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TMI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM

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TITLE: TMI-I DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM

1.0 PURPOSE

The purpose of the Direct Senior Reactor Operator Training Program is to prepare qualified personnel to perform senior reactor operator duties.

2.U SCOPE/APPLICABILITY

This procedure applies to all Direct Senior Operator candidates. Personnel placed in this program will normally fill job descriptions which do not require frequent control room participation.

3.0 DEFINITIONS

- 3.1 Candidate An individual recognized by the Manager, Plant Operations as a potential Senior Reactor Operator. Members shall also have approval of his/her immediate supervisor.
- 3.2 Extra-Person A candidate assigned to an operating shift for on-the-job training whose presence is not required to perform specific, non-training related tasks.
- 3.3 Task Examiner Any individual who, due to his qualifications and experience, is selected to certify knowledge requirements of this program.

3.4 Instructor - An instructor from the Operator Training Section of the Training Department or a "guest" instructor whose expertise in a specific subject area, e.g., a systems engineer, has resulted in his assignment, approved in writing by the Operator Training Manager or Manager Plant Training to present material on that subject area.

4.0 RESPONSIBILITIES

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4.1 The Shift Supervisor is responsible for the following:

- Assignment of individuals as alternate task examiners. (A copy of this assignment list shall be forwarded to the Supervisor, Licensed Operator Training.)
- b) Verifying adequate level of achievement and progress by the license candidates on the OJT phase of the program.
- 4.2 The <u>Supervisor</u>, <u>Licensed Operator Training</u>, <u>TMI-1</u>, is responsible for the following:
 - General supervision of the development and conduct of the Direct SRO Training Program.
 - b) Approval of the development, coordination, scheduling and administration of the Direct SRO Training Program, including course outlines, lesson plans, student handouts, simulator training, and evaluation exams.
 - Scheduling classes, students, classroom and facilities necessary to conduct the training program.
 - Interfacing with the Operations Department in all matters impacting the training programs.
 - Assuring that the program content is updated and revised to meet current requirements and supervising revision of the program content, descriptions, lesson plans, and exams.

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	 f) Evaluation of course instruction and license candi 	date progress
	to determine the effectiveness of the training pro- reporting these evaluations to the Operator Traini	gram and ng Manager.
	g) Maintaining the necessary records and reports of t	raining.
	 b) Developing and conducting oral exams. 	
	 Evaluation of candidate critiques of the training 	received.
4.3	The Operator Training Manager is responsible for the fo	llowing:
	a) Assuring the quality of the Direct SRO Training Pro	ogram by
	written approval of materials including course out	line, lesson
	and their compatability with the Direct SDO Training	es and exams
	b) Certification of candidates in accordance with AP	1058
4.4	The Manager, Plant Training is responsible for the follo	owing:
	a) To ensure that the training program is developed to	o meet the
	requirements established by the Director-TMI I, th	rough the
	Manager, Plant Operations, and that proper reccords	s and
	documentation are provided and maintained.	
	D) Certification of candidates in accordance with AP	1058.
4.0	The manager, Flant uperations is responsible for the fo	llowing:
	a) to ensure that the overall level of training of pla	ant operators
	schedules and administrative procedures	ncenc,
	b) To evaluate candidates at the completion of the sit	mulator
	program on their proficiency to direct licensed act	tivities.
	c) Certification of candidates in accordance with AP	1058.
4.6	The Director-TMI I is responsible for the following:	
	a) Final certification of candidate in accordance with	h AP 1058.
5.0 REFE	RENCES	
5.1	American Nuclear Standard (ANS) 3.1, Rev. 01/17/78, Sele	ection,
	Qualification and Training of Personnel for Nuclear Power	er Plants.
5.2	Code of Federal Regulations, Section 10CFR55, Rev. 9/1/8	80, " Operator
	Licenses".	
5.3	Regulatory Guide 1.8, Draft Rev. 9/80, Personnel Qualif	ication and
	Training.	
5.4	Administrative Procedure 1058, Requirements for Certific	cation of
5.5	VUDEC 0737 Clarification of THI Action Plan Requirement	
5.5	NURES 0757, Clarification of IML Action Flan Regul Penen	LS .
5.0 ATTA	CHMENTS	
6.1	Appendix A - Systems Guide Lists	
6.2	Appendix B - Licensed Operator OJT and System Checkouts	Charlelict
0.3	Appendix C - Administrative Procedure and Qualification	Checkiist
0.4	Appendix D - Outbuildings Checklist	
6.6	Appendix E - Frimary Systems Unecklist	
6.7	Appendix G - Watchstanding Checklist	
6.8	Appendix H - Oral Exam Summary Sheet	
6.9	Appendix I - Shift Foreman Traines Duties Checklist	
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6.10 App 6.11 App 0 <u>PROGRAM</u> 7.1 Pre A11 1. 2. 3. 4. 7.2 Seq 1.	endix J - Designation of Foreman as Final OJT Examinendix K - Simulator Training DESCRIPTION requisites: candidates for Direct Senior Reactor Operator shall Possess a degree in Engineering or applicable scie Have four years of responsible power plant experie Responsible power plant experience should be that control room operator (fossil or nuclear) or as a staff engineer involved in the day-to-day activiti facility. A maximum of two years power plant experi fulfilled by academic or related technical trainin one-for-one time basis. Two years shall be nuclea experience. Complete a minimum of three months performing the Senior Reactor Operator while under instruction as person in the control room. Satisfactorily meet the minimum medical requirement licensed personnel as specified in 10CFR55. uence: The program consists of the following:	her 1: ence. obtained as a power plant ies of the erience may be ng, on a ar power plant duties of a s an extra hts for fication will
	 e) Classroom - as determined by Supervisor Operator Training, TMI-1. The length of the Direct SRO program is dependent candidates experience and availability. The requirabove are essential to the program and must be con- time necessary to complete the on-the-job training vary from candidate to candidate, and as a result completion may vary. In no case shall the program eighteen (18) months. Due to the diversity of nuclear experience that we encountered from candidate to candidate, and the to requirements of regularly assigned job functions, committment of time devoted to the program may var Courses in Decision Analysis and Supervisory Deve waived by the Operator Training Manager due to can previous experience. The program concludes with the NRC examination. 	Licensed on the irements listed npleted. The g tasks may time to m exceed ill be time the ry. lopment may be ndidates

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7.3	Program Objectives:	
	 Conduct training in the subject areas listed below to enable the trainee to demonstrate through writte examinations, completion of specified practical fac completion of an operating examination, the ability the activities of licensed operators safely and com to obtain an NPC Semior Peactor Operator license 	as required in and oral itors, and to direct apetently and
	Subject Areas a) Supervisory course in decision analysis/superv	visory
	development.	
	 b) Supervisory control room and plant operating e directed by specific task assignments and lice operators. 	xperience, nsed senior
	c) Reactor Theory	
	 Plant Design and Operating Characteristics 	
	e) Plant Control Systems	
	T) Radiation Control and Safety	
	b) Recognizing and Mitigating Core Damage	
	i) Simulator Training	
	 Verify that the candidate has received or otherwise knowledge equivalent to those candidates who have c TMI-1 Senior Reactor Operator Replacement Training 	demonstrated ompleted the Program and
	its precursors.	
7.4	Outline:	
	1. On-the-Job Training	
	a) Secondary Systems	
	c) Administrative Procedures	
	d) Normal, Abnormal and Emergency Operating Proce	du ne s
	e) Technical Specifications	
	f) Shift Foreman Duties	
	2. Classroom Training	
	Classroom Training will be provided to meet needs e	stablished by
	candidates college level courses and training in nu	clear related
	fields shall be made to determine the extent of the	program
	needed. The topics listed below are to be used as	a guide for
	determining needed classroom training.	전 경험 전 전 것이 있는 것이 없다.
	a) Reactor Theory Review	
	Neutrons and Neutron Interactions	
	Solving Exponential Equations	
	Reactor Kinetics	
	Reactivity	
	Subcritical Multiplication	
	Neutron Sources	
	Reactor Period and Start Up Rate	
	Problem Solving in Reactor Kinetics	
	Reactivity Coefficients	

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	Flux Distribution
	DNB, FQ, F H
	Reactor Control
	Fuel Assemblies and Control Rods
	Transients and Effects on Fuel Assemblies & Control Rod
	Fission Product Poisons
	Reactor Transfent Analysis
	Excore Nuclear Instruments
b)	Heat Transfer and Fluid Flow Thermodynamics
57	Heat Transfer
	Properties of Fluids
	Steam Tables
	Core and Plant Parameters, Normal and Transient
c)	General Categories
	Facility Incidents
	Emergency Plan
	Security
	Technical Specifications
	Administrative Procedures
	Emergency, Abnormal and Normal Operating Procedures,
	including Abnormal Transient Operating Guidelines
	Primary and Secondary Chemistry Review
	Recognition and mitigation of consequences of severe core
	Damage Change Modifications
(1)	Padiation Control and Safety
	Radioactivity and Radiation
	Effects of Radiation
	Radiation Exposure Limits
	Radiation Protection Problems
	Radiation Instruments
e)	Reactor Control
	RCS Instrument Failures
	Integrated Control System (ICS)
	ICS Transients
7.5 Administ	ration
1.5.1	Program Presentation
	1. Un-the-Job Training
	a) The on-the-job training program consists of
	participation by the Direct Senicr Peactor
	Operator candidate in Shift Forema related
	activities designed to reinforce classroom study.
	maximize new learning experiences and stimulate
	interest.
	b) Primary verification of OJT tasks shall be by
	oral checkout of the candidate on individual task
	items by a task examiner. Task Examiners for
	items in Appendix 8 shall be licensed RO's and

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c) d)	<pre>Final verification shall be by oral and written checkout of task sheet sections by the Shift Supervisor (or Shift Foreman as designated in writing by the Shift Supervisor, using the Summary Sheet in Appendix J), or by a Licensed SRO Instructor. During the OJT phase the license candidates shall be assigned to various shifts with the approval of Manager, Plant Operations. During this phase, the candidate shall assume Shift Foreman duties under instruction and perform other tasks as assigned. The level of performance for each OJT Task is designated in the text or to the right of the description in parenthesis. The following are levels of performance in order of preference: (P) = Perform. The candidate shall perform each task under the direct supervision of a qualified operator. This level of</pre>
	performance is the most preferred due to the hands-on experience gained by the candidate.
	 (5) - Simulate. The candidate shall simulate each task under the direct supervision of a qualified operator. The simulation shall be done as much as possible in the vicinity of the actual component or equipment. Hands-on training, without actual manipulation of valves, breakers, etc is encouraged. When simulating Emergency and Abnormal Procedures the following is required. 1. Find all indications mentioned. 2. Locate all items used to verify that automatic actions have occurred. 3. Locate all controls necessary to perform all immediate manual and subsequent actions. 4. Walk through all Emergency and Abnormal Procedures and note indicators, check automatic actions, and simulate performing manual actions from memory.

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(D) = Discuss. The task examiner shall examine the candidate on the items to evaluate the candidate's overall understanding of the procedure.

Using the procedure as a reference the candidate shall be able to discuss:

- 1. Limits and precautions
- Major steps in procedure
 Basis for procedure seque

3. Basis for procedure sequence. When an option of the level of performance is offered the examiner shall designate by circling which one was done.

- e) Signatures obtained to meet the requirements of this program are valid for eighteen (18) months. After that time the task must be repeated.
- 2. Classroom Training
 - a) Classroom training shall be conducted by
 - qualified instructors using approved lesson plans.
 - b) The candidate will be responsible for all material presented.
- 3. Simulator

A three (3) week Simulator Program will be utilized to reinforce classroom and QUT concepts and to develop an understanding of integrated plant response. One week of the Simulator Program will be devoted to Startup Certification, as required.

a) Candidates for Direct SRO will be evaluated by the Manager, Plant Operations or his designated representative at the completion of the simulator program on their proficiency to direct licensed activities. This evaluation is part of the certification process.

The use of the simulator for training is addressed in Appendix K.

7.5.2

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- Evaluation Criteria 1. On-The-Job Training
 - a) Comprehensive oral checkouts shall be administered by the task examiners for specific task sign-offs and documented by the examiner's
 - b)

signature. The ultimate responsibility for determining adequate achievement by the student rests with the Shift Supervisor, or Licensed SRO Instructor who shall evaluate the performance of the candidate by final oral check out and/or written questioning at the end of each section of the OJT task (i.e. Primary Systems, Electrical Systems), using the Oral Examination Summary Sheet in

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Appendix H. This form shall be retained in the candidate's permanent training file.

c) All tasks which cannot be performed are to be simulated. Performance or simulation of a task shall not alone constitute successful completion of the task. Discussion and oral questioning by the task and final examiners must be included to substantiate successful completion of the task.

 Individuals failing to achieve a "pass" grade on the OJT Oral Exam checkouts shall be:

 Informed of their weak areas and given direction on the material that they should study to upgrade their performance.

 Re-examined within two weeks of the initial failure.

NOTE: Details of weak areas and scheduled date for re-exam must be forwarded to the Supervisor, Licensed Operator Training.

If an individual fails the second section check-out the Manager, Plant Operations, Operator Training Manager and candidate's supervisor shall review the Direct SRO license candidate's overall progress and performance and determine the corrective action to be taken.

2. Written Examinations

Written examinations shall be administered by representatives of the Operator Training Section as required by the Supervisor, Licensed Operator Training. Questions shall cover that material presented in the classroom, material specified for self study, and the material identified on the task sheets.

A passing grade of 80% is mandatory for all written examinations. A grade of less than 80% shall require candidate counseling by the instructor responsible for noted weak areas on the candidate's exam with suggested corrective actions that will upgrade the candidate's performance. A re-exam will be administered within two weeks. If a candidate fails the second exam the Manager, Plant Operations, Operator Training Manager, and the candidate's supervisor shall evaluate the SRO license candidate's performance and decide on the corrective action to be taken.

3. Final Examination

A comprehensive written and oral examination shall be administered by the Training Department at the conclusion of the training program. An 80% overall

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7.6	average and a 70% on individual sections for the satisfactory completion of the w The oral examination should normally com phases; a "walk through" phase, administ Instructor selected by the Supervisor, L Operator Training, and a "board" compose Instructors designated by the Supervisor Operator Training and the Manager, Plant his selected representative. A "pass" g required for the oral exam substantiated documentation with the Oral Examination in Appendix H. Upon completion of these exams, the cand training files and Training Department m shall be forwarded to the Director - TMI certify the candidate for NRC examinatio case of unsatisfactory completion of the decide on the corrective action to be ta Changes and Lesson Plan Correction The program shall be maintained to reflect the followin a) Changes in regulatory requirements U) Changes in applicable codes, standards and guides c) Significant experiences at the facility d) Significant experience throughout the industry e) Remedial action recommended by review/audit findin f) Regularly scheduled participant critiques. Changes will be incorporated per the applicable Training Procedure. Program Scheduling The program will be scheduled as needed. Records and Reports	is required ritten phase. sist of two ered by an icensed d of ; Licensed Operations or rade is by Summary Sheets idate's ecommendations I, who shall n or, in the program, ken. g: g g Department
	 A fraining Program Administrative form shall be consubmitted to the Administrative Section for each clack of lecture or lesson by the instructor who presented for the current and past schedules. Lesson plans, student is 	npleted and lassroom the material. handouts.
	completed OJT task sheets, exam keys and completed	exams and

quizzes, both written and oral, as well as any additional pertinent qualification records shall be maintained on file in the Training Department.

The Candidate Progress Report shall be maintained at the training center. Copies of the updated report shall be submitted by the Supervisor, Licensed Operator Training to the Manager, Plant Training and Manager, Plant Operations via the Operator Training Manager on a monthly basis, or more frequently on a case-by-case basis if requested by any of the reviewing individuals.

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7.9 Ev 1. 2. 3.	aluation At the conclusion of each month of training the lice candidates will be asked to complete a training crit assist in program evaluation. The completed critique reviewed by the Supervisor, Licensed Operator Trainforwarded, along with recommendations or corrective to the Manager, Plant Training, via the Operator Trainformer device and updated at the end of each program presenting the instructors presenting the course and the Supervise doer and updated at the end of each program presents of the instructors presenting the course and the Supervise doer and updated at the end of each program presents of the instructors presenting the course and the Supervise doer and updated at the end of each program presents of the doer and updated at the end of each program presents in on the course no changes in content shall be made without prior approval of the Training the presentation of the course no changes in content shall be made without prior approval of the Training the present Training to review the Direct SRO Program. The review team should consist of personne Operations and Training Departments. The team will adequacy of the program for: Meeting new requirements Adequacy of material and presentations Effectiveness In conducting the review, the team may use any recormaintained by the Training or Operations Departments them. These may include: NRC Inspections QA Audits Other Audits Regulatory Changes Industry Experience License Candidate Critiques The review team shall report the results to the Manager, Plant Operations via the S Licensed Operator Training and the Operator Training	ense ique form to les shall be ng and action taken ining all be entation by risor, ults of this tion taken to ng Manager. course Operator rvisor, Training from the assess the ds to assist ger, Plant upervisor, Manager.

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APPENDIX A SYSTEMS CHECKOUT GUIDELINE

When studying for a system checkout the candidate should be able to do the following:

1. State the functions

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- 2. State setpoints and interlocks
- 3. State the design limits and parameters during normal operations.
- 4. Draw a one line diagram including:
 - a. Major components (pumps, breakers, valves).
 - b. Power supplies to components
 - c. System interconnections
 - d. Meters and gageboards (as applicable)
- 5. Describe physical location of major components (where applicable)
- 6. State Technical Specifications and basis for each
- 7. Describe system response during normal and abnormal conditions.

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APPENDIX "B" LICENSED OPERATOR OJT CHECKLIST 1.0 PRIMARY PLANT PRACTICAL FACTORS TASK EXAMINER 1.1 REACTOR COOLANT SYSTEM (1) 1.1.1 RCS FILL AND VENT 1103-2 (S)	DATE
1.1.2 DRAINING AND N2 BLANKETING OF RCS SYSTEM 1103-11 (S) 1.1.3 RCS LEAKAGE 1303-1.1 (S) 1.1.4 LOSS OF RC/RC PRESSURE 1202-6A (S) 1.1.5 LOSS OF RC/RC PRESSURE 1202-6B (S) 1.1.6 LOSS OF RC/RC PRESSURE 1202-6C (S) 1.1.7 LOSS OF HIGH ACTIVITY IN RC 1202-11 (S) 1.1.8 INADEQUATE CORE COOLING 1202-39 (S) 1.1.9 LOSS OF RC FLOW/RC TRIP 1202-14 (S) 1.1.10 VIBRATION AND LOOSE PARTS 1203-40 (S)	
1.2 REACTOR COOLANT PUMPS AND SEALS (1) 1.2.1 RCS PUMP AND MOTOR MALFUNCTIONS 1203-16 (S) 1.2.2 ESTABLISH ALL CONDITIONS NECESSARY TO RUN REACTOR COOLANT PUMPS (START & STOP RCP'S) (P OR S)	
(1) Systems Checkout. All signatures by licensed SRO or RO.	

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.0	PRIMARY PLANT PRACTICAL FACTORS COM	TINUED	
		TASK EXAMINER	DATE
.3	REACTOR VESSEL CONSTRUCTION (1)		
.4	MAKEUP AND PURIFICATION (1) 1.4.1 LOSS OF REACTOR COOLANT MAKEUP 1203-15 (S) 1.4.2 PERFORM AN RCS BORATION		
	AND CALCULATION	de la line de la company de la	
	1.4.3 PERFORM RCS DILUTION		-
	1.4.4 PERFORM RCS DEBORATION 1.4.5 PERFORM RCS BORON CHANGE FOR ALL RODS OUT POWER REDUCTION TO 50%		-
	1.4.6 SOLUBLE POISON CONCEN- TRATION CONTROL		-
	1.4.7 MAKEUP AND PURIFICATION 1104-2 (D)		
.5	DECAY HEAT REMOVAL (1) 1.5.1 LOSS OF DECAY HEAT REMOVAL 1202-35 (S)		-
	1.5.2 DECAY HEAT REMOVAL		
.6	CORE FLOOD (1) 1.6.1 CORE FLOOD SYSTEM 1104-1 (D)		
.7	PRESSURIZER AND PRESSURE RELIEF (1) 1.7.1 PRESSURIZER FAILURE		-
	1202-29 (S) 1.7.1 PRESSURIZER OPERATION 1103-5 (D)		
1)	Systems Checkout - All signatures b	by licensed SRO or RO	

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TMI-1 DI	FCT SENTOR REACTOR OPERATOR T	DATHING BOOCDAM	0.00
111-1 011	ALCT SENTOR REACTOR OPERATOR T	MAINING PRUGRAM	0-00
1.0 PRIMA	APPENDIX LICENSED OPERATOR RY PLANT PRACTICAL FACTORS CON	"B" OJT CHECKLIST TINUED TASK EXAMINER	DATE
1.8 ONCE 1 1.8.1	HROUGH STEAM GENERATORS (1) OTSG TUBE RUPTURE		·
1.8.2	1202-5 (S) INADVERTENT CLOSURE OF MAIN STEAM ISOLATION		·
1.8.3	LOSS OF FEED TO OTSG 1202-26A (S)		· · · · · · · · · · · · · · · · · · ·
1.8.4	LOSS OF FEED TO OTSG		
1.8.5	OTSG FILL, DRAIN, LAYUP 1106-16 (D)		
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(1) System	s Checkout. All signatures by	y licensed SRO or RO.	

6	PU Nuclean TROUTEDACOPY	6211-ADM-2611.03	
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Т	MI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM	0-00	
	APPENDIX "B" LICENSED OPERATOR OJT CHECKLIST		
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2.0 2.1	PRIMARY SUPPORT SYSTEMS NUCLEAR SERVICE RIVER WATER (1) 2.1.1 RIVER WATER FAILURE 1203-19 (S) 2.1.2 NUCLEAR RIVER WATER	=	
2.2	1104-30 (D) DECAY HEAT RIVER WATER (1)		
2.3	DECAY HEAT CLOSED COOLING (1)		
2.4	INTERMEDIATE CLOSED COOLING (1) 2.4.1 LOSS OF INTERMIDIATE COOLING 1202-17 (S)	=	
	2.4.2 INTERMEDIATE CLOSED COOLING 1104-8 (D)		
2.5	RADIATION MONITORING SYSTEM (1) 2.5.1 EXCESSIVE RADIATION LEVELS 1202-12 (S)		
	2.5.2 EVALUATE AND FOLLOW- UP RMS ALARMS 2.5.3 RADIATION MONITORING CHECKS 1303-4.15 (S)		
2.6	FUEL HANDLING (1)		
2.7	WASTE DISPOSAL - SOLID (1) 2.7.1 SOLID WASTE DISPOSAL 1104-288		
2.8	WASTE DISPOSAL LIQUID (1) 2.8.1 INITIATE, MAKE, AND COMPLETE A LIQUID WASTE RELEASE (P) OR		
	(S) (1104-29 2.8.2 LIQUID WASTE DISPOSAL 1104-29) (D)		
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Title		ADMINISTRATIVE	MANUAL	Revision No.	.0.
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		APPENDIX LICENSED OPERATOR	"B" OUT CHECKLIST	14 1 1	
2.0	PRIMARY	SUPPORT SYSTEMS CONTINUED			
			TASK EXAMINER	DATE	
2.9	PRIMARY 2.9.1	SUMPS AND DRAINAGE (1) PLANT SUMPS AND DRAINAGE 1104-40 (D)			
2.10	HYDROGE	N RECOMBINER (1) HYDROGEN RECOMBINER 1104-62 (D)			
2.11	HYDROGE 2.11.1	N AND NITROGEN NUCLEAR (1) NITROGEN SUPPLY			
	2.11.2	HYDROGEN ADDITION			
2.12	PRIMARY 2.12.1	CHEMISTRY CONTROL (1) ASSIST IN RECOGNIZING & TAKING ACTION FOR OUT OF SPECIFICATION CHEMISTRY (P) OP (S)			
	2.12.2	CHEMICAL ADDITION NUCLEAR 1104-47B (D)		- <u></u>)	
2.13	FLUID BI 2.13.1	LOCK (1) FLUID BLOCK 1104-20 (D)			
.14	PRIMARY 2.14.1	SAMPLING (1) OBSERVE DRAWING AND ANALYZING A PRIMARY SAMPLE (2)		-	
	2.14.2	NUCLEAR PLANT SAMPLING			
.15	SPENT FU	JEL COOLING (1) SPENT FUEL CLEANUP PROCESS 1104-29C (D)			
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TN	MI-1 DIRE	CT SENIOR REACTOR OPERATOR	TRAINING PROGRAM	0-00
2.0 2.16 2.17	PRIMARY NUCLEAR COOLING 2.16.1 2.16.2 WASTE D 2.17.1	SUPPORI SYSTEMS CONTINUED SERVICES CLOSED (1) NSCC SYSTEM FAILURE 1200-20 (S) MSCC 1104-11 (D) ISPOSAL GAS (1) INITIATE, MAKE AND COMPLETE A LIQUID WASTE RELEASE (P)	TASK EXAMINER	DATE
	2.17.2	WASTE DISPOSAL - GASEOUS. 1104-27 (D)		
INAL	VI IN THE R OF	TION:		
	VERIFICA	SHIFT SUPERVISOR		DATE
1) 2)	Systems Signatu	SHIFT SUPERVISOR Checkout. All signatures re Required by Chem. Tech.	by licensed SRO or RO Performing Sample.	DATE
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T	MI-1 DIRECT SENIOR REACTOR OPERA	0-00	
	APP LICENSED OPE	PENDIX "B" RATOR OUT CHECKLIST	
3,0	SECONDARY SYSTEMS PRACTICAL FA	CTORS	
		TASK EXAMINER	DATE
3.1	MAIN STEAM (1)		
	3.1.1 BE ABLE TO USE STEAM TABLE PARAMETERS AND PLANT PARAMETERS - PLOT A SECONDARY HEAT BALANCE		
	RUPTURE 1203-23 (S)		
	3.1.3 MAIN STEAM 1106-14 (D)		
3.2	AUXILIARY STEAM AND AUXILIARY BOILERS (1) 3.2.1 ASSIST IN STARTUP OF	• • <u>••••</u> ••••••••••••••••••••••••••••••	
	3.2.2 AUXILIARY BOILERS 1106-4 (D)		<u> </u>
	CONNECTION 1106-19 (D)	i <u>ala sa sa sa sa sa</u>	<u></u>
3.3	EXTRACTION STEAM (1) 3.3.1 EXTRACTION STEAM HEATE YENT AND DRAINS 1106-1 (D)	R 2	-
3.4	HEATER VENTS AND DRAIN (1)		
	3.4.1 PLACE HEATER DRAIN PUMPS IN SERVICE		
	3.4.2 EXTRACTION STEAM, HEAT VENT AND DRAINS 1106-1 (D)	ER 2	
3.5	FEEDWATER (1)		
	3.5.1 ASSIST IN STARTUP OF		
	3.5.2 FEEDWATER SYSTEM 106- (D)	3	
(1)	Systems Checkout. All signatu	res by licensed SRO or RO	

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		APPENDIX LICENSED OPERATOR	"B" OJT CHECKLIST	
3.0	SECONDAR	RY SYSTEMS PRACTICAL FACTORS	CONTINUÉD	
			TASK EXAMINER	DATE
	3.5.3	STARTUP AND PLACE A FEEDWATER PUMP IN SERVICE (P) OR (S)		
3.6	EMERGENC	Y FEEDWATER (1) ASSIST IN STARTUP OF THE SYSTEM (P) OR (S)		
	3.6.2	(D)		
5.7	CONDENSA	TE (1)		
	3.7.1	ASSIST IN STARTUP OF		
	3.7.2	HIGH CATION CONDUCTIVITY IN CONDENSATE 1203-5 (S)		
	3.7.3	CONDENSATE SYSTEM 1106-2		
3.8	CONDENSE	R AIR EXTRACTION (1)		
	3.8.1	ASSIST IN STARTUP OF THE SYSTEM (P) OR (S)		
	3.8.2	MAIN AND AUX. VACUUM SYSTEM 1106-15 (D)		
3.9	GLAND SE	AL (1)		
	3.9.1	ASSIST IN STARTUP OF		
	3.9.2	TURBINE GLAND SEAL STEAM SUPPLY SYSTEM 1106-10 (D)		
3.10	MAIN TUR	BINE - GENERATOR (1)		
	3.10.1	PLACE MAIN TURBINE ON LINE AND PLACE IN AUTO (INCLUDING WARMUP) (P) OP (S) (1106-1)		
	3.10.2	TURBINE TRIP TEST		
	3.10.3	1106-1.0 (P) OR (S) TURBINE TRIP 1203-3 (S)		
(1)	SYSTEM C	HECKOUT. ALL SIGNATURES BY	LICENSED SRO OR RO.	

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			APPENDIX LICENSED OPERATOR	"B" OUT CHECKLIST	
3.0	SECONDA	RY SYSTEM	S PRACTICAL FACTORS	CONTINUED	
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	3.10.4	TURBINE 1106-1 (GENERATOR		
3.11	TURBINE 3.11.1	LUBE OIL TURBINE 1106-9 ((1) LO PUMPING		
	3.11.2	ASSIST I	N STARTUP OF		
	3.11.3	TURBINE & SUPPLY	0IL CONDITIONER		
3.12	GENERAT 3.12.1	OR STATOR STATOR C 1106-7 (1	COOLING (1) OOLING SYSTEM D)		
3.13	GENERAT 3.13.1	OR CORE ME TURBINE (1106-1 A	DNITOR (1) GENERATOR PP. C.3 (D)		
3.14	GENERAT 3.14.1	OR GAS AND HYDROGEN	SEAL OIL AND		
	3.14.2	ASSIST IN	STARTUP OF		
3.15	GENERATI 3.15.1	HYDROGEN	SEAL OIL AND		
•	3.15.2	ASSIST IN	M STARTUP OF		-
3.16	ELECTRO 3.16.1	TURBINE H	CONTROL (1)		
	3.16.2	ASSIST IN	STARTUP OF EM (P) OR (S)		
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APPENDIX "B" LICENSED OPERATOR OUT CHECKLIST	
SECONDARY SUPPORT SYSTEMS	
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SECONDARY SERVICE RIVER WATER (1) 4.1.1 SSRW 1104-31 (D)	
MECHANICAL DRAFT COOLING TOWER (1) 4.2.1 MDCT 1104-37 (D) 4.2.2 ASSIST IN STARTUP OF SYSTEM (INCLUDING TEMP. CONTROL AND DEICING (P) OR (S)	
SCREENHOUSE EQUIPMENT (1) 4.3.1 SCREENHOUSE EQUIPMENT 1104-33 (D)	
CONDENSER CIRCULATING WATER (1) 4.4.1 CIRCULATING WATER 1104-9 (D) 4.4.2 ASSIST IN STARTUP OF SYSTEM (INCLUDING DEICING (P) OR (S)	
CIRCULATING WATER CHLORINATION AND CHEM. FEED (1) 4.5.1 CIRC. WATER CHLORINATION 1104-35 (D)	
AMERTAP (1) 4.6.1 CIRCULATING WATER 1104-9 (1)	
4.6.2 ASSIST IN STARTUP OF SYSTEM (P) OR (S)	
CYCLE MAKEUP PRETREATMENT (1) 4.7.1 CYCLE MAKEUP PRETREATMENT 1104-22 (D)	
SYSTEM CHECKOUT. ALL SIGNATURES BY LICENSED SRO OR RO.	
	MI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM MI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM MI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM MECHANICAL DRAFT COOLING TOWER (1) 4.1.1 SSRW 1104-31 (D) MECHANICAL DRAFT COOLING TOWER (1) 4.2.1 MOCT 1104-37 (D) MECHANICAL DRAFT COOLING TOWER (1) 4.2.1 MOCT 1104-37 (D) MECHANICAL DRAFT COOLING TOWER (1) 4.2.1 MOCT 1104-37 (D) MECHANICAL DRAFT COOLING TOWER (1) 4.2.1 MOCT 1104-37 (D) MECHANICAL DRAFT COOLING TOWER (1) 4.2.1 MOCT 1104-37 (D) MECHANICAL DRAFT COOLING TOWER (1) 4.2.1 MOCT 1104-37 (D) MECHANICAL DRAFT COOLING TOWER (1) 4.2.1 MOCT 1104-37 (D) MECHANICAL DRAFT COOLING TOWER (1) 4.3.1 SCREENHOUSE EQUIPMENT 1 1104-33 (D) CONDENSER CIRCULATING WATER (1) 4.4.1 CIRCULATING WATER (1) 4.5.1 CIRCULATING WATER (1) 4.5.1 CIRCULATING WATER (1) 4.6.2 ASSIST IN STARTUP OF SYSTEM (P) OR (S) CYCLE MAKEUP PRETREATMENT (1) 4.7.1 CYCLE MAKEUP PRETKEATMENT (1) 4.7.1 CYCLE MAKEUP PRETKEATMENT (1) 5YSTEM CHECKOUT. ALL SIGNATURES BY LICENSED SRO OR RO.

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4.0	APPENDIX "B" LICENSED OPERATOR OUT CHECKLIST SECONDARY SUPPORT SYSTEMS CONTINUED	
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4.8	CYCLE MAKEUP DEMINERALIZER (1) 4.8.1 CYCLE MAKEUP DEMINERALIZER 1104-23 (D)	
4.9	DEMINERALIZED WATER (1)	
4.10	DOMESTIC WATER (1) 4.10.1 DOMESTIC WATER 1104-49 (D)	
4.11	RECLAIMED WATER (1) 4.11.1 RECLAIMED WATER 1104-47A (D)	
4.12	INSTRUMENT/CONTROL AIR (1) 4.12.1 INST./CONT AIR 1104-25 (D) 4.12.2 LOSS OF INSTRUMENT AIR 1202-36 (S)	
4.13	STATION SERVICE AIR (1) 4.13.1 STATION SERVICE AIR 1104-42 (D)	
4.14	INDUSTRIAL WASTE FILTER SYSTEM (1) 4.14.; IWFS 1104-50B (D)	
1.15	INDUSTRIAL WASTE TREATMENT (1) 4.15.1 IWTS 1104-50A (D)	
.16	SECONDARY CHEMISTRY CONTROL (1) 4.16.1 OBSERVE DRAWING AND ANALYZING A SECONDARY SAMPLE.	
1)	SYSTEM CHECKOUT. ALL SIGNATURES BY LICENSED SRO OR RO.	

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APPENDIX "B" LICENSED OPERATOR OJT CHECKLIST 4.0 SECONDARY SUPPORT SYSTEMS CONTINUED TASK EXAMINER	DATE
4.17 SECONDARY SERVICE CLOSED COOLING	
(1)	
4.17.1 SSCC 1104-12 (D) 4.17.2 SSCC SYSTEM FAILUR: 1203-21 (S)	
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	APPENDI) LICENSED OPERATOR	WT CHECKLIST				
INSTRUM	MENTATION AND CONTROL					
		TASK EXAMINER	DATE			
CONTROL 5.1.1	ROD DRIVE SYSTEM (1) LOCATE AND OPERATE THE FOLLOWING SWITCHES ASSOCIATED WITH THE CONTROL ROD DRIVE SYSTEM 5.1.1.1 SAFETY RODS OUT BYPASS 5.1.1.2 GROUP 7 OUT LIMIT BYPASS 5.1.1.3 BORON DILUTE BYPASS					
5.1.2	POINT OUT AND DESCRIBE THE COMPONENTS IN THE CRD CABINETS					
5.1.3	CRD SYSTEM 1105-9 (D) CONTROL ROD MOVEMENT 1303-3.1 (P) OR (S)					
5.1.6	UNANTICIPATED CRITICALITY 1203-10 (S)		·			
EXCORE 5.2.1	NUCLEAR INSTRUMENTS (1) NUCLEAR INSTRUMENTS		·			
5.2.2	CHANGE INPUTS TO THE POWER RANGE RECORDER (P) SP 1302-1.1					
INCORE 5.3.1	NUCLEAR INSTRUMENTS (1) INCORE MONITORING SYSTEM					
5.3.2	INCORE NEUTRON DETECTORS SURVEILLANCE 1301-5.3 (P)					
SYSTEM	CHECKOUT. ALL SIGNATURES BY	LICENSED SRO OR RO.				
	INSTRUM CONTROL 5.1.2 5.1.2 5.1.3 5.1.4 5.1.5 5.1.6 EXCORE 5.2.1 5.2.2 INCORE 5.3.1 5.3.2 SYSTEM	APPENDIS LICENSED OPERATOR INSTRUMENTATION AND CONTROL CONTROL ROD DRIVE SYSTEM (1) 5.1.1 LOCATE AND OPERATE THE FOLLOWING SWITCHES ASSOCIATED WITH THE CONTROL ROD DRIVE SYSTEM 5.1.1.1 SAFETY RODS OUT BYPASS 5.1.1.2 GROUP 7 OUT L'MIT BYPASS 5.1.1.3 BORON DILUTE BYPASS 5.1.2 POINT OUT AND DESCRIBE THE COMPONENTS IN THE CRD CABINETS 5.1.3 CRD SYSTEM 1105-9 (D) 5.1.4 CONTROL ROD MOVEMENT 1303-3.1 (P) OR (S) 5.1.5 CRD EQUIPMENT FAILURES 1202=8 (S) 5.1.6 UNANTICIPATED CRITICALITY 1203-10 (S) EXCORE NUCLEAR INSTRUMENTS (1) 5.2.1 NUCLEAR INSTRUMENTS (1) 5.2.2 CHANGE INPUTS TO THE POWER RANGE RECORDER (P) SP 1302-1.1 INCORE NUCLEAR INSTRUMENTS (1) 5.3.1 INCORE MONITORING SYSTEM 1105-5 (D) 5.3.2 INCORE NEUTRON DETECTORS SURVEILLANCE 1301-5.3 (P) SYSTEM CHECKOUT. ALL SIGNATURES BY	APPENDIX "B" LICENSED OPERATOR OUT CHECKLIST INSTRUMENTATION AND CONTROL TASK EXAMINER CONTROL ROD DRIVE SYSTEM (1) 5.1.1 LOCATE AND OPERATE THE FOLLOWING SWITCHES ASSOCIATED WITH THE CONTROL ROD DRIVE SYSTEM 5.1.1.1 SAFETY RODS OUT BYPASS 5.1.1.2 GROUP 7 OUT L'MIT BYPASS 5.1.2 POINT OUT AND DESCRIBE THE COMPONENTS IN THE CRD CABINETS 5.1.3 CRD SYSTEM 1105-9 (D) 5.1.4 CONTROL ROD MOVEMENT 1303-3.1 (P) OR (S) 5.1.5 CRD EQUIPMENT FAILURES 1202-8 (S) 5.1.6 UNNANT CIPATED CRITICALITY 1203-10 (S) EXCORE NUCLEAR INSTRUMENTS (1) 5.2.1 NUCLEAR INSTRUMENTS (1) 5.2.2 CHANGE INPUTS TO THE POWER RANGE RECORDER (P) SP 1302-1.1 INCORE NUCLEAR INSTRUMENTS (1) 5.3.2 INCORE MONITORING SYSTEM 1105-5 (D) 5.3.2 INCORE NUTRON DETECTORS SURVEILLANCE 1301-5.3 (P) SYSTEM CHECKOUT. ALL SIGNATURES BY LICENSED SRO OR RO.			

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		APPENDIX LICENSED OPERATOR	"B" OJT CHECKLIST	
.0 I	NSTRUM	ENTATION AND CONTROL CONTINUE	D	
			TASK EXAMINER	DATE
.4 N	ON NUCI	LEAR INSTRUMENTS (1)		
5	.4.1	RCS NNI 1105-6 (D)		
5	.4.2	CHANGE INPUTS FOR THE CONTROLLING RCS PRESSURE SIGNAL (P) 1105-6-3.4.3		
5	.4.3	CHANGE INPUTS FOR THE CONTROLLING RC FLOW		
5	.4.4	SELECT THE CONTROLLING		
		TC'S FOR THE PUMP INTER-		
		LOCKS (P) 1105-6-3.4.5		
5	.4.5	RC PUMP INTERLOCKS (P)		
5	.4.0	ALL METERS, SWITCHES AND MODULES IN THE ICS/NNI CABINETS		
.5 R	EACTOR	PROTECTION SYSTEM (1)		
5	.5.1	RPS 1105-2 (D)		
5	.5.2	RESET REACTOR PROTECTIVE		
		CHANNELS AFTER A TRIP (P) OR (S)		
.5 RI	EACTOR	PROTECTION SYSTEM CONTINUED		
5.	.5.3	DETERMINE THE CAUSE OF A TURBINE TRIP (P) OR (S)		
5.	.5.4	DETERMINE WHICH RELAY		-
		CAUSED A GENERATOR TRIP		
		FROM THE RELAY TARGET		
5	5.5	POINT OUT AND DESCRIBE		
		ALL METERS. SWITCHES AND		
		MODULES IN THE RPS		
		CABINETS		
5.	.5.0	REACTOR TRIP 1202-4 (S)		-
1) 51	YSTEM	CHECKOUT. ALL SIGNATURES BY	LICENSED SRO OR RO.	

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5.0	INSTRU	MENTATION AND CONTROL CONTINUE	.0	
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5.6	EMERGE SYSTEM 5.6.1 5.6.2 5.6.3 5.6.4 5.6.5	NCY SAFEGUARDS ACUTATION (1) ESAS 1105-3 (D) EMERGENCY CHECKLIST (P) EMERGENCY SAFEGUARDS ACTUATION SURVEILLANCE (P) RESET AN ENGINEERED SAFEGUARDS CHANNEL AFTER A TRIP (P) OR (S) POINT OUT AND DESCRIBE ALL METERS, SWITCHES, AND MODULES IN THE E.S. CABINETS		
5.7	INTEGR4 5.7.1 5.7.2	ATED CONTROL SYSTEM (1) ICS 1105-4 (D) POINT OUT AND DESCRIBE ALL CONSOLE ICS INSTRUMENTATION INPUT SWITCHES, INDICATORS AND RECORDERS		=
	5.7.3	OPERATE ICS CONTROLS IN HAND (P)		
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	MI-I UIK	ECT SENTOR REACTOR OPERATOR TR	AINING PROGRAM	0-00
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5.0	ELECTR	ICAL SYSTEMS		
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5.1	BALANC	E OF PLANT ELECTRICAL (1)		
	6.1.1	ELECTRICAL SYSTEM		
	6.1.2	POINT OUT AND DESCRIBE		
		ELECTRICAL FLOWPATH IN		
		THE SWITCHYARD AND		
		RELAY HOUSE (INCLUDE		
	6.1.3	POINT OUT AND DESCRIBE		
		COMPONENTS AND THE		
		ELECTRICAL FLOWPATH FROM		
		THE MAIN TRANSFORMER		
		THROUGH 6900 VOLT AND		
		VOLT DISTRIBUTION (INI CUDE		
		INTERLOCKS & OPERATIONS)		
	6.1.4	DEMONSTRATE UNDERSTANDING		
		OF THE GUIDANCE FOR		
		OPERATING SWITCHYARD		
		CONTROL ROOM		
	6.1.5	WALKTHROUGH THE PROCESS		
		FOR TRANSFERRING VITAL		
		POWER BUSSES FROM NORMAL		
		TO BACKUP POWER SUPPLY		
	6.1.6	EMERGENCY POWED		
		1303-4.16 (D)		
	6.1.7	BLACKOUT 1202-2 (D)		
	6.1.8	BLACKOUT 1202-2A (D)		-
	0.1.9	LOW SYSTEM (GRID) VOLTAGE		
	6.1.10	1203-41 (D)		
		(D)		
	6.1.11	EMERGENCY ELECTRICAL		
		SYSTEMS 1107-2 (D)		
))	SYSTEM	CHECKOUT. ALL SIGNATURES BY L	ICENSED SRO OR RO.	

APPENDIX LICENSED OPERATOR CTRICAL SYSTEMS CONTINUED ANCE OF PLANT ELECTRICAL CONTINUE SEL GENERATORS (1) 1 STARTUP AND SECURE A DIESEL OPERATOR, PLACE IN ES STANDBY (P) OR (S) 2 DIESEL GENERATOR 1107-3 (D)	B" OJT CHECKLIST TASK EXAMINER D	DATE
DIRECT SENIOR REACTOR OPERATOR TH APPENDIX LICENSED OPERATOR CTRICAL SYSTEMS CONTINUED ANCE OF PLANT ELECTRICAL CONTINUE SEL GENERATORS (1) .1 STARTUP AND SECURE A DIESEL OPERATOR, PLACE IN ES STANDBY (P) OR (S) .2 DIESEL GENERATOR 1107-3 (D)	B" OJT CHECKLIST TASK EXAMINER	<u>DATE</u>
APPENDIX LICENSED OPERATOR CTRICAL SYSTEMS CONTINUED ANCE OF PLANT ELECTRICAL CONTINUE SEL GENERATORS (1) .1 STARTUP AND SECURE A DIESEL OPERATOR, PLACE IN ES STANDBY (P) OR (S) .2 DIESEL GENERATOR 1107-3 (D)	B" OJT CHECKLIST TASK EXAMINER	DATE
ANCE OF PLANT ELECTRICAL CONTINUE SEL GENERATORS (1) .1 STARTUP AND SECURE A DIESEL OPERATOR, PLACE IN ES STANDBY (P) OR (S) .2 DIESEL GENERATOR 1107-3 (D)	D	DATE
ANCE OF PLANT ELECTRICAL CONTINUE SEL GENERATORS (1) .1 STARTUP AND SECURE A DIESEL OPERATOR, PLACE IN ES STANDBY (P) OR (S) .2 DIESEL GENERATOR 1107-3 (D)	D	DATE
ANCE OF PLANT ELECTRICAL CONTINUE SEL GENERATORS (1) .1 STARTUP AND SECURE A DIESEL OPERATOR, PLACE IN ES STANDBY (P) OR (S) .2 DIESEL GENERATOR 1107-3 (D)	D	
Contraction of the second s		
VAC VITAL POWER AND D.C. TRIBUTION (1) 1 POINT OUT AND DESCRIBE COMPONENTS AND ELECTRICAL' FLOWPATH IN THE 120 VAC VITAL DISTRIBUTION SYSTEM. BE ABLE TO DISCUSS INTER- LOCKS AND OPERATION. (INCLUDE PLACING AN INVERTER IN SERVICE) 2 POINT OUT AND DESCRIBE COMPONENTS AND THE ELECTRICAL FLOWPATH IN THE 125/250 VAC DIST. SYSTEM. BE ABLE TO DISCUSS INTERLOCKS AND OPERATION. (INCLUDE PLACING CHARGER IN SERVICE).		
ATED PHASE BUS DUCT COOLING		
FICATION:	/DATE	
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	FICATION: SHIFT SUPERVISOR	FICATION: SHIFT SUPERVISOR /DATE EM CHECKOUT. ALL SIGNATURES BY LICENSED SRO OR RO.

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		WICH INTATAL PROBAN	0-66
	AP	PENDIX "B"	
	LICENSED OPE	ERATOR OUT CHECKLIST	
7.0	BUILDING SUPPORT SYSTEMS		
		TASK EXAMINER	DATE
7.1	REACTOR BUILDING EMERGENCY COOLING (1) RB EMER, COOLING 1104-38 (D)		
			-
1.2	7.2.1 RE SPRAY 11(2-14 (D)		and the second second
	7.2.2 INITIATE, MAKE AND	· · · · · · · · · · · · · · · · · · ·	-
	COMPLETE A RB PURGE (P) OR (S) (1102-14)		
7.3	PENETRATION FRESSURIZATION (1)		
	7.3.1 PENETRATION COOLING	·	
	7.3.2 PENETRATION PRESSURITA	TION	 Contraction 2011 (Section 2011)
	1104-21 (D)		
.4	REACTOR BUILDING - HVAC (1)		
	7.4.1 RB RECIRCULATION		
	7.4.2 REACTOR COMPARTMENT		
	SYSTEM 1104-14C (D)		
	7.4.3 STEAM GEN.COMPARTMENT		
	7.4.4 OPERATING FLOOR		
	VENTILATION SYSTEM		
	1104-14B (D)		
.5	AUXILIARY & FUEL HANDLING-HVAC		
	7.5.1 AUX. & FH BUILDING		
	SUPPLY AND EXHAUST		
	1104-75A (D) 7.5.2 SPENT FUEL DUMP ADEA		
	1104-15B (S)		
	7.5.3 NSCC AND DECAY HEAT		•/
	PUMP 1104-15C (D)		
1)	SYSTEM CHECKOUT. ALL SIGNATURE	ES BY LICENSED SRO OR RO.	

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6	ADMINISTRATIVE MANUAL	6211-ADM-2611.0
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TI	MI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING	PROGRAM 0-00
7.0	APPENDIX "B" LICENSED OPERATOR OUT CHEC	X,L I ST
1.0	BUILDING SUPPORT STSTEMS CONTINUED	
	TAS	EXAMINER DATE
7.6	CONTROL BUILDING AND CONTROL ROOM HVAC (1) 7.6.1 CONTROL ROOM HVAC 1203-24 (D) 7.6.2 CONTROL BUILDING VENTILATION SYSTEM 1104-15 (D)	
7.7	SERVICE BUILDING - HVAC (1) 7.7.1 SERVICE BUILDING 1104-24D (D)	
7.8	REACTOR BUILDING NORMAL COOLING (1)	
7.9	FIRE PROTECTION SYSTEM (1) 7.9.1 FIRE SERVICE WATER SYSTEM 1104-45B (D) 7.9.2 FIRE SERVICE SPRINKLER SYSTEM 1104-45C (D) 7.9.3 FIRE SERVICE DELUGE SYSTEM 1104-45D (D) 7.9.4 WALK THROUGH YOUR RESPONSE TO FIRE ALARM ANNUNICATORS	
7.10	DIESEL GENERATOR BUILDING - HVAC	
	7.10.1 DIESEL GENERATOR BUILDING 1104-24M (S)	
INAL	VERIFICATION:	
	SHIFT SUPERVISOR	/DATE
1)	SYSTEM CHECKOUT. ALL SIGNATURES BY LICENSED	SRO OR RO.

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TMI-1 DIRECT SENIOR REACTOR OPERATOR OPERATOR TRAINING PI APPENDIX "B" LICENSED OPERATOR OJT CHECK 8.0 REACTOR MANIPULATIONS TASK 8.0 REACTOR MANIPULATIONS TASK 8.1 REACTIVITY CALCULATIONS 8.1.1 SHUTDOWN AND OPERATING REACTIVITY CALCULATIONS 8.1.1 SHUTDOWN AND OPERATING REACTIVITY CALCULATIONS 8.1.1 SHUTDOWN AND OPERATING REACTIVITY CALCULATIONS 8.1.2 BORON CHANGE REACTIVITY BALANCE 1103-15 (P) 8.1.3 SHUTDOWN MARGIN CALCULATION 1103-15 (P) 8.1.4 XENON POWER BLOCK 1103-15 (P) 8.2.1 CREACTIVITY CHANGES THE TRAINEE DURING HIS ON-THE-JOB TRAINING PHASE WILL PERFORM AT LEAST FIVE (5) REACTIVITY CHANGES AS DESCRIBED BELOW, WITH NO MORE THAN FOUR (4) BEING ANY COMBINATION OF ITEMS 2 AND 5. REACTIVITY CHANGES WILL BE DOCUMENTED ON THIS SHEET 8.2.1 CRITICAL APPROACH FROM SUBCRITICAL ON SOURCE DANCE INSTRUMENTION TO <th>OGRAM 0-DO</th>	OGRAM 0-DO
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8.0 REACTOR MANIPULATIONS TASK 8.1 REACTIVITY CALCULATIONS 8.1.1 SHUTDOWN AND OPERATING REACTIVITY BALANCE 1103-15 (P) 8.1.2 BORON CHANGE REACTIVITY BALANCE 1103-15 (P) 8.1.3 SHUTDOWN MARGIN CALCULATION 1103-15 (P) 8.1.4 XENON POWER BLOCK 1103-15 (P) 8.2 REACTIVITY CHANGES THE TRAINEE DURING HIS ON-THE-JOB TRAINING PHASE WILL PERFORM AT LEAST FIVE (5) REACTIVITY CHANGES AS DESCRIBED BELOW, WITH NO MORE THAN FOUR (4) BEING ANY COMBINATION OF ITEMS 2 AND 5. REACTIVITY CHANGES WILL BE DOCUMENTED ON THIS SHEET 8.2.1 CRITICAL APPROACH FROM SUBCRITICAL ON SOURCE PANCE INSTDUMENTATION TO	EXAMINER DATE
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8.2 REACTIVITY CHANGES THE TRAINEE DURING HIS ON-THE-JOB TRAINING PHASE WILL PERFORM AT LEAST FIVE (5) REACTIVITY CHANGES AS DESCRIBED BELOW, WITH NO MORE THAN FOUR (4) BEING ANY COMBINATION OF ITEMS 2 AND 5. REACTIVITY CHANGES WILL BE DOCUMENTED ON THIS SHEET 8.2.1 CRITICAL APPROACH FROM SUBCRITICAL ON SOURCE PANCE INSTRUMENTATION TO	
CRITICAL AT THE POINT OF ADDING HEAT ON THE INTERMEDIATE RANGE	
8.2.2 ANY POWER LEVEL CHANGE (INCREASE OR DECREASE) OF 10 PERCENT OF RATED POWER OR GREATER WITH CONTROL BODS IN MANUAL	
8.2.3 REACTOR SHUTDOWN FROM CRITICAL AT 15 PERCENT OF RATED TO SUBCRITICAL SHUTDOWN ON SOURCE RANGE INSTRUMENTATION	
8.2.4 BORATION OR DEBORATION DURING CRITICAL OPERATION	

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т	MI-1 DIRECT SENIOR REACTO	R OPERATOR TRAINING PROGRAM	0.00
		APPENDIX "B"	
	LICENS	SED OPERATOR OUT CHECKLIST	
8.0	REACTOR MANIPULATIONS C	ONTINUED	
		TASK EXAMINER	DATE
8.2	REACTIVITY CHANGES CONT: 8.2.5 OPERATION OF REI BRIDGE TO CHANGE GEOMETRY DURING REFUELING	INUED FUELING E CORE	
3.3	CORE POWER 8.3.1 MONITOR AND CALC QUADRANT POWER T	CULATE	
	8.3.2 MONITOR AND CALC CORE IMBALANCE 1	CULATE 203-7(P)	
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T	MI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM	0-00
9.0	APPENDIX "B" LICENSED OPERATOR OUT CHECKLIST SITE EMERGENCIES	
	TASK EXAMINER	DATE
9.1	PLANT RESPONSE TO PENETRATION OF PROTECTED AREA 1202-13 (D)	
.2	FIRE 1202-31 (D)	
.3	FL00D 1202-32 (D)	
.4	EARTHQUAKE 1202-30 (D)	
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TI	MI-1 DIRECT SENIOR REACTOR OPERATOR	TRAINING PROGRAM	0-00
10.0	APPEND LICENSED OPERATION	OIX "B" OR OUT CHECKLIST	
	PLANT COMPUTER FOR:	and a strength of the	
10.1	LEAKAGE CALCULATIONS (D)	TASK EXAMINER	DATE
10.2	SAXON PROGRAM ACCESS (P)		
10.3	CHANGING TREND RECORDER POINTS (P)		
0.4	FINDING ALARM POINTS (P)		
0.5	DISPLAY AND PRINTOUT OF VARIOUS GROUPS (P)		
0.6	OPERATOR TREND GROUPS (P)		
0.3	CHANGING TREND RECORDER POINTS (P)		
0.4	FINDING ALARM POINTS (P)		
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		APPENDIX "B LICENSED OPERATOR OJ	" CHECKLIST	
11.0	DEMONSTRATE THE PLANT COMPUTER F	USE OF THE OR:		
			TASK EXAMINER	DATE
11.1	COOLDOWN OUSIDE ROOM 1202-37 (D)	CONTROL		
1.2	PLANT STARTUP 110 AND 1103-8 (D)			
1.3	PLANT SHUTDOWN 1	102-10 (D)		
1.4	PLANT COOLDOWN 1	102-11 (D)		
1.5	PLANT HEATUP TO	25° 1102-1 (D) ·		
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		NAME :		
	PROCEDURE	QUAL	SIGNATURE	DATE
•	DOCUMENT CONTROL 1001	(1)		
2.	Procedure Revisions and Approval 1001A	(1)		
3.	Procedure Preparation 1001D	(1)		
	Procedure Utilization 1001G	(1)	and a start of the	
	Periodic Review of Procedures 1001K	. (1)		
i.	Station Organization and Chain of Command 1009	(1)		
	Technical Specifications Surveillance Program 1010	(1)		
	Controlled Key Locker Control 1011	(1)		
•	Equipment Control (Locking) AP 1001	(1)		
0.	Record Retention AP 1024	(1)		
1.	Equipment Control (Tagging) AP 1037	(1)		
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		NAME :		
	PROCEDURE	QUAL	SIGNATURE	DATE
9.	SHIFT RELIEF AND LOG ENTRIES	(1)		
10.	BYPASS AND SAFETY FUNCTIONS AND JUMPER CONTROL 1013	(1)		
11.	CORRECTIVE MAINTENANCE AND MACHINERY HISTORY 1026	(1)		
2.	PREVENTIVE MAINTENANCE 1027	(1)		
3.	OPERATOR AT THE CONTROLS 1028	(1)		
4.	CONDUCT OF OPERATIONS 1029	(1)		
5.	NUCLEAR PLANT STAFF WORKING HOURS 1031	(1)		
6.	OPERATIONS MEMOS AND STANDING ORDERS 1033	(1)		
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THE TMI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM	Revision No. 0-00
APPENDIX "C" DIRECT SRO - PROCEDURE QUALIFICATION ADMINISTRATIVE PROCEDURES	
READ AND DISCUSS THE FOLLOWING ADMINISTRATIVE PROCEDURES.	

		NAME:		
	PROCEDURE	QUAL	SIGNATURE	DATE
17.	INSTRUMENTS OUT-OF-SERVICE CONTROL 1036	(1)		
18.	CONTROL OF CAUTION AND DNO TAGS 1037	(1)		
19.	EVENT REVIEW AND REPORTING REQUIREMENTS 1044	.(1)		
20.	RULES FOR THE PROTECTION OF EMPLOYEES WORKING ON ELECTRICAL			
	AND MECHANICAL APPARATUS 1002	(1)	water and the second second	and the second

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APPENDIX "C" DIRECT SRO - PROCEDURE QUALIFICATION CODE OF FEDERAL REGULATIONS

READ AND DISCUSS THE FOLLOWING SECTIONS OF THE CODE OF FEDERAL REGULATIONS.

1.1		H/W.E.		
	CODE	QUAL	SIGNATURE (1)	DATE
1.	NOTICES, INSTRUCTIONS AND TO WORKERS: INSPECTION 10 CFR 19	(2)		
2.	STANDARDS FOR PROTECTION AGAINST RADIATION 10 CFR 20	(2)		
3.	REPORTS OF DEFECTS AND NON- COMPLIANCE 10 CFR 21	.(2)	ident das 197	
4.	DOMESTIC LICENSING OF PRODUCTION AND UTILIZATION FACILITIES 10 CFR 50	(2)		
5.	OPERATORS' LICENSE 10 CFR 55	(2) -		-
б.	PHYSICAL PROTECTION OF PLANTS AND MATERIALS 10 CFR 73	(2)		
7.	REACTOR SITE CRITERIA 10 CFR 100	(2)		
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NOTE 1: YOUR SIGNATURE INDICATES THAT THE CANDIDATE DEMONSTRATED SATISFACTORY KNOWLEDGE IN THE SUBJECT AREA. NOTE 2: SRO

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APPENDIX D: NON-LICENSED OPERATOR QUALIFICATION CHECKLIST SECTION : OUTBUILDINGS

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THE THI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM

In 0P5-5123 FS-P-6 Test 2 0P5-5144 Sewaye Lift System 3 0P5-5148 Site Air Compressors and Air Handler 4 0P5-5149 Air Handler For Site Air Cumpressor 5 1104-22 Manually Desludye Pretreatment 7 3303-MI Run the Circ Water Diesel 7 3303-MI Run the Circ Water Diesel 7 3303-MI Run the River Water Diesel 7 3303-MI Run the River Water Diesel 7 3104-22 Properiy Fill Ail Chemical Addition Tanks in Pretreat. 9 1104-22 Switch Pretreatment Manually 0 1104-22 Switch Pretreatment Manual to Auto 1 1104-32 Switch Pretreatment From Nanual to Auto 1 1104-31 Switch Pretreatment From Nanual to Auto 1 1104-31 Switch Pretreatment From Nanual to Auto 1 1104-31 Switch Pretreatment From Natural to Auto 1 1104-31 Switch Pretreatment From Natural to Auto 1 1104-31 Switch Pretreatment From Natural to Auto 1 N/A Open and Close Disconnects In Switchyar	ANE :	BOACCHIDE	SOCIAL SECURITY NUMBER:	TASK E	XAMINER SIG.	EMPLOYEE NUMBEI
20P5-5144Seuage Lift System30P5-5144Site Air Compressors and Air Handler40P5-5149Air Handler For Site Air Cumpressor51104-22Hanually Desludye Pretreatment73303-MIRun the Circ Water Diesel73303-MIRun the River Water Diesel73303-MIRun the River Water Diesel71104-22Operate Pretreatment Manually91104-22Svitch Pretreatment Manually11104-22Svitch Pretreatment Manually11104-35/36Chanye Chlorine Bottle (River Water or Circ. Water)11104-35/36Chanye Chlorine Bottle (River Water or Circ. Water)11104-31Svitch M.D.C.T.Fans for Delcelng Operating11104-31Svitch M.D.C.T.Fans for Delcelng Operating1N/AOpen and Close Disconnects in Svitchyard11104-31Reset A Fire Deluge Valve	I	PROCEDUKE 0PS-S123	FS-P-6 Test			
3OPS-S148Site Air Compressors and Air Handler4OPS-S148Site Air Compressors and Air Handler51104-22Hanually Desludye Pretreatment73303-MIRun the Circ Water Diesel73303-MIRun the River Water Diesel73303-MIRun the River Water Diesel81104-22Properly Fill All Chemical Addition Tanks in Pretreat.91104-22Operate Pretreatment Manually01104-22Switch Pretreatment Manual to Auto11104-32Switch Pretreatment Manual to Auto11104-31Manually Operate Screens and Rakes31104-33Switch H.D.C.T.Fans for Delceing Operating11104-31Switch H.D.C.T.Fans for Delceing Operating11104-31Reset A Fire Deluge Valve	2	0PS-S144	Sewaye Lift System			
4OPS-S149Air Handler For Site Air Cupressor51104-22Manually Desludye Pretreatment73303-MIRun the Circ Water Diesel73303-MIRun the River Water Diesel7104-22Properly Fill Ail Chemical Addition Tanks in Pretreat.91104-22Operate Pretreatment from Nanual to Auto11104-22Switch Pretreatment from Nanual to Auto11104-22Switch Pretreatment from Manual to Auto11104-22Switch Pretreatment from Ranual to Auto11104-35/36Chanye Chlorine Bottle (River Water or Circ. Water)21104-31Switch M.D.C.T.Fans for Deiceing Operating31104-31Switch M.D.C.T.Fans for Deiceing Operating4M/AOpen and Close Disconnects in Switchyard11104-450Reset A Fire Deluge Valve	1	OPS-5148	Site Air Compressors and Air Handler			
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APPENDIX G: NON-LICENSED OPERATOR QUALIFICATIO Section : OUTBUILDINGS	SOCIAL SECURITY NUMBER:	Manual IV Buckness and and	Operate I.W.T.S.	Operate I.W.F.S.	Line up Circ. Water Chlorination & Chemical Add. for Op.	Take Outbuilding Readings		•						
	PBACENIEC	1104-22	1004-50A	1104-508	1104-35									
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APPENDIX E: NON-LICENSED OPERATOR CHEC	TASK	Cation Demin. Resin Replacement	Decant. Spent Resin and Used Precoat Tank	Hanuelly Purge RH-A-7	Startup, Operate, & Shutdown, Misc. Evap.	Perform A Liquid Release	Perform A Gas Release	Manually Flush R4-L-6	Place R.C. Bleed Tank on Cleanup	Place BWST on Cleanup	LWST Transfer	Place Seal Injection Filter In Service	Remove Seal Injection Filter From Service	Adjust Seal Injection Flow to RCP's	Adjust #1 Seal Return Back Pressure	Adjust Makeup Bypass Flow	-	
	PROCEDURE	1104-51	1164-29N	1104-27	1104-290	1104-295	1104-27	1104-295	1104-294	1104-290	1104-29K	1104-2	1104-2	1104-2	1104-2	1104-2		
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APPENDIX E: NON-LICENSED OPERATOR CHECKLIST Section : PRIMARY

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TMI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM

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31	1104-2	Transfer HUP Cooling From DC to NSCC			
32	1104-2	Seal Return Filter Replacement			
33	1104-4	Add 0il to DH-P-IN/B		-	
34	1507-3	Move Fuel with Spent Fuel Bridge		_	
35	1507-5	Move a Control Rod with S.F. Bridge		1	
36	1507-5	Operate & Shutdown Fuel Handling Bridge		1	
37	1-1051	Operate Fuel Transfer System		1 .	
38	1104-62	Startup, Operate & Shutdown H-2 Recombiner			1
39		Reset Damper. in Air Intake Tunnel			
40	1104-45 1	Remove Air Intake Tunnel Halon System From Service			
41	1104-45 1	Return Air Intake Tunnel Halon System To Service		1	1
42	1104-4	Place Decay Heat Removal In Service			
43	1104-4	Lineup Pressurizer Aux. Spray			
**	1104-478	Adjust CA-P-1 Stroke		1	
				1	
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APPENDIX E: NON-LICENSED OPERATOR CHEC SECTION : PRIMARY	SOCIAL SECURITY NUMBER:	TASK	Lineup BMIT to all Possible Paths	Mix Boric Acid in 7% Tank	Lineup RBAI to Makeup System	Press. CF1's	Open & Close RB Doors	Open and Close Aux. Bldg. Doors	Adjust NSCC to RCP Coolers	Put 0156's on Recirc.	Change CRDM Filters (A-8 or 8-A)	Primary Readings			
		PROCEDURE	1104-478	1104-478	1104-29E	1104-1			11-1011	1106-16	1104-8				
	UAME :	MUMBER	45	46	47	48	49	50	15	52	53	54			

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APPENDIX F: NON LICENSED OPERATOR CHECKLIST SECTION : SECONDARY

WE:		SOCIAL SECURITY NUMBER:	CMDI DVC	CC MANNE	
NUMBER	PROCEDURE	I'n.K	TACK FYAMINED SIC	CE NUISE	
-	1104-23	Startup INT			MUNH
2	1104-23	Shutdown IUT			1
	1104-23	Regenerate Cation and Anion String on IVI			
-	1104-23	Regenerate Mixed Bed on INT			
5	1104-18	Neutralize [U] Waste Tank			
9	1104-18	Fill and Flush INT Chem. Moat			
1	1104-18	Determine Level of INT Vacuum Pumps			
89	1104-18	Take on Acid or Caustic For IW			
6	1104-23	Demonstrate Manual Operation of Air Valves at INT			
10	065-5001	Main Generator Core Monitur Annual			
=	0PS-S010	Main Generator Core Monitor Weekly			
12	0P5-5012	Amertap System Ball Inspection & Pump Lubrication			
13	0P5-5029	Electro-Hydraulic Control System Surveillance			
14	0P5-5032	EG-Y-1A/8 Fuel Pump/Air Start Surveillance		T	
15	005-5041	Instrument Air Compressors IA-P-IA/R			

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NAME : SOCIAL SECURITY NUMBER: ENPLOYEE WUMBER: NUMBER PROCEDURE TASK TASK EXAMINER SIG. DATE HOURS 31 Rack Out And Rack In 4160 Volt Brk 32 1106-2 Place 8th Stage Heating On Line 33 1106-8 Purge Generator With CO-2 34 1105-8 Fill Generator with H2. 35 1106-8 Fill Generator With I.A. 36 1106-10 Place Gland Seal Steam In Operation 37 1106-4 Place an Aux Boller In Service 38 1104-9 Adjust Circ Water Blowdown 39 1303-4.16 An Emergency Diesel Generator In E.S. St/By 40 1303-4.16 Perform Emergency Diesel Run Surveillance 41 Place Turbine Bypass EF-V-30A/B Valve In Manual Auto 1300-3F A/B Test Run EF-P-2 A/B 42 43 1300-3G A/B Test Run EF-P-1 44 Rdgs.Sheet Complete Secondary Readings . TOTAL HOURS THIS PAGE

APPENDIX F: NON LICENSED OPERATOR CHECKLIST SECTION : SECONDARY

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TMI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM

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	APPENDIX ' WATCHSTANDING C	"G" HECKLIST	
PERF	FORM THE FOLLOWING EVOLUTIONS AND ATTACH	H COMPLETED REA	ADING SHEETS
			CANDIDATES SIGNATURE
1.	TAKE AD OUTBUILDING READINGS		OF ON COPPLETION
	FOR ALL SHIFTS	(7-3) (3-11) (11-7)	
2.	TAKE AD PRIMARY PLANT READINGS	17.01	
	FOR ALL SHIFTS	(7-3) (3-11) (11-7)	
	TAKE AD SECONDARY PLANT READINGS		
	FOR ALL SHIFTS	(7-3) (3-11)	
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CANDIDATE:	APPENDIX "H" ORAL EXAM SUMMARY SHEET CONTINUE EXAMINER:	D DATE:
SIGNATURE OF EXAMIN	ER* REVIEWED BY:	ICENSED OPER TENC
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	TMI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM	0-00
	APPENDIX "I" SHIFT FOREMAN TRAINEE DUTIES CHECKLIST	
		SIGNATURE (SF/SS)
1.	CONDUCT SHIFT RELIEF AS A SHIFT FOREMAN	
2.	MAKE A LEFT HAND LOG ENTRY IN ACCORDANCE WITH AP 1012	
3.	REVIEW ONE COMPLETE SET OF OPERATOR READING SHEETS	
4.	DEMONSTRATE PROPER HANDLING OF A CHEMICAL DEVIATION SHEET	
5.	DEMONSTRATE PROPER HANDLING OF TECH SPEC SURVEILLANCE DATA AND SCHEDULES.	
6.	DEMONSTRATE PROPER HANDLING OF OPS SURVEILLANCE DATA AND SCHEDULING	
7.	DEMONSTRATE PROPER HANDLING OF ISI DATA IN THE ALERT RANGE	
3.	DEMONSTRATE CORRECT HANDLING OF WORK REQUEST (COMPLETION/INITIATION)	
	DEMONSTRATE PROPER HANDLING OF QA HOLD POINT STAMP	
0.	MAKE THE PROPER LOG AND CHECKLIST ENTRIES NECESSARY TO COMPLETE A VALVE LINEUP CHANGE WITHOUT A TCN.	
1.	AUDIT THE TCN AND STP BOOKS	
2.	PROPERLY ISSUE A JUMPER OR LIFTED LEAD	
3.	DEMONSTRATE KNOWLEDGE OF ALL COMMUNICATION SYSTEMS INCLUDING NRC PHONES NAWAS LINES, AND EMERGENCY PLANNING PHONED	
4.	DEMONSTRATE PROPER METHOD OF RECALL OF FLANT PERSONNEL INCLUDING USE OF RECORD-A-PHONE	
5.	PROPERLY APPROVE AND DISCUSS TESTING REQUIREMENTS FOR ESAS COMPONENT TAG OUTS.	
6.	CONDUCT A CREW TURNOVER BRIEFING	
7.	DEMONSTRATE KNOWLEDGE OF DUTIES OF MEMBERS ON THE FIRE BRIGADE	

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ADMINISTRATIVE MANUAL	6211-ADM-2611.0
Title	Revision No.
TMI-1 DIRECT SENIOR REACTOR OPERATOR TRAINING PROGRAM	0-00
APPENDIX "J" DESIGNATION OF FOREMAN AS FINAL OJT EXAMINER	
	DATE:
SHIFT SHIFT FOREMAN, are hereby designate	ed to serve as
the final verification examiner for Replacement SRO's on	Shift for QJT
sections noted below:	
List Sections:	
SHIFT, SHIFT SU	IPERVISOR
c: Operator Training Section	

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APPENDIX "K" SIMULATOR TRAINING

A nuclear plant simulator provides a means of training individuals to control the plant during routine conditions and to cope with nuclear plant transients and accidents. This training should prepare the candidate to supervise proficiently routine evolutions and carryout abnormal/emergency actions for the control room. Candidates shall practice manipulating the controls of an applicable simulator as described in reference 5.6.

As a minimum th license candidate shall participate in training sessions that include those listed below. The use of a simulator for performance of reactivity manipulations addressed in this program is authorized.

The following control manipulations and plant evolutions are required: Those control manipulations which cannot be performed at the plant shall be performed on a simulator.

- Plant or reactor startups to include a range that reactivity feedback from nuclear heat addition is noticable and heatup rate is established.
- (2) Plant shutdown.
- (3) Manual control of steam generators or feedwater, or both, during startup and shutdown.
- (4) Any significant (10 percent) power changes due to manual changes in control rod position or recirculation flow.
- (5) Loss of coolant including:
 - (a) Significant steam generator tube leaks
 - (b) Inside and outside primary containment
 - (c) Large and small, including leak determination
 - (d) Saturated reactor coolant response
- (6) Loss of core coolant flow/natural circulation.
- (7) Loss of all feedwater (normal and emergency).
- (8) Boration or dilution, or both during power operation.
- (9) Any reactor power change of 10 percent or greater where load change is performed with load limit control.
- (10) Loss of instrument a'r (if simulated plant specific).
- (11) Loss of electrical power (or degraded power sources, or both)
- (12) Loss of condenser vacuum.
- (13) Loss of service water if required for safety.
- (14) Loss of shutdown cooling.

FORM-1000-ADM-1218 01-2 (11/82)

- (15) Loss of component cooling system or cooling to an individual component.
- (16) Loss of protective system channel.
- (17) Mispositioned control rod or rods (or rod drops)
 - (18) Inability to drive control rod
 - (19) Condition requiring use of emergency boration or standby liquid control system.

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(20) (21) (22) (23) (24) (25)	Fuel cladding failure or high activity in reactor coo gas. Turbine or generator trip. Loss of normal feedwater or normal feedwater system f Malfunction of automatic control system(s) which affe Malfunction of reactor coolant pressure/volume contro Reactor trip	ailure. ect reactivity. I system.
(26) (27)	Main steam line break (inside or outside containment) Nuclear instrument failure(s).	•
articipat anipulatir lant exerc	ion at simulators shall be in groups of no more than f ng the controls or directing the activities of individ tises.	our (4) people uals during
n examinat ower with ertificati (1) (2) (3) (4) (5)	tion using the simulator shall be conducted while at plant malfunctions and while starting up the reactor. ion examination shall demonstrate the candidates abili Manipulate the controls in a safe and competent manne Predict instrument response and use instrumentation a Follow the facility procedures. Recognize significance of alarms and annunicators, di problems they represent, and respond with appropriate Communicate properly and effectively.	operating The ty to: r. vailable. agnose the actions.
startup c ompleted o uthor:zed.	ertification is required by each candidate who has no ne for Unit 1. The use of the simulator for this req	t previously uirement is

LICENSED OPERATOR REQUALIFICATION TRAINING PROGRAM DESCRIPTION

TMI - 1

Submitted: 17/7/81

S. L. Newton

DATE

Operator Training Manager

Approved: Klikeleek

R. A. Knief

Date

Manager, Plant Training

Son 17/25/81 Approved: M. J. Ross Date

Manager of Operations

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Appendices

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1.0 PURPOSE

The goal of the operator requalification program is to enhance nuclear plant safety and reliability by maintaining a high level of skill and knowledge in licensed senior operators and licensed operators.

To achieve this goal, the operator requalification program is designed to be sufficiently broad in scope to review areas of knowledge necessary for safe plant operation and flexible enough to cover recent operating experience and operational changes so that proficiency can be enhanced and operational safety maintained.

The regualification program described in this document is implemented utilizing four interrelated segments. These segments are:

- 1) Pre-Planned Lecture Series
- 2) Skills Training and Evaluation
- 3) Operational Review Program
- 4) Annual Examination at 1 Evaluation

Each program segment is described separately. The operator requalification program shall be conducted on a cyclic basis so that all program requirements are completed in a period not to exceed two years.

Successive requalification programs shall be conducted on a schedule enabling a continuing program to exist.

The operator requalification program shall be established with fixed performance standards and specified remedial training requirements in the event of deficiencies occurring. The program training materials, performance results and records shall be maintained in a fully auditable manner.

Guidelines percaining to duration of training in the requalification program are estimates of the time needed for retraining of licensed personnel. The training sessions needed to cover special nuclear plant operating situations or significant nuclear industry operating experiences may result in adjustments to these estimates. The duration of training shall also be adjusted as appropriate in order to assure that licensed personnel performance meets or exceeds established performance standards.

2:0. REFERENCES

a)	10 C	FR 55	, Append	ix A,	Draft	Revision	5/14/80
----	------	-------	----------	-------	-------	----------	---------

- b) NRR letter of March 28, 1980 on Qualifications of Reactor Operators
- c) NUREG 0737, Enclosure 3
- d) ANE 3.1, Draft Revision 10/80

- e) Reg. Guide 1.8, Draft Revision 9/80
- f) INPO-Nuclear Power Plant Regualification Program of Licensed Personnel

3.0 DEFINITIONS

The definitions given below are of a restricted nature for the purpose of this program.

<u>Drill</u> - A supervised training exercise conducted in a work environment for the purpose of developing and maintaining skills required to cope with plant abnormal/emergency conditions and including an evaluation of performance.

<u>Contact Hour of Instruction</u> - A one hour period in which the course instructor is present or immediately available for instructing or assisting students: lectures, seminars, discussions, problem solving sessions, and examinations are considered contact periods under this definition.

Licensed Operator (RO) - Any individual who possesses an operator's license pursuant to Title 10, Code of Federal Regulations, Part 55, "Operators' Licenses".

Licensed Senior Operator (SRO) - Any individual who possesses a

senior operator's license pursuant to Title 10, Code of Federal Regulations, Part 55, "Operators' Licenses".

<u>Annual</u> - As referred to in the operator requalification program, is twelve (12) months, not to exceed fifteen (15) months, in order to accomodate plant operations. March 1, and subsequent anniversaries of this date, will be considered the starting date of each annual cycle of requalification program operation.

Shall, Should, and May - The word "shall" is used to denote a requirement; the word "should" to denote a recommendation; and the word "may" to denote permission - meither a requirement nor a recommendation.

4.0 PROGRAM DESCRIPTION

4.1 OBJECTIVES

The Requalification Program objectives are to:

- 1) Maintain nuclear plant operational safety and reliability.
- 2: Assure that licensed personnel maintain the bigb level of skill and knowledge required to accomplish routine and emergency duties.

 Establish a system for evaluating and documenting licensed personnel proficiency and competency.

4.2 PRE-PLANNED LECTURE SERIES

The operator requalification program shall include pre-planned training sessions conducted on a regular and continuing basis. The training sessions shall include two types of lecture series as follows:

1) Fundamentals Review Lecture Series

2) Operational Proficiency Lecture Series

4.2.1 Fundamental_ Review Lecture Series

The Fundamentals Review training sessions cover areas in which the knowledge required of a licensed individual is relatively constant.

4.2.1.1 Lecture Series Topics

The Fundamentals Review lecture topics are selected on an as-needed basis and shall include the following:

Theory and Principles of Reactor
 Operation.

- Heat Transfer, Fluid Flow and Thermodynamics.
- Features of Facility Design including Plant Systems.
- Gereral and Specific Plant Operating Characteristics including Expected Response to Equipment Failure.
- Flant Instrumentation and Control Systems.
- 6) Plant Protection Systems.
- 7) Engineered Safety Systems.
- Radiation Control and Safety and Plant Chemistry.
- Applicable Portions of Title 10,
 Chapter I, Code of Federal Regulations.

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10) Fuel Handling and Core Parameters

4.2.1.2 Lecture Series Topic Selection

The trpics presented in the Fundamentals Review series should reflect the general results of the annual examinations and performance of the licensed personnel as evaluated by the Manager of Operations and the Operations and Maintenance Director.

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The scope of the lecture series should be determined by the Supervisor, Licensed Operator Training and approved by the Operator Training Manager. The depth of coverage in each topic should reflect typical deficiencies identified by the annual examinations.

4.2.2 Operational Proficiency Lecture Series

The Operational Proficiency training sessions cover (areas which involve essential plant operational guidelines.

4.2.2.1

Lecture Series Topics

The Operational Proficiency lecture topics are selected to ensure coverage of essential plant operational guidelines and to ensure operational changes and experiences are integrated into licensed individual's training. The lecture topics should include the following:

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- Normal, Abnormal and Emergency Operating Procedures and Changes Thereto.
- 2) Technical Specifications.
- 3) Administrative Procedures, Conditions

and Limitations and Changes Thereto.

- 4) Major Operational Evolutions.
- 5) Facility Design and License Changes.
- 6) Operating History and Problems.
- Related Nuclear Industry Operating Experience.
- Mitigation of Accidents Involving
 a Degraded Core.

4.2.2.2 Lecture Series Topic Selection

The topics presented in the Operational Proficiency Series shall include all the topics listed in Section 4.2.2.1, unless the applicable information is covered in another appropriate manner, such as staff discussion sessions or Operational Review Program (Section 4.4) discussion sessions.

The scope of the lecture series should be determined by the Supervisor, Licensed Operator Training and the Operator Training Manager taking into account the Manager of Operation's and the Operations and Maintenance Director's inputs. The depth of the coverage in each topic should

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reflect the knowledge required of the Licensed Senior Operator

4.2.3 Pre-Planned Lecture Series Schedule

The Pre-planned Lecture Series shall be scheduled on an annual basis. Lectures may be deferred due to unanticipated events, but should be conducted as soon as practicable thereafter and within the annual cycle.

The lecture series shall be held on a continuing basis with a weekly schedule of lectures designed to be repeated for each shift when that shift is designated for its training week. The program and schedule will be determined by unit operations or projected operations and must take into account planned and unplanned outages and available simulator time. It shall typically involve approximately 240 contact hours of instruction divided among the program topics and appropriately scheduled throughout the year. Records of the topics covered in each session shall be maintained by the Training Department.

4.2.4 Pre-Planned Lecture Series Attendance

Attendance of all licensed personnel shall be recorded. Absences should be approved in advance by

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the Manager of Operations or the Operations and Maintenance Director, Unit 1, and should be limited to one training week per year. Additional absences unless approved by the Manager of Operations shall result in the individual being removed from licensed duties and placed in an accelerated regualification program until such time as the missed material is made up. In any case, the individual who misses training shall be responsible for the material presented in his absence and shall take the quiz that was given on the missed material. Those personnel whose annual written examination scores indicate that a mandatory upgrading of their knowledge level is required, as indicated by a grade of less than 80% in any section, must attend the applicable pre-planned lecture series presentation. Mandatory attendance requirements shall be determined by the Supervisor, Licensed Operator Training.

4.2.5 <u>Pre-Planned Lecture Series Training Methods</u> The pre-planned lecture series shall use training sessions supported by prepared lesson plans.

4.2.5.1 Lecture Presentation

For each training session in the lecture

series, a lesson plan shall be prepared, reviewed, and approved in accordance with Training Department Procedures TD 1103 and TD 1104.

The incorporation of training aids such as trainee bandouts, films, slides, models, transparencies, and videotape presentations is encouraged. The lesson plan shall become the reference source for the information covered during the lecture and shall be retained as part of the. program records.

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In the event that videotape or film presentations or computer-based instruction are used, an instructor shall be available to embellish, explain or emphasize the presentation and to respond to any questions or comments from the trainees.

4.2.5.1 Study Periods

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Study periods should be scheduled in conjunction with the Pre-Planned Lecture Series to provide licensed trainees an

opportunity to reinforce the lecture series learning experience and to study new or additional materials. While individual self-study is encouraged, it should not be substituted for training sessions implemented by an instructor conducted as part of the Pre-planned Lecture Series.

4.2.5.3 Instructor Qualifications

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 Training Department and facility instructors who teach systems, integrated responses and transient courses shall have demonstrated their competence to the NRC by successful completion of a senior operator examination.

2. "Guest" lecturers who are experts in a particular subject area need not possess the above qualification. The scheduling and appearance of "Guest" lecturers shall be approved in writing by the Operator Training Manager or Manager, Plant Training.

3. Training Department instructors should attend the first Instructor Development Program presented following their selection as instructors. Waivers may be approved by the Manager, Plant Training based on previous experience.

4. .6 Pre-Planned Lecture Series Evaluation

The lecture series shall be evaluated by conducting evaluations of the trainee's knowledge, effectiveness of the overall lecture series, and effectiveness of the instructors.

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.1 Lecture Series Quiz

After each week of lectures, all trainees shall take a written, closed-book quiz covering the lecture topic(s).

4.2.6.1.1 Quiz Administration

During or at the completion of a group of lectures, an evaluation of trainee knowledge shall be made. The evaluation shall contain questions related to the
lesson plan objectives covered during the lectures. All lecture topics covered during the training session should be represented by question. in the evaluation. In addition, questions shall be included from selected abnormal and emergency procedures as designated by the Supervisor Licensed Operator Training, such that all abnormal and emergency procedures are tested on a biannual basis.

A variety of question types may be used, but questions requiring analysis or detailed discussion should predominate. An answer key with predetermined question point values should be prepared for evaluation and record keeping purposes.

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4.2.6.1.2 Quiz Standards

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Quizzes shall be evaluated and a grade determined for each trainee. A performance standard of 80% shall be established for a written quiz. Trainees who do not meet these performance standards should complete a remedial review process within six (6) weeks consisting of: 1) Trainee review of the training session material associated with identified knowledge deficiencies.

- Trainee review of associated reference material identified by the instructor.
- 3) Administration of a second quiz covering at least the identified knowledge deficiencies.

If the second quiz is completed satisfactorily, the trainee should receive credit for completion of the required lecture. If the second quiz is unsatisfactory, the Supervisor Licensed Operator Training shall notify the Manager of Operations with copies to the Operations and Maintenance Director and the · Director, TMI-1, via the Operator Training Manager and Manager, Plant Training of the evaluation results and provide a recommendation regarding the trainee's removal from licensed duties and entrance into an accelerated requalification program (Section 4.6.1).

Trainees whose attendance at all or part of the training sessions covered by a particular guiz is mandatory based on the previous annual

examination, but must achieve a score of at least 80% on the applicable sections of the first quiz or be assigned an accelerated requalification program per section 4.6.1.

4.2.6.2 Lecture Series Effectiveness

The effectiveness of the lecture series should be evaluated by the trainees, Manager of Operations and the Supervisor, Lice sed Operator Training. The results of this evaluation should be factored into subsequent regualification training.

4.2.6.2.1

Lecture Series Evaluation

An overall evaluation of the annual lecture series should be conducted on at least an annual basis. The evaluation abould encompass the instructors, training materials, presentation techniques, quiz techniques, and classroom facilities.

This evaluation should be conducted by individuals designated by the Supervisor, Licensed Operator Training and should consider input from the trainees, Manager of Operations and the program instructors. Significant problems should be considered and resolved by the Supervisor, Licensed Operator Training and the Operator Training Manager.

4.2.6.2.2

Presentation Evaluation

Evaluation of selected lecture series training sessions shall be conducted periodically. The evaluations should be directed toward ensuring overall quality of instruction for the lecture series. The evaluation should encompass instructor preparation, presentation techniques, and

technical content of the lecture. This evaluation should be conducted by personnel designated by the Manager, Plant Training or the Operator Training Manager.

Problems identified by these evaluations should be resolved by the instructor and the Supervisor, Licensed Operator Training.

4.3 SKILLS TRAINING AND EVALUATION

In orde to maintain an acceptable level of skills and familiarity associated with the nuclear plant systems, controls, and operational procedures, each licensed individual shall participate in frequent and varied plant evolutions. Each licensed individual shall demonstrate operational proficiency by participating in the following activities:

- 1) Reactivity Manipulations and Plant Evolutions
- 2) Nuclear Plant Simulator Exercises
- 3) Plant Drill Program

To maintain these skills, licensed operators shall actually manipulate the controls while licensed senior operators may either manipulate or actively supervise manipulation of the controls. Training to achieve proficiency should be planned so that skills training exercises are repeated until proficiency is demonstrated.

4.3.1 Reactivity Manipulations and Plant Evolutions

During the two year term of the NRC license, each licensed individual shall participate in a variety of reactivity control manipulations and plant evolutions.

.3.1	.1	Normal	Plant	Evolutions	Sec. 1	·		1.2.1
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On an annual basis, each licensed individual shall participate in the following plant evolutions either at the nuclear plant or at a nuclear plant simulator:

 Plant or reactor startup and power escalation to a range where reactivity feedback from nuclear beat addition is noticeable and beatup rate is established.

2) Plant shutdown.

- 3) Manual control of steal generator water level and/or feedwater flow during plant startup and/or sbutdown.
- Boration and/or dilution during power operation.
- Reactor power changes of 16% or greater where rod control is in manual.
- 6) Reactor power changes of 10% or greater where load change is performed with load control on manual.
- Operation of turbine controls in manual during turbine startup.
- 8) Decay heat removal system operation.
- 9) Incore monitoring system operation.
- IO) Control room calculations including beat balance, coolant inventory balance, and reactivity balance.

Individual performance during these plant evolutions shall be monitored and deficiencies corrected so that satisfactory proficiency is demonstrated.

4.3.1.2 <u>Abnormal/Emergency Plant Evolutions</u> On an annual basis, each licensed

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individual shall participate in training exercises covering the following plant abnormal/emergency conditions either at a nuclear power plant simulator or during the plant drill program:

- 1) Reactor trip
- 2) Turbine or generator trip
- 3) Loss of coolant including:
 - a) Significant steam generator leaks
 - b) Significant pressurizer leaks
 - (c) Large and small leaks located inside and outside primary containment (including leak rate determination for small leaks inside containment)
 - d) Saturated reactor coolant system
 response

- Loss of coolant flow/natural circulation
- Loss of all feedwater (normal and emergency)
- 6) Control room inaccessibility
- 7) Loss of shutdown cooling.

On a two-year cyclic basis, each licensed individual shall participate in training exercises covering the following plant abnormal/emergency conditions either at a nuclear plant simulator or during the plant drill program.

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- 1) Nuclear instrumentation failure(s)
- 2) Loss of protective system channel(s)
- Mispositioned control rod(s) (or rod drops)
- 4) Inability to drive control rods
- Conditions requiring use of emergency boration or standby liquid control system.
- Fuel cladding failure or high activity in reactor coolant or offgas
- Malfunction of automatic control system(s) which affect reactivity
- Malfunction of reactor coolant pressure/volume control system
- 9) Loss of instrument air
- Loss of electrical power and/or degraded power sources
- 11) Loss of condenser vacuum

- 12) Loss of service water
- Loss of component cooling system or cooling to an individual component
- 14) Loss of normal feedwater or normal feedwater system failure
- 15) Main steam line break (inside or outside containment).

Response to abnormal/emergency conditions should include use of alternate methods of accomplishing a given function, such as alternate methods of core cooling. Exercises involving multiple failures and/or operator error shall be included. Utilization of applicable plant procedures and technical specifications during the training exercises should not be minimized.

Individual and operational team performance during the abnormal/emergency training exercises shall be monitored and deficiencies corrected so that satisfactory proficiency is demonstrated.

4.3.1.3 <u>Performance of Reactivity Manipulations</u> and Plant Evolutions Reactivity manipulations and plant evolutions shall be conducted either at the nuclear plant or at a suitable nuclear plant simulator.

4.3.1.3.1 Performance of Normal Plant

Evolutions

Normal plant evolutions shall be either performed at the nuclear plant or performed at a suitable nuclear plant simulator (Section 4.3.2).

4.3.1.3.2

<u>Performance of Abnormal/</u> <u>Emergency Plant Evolutions</u> Abnormal/emergency plant evolutions shall be either performed at a nuc' ar plant simulator (Section 4.3.2) or conducted during the plant drill program (Section 4.3.3).

In the event that an actual abnormal/emergency condition occurs at the plant and performance of the licensed

personnel coping with the condition is satisfactory (as determined by the Manager of Operations), credit for completion of a training exercise may be taken.

4.3.2 Nuclear Plant Simulator Exercises

A nuclear plant simulator provides a means of training licensed individuals to conduct normal plant evolutions and to cope with nuclear plant transient and accident conditions. Licensed individuals shall participate in a structured training program utilizing a nuclear plant simulator on an annual basis.

During the simulator training sessions, the team concept should be utilized, emphasizing individual roles in reporting, assignment of operational duties, use of plant procedures and use of technical specifications. Assignment to simulator training sessions which are direct interactions with the nuclear plant control panel should be made on a crew basis with other licensed individuals integrated into

the crews not to exceed one additional individual per crew.

Each licensed individual shall complete nuclear plant simulator training sessions involving at least twenty (20) hours of direct interaction with the simulator nuclear plant control panel on an annual basis. Brief post-evolution critiques conducted at the simulator nuclear plant control panels may be considered as part of the direct interaction session.

Lecture sessions conducted in conjunction with a simulator training program covering topics derignated in the Pre-planned Lecture Series may be credited toward fulfilling the requirements of Section 4.2.

Reactivity manipulations, plant evolutions, and exercises which should be considered in the simulator training program include:

- 1) Normal Plant Evolutions (Section 4.3.1.1).
- Abnormal/Emergency Plant Evolutions (Section 4.3.1.2).
- 3) Verification of Plant Operating Procedure Adequacy
- Demonstration of plant response to conditions identified from nuclear industry operating

experiences.

Instructors presenting simulator training sessions scall have demonstrated their competence to the NRC by successful completion of a Senior Operator Examination.

4.3.3 Plant Drill Program

Plant drills provide a means of training licensed individuals in responding to plant achormal/emergency conditions. Licensed individuals shall participate in the plant drill program on an annual basis.

4.3.3.1 Plant Drills

On an annual basis each licensed individual shall participate in plant drills. Plant drills shall be conducted so that each licensed individual actively participates in drills covering abnormal/emergency plant evolutions (Section 4.3.1.2) which are not adequately covered in the nuclear plant simulator training program (Section 4.3.2).

Participation in a plant drill involves either responding to drill conditions or being an assigned monitor for observing/evaluating response to a drill. Plant drills may be structured to review or carry out actions required to respond to abnormal/emergency plant conditions.

Plant drills may be conducted with the approval of the Manager of Operations, on an individual or team basis and may involve:

- 1) Reviewing plant procedure steps.
- Identifying actions required to establish stable plant conditions.
- Identifying equipment control locations and functions.
- Identifying expected plant instrumentation and alarm response.
- Reviewing communications necessary to gather information or coordinate team actions.
- Identifying supplementary actions
 aimed at mitigating results or causes
 of plant abnormal/emergency conditions.

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4.3.3.2 Plant Drill Scenarios

Each plant drill must be carefully planned and monitored in order to ensure actual abnormal/emergency conditions are not inadvertently initiated during the drill and in order to evaluate individual or team response to the drill conditions.

Each plant drill structured to include carrying out actions should be planned in a prepared drill scenario, approved by the Manager of Operations. The Operations and Maintenance Director and Director TMI-1 shall approve scenerios for major emergency drills. The drill scenario should include the following: 1) Purpose/objectives of the drill

- 2) Initial conditions
- 3) General description
- 4) Method of initiation
- 5) Precautions and limitations
- 6) Sequence of expected actions
- Point of termination/conditions under which the drill is to be secured
- 8) Final conditions
- 9) Monitors required/location

The drill performance should be evaluated and deficiencies identified. A drill critique should be held by the Senior auitor with the drill participants. Deficiencies identified should be reviewed by the Supervisor, Licensed Operator Training. Additions or modifications to training programs required to correct performance deficiencies should be initiated by the Supervisor, Licensed Operator Training and/or the Manager of Operations.

4.3.4' Skills Evaluation System

Licensed personnel performance and competency related to performing licensed duties shall be periodically evaluated. Repeated errors or other indicators of degraded proficiency should be reviewed by the Manager of Operations and the Operator Training Manager and appropriate training initiated.

Evaluation of licensed personnel job performance should be utilized to relate job performance to requalification training. The Manager of Operations should provide the Supervisor, Licensed Operator

Training or the Operator Training Manager with periodic observations identifying job performance results related to requalification training. Each licensed individual's performance shall be evaluated during the following situations: 1) Nuclear Plant simulator exercises

2) Plant drills

The Supervisor, Licensed Operator Training should ensure evaluations are conducted during the simulator and drill exercises.

4.3.5 Skills Training Participation

In the event that skills training guidelines for participation in normal plant evolutions (Section 4.3.1.1), abnormal/emergency plant evolutions (Section 4.3.1.2) or plant drills (Section 4.3.3.1) are not met, exercises which will fulfill the requirements should be scheduled and completed within twelve (12) weeks of the required period. If the requirements are not completed within the twelve (12) week period, the Supervisor, Licensed Operator Training shall notify plant management and the Director TMI-I via the Operator Training Manager and the Manager, Plant Training and provide a

recommendation regarding the licensed individual's removal from licensed duties.

Based on schedule constraints, satisfactory participation shall be defined as meeting the requirements on a quarterly basis. If an individual bas not spent six shifts in the control room over a given quarter, he shall have one month in which to make up the missed shifts. Failure to do so shall result in his being placed in the Inactive Status Retraining program, Section 4.6.2.

4.4 OPERATIONAL REVIEW PROGRAM

The operational review program provides a system for on-shift review of selected operational experiences and changes to existing operating guidance or equipment. The operational review program enables continuing updating of on-shift personnel and establishes a means of disseminating new or changing information on a short term basis.

4.4.1 Modification Review

A continuing system shall be established by the Manager of Operations so licensed individuals review documented plant design changes, equipment modifications, procedure changes and technical specification changes. Selected changes and modifications should be analyzed and information pertinent to the basis for the changes and their operational implications collected. This information should be formally transmitted to all licensed individuals with acknowledgement of review required. Changes to emergency procedures, technical specifications and safety related systems shall be reviewed prior to the licensee assuming shift operation responsibilities.

The Manager of Operations and the Supervisor, Licensed Operator Training should specify changes and modifications to be analyzed, with information for review transmitted in accordance with the urgency of the situation. The Manager of Operations should ensure that all on-shift licensed personnel review the selected information in a timely manner.

Expanded coverage of plant design changes, equipment modifications, procedure changes and technical specification changes in the Operational Proficiency Lecture Series (Section 4.2.2) should be considered by the Supervisor, Licensed Operator Training.

On-shift supervisory (SRO) personnel should provide guidance to on-shift operators in interpreting and reviewing changes and modifications. An on-shift discussion period to review changes and modifications is encouraged.

4.4.2 Operating Experience Review

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A continuing system shall be established by the Manager of Operations so licensed individuals review operating experience from TMI and from applicable segments of the nuclear industry. Selected operational events and reportable occurrences at the

facility should be analyzed and information pertinent to the event collected. Selected operational information from the nuclear industry should be analyzed. The following sources of information shall be considered:

1) Licensee Event Reports

2) Audit, evaluation, and inspection reports

 Publications and periodicals covering nuclear industry information.

4) NSAC/INPO Significant Event Reports

This information should be formally transmitted to all licensed individuals with required acknowledgement of review. The Manager of Operations should a sure that all on-shift licensed personnel review the information in a timely manner.

Technical Functions personnel assigned to assess plant operating experience and the Supervisor, Licensed Operator Training shall specify operating experience to be analyzed for training purposes, with information transmitted to all licensed personnel for review. Subsequent coverage of operating experience in the Operational Proficiency Lecture Series (Section 4.2.2) should be considered by the Supervisor,

Licensed Operator Training. Selected nuclear plant accident/transient situations shall be analyzed and integrated into the Nuclear Plant Simulator Exercises (Section 4.3.2) or the Plant Drill Program (Section 4.3.3).

4.5 ANNUAL REQUALIFICATION EXAMINATION

In order to determine each licensed individual's knowledge of topics covered in the requalification program and provide a basis for determining areas in which retraining is needed, an annual requalification examination shall be given. The annual examination shall be given to all licensed individuals prior to the completion of each annual requalification program cycle and shall consist of an oral examination and a written examination.

4.5.1 Annual Written Examination

An annual written examination shall be administered to all licensed individuals.

4.5.1.1 <u>Written Examination Content</u> The written examination shall contain examination guestions covering the

following topics:

- Theory and Principles of Reactor Operation
- Heat Transfer, Fluid Flow and Thermodynamics
- 3) Peatures of Facility Design
- General and Specific Plant Operating Characteristics
- 5) Plant Instrumentation and Control Systems
- 6) Plant Protection System
- 7) Engineered Safety Systems
- 8) Radiation Control and Safety
- 9) Applicable Portions of Title 10,
- Chapter I, Code of Pederal Regulations
- 10) Fuel Handling and Core Parameters
- 11) Normal, Abnormal and Emergency Operating Procedures
- 12) Technical Specifications
- 13) Administrative Procedures, Conditions and Limitations
- 14) Nuclear Industry Operating Experience

The topics shall be grouped into at least six (6) examination categories for evaluation purposes. The examination shall be structured so that the level of questioning is consistent with the individual's license level (RO or SRO).

4.5.1.2 <u>Written Examination Administration</u> The written examination should be prepared under a structure enabling consistency of questioning and minimizing possible compromise of examinations prior to administration. The following guidelines should be considered:

- An examination question file shall be assembled taining questions and answers on each examination topic.
- Questions should be formulated from many sources including:
 - a) Licensed personnel training program.
 - b) Regualification programs
 - c) Plant documents such as technical specifications and procedures.
 - d) Previous examinations.
 - e) Manager of Operations.

- f) Operating experience.
- g) Plant design changes and system modifications.
- 3) The examination should include a variety of question types, but questions requiring analysis and/or explanation should predominate.
- Questions and answers should be reviewed and approved by the Supervisor, Licensed Operator Training prior to use.
- 5) The Supervisor, Licensed Operator Training shall designate individuals to update the examination guestions and answers file annually.

Personnel designated by the Supervisor, Licensed Operator Training shall prepare the written examination(s) utilizing the examination question file. The number of different examinations prepared should be consistent with preventing examination compromise. The examination may be administered in segments. The examination, answer key and grading system

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should be reviewed and approved by the Supervisor, Licensed Operator Training, Operator Training Manager, or Manager of Operations prior to examination administration.

The Supervisor, Licensed Operator Training shall designate qualified personnel to grade the written examination and shall review the graded examinations for grading techniques and consistency.

4.5.1.3 Examination Performance Standards

A licensed individual receiving a grade of less than 70% in any examination category or an overall grade of less than 80% shall be relieved of his license duties and placed in an accelerated gualification program (Section 4.6.1).

Under special circumstances where a grade of less than 70% has been scored in a single section, the Director, ThI-1 may document the special circumstances and authorize an oral and written reexamination of the failed section within one (1) week. If the oral exam is

completed satisfactorily and a grade of 80% or greater is scored on the written section, the individual may return to shift in a licensed status with the approval of the Director, TMI-1.

4.5.2 Annual Oral Examination

An annual oral examination shall be administered to all licensed individuals.

4.5.2.1 Oral Examination Content

The oral examination should contain questions covering the following areas: 1) bicensed duties and responsibilities

- of the operating position corresponding to the individual's license level.
- Actions in the event of abnormal conditions.
- Actions in the event of emergency conditions.
- Interpretation of instrumentation responses.
- 5) Plant transient and accident response.

6) Plant modifications.

- 7) Procedure changes.
- 8) Technical Specifications.
- 9) Emergency Plan.
- 10) Plant Operating bistory and problems.
- Related nuclear industry operating experiences.
- 4.5.2.2 Oral Examination Administration

The oral examination shall be conducted under a structure enabling consistency of questioning and evaluation. The following guidelines should be considered.

- A checklist identifying the areas to be covered shall be used.
- Overall evaluation shall be made on a pass/fail basis.
- Comments on individual strengths and weaknesses shall be made.

The Supervisor, Licensed Operator Training and the Manager of Operations shall establish the oral examination schedule.

Personnel assigned to conduct an oral evaluation shall be designated by the

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Supervior, Licensed Operator Training and approved by the Manager of Operations. Oral examinations shall be conducted by a Licensed Senic. Operator or personnal wbo have successfully completed education and training programs required for a senior operator's license. Each oral examination should be structured so that an examination time of two (2) hours or more is appropriate.

The Oral examination should involve sessions conducted in the plant control room and in plant areas occupied by individuals whose actions are directed by the licensed operator.

4.5.2.3 Examination Performance Standards

A failing overall oral examination grade shall require the licensed individual to be removed from his license duties and be placed in an accelerated regualification program (Section 4.6.1).

4.6 SPECIAL RETRAINING PROGRAMS

Specific retraining programs for licensed individuals may be required to upgrade or refresh knowledge and skills related to licensed duties.

4.6.1 Accelerated Regualification Program

The accelera i requalification program is for licensed individuals baving identified deficiencies requiring assignment to a special retraining effort.

4.6.1.1 Required Attendance

Licensed individuals meeting one or more of the following criteria shall be assigned to an accelerated regualification program:

- Annual regualification written examination performance deficiencies per Section 4.5.1.3.
- Annual regualification oral
 examination performance deficiencies
 per Section 4.5.2.3.
- Pre-planned lecture series quiz performance deficiencies per Section 4.2.6.1.2.

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 Significant licensed duty performance deficiencies identified by the Manager of Operations and/or the Supervisor, Licensed Operator Training.

4.6.1.2 Program Content

The accelerated requalification program content shall be specifically structured to upgrade knowledge and skills identified as deficient. Examination categories and areas in which performance standards were not met shall be covered in the program.

The Supervisor, Licensed Operator Training shall be responsible for formulating individual accelerated requalification programs. They shall be approved by the Operator Training Manager and the Manager of Operations.

4.6.1.3 Program Administration

The accelerated regualification program may involve a variety of training exercises including: 1) Directed self-study.

- Oral intervia.s and discussion sessions.
- 3) Pre-planned lectures
- Skills training exercises at the plant or utilizing a simulator.

Personnel assigned to implement the training exercises shall be qualified in accordance with Section 4.2.5.3 or 4.3.2 as appropriate.

Program duration should be dictated by the extent of training required and trainee's performance.

4.6.1.4

Performance Standards

Successful completion of the accelerated requalification program shall be determined by administering an examination. The examination shall cover all categories of the requalification written examination and/or all areas of the requalification oral examination originally failed. The examination format should be similar to the original

examination, and the examination shall be conducted by individuals designated by the Supervisor, Licensed Operator Training.

Performance standards for the accelerated requalification program shall be as follows:

- A score of at least 80% on each accelerated regualification written examination category.
- A passing evaluation on the accelerated regualification oral examination.

In the event that these standards are not met, the individual's suitability for resuming licensed duties will be reviewed by the Supervisor, Licensed Operator Training. He shall provide a recommendation to plant management and the Director TMI-1, via the Operator Training Manager and Manager, Plant Training regarding the individual's permanent removal from licensed duties or additional upgrading efforts to be considered.
If appropriate, another accelerated requalification program shall be structured to correct deficiencies.

4.6.2 Inactive Status Retraining

If a licensed individual has not actively carried out licensed duties for a period in excess of four (4) months, a special retraining program and/or evaluation is required prior to resuming licensed duties.

Active status can be maintained by performance of licensed duties and participation in the licensed personnel requalification program. Performance of licensed duties involves manipulation of controls which directly affect reactivity or power level of the reactor or directing the licensed activities of licensed operators, and for those licensed individuals not normally assigned to an operating shift, completing the requirements of Section 5.5 for participation in control room operations.

In the event that a licensed individual does not maintain an active status, the Supervisor, Licensed Operator Training shall designate, subject to

approval of the Manager of Operations, a Licensed Senior Operator to conduct an oral examination similar in scope and format to an annual oral examination prior to resuming licensed duties. In addition, evaluation of performance in the current Pre-planned Lecture Series shall be conducted. If performance in the Pre-planned Lecture Series is unsatisfactory, a written examination similar in scope and format to the annual written examination (Section 4.5.1) shall be administered to the licensed individual prior to resuming licensed duties. Consideration should be given by the Manager of Operations to assigning the licensed individual to a training status on an operating shift prior to resuming licensed duties.

The performance standards applied to the annual requalification examination shall be used in evaluating the results of the oral and written examinations. If the performance standards are not met, the licensed individual shall complete an accelerated requalification program prior to resuming licensed duties.

Licensed duties may be resumed only upon certification of the Director TMI-1, which must be forwarded to the NRC.

4.6.3 Newly Licensed Individuals

Newly licensed individuals shall enter the requalification program and participate in the annual program cycle upon receipt of their license. Newly licensed individuals successfully completing their NRC licensing examination less than three(3) months prior to an annual requalification examination may be excused from taking the current annual written and oral examinations.

4.7 REQUALIFICATION PROGRAM EVALUATION

A regualification program review and evaluation shall be conducted on an annual basis by the Supervisor, Licensed Operator Training. The areas encompassed by the review should include:

- Inspection, audit and evaluation reports of requalification training completed by outside organizations and facility personnel.
- Licensed individual performance evaluations related to licensed duties.

- 3) Program oral and written examination results.
- Plant operational problems related to licensed individual knowledge or skills deficiencies.
- Licensee Event Reports related to licensed individual performance from the plant and the nuclear industry.
- Changes in job assignments related to licensed duties and/or safety related functions of licensed operators.
- Regulations and standards affecting licensed operator retraining.
- Assessment of licensed personnel performance deficiencies related to training prepared by the Manager of Operations.

Requalification program curriculum deficiencies and licensed operator retraining needs determined by the review shall be identified, recommended corrective actions structured and a report formulated for review by the training organization through the Director of Nuclear Assurance and the operations organization through the Director, TMI-1.

Requalification program deficiencies or required changes which need immediate action or significant program modification should be evaluated by the Supervisor, Licensed Operator Training as soon as practicable. Necessary corrective action shall be structured by the Supervisor, Licensed Operator Training and reviewed as above.

4.8 RESPONSIBILITIES

- A. Supervisor, Licensed Operator Training is responsible for the following:
- Determining the scope of the Pundamentals Peview and
 Operational Proficiency Lecture Series.
- Determining mandatory attendance requirements at requalification lectures based on weaknesses on previous .
 annual requalification examination.
- Designating those abnormal and emergency procedures to be the basis of the guiz questions for each six weeks' cycle such that all abnormal and emergency procedures are tested biannually.
- Preparing notification for plant management and the Director, TMI-1 of unsatisfactory guiz results from the lecture series.
- Designating individuals to conduct overall evaluation of the annual lecture series and resolving problems described in these evaluations.
- Resolving problems identified by evaluation of lecture series training sessions.

- Reviewing plant drill critiques and initiating additions or modifications to training programs to correct performance deficiencies noted.
- Ensuring performance evaluations are conducted during simulator and drill exercises.
- Preparing notification for plant management and the Director, TMI-1 of unsatisfactory skills training participation.
- Specifying changes and modifications to be analyzed for review in the operational review program.
- Determining expanded coverage of plant design changes, equipment modifications, procedure changes, and technical specification changes in the Operational Proficiency Lecture Series.
- Specifying, in conjunction with Technical Functions,
 operating experience to be analyzed for training purposes
 and integrating the information into the training program.
- Designating personnel to prepare and grade the annual written examination, approving the examination, answer key and grading system, and reviewing the graded

examination for grading techniques and consistency.

- Designating personnel to review and update the examination guestion file.
- Establishing the annual oral examination schedule.
- Designating personnel to conduct oral examinations.
- Identifying significant licensed performance deficiencies requiring accelerated regualification programs.
- Formulating individual accelerated requalification programs and designating individuals to conduct the associated examinations.
- Preparing recommendations to plant management and the Director, TMI-1 regarding the permanent removal from licensed duties or additional upgrading efforts to be considered for those individuals failing to meet the standards of the accelerated regualification program.
- Designating licensed SRO's to conduct oral examinations for those individuals failing to maintain an active status.

- Conducting an annual regualification program review and evaluation, submitting a report on this review, and taking immediate corrective action where necessary.
- Establishing the requalification program records identified in Section 4.9.
- B. Operator Training Manager is responsible for the following:
- Approving the scope of the Fundamentals Review and
 Operational Proficiency Lecture Series.
- Approving scheduling and appearance of "Guest" lecturers.

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- Designating personnel to evaluate selected lecture series training sessions.
- Reviewing repeated personnel errors or other indicators of degraded proficiency and initiating appropriate training.
- Approving the annual written examination, answer key and grading system.

- Approving accelerated regualification programs.
- C. Manager, Plant Training is responsible for the following:
- Approving waivers of attendance requirements for the Instructor Development Program.
- Designating personnel to evaluate selected lecture series training sessions.
- Ensuring that the Training Department's Administrative Support Section maintains the records identified in Section 4.9.
- D. Manager of Operations is responsible for the following:
- Providing inputs to the Training Department on topics to be presented in the Fundamentals Review and Operational Proficiency Lecture Series.
- Approving absences of licensed personnel from lecture series.
- Approving Plant Drill Scenerios, the conduct of plant drills and additions or modifications to training

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programs as a result of drill critiques.

- Reviewing repeated personnel errors or other indicators of degraded proficiency and initiating appropriate training.
- Providing periodic observation to the Training Department identifying job performance results related to requalification training.
- Establishing a continuing system so that licensed personnel review documented plant design changes, equipment modifications, procedure changes and technical specification changes, specifying the changes and modifications to be analyzed, and ensuring that on-shift licensed personnel review the selected information in a timely manner.

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Establishing a continuing system so that licensed personnel review operating experience from TMI and applicable segments of the nuclear industry and ensuring that on-shift licensed personnel review the information in a timely manner.

Approving the annual written examination, answer key, and

grading system.

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- Establishing the annual oral examination schedule.
- Approving personnel designated to conduct oral examinations.
- Identifying significant licensed performance deficiencies requiring accelerated requalification programs.
- Approving accelerated regualification programs.
- Approving SRO's designated to conduct oral examinations
 for those individuals failing to maintain an active status.
- Establishing and maintaining operational review series participation records.

4.9 REQUALIFICATION PROGRAM RECORDS

Records of licensed individuals' performance in the requalification program shall be maintained in an auditable manner. The Supervisor, Licensed Operator Training is responsible for establishing the following regualification program records:

- Oral and written examination results for each licensee.
- 2) Written examination questions and answer keys.
- 3) Lecture series attendance records.
- 4) Lecture series lesson plans.
- 5) Plant drill participation records.
- Reactivity manipulation and plant evolution participation records.
- 7) Simulator training participation records.

These records shall be maintained by the Training Department's Administrative Support Section.

Operational review series participation records shall be established and maintained by the Operations Department.

4.10 PROGRAM APPROVAL

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The Director of Unit 1, through the Operations and Maintenance Director and the Manager of Operations, certifies operators for requalification and relicensing. The Manager of Operations retains the responsibility to onsure that the overall level of training of plant operators is satisfactory through the approval of program

content, schedules and administrative procedures and changes thereto.

- The Manager, Plant Training through the Operator Training Manager, is responsible to ensure that the training program is developed to meet the requirements established by the Director of Unit 1 through the Manager of Operations and that proper records and documentation are provided and maintained.
- Lesson plans for implementation of the training program shall be reviewed by the Supervisor, Licensed Operator Training and approved by the Operator Training Manager.
- All changes to this program description shall be approved by the Manager of Operations and Manager, Plant Training.

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APPENDIX A

OFF SHIFT LICENSED OPERATOR

WATCH STANDING DOCUMENTATION

1.

I certily that _____ has satisfactorily Off Shift Licensed Operator

assumed and performed the SS/SF/CRO duties (actual or under (Circle One)

instruction) for the 11-7/7-3/3-11 shift on (Circle One) :

Date

Shift Supervisor

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