

Appendix

NOTICE OF VIOLATION

Northern States Power Company

Docket No. 50-282

As a result of the inspection conducted on June 20-21, 1984, and in accordance with the General Policy and Procedure for NRC Enforcement Actions, (10 CFR Part 2, Appendix C), the following violation was identified:

The Prairie Island Nuclear Generating Station, Unit 1 Technical Specification 4.2 states: "Inservice testing of ASME Code Class 1, Class 2 and Class 3 pumps and valves shall be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g)."

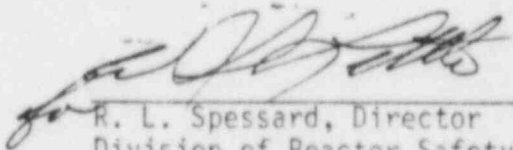
Contrary to the above, five motor operated, Category A Residual Heat Removal System valves are not leak tested or trended per the requirements of IWV-3420 of Section XI of the ASME Code.

This is a Severity Level V violation. (Supplement II).

The inspection showed that action would be taken to correct the identified item of noncompliance and to prevent recurrence. Consequently, no reply to this item of noncompliance is required and we have no further questions regarding this matter.

Dated

6/29/84

  
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R. L. Spessard, Director  
Division of Reactor Safety

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