UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

OMAHA PUBLIC POWER DISTRICT

DOCKET NO. 50-285

FORT CALHOUN STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 80
License No. DPR-40

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by the Omaha Public Power District (the licensee) dated March 9, 1984, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, Facility Operating License No. DPR-40 is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3.B. of Facility Operating License No. DPR-40 is hereby amended to read as follows:

B. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 80, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

for James R. Miller, Chief
Operating Reactors Branch #3
Division of Licensing

Attachment:
Changes to the Technical
Specifications

Date of Issuance: July 9, 1984

ATTACHMENT TO LICENSE AMENDMENT NO. 80

FACILITY OPERATING LICENSE NO. DPR-40

DOCKET NO. 50-285

Revise Appendix "A" Technical Specifications as indicated below. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove Pages	Insert Pages
i	i
iii	iii
	2-16b
3-20d	3-20d
5-21	5-21

TECHNICAL SPECIFICATIONS

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2.0 LIMITING CONDITIONS FOR OPERATION
2.1 Reactor Coolant System (Continued)

2.1.8 Reactor Coolant System Vents

Applicability

Applies to the status of the reactor coolant gas vent system. This specification is applicable while in modes 1, 2, or 3.

Objective

To ensure capability of venting non-condensible gases from the reactor coolant system, the following gas vent system requirements must be met:

(1) At least one reactor coolant system vent path consisting of at least two valves in series powered from emergency buses shall be OPERABLE and closed at each of the following locations:

a. Reactor vessel head.

b. Pressurizer steam space.

(2) With one of the above reactor coolant system vent paths inoperable, startup and/or power operation may continue provided power is removed from the valve actuators of all the inoperable valves; restore the inoperable vent path to OPERABLE status within 30 days or be in HOT STANDBY within 12 hours and in COLD SHUTDOWN within the following 30 hours.

(3) With both of the above reactor coolant system vent paths inoperable, maintain the inoperable vent path closed with power removed from the valve actuators of all the inoperable valves in the inoperable vent paths and restore at least one of the vent paths to OPERABLE status within 72 hours or be in HOT STANDBY within 12 hours and in COLD SHUTDOWN within the following 30 hours.

Basis

The purpose of this specification is to ensure a method and system is available to remove steam and/or non-condensible gases from the reactor coolant system, which may inhibit core cooling during natural circulation. The Power Operated Relief Valves are not to be considered a vent path for the purpose of this specification.

TABLE 3-5
(Continued)

		Test	Frequency	USAR Section Reference
10c.	(Continued)	4. Automatic and/or manual initiation of the system shall be demonstrated	At least once per plant operating cycle.	
11.	Containment Cooling and Iodine Removal Fuseable Linked Dampers	1. Demonstrate damper action. 2. Test a spare fuseable link.	1 year, 2 years, 5 years, and every 5 years thereafter.	9.10
12.	Diesel Generator Under-Voltage Relays	Calibrate	During each refueling outage.	8.4.3
13.	Motor Operated Safety Injection Loop Valve Motor Starters (HCV-311, 314, 317, 320, 327, 329, 331, 333, 312, 315, 318, 321)	Verify the contactor pickup value at ≤85% of 460 V.	During each refueling outage.	
14.	Pressurizer Heaters	Verify control circuits operation for post-accident heater use.	During each refueling outage.	
15.	Spent Fuel Pool Region I Racks	Test neutron poison samples for dimensional change, hardness change, and neutron attenuation change.	Intervals of 1, 2, 4, 7, 11, 15, 20, and 25 years after installation.	
16.	Reactor Coolant Gas Vent System	1. Verify all manual isolation valves in each vent path are in the open position.	During each refueling outage just prior to plant start-up.	
		2. Cycle each automatic valve in the vent path through at least one complete cycle of full travel from the control room. Verification of valve cycling may be determined by observation of position indicating lights.	During each refueling outage.	
		3. Verify flow through the reactor coolant vent system vent paths.	During each refueling outage.	

Amendment No. 41, 54, 60, 75, 77, 80

5.0 ADMINISTRATIVE CONTROLS

5.14 Systems Integrity

A program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids during a serious transient or accident to as low as practical levels shall be implemented. This program shall include the following:

1. Provisions establishing preventive maintenance and periodic visual inspection requirements, and

2. Integrated leak test requirements for each system at a frequency not to exceed refueling cycle intervals.

5.15 Iodine Monitoring

A program which will ensure the capability to accurately determine the airborne iodine concentration in vital areas under accident conditions shall be implemented. This program shall include the following:

1. Training of personnel,

2. Procedures for monitoring, and

3. Provisions for maintenance of sampling and analysis equipment.

5.16 Sampling and Analysis of Plant Effluents

A program which will ensure the capability to obtain and analyze radioactive iodines and particulates in plant gaseous effluents under accident conditions shall be implemented. The program shall include the following:

1. Training of personnel,

2. Procedures for sampling and analysis, and

3. Provisions for maintenance of sampling and analysis equipment.