OMAHA PUBLIC POWER DISTRICT
Fort Calhoun Station Unit No. 1

SEPTEMBER 1995
Monthly Operating Report

1. OPERATIONS SUMMARY

During the month of September, Fort Calhoun Station operated at a nominal 100% power. Normal plant maintenance, surveillance and equipment rotation activities occurred during the month, in addition to scheduled on-line modification activities. Monitoring of the leaking Control Element Drive Mechanism (CEDM) mechanical seal continued.

2. SAFETY VALVES OR PORV CHALLENGES OR FAILURES WHICH OCCURRED

During the month of September, no Power Operated Relief Valve (PORV) or Primary System Safety Valve challenges or failures occurred.

3. RESULTS OF LEAK RATE TESTS

Although above normal, the September Reactor Coolant System (RCS) leak rate was steady at approximately 0.4 gpm throughout the month. This leak rate remained steady following the reactor trip and resultant surveillance testing of the CEDMs on August 26, 1995.

The major contributor to the increase in RCS leakage has been classified as "Known" leakage. This leakage is being collected in the Reactor Coolant Drain Tank (RCDT). The apparent leakage source was CEDM #15. The "Known" leak rate stabilized at 0.280 gpm.

The remainder of the leakage has been classified as "Unknown" leakage. The "Unknown" leak rate varied from 0.1 to 0.25 gpm. The identified sources of "Unknown" leakage were Chemical and Volume Control System valves (located in the Auxiliary Building) leaking through or minor Charging System packing leaks.

4. CHANGES, TESTS AND EXPERIMENTS REQUIRING NUCLEAR REGULATORY COMMISSION AUTHORIZATION PURSUANT TO 10CFR50.59

Amendment No.	Description
No. 170	This amendment revised the Technical Specifications (TS) to relocate the axial power distribution limits to the Core Operating Limits Report (COLR).
No. 171	This amendment changed several sections in the TS in accordance with the guidance of Generic Letter 93-05, "Line Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing During Power Operation."

5. SIGNIFICANT SAFETY RELATED MAINTENANCE FOR THE MONTH OF SEPTEMBER 1995

- Rebuilt packing cooling pump for Charging Pump CH-1A
- Replaced packing cooling pump for Charging Pump CH-1C due to lack of discharge pressure
- Replaced Raw Water Pump AC-10C due to normal wear
- Rebuilt Charging Pump CH-1A due to excessive leakage
- Replaced relay 41C for Emergency Diesel Generator #2 due to field flash failure

6. OPERATING DATA REPORT

Attachment I

7. AVERAGE DAILY UNIT POWER LEVEL

Attachment II

8. UNIT SHUTDOWNS AND POWER REDUCTIONS

Attachment III

9. REFUELING INFORMATION, FORT CALHOUN STATION UNIT NO. 1

Attachment IV

ATTACHMENT I
OPERATING DATA REPORT

DOCKET NO. 50-285
UNIT FORT CALHOUN STATION
DATE OCTOBER 06, 1995
COMPLETED BY D. L. LIPPY
TELEPHONE (402) 533-6843

OPERATING STATUS

1. Unit Name: FORT CALHOUN STATION
2. Reporting Period: SEPTEMBER 1995 NOTES
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 502
5. Design Elec. Rating (Net MWe): 478
6. Max. Dep. Capacity (Gross MWe): 502
7. Max. Dep. Capacity (Net MWe): 478
8. If changes occur in Capacity Ratings (3 through 7) since last report, give reasons:
N/A
9. Power Level to which restricted, if any (Net MWe): N/A
10. Reasons for restrictions, if any:
N/A

	THIS MONTH	YR-TO-DATE	CUMULATIVE
11. Hours in Reporting Period.....	720.0	6551.0	192985.0
12. Number of Hours Reactor was Critical	720.0	5081.1	151499.0
13. Reactor Reserve Shutdown Hours.....	.0	.0	1309.5
14. Hours Generator On-line.....	720.0	4997.2	149771.5
15. Unit Reserve Shutdown Hours.....	.0	.0	.0
16. Gross Thermal Energy Generated (MWH)	1077617.7	7232110.8	199380811.3
17. Gross Elec. Energy Generated (MWH)..	359274.0	2405192.0	65810074.2
18. Net Elec. Energy Generated (MWH)....	342887.4	2291503.4	62783295.7
19. Unit Service Factor.....	100.0	76.3	77.6
20. Unit Availability Factor.....	100.0	76.3	77.6
21. Unit Capacity Factor (using MDC Net)	99.6	73.2	70.4
22. Unit Capacity Factor (using DER Net)	99.6	73.2	68.7
23. Unit Forced Outage Rate.....	.0	5.3	4.0

24. Shutdowns scheduled over next 6 months (type, date, and duration of each):
NONE.
25. If shut down at end of report period, estimated date of startup: _____
26. Units in test status (prior to comm. oper.):

		Forecast	Achieved
INITIAL CRITICALITY			
INITIAL ELECTRICITY	N/A		
COMMERCIAL OPERATION			

ATTACHMENT II
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-285
UNIT	FORT CALHOUN STATION
DATE	OCTOBER 06, 1995
COMPLETED BY	D. L. LIPPY
TELEPHONE	(402) 533-6843

MONTH SEPTEMBER 1995

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	467	17	476
2	469	18	474
3	469	19	476
4	468	20	478
5	468	21	481
6	469	22	482
7	471	23	484
8	472	24	484
9	475	25	484
10	475	26	484
11	476	27	483
12	477	28	482
13	476	29	480
14	475	30	480
15	475	31	N/A
16	476		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

ATTACHMENT III
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-285
UNIT NAME Fort Calhoun St.
DATE October 6, 1995
COMPLETED BY D. L. Lippy
TELEPHONE (402) 533-6843

REPORT MONTH September 1995

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
None.									

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

4
Exhibit F - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit H - Same Source

(9/77)

Attachment IV
Refueling Information
Fort Calhoun Station - Unit No. 1

Report for the month ending September 30, 1995

1. Scheduled date for next refueling shutdown. — September 21, 1996

2. Scheduled date for restart following refueling. — November 2, 1996

3. Will refueling or resumption of operations thereafter require a technical specification change or other license amendment? — Yes

   a. If answer is yes, what, in general, will these be? — Enrichment limit of spent fuel racks is to be increased to at least 4.5 w/o from 4.2 w/o. This is necessary based upon the preliminary Cycle 17 core pattern development.

   b. If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload. — N/A

   c. If no such review has taken place, when is it scheduled? — N/A

4. Scheduled date(s) for submitting proposed licensing action and support information. — January 1996 (for spent fuel rack enrichment limit change)

5. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures. — N/A

6. The number of fuel assemblies:
   a) in the core — 133 Assemblies
   b) in the spent fuel pool — 618 Assemblies
   c) spent fuel pool storage capacity — 1083 Assemblies

7. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity. — 2007 Outage

Prepared by Kevin Hobbs — Date 10-10-95

LIC-95-0196
Enclosure 2

OMAHA PUBLIC POWER DISTRICT

FORT CALHOUN STATION UNIT NO. 1

"CORRECTED PAGES TO THE AUGUST 1995 MONTHLY OPERATING REPORT"