



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

OMAHA PUBLIC POWER DISTRICT

DOCKET NO. 50-285

FORT CALHOUN STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 81
License No. DPR-40

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Omaha Public Power District (the licensee) dated March 9, 1984 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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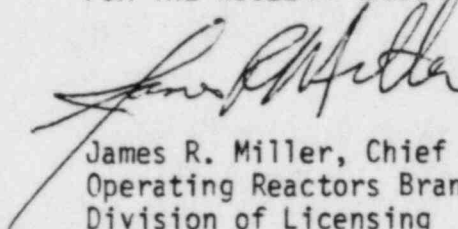
2. Accordingly, Facility Operating License No. DPR-40 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-40 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 81, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of issuance: July 12, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 81

FACILITY OPERATING LICENSE NO. DPR-40

DOCKET NO. 50-285

Revise Appendix "A" Technical Specifications as indicated below. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages

i
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3-13 (Table 3-3)

Insert Pages

i
2-97
2-98 (Table 2-9)
3-13 (Table 3-3)

TECHNICAL SPECIFICATIONS

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2.0 LIMITING CONDITIONS FOR OPERATION

2.21 Post-Accident Monitoring Instrumentation

Applicability

Applies to post-accident monitoring instrumentation not included as part of the Reactor Protective System or Engineered Safety Features. This specification is applicable while in modes 1, 2 and 3.

Objective

To assure that instrumentation necessary to monitor plant parameters during post-accident conditions is operable or that backup methods of analysis are available.

Specifications

Post-accident instrumentation shall be operable as provided in Table 2-9. If the required instrumentation is not operable, then the appropriate action specified in Table 2-9 shall be taken.

Basis

Post-accident monitoring instrumentation provides information, during and following an accident, which is considered helpful to the operator in determining the plant condition. It is desirable that this instrumentation be operable at all times during operation of the plant. However, none of the post-accident monitors are required for safe shutdown of the plant nor are any control or safety actions initiated by the monitors.

In general, the post-accident monitors provide wide range capabilities for parameters which are beyond the range of normal protective and control instrumentation. They also provide remote sampling and analysis capability to reduce personnel exposure under post-accident conditions. Because the information necessary to assess the effect of an accident (i.e., core damage) can be obtained from other sources and by manual methods, it is not necessary that the post-accident monitors be operable at all times.

TABLE 2-9

Post-Accident Monitoring Instrumentation Operating Limits

<u>Instrument</u>	<u>Minimum Operable Channels</u>	<u>Action</u>
1. Containment Wide Range Radiation Monitors (RM-091A & B)	2	(a)
2. Wide Range Noble Gas Stack Monitor		
RM-063L (Noble Gas Portion Only)	1	(a)
RM-063M (Noble Gas Portion Only)	1	(a)
RM-063H (Noble Gas Portion Only)	1	(a)
3. Main Steam Line Radiation Monitor (RM-064)	1	(a)

(a) With the number of OPERABLE channels less than required by the minimum channels operable requirements, initiate the pre-planned alternate method of monitoring the appropriate parameter(s) within 72 hours, and

1. either restore the inoperable channel(s) to OPERABLE status within 7 days of the event, or
2. prepare and submit a special report to the Commission pursuant to specification 5.9.3 within 14 days following the event outlining the action taken, the cause of the inoperability, and the plans and schedules for restoring the system to OPERABLE status.

TABLE 3-3

MINIMUM FREQUENCIES FOR CHECKS, CALIBRATIONS AND TESTING
OF MISCELLANEOUS INSTRUMENTATION AND CONTROLS

<u>Channel Description</u>	<u>Surveillance Function</u>	<u>Frequency</u>	<u>Surveillance Method</u>
1. Primary CEA Position Indication System	a. Check	S	a. Comparison of output data with secondary CEAPIS.
	b. Test	M	b. Test of power dependent insertion limits, deviation, and sequence monitoring systems.
	c. Calibrate	R	c. Physically measured CEDM position used to verify system accuracy. Calibrate CEA position interlocks.
2. Secondary CEA Position Indication System	a. Check	S	a. Comparison of output data with primary CEAPIS.
	b. Test	M	b. Test of power dependent insertion limit, deviation, out-of-sequence, and overlap monitoring systems.
	c. Calibrate	R	c. Calibrate secondary CEA position indication system and CEA interlock alarms.
3. Area, Process, and Post-Accident Radiation Monitors	a. Check	D	a. Normal readings observed and internal test signals used to verify instrument operation.
	b. Test	M	b. Detector exposed to remote operated radiation check source or test signal.
	c. Calibrate	R	c. RM-063L, M, and H and RM-064 - One time factory calibration is acceptable provided linearity solid sources are used to check the integrity of the detectors. RM-091A and B - In situ calibration by electronic signal substitution is acceptable for all range decades above 10 R/hr. In situ calibration for at least one decade below 10 R/hr shall be by means of calibrated radiation source. All other monitors - Exposure to known radiation source.