

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Vermont Yankee Nuclear Power Station	DOCKET NUMBER (2) 0 5 0 0 0 0 2 7 1 1	PAGE (3) 1 OF 0 2
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TITLE (4)
1984 Type C Testing

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	6	1 6 8 4	8 4	0 1 1	0 0	0	7	1 6 8 4			0 5 0 0 0
											0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) N	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 1 0 0	20.406(a)(1)(I)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.406(a)(1)(II)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.406(a)(1)(III)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME James P. Pelletier, Plant Manager	TELEPHONE NUMBER AREA CODE: 8 0 2 2 5 7 - 7 7 1 1
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
X	S B	V R	3 4 0	Y	X	B B	V A	1 8 0	Y
X	A A	V C	6 3 1	Y	B	S J	V A	3 9 1	Y

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing Type C Leak Rate Testing MSIV-86B, CRD-412A, PCAC V16-19-8 and FDW-96A were found to have seat leakage in excess of that permitted by Tech. Spec. Section 3.7.A.4.

Vermont Yankee will perform maintenance on all of the above valves and retest them to insure that seat leakage is within Tech. Spec. allowables prior to plant startup following the 1984 refueling outage.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Vermont Yankee	DOCKET NUMBER (2) 0 5 0 0 0 2 7 1	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	- 0 1 1	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 365A's) (17)

While performing Type C Leak Rate Testing, MSIV-86B, CRD-412A, PCAC V16-19-8 and FDW-96A were found to have seat leakage in excess of that permitted by Tech. Spec. section 3.7.A.4.

For MSIV-86B, CRD-412A and PCAC V16-19-8, a second isolation valve, in the applicable system, was tested and met the acceptance criteria. For FDW-96A, containment was provided by the water seal on the inboard check valve and the plant's capability to maintain a pressure greater than P_a on the feedwater system.

Vermont Yankee will perform maintenance on all of the above valves and retest them to insure that seat leakage is within Tech.Spec. allowables prior to Plant startup.

No similar events have been reported on MSIV-86B, CRD-412A or PCAC V16-19-8 in the last five years.

A similar event was reported on FDW-96A as LER 83-10.



VERMONT YANKEE NUCLEAR POWER CORPORATION

P. O. BOX 157
GOVERNOR HUNT ROAD
VERNON, VERMONT 05354

July 16, 1984

VYV84-370

U. S. Nuclear Regulatory Commission
Document No. 50-271
Washington, D. C. 20555

REFERENCE: Operating License DPR-28
Docket No. 50-271
Reportable Occurrence No. LER 84-11

Dear Sirs:

As defined by 10CFR50.73, we are reporting the attached Reportable Occurrence as LER 84-11.

Very truly yours,

James P. Pelletier
Plant Manager

RDP/cjm

cc: Regional Administrator
USNRC Office of Inspection and Enforcement
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

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