Tech. Spec. 6.9.A.2

Boston Edison
Pilgrim Nuclear Power Station
Rocky Hill Road
Plymouth, Massachusetts 02360

L. J. Olivier
Vice President Nuclear Operations
and Station Director

October 12, 1995
BECo Ltr. #95-107

U.S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Docket No. 50-293
License No. DPR-35

SEPTEMBER 1995 MONTHLY REPORT

In accordance with PNPS Technical Specification 6.9.A.2, a copy of the Operational Status Summary for Pilgrim Nuclear Power Station is attached for your information and planning. Should you have any questions concerning this report please contact me directly.

L.J. Olivier

WJM/laa/9458

Attachment

cc: Mr. Thomas T. Martin
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, PA 19406

Senior Resident Inspector

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OPERATING DATA REPORT

DOCKET NO. 50-293
DATE 10/12/95
COMPLETED BY: W.J. Munro
TELEPHONE (508) 830-8474

OPERATING STATUS		NOTES
1.	Unit Name	Pilgrim I
2.	Reporting Period	September 1995
3.	Licensed Thermal Power (MWt)	1998
4.	Nameplate Rating (Gross MWe)	678
5.	Design Electrical Rating (Net MWe)	655
6.	Maximum Dependable Capacity (Gross MWe)	696
7.	Maximum Dependable Capacity (Net MWe)	670

8. If Changes Occur in Capacity Ratings (Item Number 3 Through 7) Since Last Report, Give Reasons:

NONE

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any: N/A

		This Month	Yr-to-Date	Cumulative
11.	Hours in Reporting Period	720.0	6551..0	199943.0
12.	Hours Reactor Critical	720.0	4857.0	124057.1
13.	Hours Reactor Reserve Shutdown	0.0	0.0	0.0
14.	Hours Generator On-Line	720.0	4753.8	119614.9
15.	Hours Unit Reserve Shutdown	0.0	0.0	0.0
16.	Gross Thermal Energy Generated(MWH)	1414138.0	9213921.0	21196137.0
17.	Gross Electrical Energy Generated(MWH)	483730.0	3161870.0	71781544.0
18.	Net Electrical Energy Generated(MWH)	466384.0	3042769.0	69000546.0
19.	Unit Service Factor	100.0	72.6	59.8
20.	Unit Availability Factor	100.0	72.6	59.8
21.	Unit Capacity Factor (Using MDC Net)	96.7	69.3	51.5
22.	Unit Capacity Factor (Using DER Net)	98.9	70.9	52.7
23.	Unit Forced Outage Rate	0.0	5.2	12.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each) NONE
25. If Shutdown at End of Report Period, Estimated Date of Startup - Unit Operating

AVERAGE DAILY UNIT POWER LEVEL

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MONTH September 1995

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	657	17	488
2	659	18	661
3	658	19	658
4	658	20	659
5	660	21	659
6	660	22	660
7	663	23	597
8	663	24	536
9	660	25	662
10	662	26	661
11	660	27	661
12	659	28	659
13	660	29	656
14	662	30	654
15	660		
16	660		

This format lists the average daily unit power level in MWe-Net for each day in the reporting month, computed to the nearest whole megawatt.

BOSTON EDISON COMPANY
PILGRIM NUCLEAR POWER STATION
DOCKET NO. 50-293

OPERATIONAL SUMMARY FOR SEPTEMBER 1995

The unit started the reporting period at 100 percent Core Thermal Power (CTP), where it was essentially maintained until September 17,1995, when reactor power was reduced to perform a thermal backwash of the main condenser. While attempting to align the Salt Service Water System sluice gates for the backwash, the rear sluice gate failed to function properly and the backwash was cancelled. Reactor power was increased and the unit attained 100 percent CTP on September 18, 1995 where it was maintained until September 23, 1995. A power reduction to approximately 50 percent CTP was initiated on September 23, 1995 to facilitate a thermal backwash. Following a successful backwash reactor power was again returned to 100 percent CTP where it was maintained for the remainder of the reporting period.

SAFETY RELIEF VALVE CHALLENGES

MONTH OF SEPTEMBER 1995

Requirement: NUREG-0737 T.A.P. II.K.3.3

There were no safety relief valve challenges during the reporting period.

An SRV challenge is defined as anytime an SRV has received a signal to operate via reactor pressure signal (ADS) or control switch (manual). Reference BECo Ltr. #81-01 dated January 5, 1981.

REFUELING INFORMATION

The following refueling information is included in the Monthly Report as requested in an NRC letter to BECo, dated January 18, 1978:

For your convenience, the information supplied has been enumerated so that each number corresponds to equivalent notation utilized in the request.

1. The name of this facility is Pilgrim Nuclear Power Station, Docket Number 50-293.

2. Scheduled date for next refueling shutdown: March 29, 1997.

3. Scheduled date for restart following next refueling: May 12, 1997.

4. Due to their similarity, requests 4, 5, & 6 are responded to collectively under #6.

5. See #6.

6. The new fuel loaded during the 1995 refueling outage (RFO-10) is of a different design than that loaded in the previous refueling outage and consists of 136 new fuel assemblies.

7. (a) There are 580 fuel assemblies in the core.

   (b) There are 1765 fuel assemblies in the spent fuel pool.

8. (a) The station is presently licensed to store 3859 spent fuel assemblies. The spent fuel storage capacity is 2891 fuel assemblies. However, 23 spent fuel locations cannot be used due to refuel bridge limitations.

   (b) The planned spent fuel storage capacity is 3859 fuel assemblies.

9. With present spent fuel in storage, the spent fuel pool now has the capacity to accommodate an additional 1103 fuel assemblies.

PILGRIM NUCLEAR POWER STATION MAJOR SAFETY RELATED MAINTENANCE

DOCKET NO: 50-293
NAME: Pilgrim I
DATE: 10/12/95
COMPLETED BY: W.J. Munro
TELEPHONE: (508) 830-8474
REPORT MONTH: September 1995

SYSTEM	COMPONENT	MALFUNCTION	CAUSE	MAINTENANCE	CORRECTIVE ACTION TO PREVENT RECURRENCE	ASSOCIATED LER

No major safety related maintenance was completed during this reporting period.

UNIT SHUTDOWNS AND POWER REDUCTIONS

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TELEPHONE: (508) 830-8474
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NO.	DATE	TYPE 1	DURATION (HOURS)	REASON 2	METHOD OF SHUTTING DOWN REACTOR	LICENSE EVENT REPORT	SYSTEM CODE 4	COMPONENT CODE 5	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
07	09/17/95	S	0.0	H	N/A	N/A	N/A	N/A	Power reduction to facilitate a thermal backwash of the main condenser. The backwash was cancelled due to equipment problems.

There were no unit shutdowns or significant power reductions during the reporting period.

1	2	3	4&5
F-Forced S-Sched	A-Equip Failure B-Main or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Admin G-Operator Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparations of Data Entry Sheet Licensee Event Report (LER) File (NUREG-1022)