Valve Setpoint Verification" respectively.

On April 3 and 7, 1995, with Unit 2 in Mode 6 (Refueling), the "as found" set pressure for Main Steam Safety Valves 2-MS-SV-201A and 2-MS-SV-201C respectively were found to be outside the lift set pressure of 1085 psi +/-3% allowed by Technical Specifications 3.7.1.1. Additionally, on April 17 and 19, 1995, the "as found" set pressure for Pressurizer Safety Valves 2-RC-SV-2551C and 2-RC-SV-2551B respectively were found to be outside the lift set pressure of 2485 psi +/-1% allowed by Technical Specifications 3.4.2 and 3.4.3.1.

2.0 Significant Safety Consequences and Implications

These events posed no significant safety implications because the "as found" set pressure settings for the Safety Valves are bounded by the safety analyses for overpressure transients and the acceptance criteria for overpressure accidents would have been met. Therefore, the health and safety of the public were not affected at any time during this event.

These events are reportable pursuant to 10 CFR 50.73 (a) (2) (i) (B) for conditions prohibited by Technical Specifications 3.4.2, 3.4.3.1, 3.7.1.1.

3.0 Cause of the Event

The cause of the event has been determined to be setpoint drift. The industry has experienced a history of setpoint drift for safety valves of this type and is a common occurrence.

4.0 Immediate Corrective Actions

The Main Steam Safety Valves were repaired and tested within the allowable TS setpoint limit tolerances. Repairs included lap polishing of the nozzle and replacement of the disk for 2-MS-SV-201-A, and lap polishing of the nozzle and disk for 2-MS-SV-201-C. The remaining testing was performed satisfactory.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HOURS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
North Anna Power Station Unit 2	05000339	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	3 OF 4
		95	001	01	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

The Pressurizer Safety Valves failed the initial As-Found Lift Pressure test at Wyle Laboratories. Subsequently, the Pressurizer Safety Valves were tested as allowed by the ASME Code, and three consecutive lifts of the valves within the allowable TS setpoint limit tolerances were performed. Repairs were not required on these valves. The remaining testing was performed satisfactory.

5.0 Additional Corrective Actions

No additional actions are required.

6.0 Actions to Prevent Recurrence

An evaluation has been conducted to increase the tolerance for the Pressurizer Safety Valve as-found lift setting from +/-1% to an average of +2/-3% with no single valve outside of 3%. This will provide the maximum positive tolerance without reduction in the nominal lift setpoint. A Technical Specifications change request has been developed and submitted to the NRC.

7.0 Similar Events

Previous similar events where Main Steam Safety Valves have been outside the requirements of Technical Specifications 3.7.1.1 have occurred at North Anna Unit 2 on April 21, 1983 (LER 83-027/03L-0), February 21, 1986 (LER 86-001-01), January 21, 1988 (LER 87-009-01), April 12, 1989 (LER 89-005-00), October 17, 1990 (LER 90-005-00), and March 31, 1992 (LER 92-003-00).

Previous similar events where the Pressurizer Safety Valves have been outside the requirements of Technical Specifications 3.4.2 and 3.4.3 have occurred at North Anna Unit 2 on March 23, 1982 (LER 82-014/03L-0), September 13, 1987 (LER 87-008-00), April 12, 1989 (LER 89-006-00), October 17, 1990 (LER 90-005-00), and March 31, 1992 (LER 92-003-00).

8.0 Additional Information

During this period, Unit 1 was operating at 100% power and was not affected by these events.

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HOURS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714) U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
North Anna Power Station Unit 2	05000339	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	4 OF 4
		95	001	01	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

The following are the Technical Specifications setpoints and "as found" pressures:

Main Steam Safety Valves	Set Pressure (psig)	As Found (psig)
2-MS-SV-201A	1085 +/-32.5	1127
2-MS-SV-201C	1085 +/-32.5	1121
Pressurizer Safety Valves	Set Pressure (psig)	As Found (psig)
2-RC-SV-2551B	2485 +/-24.8	2442
2-RC-SV-2551C	2485 +/-24.8	2511