

Mr. William R. McCollum
Catawba Site Vice President
Duke Power Company
P. O. Box 1006
Charlotte, NC 28201

October 13, 1995

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - REQUEST FOR RELIEF NO. 94-04,
94-05 SECOND TEN YEAR INTERVAL INSERVICE INSPECTION PROGRAM PLAN,
CATAWBA NUCLEAR STATION, UNIT 1 (TAC NO. M92198)

Dear Mr. McCollum:

The NRC staff, with assistance from its contractor, Idaho National Engineering Laboratory, is reviewing and evaluating the second ten-year interval inservice inspection program plan and the associated requests for relief from the ASME Boiler & Pressure Vessel Code, Section XI requirements for Catawba Nuclear Station, Unit 1. Additional information is required from Duke Power Company in order for the staff to complete its review. Please provide your responses within sixty days of receipt of this letter with a copy also sent to the following:

Mr. Michael T. Anderson
INEL Research Center
2151 North Boulevard
PO Box 1625
Idaho Falls, Idaho 83415-2209

This requirement affects nine or fewer respondents, and therefore, it is not subject to the Office of Management and Budget review under P.L. 96-511.

Sincerely,

Original signed by:
Robert E. Martin, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No . 50-413

Enclosure: Request for
Additional Information

cc w/encl: See next page

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DATE	10/16/95		10/17/95		11/3/95				

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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

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Mr. W. R. McCollum
Duke Power Company

Catawba Nuclear Station

cc:

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REQUEST FOR ADDITIONAL INFORMATION FOR THE
SECOND 10-YEAR INTERVAL INSPECTION PROGRAM PLAN
AND ASSOCIATED REQUESTS FOR RELIEF
FOR
DUKE POWER COMPANY
CATAWBA NUCLEAR STATION, UNIT 1
DOCKET NUMBER 50-413

Request for Additional Information - Second 10-Year Interval Inservice
Inspection Program Plan

1. Scope/Status of Review

Throughout the service life of a water-cooled nuclear power facility, 10 CFR 50.55a(g)(4) requires that components (including supports) that are classified as American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code Class 1, Class 2, and Class 3 meet the requirements, except design and access provisions and preservice examination requirements, set forth in the ASME Code Section XI, "Rules for Inservice Inspection of Nuclear Power Plant Components", to the extent practical within the limitations of design, geometry, and materials of construction of the components. This section of the regulations also requires that inservice examinations of components and system pressure tests conducted during the successive 120-month inspection interval comply with the requirements in the latest edition and addenda of the Code incorporated by reference in 10 CFR 50.55a(b) on the date 12 months prior to the start of a successive 120-month interval, subject to the limitations and modifications listed therein. The components (including supports) may meet requirements set forth in subsequent editions and addenda of the Code that are incorporated by reference in 10 CFR 50.55a(b) subject to the limitations and modifications listed therein. The licensee, Duke Power Company, has prepared the *Catawba Nuclear Station, Unit 1, Second 10-Year Interval Inservice Inspection Program Plan* to meet the requirements of the 1989 Edition of Section XI of the ASME Code.

As required by 10 CFR 50.55a(g)(5), if the licensee determines that

certain Code examination requirements are impractical and requests relief, the licensee shall submit information to the Nuclear Regulatory Commission (NRC) to support that determination.

The staff has reviewed the available information in the *Catawba Nuclear Station, Unit 1, Second 10-Year Interval Inservice Inspection Program Plan*, Revision 0, submitted December 29, 1994, and the requests for relief from the ASME Code Section XI requirements that the licensee has determined to be impractical.

2. Additional Information Required

Based on the above review, the staff has concluded that additional information and/or clarification is required to complete the review of the ISI Program Plan.

- A. Please address the degree of compliance and/or exceptions to augmented examinations that have been established by the NRC when added assurance of structural reliability is deemed necessary. Examples of documents that address augmented examinations are:
- (1) Branch Technical Position MEB 3-1, "High Energy Fluid Systems, Protection Against Postulated Piping Failures in Fluid Systems Outside Containment";
 - (2) Regulatory Guide 1.150, *Ultrasonic Testing of Reactor Vessel Welds During Preservice and Inservice Examinations*;
 - (3) Code of Federal Regulations, Part 10, 50.55a(g)(6)(ii)(A), which states that all licensees must augment their reactor vessel examinations by implementing once, during the inservice inspection interval in effect on September 8, 1992, the examination requirements for reactor vessel shell welds specified in Item B1.10 of Examination Category B-A of the 1989 Code. In addition, all previously granted relief

for Item B1.10, Examination Category B-A, for the interval in effect on September 8, 1992, is revoked by the new regulation. For licensees with fewer than 40 months remaining in the interval on the effective date, deferral of the augmented examination is permissible with the conditions stated in the regulations.

Discuss these and any other augmented examinations that may have been incorporated in the *Catawba Nuclear Station, Unit 1, Second Ten-Year Inspection Interval Inservice Inspection Program*, Revision 0.

- B. Paragraph 10 CFR 50.55a(b)(2)(iv) requires that certain ASME Code Class 2 piping welds in the Residual Heat Removal (RHR), Emergency Core Cooling (ECC), and Containment Heat Removal (CHR) systems be examined. Portions of these systems are critical to the safe shut down of the plant and should not be completely excluded from inservice volumetric examination based on piping wall thickness. Other utilities/licensees have deemed it technically prudent to perform augmented volumetric examinations on welds in those portions of lines excluded from examination. In consideration of the safety significance of the subject systems, has Duke Power Company planned and/or scheduled the examination of a sample to assure the continued integrity of thin-wall piping welds in the subject systems? (A 7 1/2% sample is consistent with the extent of examination required for Class 2 piping.)
- C. The licensee stated in Section 4, page 10, for Examination Category B-K-1, Items B10.10, B10.20, and B10.30, that integral attachments are not applicable for Catawba Nuclear Plant, Unit 1. Verify that this is the case. It should be noted that Class 1 piping component supports typically have some integral attachment welds.
- D. The licensee stated in Section 4, page 10, for Examination Category B-L-1, that Item B12.10, pump casing welds, is not applicable. Verify that the pump casing design for the reactor coolant pumps does

not include pump casing welds. What is the model type and number of the pumps?

- E. In Section 9.0, "Requests for Relief from ASME Code Requirements," the licensee stated that the relief request list contains some of the requests for relief that affect the Catawba Nuclear Station and that for a list of all requests for relief, one should see the Station Regulatory Compliance Section. This statement implies that there are other relief requests in addition to those submitted for review. Verify that all requests for relief have been submitted. If additional relief requests are required, the licensee should submit them for staff review.

The schedule for timely completion of this review requires that the licensee provide, by the requested date, the above requested information and/or clarification with regard to the Catawba Nuclear Station, Unit 1, Second 10-Year Interval Inservice Inspection (ISI) Program Plan.