



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

October 13, 1995

Mr. George A. Hunger, Jr.
Director-Licensing, MC 62A-1
PECO Energy Company
Nuclear Group Headquarters
Correspondence Control Desk
P.O. Box No. 195
Wayne, PA 19087-0195

SUBJECT: REPAIR OF FLAWS FOUND IN CORE SPRAY DOWNCOMER JOINTS, PEACH BOTTOM
ATOMIC POWER STATION, UNIT NO. 3 (TAC NO. M92639)

Dear Mr. Hunger:

By letter dated October 9, 1995 as supplemented by letter dated October 12, 1995, you informed the NRC of cracks discovered in the four core spray system downcomers within the Peach Bottom Unit 3 reactor vessel. The cracks were discovered through an inservice visual inspection requested in NRC Bulletin 80-13, "Cracking in Core Spray Spargers." You supplemented the visual inspections with ultrasonic inspections and determined the extent of the cracking as detailed in your October 9, 1995 letter.

In your letters, you informed the staff that you planned to install clamps over the affected sections of piping as permanent repairs to the flaws. Details of the clamp design were included in General Electric Company's (GE) reports GENE-771-99-0295, Revision 2, "Core Spray Line Downcomer Bracket Stress Assessment Report," and GENE-771-98-0295, Revision 2, "Core Spray Line Seismic Assessment Report," which you forwarded in your October 12, 1995 letter. We are aware that installation of the clamps was completed on October 12, 1995.

The GE reports demonstrated that the clamps are sufficiently designed to withstand stresses experienced during normal plant operation, transients and postulated loss-of-coolant accidents and maintain the functionality of the core spray piping. The staff concurs with this conclusion.

Your October 12, 1995 submittal also addressed the impact of expected core spray system leakage within the reactor vessel during a postulated loss-of-coolant accident. With the clamps installed and assuming a 360 degree through-wall crack at the flaw location, you calculated that core spray loop "A" would experience a total of 343 gpm of leakage and the "B" loop would experience a total of 78 gpm of leakage. You concluded that these leakage rates were within the margin between the design flow rate for each loop (6250 gpm at 105 psid) and the nominal loop flow rates assumed in your licensing basis loss-of coolant accident analysis (5000 gpm at 105 psid). The staff concurs with this conclusion.

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G. Hunger, Jr.

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Based on the above, we conclude that operation of the Peach Bottom Unit 3, with cracked but clamped core spray downcomers, is acceptable. Ongoing interaction between the staff and the Boiling Water Reactor Vessel and Internals Project may result in further actions involving individual licensee core spray piping.

If you have any questions on this matter, do not hesitate to contact the NRC Project Manager, Joe Shea, at (301) 415-1428.

Sincerely,

/s/

John F. Stolz, Project Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-278

cc: See next page

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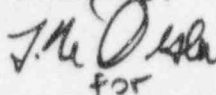
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Sincerely,

Handwritten signature of John F. Stolz in cursive script.

for
John F. Stolz, Project Director
Project Directorate I-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-278

cc: See next page

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Units 2 and 3

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