JUL 1 3 1984

Docket No. 50-302

Mr. Walter S. Wilgus Vice President, Nuclear Operations Florida Power Corporation ATTN: Manager, Nuclear Licensing & Fuel Management

& Fuel Management
P. O. Box 14042, M.A.C. H-2
St. Petersburg, Florida 33733

Dear Mr. Wilgus:

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SUBJECT: CRYSTAL RIVER UNIT 3 - NRC COMMENTS ON THE OFFSITE DOSE CALCULATION MANUAL

Amendment No. 69 to the Crystal River Unit 3 operating license was issued on June 27, 1984 to implement the revised radioactive effluent Technical Specifications (RETS). In conjunction with our review of the RETS, we reviewed the proposed Crystal River Unit 3 Offsite Dose Calculation Manual (ODCM) and noted a number of areas in the manual that require revision or clarification. This letter forwards our comments on the ODCM. Florida Power Corporation should resolve the attached comments and revise the ODCM accordingly. In addition, we request that a section be added to the ODCM to provide for total dose calculation, including direct radiation, as required by RETS Section 4.11.3.

Sincerely,

"ORIGINAL SIGNED BY:"

George W. Rivenback, Acting Chief Operating Reactors Branch #4 Division of Licensing

Enclosure: As Stated

cc w/enclosure: See next page

ORB#4:0L HSilver;cf 7/184

ORB/4:DL GRivenbark 7/1/84

8407250028 840713 PDR ADOCK 05000302 PDR Crystal River Unit No. 3 Florida Power Corporation

cc w/enclosure(s):

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Ulray Clark, Administrator Radiological Health Services Department of Health and Rehabilitative Services 1323 Winewood Blvd. Tallahassee, Florida 32301 NRC COMMENTS ON THE PROPOSED CRYSTAL RIVER UNIT 3
OFFSITE DOSE CALCULATION MANUAL (ODCM)

JUNE 1984

1. Sections 1.3-1

It is not clear why there are two definitions for "0 $_{\rm i}$ in Section 1.3-1. Was the second definition meant to be written as

" Q_i = (Muclide Concentration in the Effluent Stream) x (Flow)"? What is the significance of the "*"?

Ettivent Stream Flow Rate Nuclide Concentration

Flow F

- Reactor Building Purge Exhause Duct + 30,000 cfm + 2.9 x 107
 cc/sec
- 2) Auxiliary Building and Fuel Handling Area Exhaust Duct s 136,000 cfm = 7.3 x 107 cc/sec
- 3) Waste Gas Decay Tank Release Line > 30 c/m max + 2.4 x 109 cc/.sc

In order for a gaseous release to be within the limits of specification 1.1-1, the following equations must be satisfied:

where:

- The allowable total body doze rate due to noble gas gamma emissions in mrem/yr.
- 3000 . The allowable skin dose rate due to mobile gas beta emissions in
- 1500 . The allowable organ dose rate in meem/yr.

2. Sections 1.3-1 and 1.3-2

The titles for Section 1.3-1 and 1.3-2 should be changed as these sections are not setpoint calculations, but are the pre-release calculations to ascertain 10 CFR Part 20 compliance.

3. Section 1.3-2

625

The Licensee should state that the activity of all components in the expression were obtained from the most recent analysis. Specifically, $C_{\rm t}$, $C_{\rm s}$, and $C_{\rm fe}$ are obtained on a monthly or quarterly basis.

SETPOINT CALCULATION 1.3-2 TIQUID RAD WASTE RELEASE

L. INTRODUCTION

Prior to initiating a release of liquid radwaste, it must be determined that the concentration of radionuclides to be released and the flow rates at which they are released will not lead to a release concentration greater than the limits of specification (.1-2 at the point of discharge.

II. INFORMATION REQUIRED

Results of appropriate Nuclide Analysis from Section 1.2

IIL CALCULATION

Discharge Concentration •
$$\left[\frac{11C_{r1}}{MPC_{r1}} + \frac{C_{f}}{MPC_{f}} + \frac{C_{g}}{MPC_{g}} + \frac{C_{g}}{MPC_{g}} + \frac{C_{g}}{MPC_{g}} + \frac{C_{g}}{MPC_{g}} + \frac{C_{g}}{MPC_{g}} + \frac{C_{g}}{MPC_{g}} \right]$$

wheret

IIC. 1 . Sura of the concentrations of isotope i, in the gamma spectrum

Ci . lodire 131 concentration

CG . Dissolved or entrained noble gas concentration

Cr . Tritium Concentration

C. . Gross alpha emitters concentration

C. . Sr-49, 90 concentration

CFe . Fe-55 concentration

R . Effluent Stream Flow Rate (600gpm-MM-L7; 100gpm-RM-L2)

F . Dilution Stream Flow Rate (10,000gpm; Ref: FSAR Sec. 11.2.1.1)

MPC . 10CFR20 Appendix B, Table II, Column 2 Maximum Permissible Concentration by isotope.

If the Calculated Discharge Concentration is less than or equal to 1, the discharge may be initiated. If the calculated discharge concentration is greater than 1, action must be taken to reduce the effluent concentration or effluent stream flow rate prior to initiating discharge.

4. Section 1.4-5

The term C should be written as Σ μρτ.

where I I. he Ith nuclide

Serpoint Adjustment 1.9-5 Plant Discharge Line Monitor (RM-L2) (Batch Type Releases)

INTRODUCTION

Following completion of the analyses required by Section 1.2-4 and determination of release rates and concentration limits in accordance with Section 1.3-2, the monitor setpoint requires adjustment to ensure that alarm and pathway isolation occur if the nuclide concentration or release rate exceeds the limits determined.

METHODOLOGY

Evaporator Condensate Storage Tank or Laundry and Shower Sump Tank contents are circulated through radiation monitor RM-L2 and returned to the auxiliary building sump to obtain the actual count rate at RM-L2 for the concentration contained in the tank for release. The observed count rate is adjusted for release flow, background and statistical counting variations, particular to this release flow path. The resulting value is used as the alarm/trip setpoint and RM-L2 is adjusted to this value prior to initiating the release.

CALCULATION

Monitor Setpoint • [Net CPM x Admin. Factor] FR - FD - 8kg - 3.3 r 8kg

wheres

Net CPM . The observed monitor count rate, in counts per minute.

Admin. Factor - Administration Factor to account for error in setpoint determination. Admin. Factor - 0.3.

C/MPC * The ratio of the actual concentration of the tank contents to be released to the Maximum Permissible Concentration (MPC) as listed in 10 CFR 20, Table II, Column 2 for unrestricted areas.

FR . The release flow rate of waste to be dischar; & in gallons per minute.

FD . The dilution flow from the Nuclear Services Sea Water system in gallons per minute.

BKG = The background count rate at RM-L2 in counts per minute (cpm).

3.3 BKg • A statistical spread on the background count rate which represents a 99.93% confidence level on monitor counting. This factor is included to prevent inadvertent high/strip alarms due to random counts on the monitor. Only the positive (-) side of the spread is applied.

5. Section 2.2-1 and Section 2.2-2

Should the second equation for D_p be written as $D_p = \frac{(D_{L-2}) k_2 + (D_{L-1}) k_1 + (D_L) k}{3}$

As written "k" has the same value for all three terms.

DOSE PROJECTION METHODOLOGY 2.2-1 GASEOUS RADVASTE

I. INTRODUCTION

Crystal River Unit 3 operating practices require use of the waste gas system (Waste Gas Decay Lanks). The normal release paths for gaseous effluents are via ventilation exhaust treatment systems (HEPA and Charcoal Filters). The operability of the ventilation exhaust treatment systems is controlled by the Crystal River Unit 3 Technical Specifications.

As long as these practices and specifications are maintained, the radwaste reduction requirements of Crystal River Unit 3 Technical Specification 3.7.13.) are met, and there is no need to project doses prior to the release of gaseous radwaste.

B. CALCULATIONS

Dose projection calculations will be necessary if either system is not available for use. The dose projection will be made using whichever formula below yields the higher projected dose.

- 1) Dp . (DL) k
- 2) Dp . (DL-2) k · (DL-1) k · (DL) k

wheres

- Dp . Projected Dose.
- DL . Dose, calculated in accordance with section 4.3-1 of this manual for the most recently completed (reference) month.
- A constant to account for the use of the waste gas system during the reference month.
 - k a wif the waste gas system was used.
 - k . I if the system was not used.

Assumptions:

- Xe-133 and I-131 are major dose contributors.
- 2) I-131 is removed by charcoal filters
- 3) Nominal holdup time is ten days (two Xe-133 haif-lives).

References:

1) T.S. 3.6.4.2, 3.7.8.1

2) FSAR 3.3.1, 5.5.2

Calculation of Dose Factors in the Ground Plane Pathway ($R_i^G [D/Q]$)

R^G₁(D/Q) * K' K" (SF) DFG₁ (1-e^{-k₁t}) / k₁) units * m² · mrem/yr per uCl/sec

where:

K' A constant unit of conversion, 106 pCi/vCl.

K" A constant unit of conversion, 8760 hr/yr

SF The shielding factor, (dimension bas, 0.7)

1 The decay constant for the ith radionuclide, sec to the exposure period, 3.15 x 10⁷ sec (1 year)

DFG:

The ground plane dose conversion factor for the ith radionuclide (mrem/hr per/Ci/m²)

E-6

6. Calculation of Dose Factors in the Ground Plane Pathway

The t should be for 15 years and not for one year.

The units for DFG, should be uC, not C.

Reference: The equation deriving RG(D/d) was taken from NUREG 0(1), Section 3.3.1.2.

7. The total dose calculation including direct radiation was not addressed.

C68

C69

C41

C70

C71

C72

C73

5

55W

SW

WSW

WNW

NW

NNW

1430

1300

1780

2100

4400

3600

3600

2000

d. The following sample locations were mitted:

Samule Location Desitted

Table 5.1-2 C27
Table 5.1-1 C14H, C09, C43
Figure 5.1 C14H, C48

fote: CO9 is identified in Figure 5.1

Table 3.1-1 •
Environmental Radiological Monitoring
Stations Locations

STATION	LOCATION	DIRECTION FROM PLANT	DISTANCE FROM PLANT (mi)
C04	State Park Old Dam on River near road intersection	ENE	6.3
C07	Crystal Piver Public Water Plant	ESE	7.3
C10	Indian Waters Public Water Supply	ESE	1.9
CI3	Mouth of Intake Canal	wsw	3.4
CI4H	Head of Discharge Canal	NW	0.1
C14G	Discharge Canal at Gulf of Mexico	*	2.4
CIS	Yankeetown City Well	N	\$ 2
C19	NW Corner State Roads 433 & 495	ENE	1.1
C29	Discharge Area	N	2.0
C30	Intake Area	wsw	3.6
C+0	Near N.E. Site Boundary near excavated pond & pump station	t	3.5
C41	Onsite meteorological tower	\$SW	0.4
C46	North Pump Station	N	0.4
C47	University of Florida, Gainesville	NNE)2
C49	To be determined from annual milk		

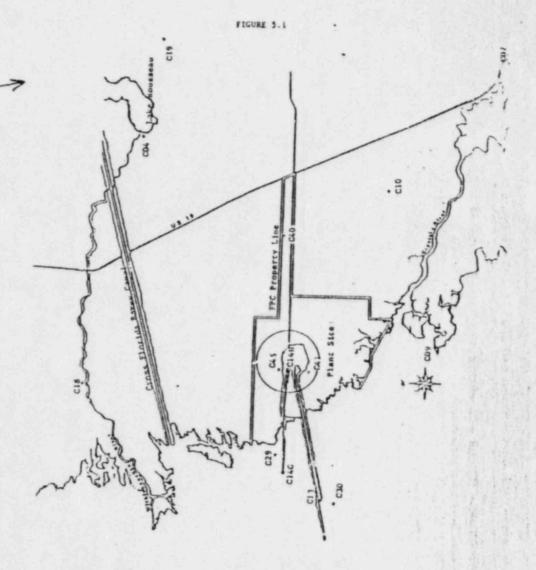
8. The following sample locations were omitted:

Sample Location Omitted

Table 5.1-2 Table 5.1-1 Figure 5.1

C27 C14M, C09, C48 C14M, C48

Note: CO9 is identified in Figure 5.1



Environmental Monitoring Sample Station Locations

9. Section 4.3-3

The Licensee should consider use of a near field average dilution factor in the expression for the liquid dose calculation.

DOSE CALCULATION 4.3-3 (LIQUID EFFLUENTS)

The dose or dose commitment to a MEMBER OF THE PUBLIC from radioactive materials in liquid effluents released to UNRESTRICTED AREAS is calculated as follows:

where:

- D, . The cumulative dose commitment to the total body or any organ, . . from the liquid effluents for the total time period Tact, in mrem.
- The length of the lth time period over which Cit is averaged for all liquid releases, in hours.
- Cil The average concentration of radionuclide, i, in undiluted liquid effluent during time period at from any liquid release, in "Ci/mi.
- Air * The site related ingestion dose commitment factor to the total body or any organ r for each identified principal gamma and oeta emitter as shown in Table 4.4-17 of this manual, in mrem-inl per hour w Ci.

References:

1) NUREG 0113, Section 4.3

Calculation of Inhalation Pathway Dose Factor (P,)

The P; should be R; in both the title and the expression.

79 P.R.

PATHWAY DOSE FACTOR (P)

R:

K' (BR) DFA = mrer/year per ,Ci/m)
NUREG 0133)

(from section 5.2.1.1 of

where:

A constant unit of conversion - 106 pCI/uCi

BR . The Breathing Rate of the represented age group: (from Table E-5 of Regulatory Guide 1.109).

> 1900 m³/yr - infant 3700 m³/yr - child \$000 m³/yr - teen \$000 m³/yr - adult

DFA; The maximum organ inhalation dose factor for the represented age group for the ith radionuclide, in mrem/pCi.

The values of DFA1 are listed in Regulatory Guide 1.109 as follows:

DFA; (infant) - Table E-10 DFA; (child) - Table E-9 DFA; (cen) - Table E-8 DFA; (adult) - Table E-7 Calculation of Ingestion Dose Factor Grass-Cow-Meet Pathway

The term e - 1th should be e -1t f

			Calculation of Ingestion Dose Factor Grass-Cow-Meat Pathway (R (D/Q))	79 po to
8	[0/0]	- K	$\left[\frac{Q_{F}(U_{ap})}{\frac{\lambda_{1}+\lambda_{n}}{\lambda_{2}+\lambda_{n}}}\right]F_{f}(c) (DFL_{\underline{t}})_{s}\left[\frac{f_{p}f_{s}}{Y_{p}}+\frac{(1-f_{p}f_{s})e^{-\lambda_{1}}}{Y_{s}}\right]$	6) 115
	here:		s = a - area/yr per uct/sec	Reference Table, R.G. 1.109
	K'		A constant of unit conversion, 106 pCI/wCI.	
	QF		The cow's consumption rate, 30 kg/day (wet weight)	E-3
*	Uap		The receptor's meat consumption rate for age (a), in kg/yr	E-3
-			Infant - 0 Child - 41 Teen - 63 Adult - 110	
	Yp	•	The agricultural productivity by unit area of pasture feed grass 0.7 kg/m ²	E-13
	Y,	•	The agricultural productivity of unit area of stored feed 2.0 kg/ α . 3	E-13
-	Ft	•	The stable element transfer coefficients, in days/kg.	E-1
	•	•	Fraction of deposited activity retained on cow's feed grass 1.0 radioiodine 0.2 particulates	E-13 E-13
	tt	•	Transport time from pasture to receptor, in sec. 1,73x106 sec (20 days)	E-13
	th	٠	Transport time from crop field to receptor, in sec. 7.78x106 sec. (90 days)	E-15)
	(DFI	-i)a =	The maximum organ ingestion dose factor for the ith radionuclide for the receptor in age group (a), in mrem/pCi	E-11 to E-14
	4.		The decay constant for the ith radionuclide, in sec -1	E-11 10 E-117
	٠.,	٠	The decay constant for removal of activity on leaf and plas surfaces by weathering, 3.73 x 10-7 sec -1 (corresponding to a 14 day half-life).	nt
	to.		Fraction of the year that the cow is on pasture (dimension)	ess) + 1 *
	t,		Fraction of the cow feed that is pasture grass while the co- pasture (dimensionless) * 1 *.	w is on
1		stor	t cattle are considered to be fed from two potential souled feeds. Following the development in Regulatory Guide Is will be considered unity, in lieu of site specific informal land census report by the licensee.	1.109, the values of to
	Note:		above equation does not apply to the concentration of tritistion is provided in NUREG 0133, section 5.3.1. # to determine	

Reference: The equation deriving 3 1 (D/Q) was taken from NUREG 2133, Section 5.3.1.3.

Calculation of Ingestion Dose Factor Vegetation Pathway (R | D/Q)

Calculation of Ingestion Dose Factor Vegetation Pathway

The term $\frac{1}{(1+w)}$ should be $\frac{1}{{\lambda_1 + \lambda_w \choose 1}}$

The defintion for $t_{\rm L}$ should state it is for <u>leafy</u> vegetation instead of stored vegetation.

The defintion for Y_{ψ} should use a capital letter Y.

where	: 0	mits = m · mrem/yr per uci/sec	Reference Table, R.G. 1.151
K'		A constant of unit conversion, 106 pCi/aCi.	1119
U		The consumption rate of fresh leafy vegetation by the receptor in age group (a), in kg/yr.	
		Infant - 0	E-3
		Child - 26	
		Teen - +2	
		Adult - 64	
US		The consumption rate of stored vegetation by the receptor	
75.4		2. 8. and 101' 111 v 81 11	E-3
		Infant - 0	
		Child - 320	
		Teen - 630	
		Adult - 520	
(DF	Lilas	The maximum organ ingesting dose factor for the ith radionuclide for the receptor in age group (a), in mrem/pCl.	E-11 to E-19
1L	•	The fraction of the annual intake of fresh leafy vegetation grown locally. (default 1.0)	
18	٠	The fraction of the annual intake of stored vegetation grown locally. (default 0.76)	
t _L		Liefy	146
		The average time between harvest of stored egetation and it consumption, \$.6 x 10 ⁴ , seconds, (1 day)	. 01
th		The average time between harvest of stored vegetation and it	
		consumption, 5.13 x 106 seconds, (60 days)	
(1.		E-15
(An	, .	The vegetation areal density, 2.0 kg/m ²	E-13
		Fraction of deposited activity retained on the vegetation	
		1.0 radiosocine 0.2 particulates	E-13
*1		The decay constant for the ith radionuclide, in sec -1	
**	•	The decay constant for removal of activity on leaf and plant surfaces by weathering, 5.73 x 10-7 sec -1 (corresponding to a 14 day half-life).	

The above equation does not apply to the concentrations of tritium in vegetation. A separate equation is provided in NGREG 0133, section 5.3.1.5 to determine tritium values.

References The equation deriving R (D/Q) was taken from NUREG 0133, Section 5.3.1.5.

DOSE CALCULATION 4.3-1 (NOBLE GAS)

The dose to an individual at or beyond the SITE BOUNDARY due to noble gases released in gaseous effluents is calculated as follows:

wherei

- Dy

 The dose to an individual at or beyond the SITE SOUNDARY due to gamma emissions from noble gases in gaseous effluents in mead/time period.
- Ds a The dose to an individual at or beyond the SITE BOUNDARY due to beta emissions from noble gases in gaseous effluents in mrad/time period.

3.17 x 10-8 . The number of years in one second.

- M_j The air dose factor due to gamma emissions for each identified noble gas radionuclide, in mrad/year per uCi/m³.
- NI a The air dose factor due to beta emissions for each identified noble gas radionuclide, in mrad/year per uCi/m).
- (X/Q) . The highest calculated annual average relative concentration for areas at or beyond the UNRESTRICTED AREA Boundary, 2.3 x 10-6 sec/m³.
- Q1 The average release of hobie gas radionuclides in gaseous releases from all vents/in 4Ci. Releases shall be cumulative over the time period of concern.

7 yps

Section 4.3-1

The units for Q, should be uC, not C1.

SETPOINT CALCULATION 1.3-1

I. INTRODUCTION

Prior to initiating a release of gaseous radwaste, it must be determined that the concentration of radionuclides to be released, and the flow rates at which they are released will not exceed the dose rate limitations of Specification 1.1-1.

II. INFORMATION REQUIRED

Results of appropriate Nuclide Analysis from Section 1.2

IL CALCULATIONS

A. Noble Gas Gamma Emissions

Eq. 1 Dose Rate, 1 Ki (X/Q) Qi) mremlyr.

B. Noble Gas Beta Emissions

Eq. 2 Dose Rate = 1; (Li + 1.1Mi) ((X/Q) Qi) mrem/yr.

C. Iodine 131, Tritium, Radioactive Particulates

Eq. 3 Dose Rate I,T,P . 11 P1 (X/Q) Q1 mrem/yr.

wheres

- Ki . The total body dose factor due to gamma emissions for each identified noble gas radionuclide, in mrem/yr per wCi/m. (See Table 1.5-1).
- L1 The skin dose factor due to beta emissions for each identified noble gas radionuclide, in mrem/yr per "Ci/m". (See Table 1.3-1).
- Mi . The air dose factor due to gamma emissions for each identified noble gas radionuclide, in mrad/yr per uCi/m³ (unit conversion constant of 1.1 mrem/mrad converts air dose to skin dose). (See Table 1.5-1).
- Pi The dose parameter for radionuclides other than noble gases for the inhalation pathway, in mrem/yr per "Clim". (See Table 1.5-2).
- Qi . The release rate of radionuclides, i, in gaseous effluent from individual release sources, in «Ci/sec ther unit, unless otherwise specified).
- (X/Q) = 2.5x10-6 secim3. (Fb) all vent releases. The highest calculated annual average relative concentration for any area at or beyond the unrestricted area boundary.

Section 1.3-1

The word "For" is spelled as "fbr" in the $\frac{x}{0}$ definition.

Section 1.3-3

a. Shouldn't the value 3.26 x 10^8 cc be 1.36 x 10^7 cc? The 3.26 x 10^8 value is not used in the calculation.

b. The " ν " is missing from most of the units, e.g., 2.08 ν C1 and 6.62 x 10^{-1} ν C4.

In order to determine the total quantity of lodine 131 collected on the filter, the values of Cv above are multiplied by the volume assumed to have passed through the filter, $(3.26 \times 10^8 \text{ cc})$.

QI . Cystiak

wherei

Q1 . The total quantity of lodine 131 collected on the filter in 4Ci.

Cy . The concentration of lodine 13t in the vent in Cl/cc.

If The assumed total volume of vent atmosphere that has passed through the filter, 1.36 x 107 cc (1 CFM for 3 hours).

k . The lodine removal efficiency of the filters: 95%

Solving for QI for the Reactor Building vent yields:

Q1(RB) . 2.044C1 2.04 4C;

Solving for Q for the Auxillary Building and Fuel Handling Area vent yields: -

Q1(AB) . 6.62 x 10-1 MCI

These values are translated to counts per minute for the lodine monitoring channels through use of the appropriate calibration curves.