U.S. NUCLEAR REGULATORY COMMISS APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/86 LICENSEE EVENT REPORT (LER) FACILITY HAME (1) DOCKET NUMBER (2) Prairie Island Unit 1 0 | 5 | 0 | 0 | 0 | 2 | 8 | 2 1 OF 01 TITLE (4) Reactor Trip on Startup EVENT DATE (S) LER NUMBER (6) REPORT DATE (7) OTHER FACILITIES INVOLVED (8) SEQUENTIAL DOCKET NUMBER(S) MONTH DAY DAY YEAR YEAR MONTH 0 |5 |0 |0 |0 | 018 311 8 14 814 01016 010 019 218 814 0 |5 |0 |0 |0 | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 19 CFR §: (Check one or more of the following) (11) MODE (9) 20,408(a) 80.73(a)(2)(iv) 73.71(b) 20.406(a)(1)(i) 50.36(e)(1) 80.73(a)(2)(v) 73.71(e) POWER LEVEL (10) 0 2 . 0 OTHER (Specify in Abstract below and in Toxt, NRC Form 366A) 20.406(a)(1)(ii) 50.36(e)(2) 50.73(a)(2)(vii) 20.406(a)(1)(W) 50.73(a)(2)(viii)(A) 50.73(a)(2)(i) 20.406(a)(1)(h) 90.73(a)(2)(ii) 50.73(a)(2)(viii)(B) 20.405(a)(1)(v) 50.73(a)(2)(Ni) 80.73(a)(2)(x) LICENSEE CONTACT FOR THIS LER (12) TELEPHONE NUMBER AREA CODE Arne A. Hunstad, Staff Engineer 6 1112 318181-11111211 COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) REPORTABLE TO NPROS MANUFAC-TURER SYSTEM MANUPAC-TURER TO NPROS CAUSE COMPONENT CAUSE SYSTEM COMPONENT X S, J F, T F, 1, 8,0 Y YEAR SUPPLEMENTAL REPORT EXPECTED (14) MONTH DAY EXPERTED SUBMISSION DATE (15) YES III yes, complete EXPECTED SUBMISSION DATE X NO During trip recovery, the reactor tripped at about 20% power on low steam generator level plus steam flow/feedwater flow mismatch. Faulty feedwater flow transmitter was recalibrated and will be replaced.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)

Prairie Island Unit 1

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TEXT (If more space is required, use additional NRC Form 386A's) (17)

During recovery from the reactor trip of 8-30-84, reactor power was at about 20%. The Loop A feedwater (SJ) flow channel had drifted low resulting in a steam flow/feedwater flow mismatch indication. Difficulty in controlling feedwater flow at this low power level was resulting in some steam generator level swings. At 0002 on August 31, 1984, the mismatch indication and a low steam generator level indication occurred together, satisfying the logic for reactor trip.

The feedwater flow transmitter (FT) had drifted out of calibration. The transmitter, a Foxboro D/P Cell Electronic Transmitter Model El3DH Style B, was recalibrated and unit restart was successful. A similar event involving feedwater flow controllability is described in RE 84-1 dated January 3, 1984. It is planned that feedwater regulating valve (FCV) trim be replaced at the next refueling. The faulty feedwater flow transmitter is being observed closely and will be replaced at the first opportunity. Feasibility of improving feedwater flow indication in the low flow range is being studied. Operators will practice controlling steam generator level under similar conditions on the simulator.





## Northern States Power Company

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September 28, 1984

U S Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

> PRAIRIE ISLAND NUCLEAR GENERATING PLANT Docket No. 50-282 License No. DPR-42 50-306 DPR-60

## Reactor Trip On Startup

The License Event Report for this occurrence is attached.

This event was reported via Emergency Notification System per 10 CFR Part 72 on August 31, 1984.

For David Musolf
Manager - Nuclear Support Services

DMM/EFE/dab

c: Regional Administrator-III, NRC NRR Project Manager, NRC Resident Inspector, NRC MPCA Attn: J W Ferman

Attachment

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