

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 2 8	PAGE (3) 1 OF 0 2
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TITLE (4)  
Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																									
0 9	0 5	8 4	8 4	0 1 5	0 0 1	0 0	3 8	4			0 5 0 0 0																																									
<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:15%;">OPERATING MODE (9) 1</td> <td colspan="11">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="5">POWER LEVEL (10) 110.0</td> <td>20.402(b)</td> <td>20.405(c)</td> <td><input checked="" type="checkbox"/></td> <td>50.73(a)(2)(iv)</td> <td>73.71(b)</td> </tr> <tr> <td>20.405(a)(1)(i)</td> <td>50.38(c)(1)</td> <td><input type="checkbox"/></td> <td>50.73(a)(2)(v)</td> <td>73.71(c)</td> </tr> <tr> <td>20.405(a)(1)(ii)</td> <td>50.38(c)(2)</td> <td><input type="checkbox"/></td> <td>50.73(a)(2)(vii)</td> <td rowspan="3">OTHER (Specify in Abstract below and in Text, NRC Form 356A)</td> </tr> <tr> <td>20.405(a)(1)(iii)</td> <td>50.73(a)(2)(i)</td> <td><input type="checkbox"/></td> <td>50.73(a)(2)(viii)(A)</td> </tr> <tr> <td>20.405(a)(1)(iv)</td> <td>50.73(a)(2)(ii)</td> <td><input type="checkbox"/></td> <td>50.73(a)(2)(viii)(B)</td> </tr> <tr> <td>20.405(a)(1)(v)</td> <td>50.73(a)(2)(iii)</td> <td><input type="checkbox"/></td> <td>50.73(a)(2)(ix)</td> <td></td> </tr> </table>												OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)											POWER LEVEL (10) 110.0	20.402(b)	20.405(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)	20.405(a)(1)(i)	50.38(c)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	73.71(c)	20.405(a)(1)(ii)	50.38(c)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 356A)	20.405(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	20.405(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	20.405(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)	
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LICENSEE CONTACT FOR THIS LER (12)

NAME Glenn B. Kirk, Compliance Section Engineer	TELEPHONE NUMBER 6 1 5 8 7 1 0 - 6 1 4 6
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)       NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	1 2	3 0	8 4

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

An automatic reactor trip occurred with unit 2 in mode 1 at 100 percent reactor power following a turbine generator trip due to a failure of the turbine generator electro-hydraulic control system.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR 8 4	SEQUENTIAL NUMBER - 0 1 5	REVISION NUMBER - 0 0	0 2	OF 0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

Unit 2 was in mode 1 (2235 psig, 579<sup>0</sup>F) at 100 percent reactor power on 09/05/84. At 0523C the operator received a turbine generator electrohydraulic control system (EHC) high-low level alarm. An assistant unit operator was immediately dispatched to the EHC tank to investigate the problem. At 0529C the operator received an EHC low oil pressure alarm. At 0530C the reserve EHC pump automatically started. At 0532C the generator turbine tripped on EHC low-low pressure and level resulting in an automatic reactor trip.

Investigation into the EHC failure revealed a crack at a fitting in the EHC line to the 2A2 interceptor valve. Further inspection determined that a support clamp on the EHC line had become loose allowing the line to vibrate and subsequently crack. The crack was repaired and the support clamp retightened on 09/05/84.

Following the reactor trip, steam generator blowdown inboard containment isolation valve 2-FCV-1-182 did not indicate closed (both red and green lights were lit) in the main control room. Investigation revealed the valve had closed as required and the limit switch was adjusted so that valve position indicator lights read properly.

Additionally, following the reactor trip a steam generator auxiliary feedwater loop 1 pipe break signal closed LCV-3-174 and prevented flow to the number one steam generator from the turbine driven auxiliary feedwater pump. Investigation revealed that the pipe break signal was invalid. During troubleshooting the turbine driven auxiliary feedwater pump was run twice for 15 minutes with no indication of low pressure on auxiliary feedwater loop 1 and auxiliary feedwater level control LCV-3-174 operated correctly. Subsequent inspections revealed that the wiring on pressure switch PS-3-160A was incorrectly terminated inside the pressure switch housing. The pressure switch was rewired properly on 09/05/84. At this time no conclusion has been made as to when the miswiring occurred or why testing did not reveal the error. To further investigate the behavior of the miswired switch a special test will be performed during the unit 2 refueling outage scheduled for early October 1984. This special test will place the switch back into the as-found miswired condition and perform several tests on the switch. A supplemental LER will be submitted with the results and conclusions of the special tests. The pressure switches on the remaining loops (both unit 1 and unit 2) were checked and all found wired correctly. During this inspection the auxiliary feedwater loop 3 pipe break pressure switch PS-3-140B was found corroded and replaced. All of these pressure switches are scheduled to be replaced with new class 1E switches under NUREG 0588 program.

All other personnel and equipment performed and responded as expected. There was no effect on public health or safety. For 1984 there have been three (3) automatic reactor trips on unit 2, but this is the first reactor trip due to EHC failure.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant  
Post Office Box 2000  
Soddy Daisy, Tennessee 37379

October 3, 1984

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

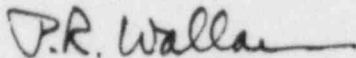
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 2 - DOCKET NO.  
50-328 - FACILITY OPERATING LICENSE DPR-79 - REPORTABLE OCCURRENCE REPORT  
SQRO-50-328/84015

The enclosed licensee event report provides details concerning an automatic reactor trip due to failure of the turbine generator electrohydraulic control system. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



P. R. Wallace  
Plant Manager

Enclosure  
cc (Enclosure):

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NRC Inspector, NUC PR, Sequoyah