



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 81 TO FACILITY OPERATING LICENSE NO. DPR-40

OMAHA PUBLIC POWER DISTRICT

FORT CALHOUN STATION, UNIT NO. 1

DOCKET NO. 50-285

Introduction:

By application dated March 9, 1984, the Omaha Public Power District (the licensee) requested an amendment to the Technical Specifications (TS) for the Fort Calhoun Station, Unit No. 1. The amendment request was in response to the Commission's Generic Letter No. 83-37 entitled, "NUREG-0737 Technical Specifications."

The Generic Letter, which was issued on November 1, 1983, advised licensees to submit new TS for the following NUREG-0737 items:

1. Reactor Coolant System Vents (II.B.1)
2. Post-Accident Sampling (II.B.3)
3. Long Term Auxiliary Feedwater System Evaluation (II.E.1.1)
4. Noble Gas Effluent Monitors (II.F.1.1)
5. Sampling and Analysis of Plant Effluents (II.F.1.2)
6. Containment High-Range Radiation Monitor (II.F.1.3)
7. Containment Pressure Monitor (II.F.1.4)
8. Containment Water Level Monitor (II.F.1.5)
9. Containment Hydrogen Monitor (II.F.1.6)
10. Instrumentation for Detection of Inadequate Core Cooling (II.F.2)
11. Control Room Habitability (III.D.3.4)

The Generic Letter contained TS which would be acceptable to the staff.

The licensee proposed TS for the above items except Post-Accident Sampling, Long Term Auxiliary Feedwater System Evaluation, Instrumentation for Detection of Inadequate Core Cooling, and Control Room Habitability Requirements. The licensee advised the staff that proposed TS for these items will be the subject of other amendment requests. As such, the staff will review these other items separately when submitted. Our evaluations of the proposed TS for reactor coolant system vents (II.B.1), sampling and analysis of plant effluents (II.F.1.2), Containment Pressure Monitor (II.F.1.4), Containment Water Level Monitor (II.F.1.5), and Containment Hydrogen Monitor (II.F.1.6) will be the subject of separate evaluations and licensing actions. Our evaluation of the proposed TS for the post-accident radiation monitoring instrumentation follows.

Evaluation of Post-Accident Radiation Monitoring Instrumentation  
(II.F.1.1 and II.F.1.3)

The licensee proposes to add new TS number 2.21 entitled "Post-Accident Monitoring Instrumentation." This TS addresses operability requirements for noble gas effluent monitors (II.F.1.1), containment high-range radiation monitors (II.F.1.3), and main steam line radiation monitor (II.F.1.1). Regarding applicability, the licensee proposes that the TS be applicable in modes 1, 2, and 3 (power operation, hot standby, and hot shutdown, respectively). The applicability modes of the Fort Calhoun proposal are consistent with the staff guidance. Therefore, the applicability proposed by the licensee is acceptable.

The licensee proposes to add the operability requirements in a new table 2-9 entitled, "Post-Accident Monitoring Instrumentation Operating Limits." Regarding the minimum operable channels for the containment wide range radiation monitors, wide-range noble gas stack monitors, and main steam line radiation monitor, these are consistent with the staff guidance. Therefore, the minimum operable channels for these monitors are acceptable. If the minimum operable channels are not met during the applicable modes, the licensee proposes corrective actions which are consistent with the corrective actions contained in the staff guidance. Therefore, the corrective action requirements for these monitors are acceptable.

The licensee did not propose TS for alarm/trip setpoint or measurement range for the various instruments. The licensee stated that only engineered safety features and reactor protective system setpoints are presently included in the Fort Calhoun TSs. In addition, the present Fort Calhoun TSs do not contain instrument measurement range for any instrument. The licensee states that because no control or safety functions are initiated by the post-accident monitors, the range and setpoints for these monitors will be included in the operating manuals and not in the TSs. Since no Fort Calhoun TSs contain measurement range for any instrument, we will not require measurement range for the subject instruments to be included in the TSs at this time. In addition, since only the RPS and ESF instruments contain trip setpoints in the Fort Calhoun TSs, we will not require alarm/trip setpoints for non RPS/ESF instruments, such as the subject radiation monitors, to be included in the TSs at this time. We do, however, agree with the licensee's proposal to put the alarm/trip setpoints and measurement ranges in the operating manuals.

Based upon the above discussion, the staff finds the operability requirements and corrective action statements for the containment wide range radiation monitors, wide range noble gas stack monitors, and main steam line radiation monitor acceptable.

The licensee proposed to amend Table 3-3, item 3 of the present TS to include surveillance requirements for the subject radiation monitors. The staff guidance recommends a channel check on a shift basis. The licensee proposes

a daily channel check to be consistent with daily channel checks for other area and process monitors contained in the TSs. Since the licensee's present TSs require similar monitors to be channel checked on a daily basis, we will not require the post-accident radiation monitors to be channel checked on a shift basis at this time. Therefore, a daily basis is acceptable.

The licensee proposed to perform a monthly test on the monitors. This is consistent with the staff guidance. The licensee proposes to calibrate the monitors at least once every 18 months. This is also consistent with the staff guidance.

Based upon the above discussion, the staff finds the surveillance requirements for the containment wide range radiation monitors, wide range noble gas stack monitors, and main steam line radiation monitor acceptable.

#### Environmental Consideration

This amendment involves a change in the installation or use of a facility component located within the restricted area. The staff has determined that the amendment involves no significant increase in the amounts of any effluents that may be released offsite and that there is no significant increase in individual or cumulative occupation radiation exposure. The Commission has previously issued a proposed finding that this amendment involves no significant hazards consideration and there has been no public comment on such finding. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR §51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

#### Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: July 12, 1984

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