NRC FORM 366
(7-77)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: |__|__|__|__|__|__|1 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

|0|1| |A|R|A|N|0|2|2 LICENSEE CODE |0|0|-|0|0|0|0|0|-|0|0|3 LICENSE NUMBER |4|1|1|1|1|4 LICENSE TYPE |__|__|5 CAT

|0|1| REPORT SOURCE |L|6 |0|5|0|0|0|3|6|8|7 DOCKET NUMBER |1|0|0|9|8|1|8 EVENT DATE |0|7|1|6|8|4|9 REPORT DATE

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

|0|2| On 10/9/81, while in Mode 3, the steam driven Emergency Feedwater Pump, 2P7A, tripped on overspeed while the
|0|3| monthly surveillance test was being performed. On 10/11/81, the EFW Pump, 2P7A, tripped on overspeed
|0|4| following a manual start during a unit trip recovery. On 10/12/81, the EFW Pump, 2P7A, tripped on overspeed
|0|5| after receiving an auto start signal following a turbine/reactor trip. The electric driven EFW Pump, 2P7B,
|0|6| was available at all times. Reportable per Technical Specification (T.S.) 6.9.1.9.b.
|0|7|
|0|8|

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP SUBCODE	VALVE SUBCODE
0 9	H H 11	D 12	/ 13	T U R B I N 14	Z 15	Z 16

LER/RO REPORT NUMBER 17	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO
	8 1	---	0 3 8	/	0 3	X	---	1

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
X 18	Z 19	Z 20	Z 21	0 0 0 0 22	Y 23	Y 24	A 25	T 1 4 7 26

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

|1|0| The 10/9/81, occurrence was believed to be caused by an oil leak in the turbine hydraulic system. The oil
|1|1| leak was repaired, and the EFW Pump, 2P7A was tested, found to be operable, and returned to service. The
|1|2| 10/11/81, occurrence was attributed to condensation in the steam lines which resulted from insulation not
|1|3| being reinstalled and a steam trap not being unisolated after a maintenance activity. The insulation was
|1|4| replaced and the trap was unisolated. Also, the trap bypasses were opened to ensure that condensation (cont'd)

	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
1 5	G 28	0 0 0 29	NA 30	A 31	Operator Observation 32

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
1 6	Z 33	Z 34	NA 35	NA 36

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION
1 7	0 0 0 37	Z 38	NA 39

PERSONNEL INJURIES

	NUMBER	DESCRIPTION
1 8	0 0 0 40	NA 41

LOSS OF OR DAMAGE TO FACILITY

	TYPE	DESCRIPTION
1 9	Z 42	NA 43

PUBLICITY

	ISSUED	DESCRIPTION	NRC USE ONLY
2 0	N 44	NA 45	

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PDR ADOCK 05000368
S PDR

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Cause Description and Corrective Actions (Continued)

would not accumulate in the lines. The EFW Pump, 2P7A, was tested, found operable and returned to service. The 10/12/81, occurrence was caused by the steam supply control valve not being properly reset after maintenance. The valve limits were reset properly and the EFW Pump, 2P7A, was tested, and returned to operable status. The design of the governor for the turbine driver was determined to have a high sensitivity to minor system deficiencies which should not by themselves cause the driver to overspeed. Consequently, a design change to reduce sensitivity to overspeed was installed. The new design changed the startup method so that steam is admitted to the turbine through a new bypass line for 15 seconds prior to opening the steam inlet valves (2CV-0340-2). This allows the turbine speed to increase to the idle speed which pressurizes the hydraulic portion of the governor system. This facilitates the governor to respond quickly enough to prevent an overspeed trip when 2CV-0340-2 opens after the 15 second time delay. This design change was completed and the system was released to operations on 11/17/82.

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Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Licensee Event Report
No. 81-038/03X-1

Gentlemen:

In accordance with Arkansas Nuclear one - Unit 2 Technical Specification 6.9.1.9b, attached is the subject report concerning steam driven Emergency Feedwater Pump (EFW) 2P7A. This is an update to a previous submittal dated October 30, 1981. Change bars have been added to indicate the updated material.

Very truly yours,

John R. Marshall
Manager, Licensing

JRM:RJS:ac

Attachment

cc: Mr. Richard C. DeYoung
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Mr. Norman M. Haller, Director
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