UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 9, 1984

Docket No. 50-395

Mr. O. W. Dixon, Jr.
Vice President Nuclear Operations
South Carolina Electric & Gas Company
P.O. Box 764
Columbia, South Carolina 29218

Dear Mr. Dixon:

Subject: V. C. Summer Nuclear Station - Full Load Rejection Capability

Reference: (a) Letter dated August 24, 1984 from O. W. Dixon, Jr. (SCE&G)
To H. R. Denton, (NRC)

The information provided by reference (a) was not sufficient for us to make a determination as to whether or not the requested amendment contains significant hazards considerations. Therefore, we are requesting a response to the enclosed questions and any additional information that you wish to provide concerning this matter.

The recording and/or recordkeeping requirements contained in this letter affect fewer than ten respondents; therefore, OMB clearance is not required under P.L. 96-511.

Sincerely,

Elinor G. Adensam

Elinor G. Adensam, Chief
Licensing Branch #4
Division of Licensing

Enclosure:
As stated

cc: See next page

DESIGNATED ORIGINAL

Certified By

SUMMER

Mr. O. W. Dixon, Jr.
Vice President, Nuclear Operations
South Carolina Electric & Gas Company
P.O. Box 764 (Mail Code F-04)
Columbia, South Carolina 29218

cc: Senior Vice President
South Carolina Public Service Authority
223 North Live Oak Drive
Moncks Corner, South Carolina 29461

J. B. Knotts, Jr., Esq.
Bishop, Liberman, Cook, Purcell
and Reynolds
1200 17th Street, N.W.
Washington, D. C. 20036

Mr. Mark B. Whitaker, Jr.
Group Manager - Nuclear Engineering
& Licensing
South Carolina Electric & Gas Company
P.O. Box 764
Columbia, South Carolina 29218

Resident Inspector/Summer NPS
c/o U.S. Nuclear Regulatory Commission
Route 1, Box 64
Jenkinsville, South Carolina 29065

James P. O'Reilly, Regional Administrator
U.S. Nuclear Regulatory Commission,
Region II
101 Marietta Street, N.W., Suite 2900
Atlanta, Georgia 30323

Enclosure

1. What is the level in feet of the steam generator water level-low low trip setpoint of 30% (allowable value; 28.2%) compared to the zero level assumed in the safety analysis of the loss of normal feedwater accident?

2. At what level is steam generator heat transfer capability reduced?

3. What is the lowest steam generator level that the loss of normal feedwater accident can commence and still not reach the level where steam generator heat transfer capability is reduced at any time during the accident?

4. What is the effect on the overtemperature Δt setpoint by changing T4 from 33 secs. to 28 secs?

5. What is the effect on the safety analyses listed in FSAR Table 7.2-4 that have a correlation with the overtemperature Δt trip?