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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (18)

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YES (If yes, complete EXPECTED SUBMISSION DATE)

While shutdown for a recirculation pipe replacement outage and during the performance of the "PRIMARY CONTAINMENT PERIODIC TYPE B AND C LEAKAGE TESTS" procedure (HNP-2-3952), as required by Tech. Specs. section 4.6.1.2.d, plant personnel determined that the results of numerous leakage tests for primary containment isolation valves were in excess of the limits specified in either Tech. Specs. sections 3.6.1.2. b or c, or the ASME SEC. XI criteria specified in HNP-2-3952, or 10CFR50, Appendix J, Paragraph III.C.3.b.

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EXPECTED SUBMISSION DATE (15) MONTH

YEAR

The cause for the unacceptable leakages is postulated to be the result of normal equipment wear.

The primary containment isolation valves (identified in the LER's text) were repaired, functionally tested satisfactorily, and returned to service prior to start up (start up commenced at approximately 03:36 CDT on 08/29/84).

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SUPPLEMENTAL REPORT EXPECTED (14)

IE 22/

U.S. NUCLEAR REGULATORY COMMISSION

APPHOVED OMB NO. 3150-0104 EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)		LE	R NUMBER (6)	PAGE (3)		
		YEAR		SEQUENTIAL NUMBER	REVISION NUMBER		
EDWIN I. HATCH, UNIT 2	0  5  0  0  0   3   6   6	8 4	_	01014	-0,1	0   2 OF	1 14

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This report is required by 10CFR50.73 (a)(2)(v)(C) and 10CFR50.73 (a)(2)(ii) due to the assumption that the plant did not meet the requirements of Tech. Specs. sections 3.6.1.2.b and 3.6.1.2.c, 10CFR50 Appendix J. Article III, A.1.d, and ASME section XI.

When the "PRIMARY CONTAINMENT PERIODIC TYPE B AND TYPE C LEAKAGE TESTS" procedure (HNP-2-3952) was performed, the plant was in cold shutdown for the recirculation pipe replacement outage.

- A. At the time of discovery, plant personnel assumed that the leakage of the following tests was such that the plant could not meet the .60 La requirement of Tech. Specs. section 3.6.1.2.b.l. (Refer to Table 1 for "VALVE DESCRIPTION", "AS FOUND LEAKAGE", "AS LEFT LEAKAGE", and "REPAIR SUMMARY". The table is arranged in alphanumeric order by valve MPL number.)
  - On 01/17/84, RHR Return to Recirculation Isolation Valve (2E11-F015B) was tested.
  - 2. On 01/17/84, Core Spray Inboard Isolation Valve (2E21-F005A) and Core Spray Isolation Check valve (2E21-F006A) (i.e., both valves in penetration X-16A) were tested.
  - On 01/19/84, RCIC Steam Supply Inboard Isolation valve (2E51-F007) and RCIC Steam Supply Outboard Isolation Valve (2E51-F008) were tested.
  - On 01/26/84, RHR Shutdown Cooling Suction Isolation Valve (2E11-F008) was tested.
  - 5. On 01/28/84, Vent Purge Return Inboard Isolation valve (2T48-F319) and Vent Purge Return Outboard Isolation valve (2T48-F320) (i.e., both valves in penetration X-26) were tested.
  - On 01/31/84, Vacuum Relief Inboard Isolation Valve (2T48-F310) was tested. 6.
  - 7. On 01/31/84, Vent Purge Outlet Inboard Isolation Valve (2T48-F318) was tested.
  - 8. On 02/01/84, LOCA H2 Recombiner Inboard Isolation Valve (2T49-F002B) was tested.
  - 9. On 02/01/84, H2/02 Sample Outboard Isolation valve (2P33-F010) was tested.
  - 10. On 02/01/84, Vacuum Relief Inboard Isolation Valve (2T48-F309) and Vacuum Relief Outboard Isolation Valve (2T48-F324) were tested (one test for Penetration X-205). Following the repair and cleaning of Vent Purge Supply Inboard and Outboard Isolation Valves, 2T48-F307 and 2T48-F103, the X-205 penetration had an as-left leakage of 700 ACCM which was acceptable.

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APPROVED OMB NO. 3150-0104 EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)
		YEAR SEQUENTIAL REVISION NUMBER
EDWIN I. HATCH, UNIT 2	0  5  0  0  0   3  6	684 - 01014 - 011013 OF1 14

TEXT (If more space is required, use edditional NRC Form 366A's) (17)

- 11. On 02/02/84, Drywell Differential Pressure System Discharge Inboard Isolation Valve (2T48-F209) was tested.
- 12. On 03/01/84, the CRW Pump Discharge Inboard Isolation Valve (2G11-F019) was tested.
- 13. On 03/02/84, TIP Drive Ball Inboard Isolation valve (2C51-J004A) was tested.
- 14. On 03/02/84, TIP Drive Ball Inboard Isolation valve (2C51-J004B) was tested.
- 15. On 03/02/84, TIP Drive Ball Inboard Isolation Valve (2C51-J004C) was tested.
- 16. On 03/02/84, TIP Drive Ball Inboard Isolation Valve (2C51-J004D) was tested.
- 17. On 03/02/84, TIP N2 Purge Outboard Isolation Solenoid Valve (2C51-F3012) was tested.
- On 03/02/84, Containment Spray Inboard Isolation Valve (2E11-F016A) was 18. tested.
- On 03/10/84, Containment Spray Isolation Valve (2E11-F028B) was tested. 19.
- 20. On 03/05/84, Chemical Pump Discharge Outboard Isolation Valve (2G11-F852) was tested. The redundant valve was operable. Further investigation proved 2G11-F851 to be the cause of failure, thus 2E11-F852 was operable.
- 21. On 08/11/84, Drywell Pneumatic Return Inboard Isolation Check Valve (2P70-F091) was tested.
- 22. On 01/31/84, Vacuum Relief Inboard Isolation valve (2T48-F311) was tested.
- At the time of discovery, plant personnel assumed that the leakage of the following tests was such that the plant could not meet the .009 La requirement of Tech. Specs. section 3.5.1.2.b.2.:
  - On 01/19/84, Dirty radwaste pump discharge outboard isolation valve (2G11-F004) was tested. The dirty radwaste pump discharge inboard isolation valve (2G11-F003) was found with an acceptable leakage of 20 ACCM; thus, this penetration remained leaktight.
  - 2. On 02/10/84, Primary feedwater inboard isolation valve (2B21-F010A) was tested.
  - 3. On 02/11/84, Primary feedwater outboard isolation valve (2B21-F077A) was tested. This leakage test failed due to the leakage of valve 2B21-F010A.
  - On 02/10/84, Primary feedwater inboard isolation valve (2B21-F010B) was tested.
  - On 02/12/84, Primary feedwater outboard isolation valve (2B21-F077B) was tested. This leakage test failed due to the leakage of valve 2B21-F010B.

U.S. NUCLEAR REGULATORY COMMISSION

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FACILITY NAME (1)	DOCKET NUMBER (2:	LEF	R NUMBER (6)	PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
EDWIN I. HATCH, UNIT 2	0  5  0  0  0   3   6	8 4 _	0,0,4	0 1	0 4 0	DF 1 14

TEXT IN more space is required, use additional NRC Form 386A's) (17)

- C. At the time of discovery, plant personnel assumed that the leakage of the following test was such that the plant could not meet the 11.5 SCFH requirement of Tech. Specs. section 3.6.1.2.c.:
  - On 1/16/84, Primary leakage test on penetration X-7C showed a leakage rate of 30 This tested the inboard MSIV (2B21-F022C), the outboard (2B21-F028C), and a MSIV leakage control system isolation valve (2E32-F001K). After the as-found test was performed, 2B21-F022C and 2B21-F028C were rebuilt as part of a pre-planned maintenance schedule. The X-7C penetration was retested on 5/30/84 with an as-found leakage rate of 0 SCFH.
- D. At the time of discovery, plant personnel assumed that the leakage of the following tests was such that the plant could not meet the requirements of the individual valve leakage rates specified as a result of ASME section XI:
  - On 01/20/84, the suction inboard isolation valves (2E11-F004A and 2E11-F030A) for the "A" RHR pump would not pressurize. investigation revealed that only the 2E11-F004A was leaking significantly.
  - 2. On 01/26/84, the core spray test line inboard isolation valve (2E11-F007A) was tested.
  - On 01/19/84, the HPCI pump suction outboard isolation valve (2E41-F042) and the HPCI pump suction inboard isolation valve (2E41-F051) (i.e., both valves in penetration X-207) were found to have a combined leakage of 1600 ACCM. Further investigation revealed that only 2E41-F051 was leaking significantly.
  - 4. On 01/20/84, the torus purification suction inboard isolation valve (2G51-F002) was tested.
  - 5. On 01/27/84, the suction inboard isolation valves (2E11-F004B and 2E11-F030B) for the "B" RHR pump would not pressurize. investigation revealed that only the 2E11-F004B was leaking significantly.
  - On 02/04/84, the "A" RHR testable check valve (2E11-F050A) was tested. 6.
  - 7. On 02/04/84, the "B" RHR testable check valve (2E11-F050B) was tested.
  - On 02/29/84, the HPIC turbine exhaust inboard isolation valve (2E41-F021) 8. and the HPCI turbine exhaust outboard isolation valve (2E41-F049) (i.e., both valves in penetration X-214) were found to have a combined leakage of 1750 ACCM. This leakage did not exceed the requirements of ASME section XI (1800 ACCM); however, on 07/09/84, the valves were overhauled and then successfully retested with an as-left leakage of 40 ACCM.
  - On 02/29/84, the RCIC turbine exhaust inboard isolation valve (2E51-F001) and the RCIC turbine exhaust outboard isolation valve (2E51-F040) (i.e., both valves in penetration X-212) would not pressurize.

NAC Form 366 (9-83)	LICENSEE EVENT REPO	RT (LER	) TE	EXT	CONT	rinu	ATIO	N			APPROVED EXPIRES	D OMB			
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	following test was such that the Appendix J, Article III. A 1.d. On 08/07/84, the Chilled Water Supply Isolation Valve (2)	r Supp	1y	Isc	olati	on	Valv								
NOTE:	The component failure inform COMPONENT " sheets are arran										OF 1	EACH	FAI	LED	

NRC Form 366A (9-83)

# LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES 8/31/85

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

# IDENTIFICATION OF EACH FAILED COMPONENT

MPL NUMBER	MANUFACTURER	MODEL NUMBER
. 2B21-F010A	 Rockwell	. Fig 970NTY(WCC) .
2221 2222	Rockwell	D. 0701/11/20\
. 2B21-F022C	Rockwell	. 1612 JMMNTY .
. 2B21-F028C	Rockwell	. 1612 JMMNTY .
. 2C51-J004A	General Electric	.112C2391P001 Rev. 20.
. 2C51-J004B	General Electric	.112C2391P001 Rev. 20.
. 2C51-J004C	General Electric	.112C2391P001 Rev. 20.
. 2C51-J004D	General Electric	.112C2391P001 Rev. 20.
. 2C51-F3012	General Electric	. CAT 117C1 .
. 2E11-F004A	Walworth Comp.	. Fig. S206WE .
. 2E11-F004B	 Walworth Comp.	. Fig. S206WE .
. 2E11-F007A	Walworth Comp.	. Fig. S206WE .
. 2E11-F008	 William Powell	. 19023 WE .
. 2E11-F015B	 William Powell	. 19023 WE .
. 2E11-F016A	 Walworth Comp.	. 5281 WE .
. 2E11-F028B	 Walworth Comp.	. 5206 WE .
. 2E11-F050A		.Fig. 970(WCC) JMMNTY.
. 2E11-F050B	 Rockwell	.Fig. 970(WCC) JMMNTY.
. 2E21-F005A	William Powell	. Fig. 16023 WE .
. 2E21-F006A	Rockwell	.Fig. 970(WCC)JMMNTY .
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NRC Form 366A (9-83)

# LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)			LER NUMBER (6)	PAGE (3)		
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EDWIN I. HATCH, UNIT 2	0  5  0  0  0   3   6	1 6 8	14	01014	-011	0 1 7 OF	1 14

TEXT Iff more space is required, use additional NRC Form: 366A's) (17)

# IDENTIFICATION OF EACH FAILED COMPONENT

MPL No.	MANUFACTURER	MODEL NO.
. 2E41-F021	Atwood & Morrell & Comp.	. DWG.21212-C .
00/1 00/0	Walworth Comp.	
. 2E41-F051	Fisher Controls Comp.	. Fig. 9120 .
. 2E51-F007	William Powell	
. 2G11-F004	Pacific Valves	. 153G-7-WE-CC .
. 2G11-F019	Pacific Velves	. 153G-7-WE-CC .
. 2G11-F852	Hancock Mfg. Co.	. Type 950W .
. 2G51-F002	William Powell	. Fig. 1503WE .
. 2P33-F010	Fisher Controls Comp.	. 657-ES .
. ZP64-F045	Walworth Comp.	Fig 5275
	Rockwell	
. 2148-F209	Pacific Valves	. 1536-7-WE-CC
. 2T48-F307	Fisher Controls Comp.	Fig. 9220
. 2T48-F103	Fisher Controls Comp.	Fig. 9220
		. Fig. 9220
**********************	Fisher Controls Comp.	
. 2T48-F318	***************************************	. Fig. 9220 .
	Fisher Controls Comp.	. Fig. 9220
. 2T49-F002B		
	wiiiigm Lowell	. Fig. 1503 WE .

NRC For (9-83)	n 3668	LICE	ENSEE EVE	NT REPORT	(LER) FAIL	URE	CONTINU	ATION	A	CLEAR REGULA PPROVED OMB I XPIRES 8/31/85	NO. 3150-0104
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Х	JM	ISV	G 0 8 0	N		Х	JM	I S V	R 3 4 0	Y	
Х	JM	ISV	G 0 8 0	N		X	JM	I S V	P 0 3 2	N	
X	JM	IISIV	G 0 8 0	N		X	BBB	I S V	F. 1 3 0	Y	
Х	ВО	IISIV	W 0 3 0	Y		X	B   B	I S V	F 1 1 3 10	N	
Х	Вјо	IISIV	W 0 1310	Y		Х	N H I	I <sub>I</sub> S <sub>I</sub> V	F   1   3   0	Y	
Х	B10	IISIV	W 0 3 10	Y		Х	N   H	I <sub>I</sub> S <sub>I</sub> V	F   1   3  0	Y	
Х	В10	IISIV	P 3 1015	Y		Х	B   B   1	IISIV	F   1 3 0	Y	
Х	ВІО	II IS I V	P \$ 1015	Y		Х	B   B	IJSJV	F   1   3  0	Y	
Х	B10	II IS IV	w p 1310	Y		Х	B   B   1	IISIV	F   1   3   0	Y	
Х	B 10	II IS I V	w p   3  0	Y		Х	J M	IISIV	P   1   3  0	N	
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NRC FORM 3668 (9-83)

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U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104 EXPIRES 8/31/85

ACILITY NAME (1)	DOCKET NUMBER (2)		L	ER NUMBER (6)	PAGE (3)			
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EDWIN I. HATCH, UNIT 366	0  5  0  0  0   3   6	6 8	4	0,0,4	-0,1	0,9	OF 1	14

TEXT If more space is required, use additional NRC Form 366A's) (17)

# TABLE NUMBER 1

(sheet 1 of 6)

# PRIMARY CONTAINMENT PERIODIC TYPE B & TYPE C LEAKAGE TESTS FAILURE SUMMARY

. VALVE DESCRIPTION	. F	AS OUND AKAGE	: 1	AS LEFT LEAKAGE	. REPAIR SUMMARY .
. 1. Primary Feedwater Inboard Isolation . Valve 2B21-F010A **	:		:	40 accm	installed new valve .
. 2. Primary Feedwater Inboard Isolation . Valve 2B21-F010B **	:	*		0	welded up steam cut . on seat area; then . machined seat; replaced hinge pins; then repacked valve .
. 3. Main Steam Inboard Isolation Valve . 2B21-F022C ***	. 3	O SCFH	:	0	rebuilt, and repacked valve
. 4. Main Steam Outboard Isolation Valve . 2B21-F028C ***	. 30	SCFH	:	0	rebuilt, and . repacked valve .
. 5. Tip Drive Ball Valve Inboard Isolation . 2C51-J004A **		0 accm		0	. New ball valve . installed .
. 6. Tip Drive Ball Valve Inboard Isolation . 2C51-J004B **	. 300	accm	:	0	. New ball valve installed
. 7. Tip Drive Ball Valve Inboard Isolation . 2C51-J004C **	. 2150	accm	·····	0	. New ball valve . installed .
. 8. Tip Drive Ball Valve Inboard Isolation . 2C51-J004D **	: 210X	ассв		0	. New ball valve installed

Would not pressurize Redundant isolation valve/ barrier in penetration was operable.

Both valves in penetration failed test.

ASME test only.

NAC Form 366A

#### LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) PAGE (3)
		YEAR SEQUENTIAL REVISION NUMBER
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TEXT (If more space is required, use additional NRC Form 386A's) (17)

# TABLE NUMBER 1

(sheet 2 of 6)

# PRIMARY CONTAINMENT PERIODIC TYPE B & TYPE C LEAKAGE TESTS FAILURE SUMMARY

. VALVE DESCRIPTION	. AS . FOUND . LEAKAGE	. AS . LEFT . LEAKAGE	REPAIR SUMMARY
. 9. Tip N <sub>2</sub> Purge Outboard Isolation . Solenoid Valve 2C51-F3012 **	. 60 accm	. 25 accm	. cleaned internals, . inspected seat and . orfices; then . reassembled .
. 10. RHR Pump Suction Inboard Isolation . Valve 2Ell-F004A ****	: *	. 350 accm	. machined seats, . aligned wedge and . repacked the valve
. 11. RHR Pump Suction Inboard Isolation . Valve 2E11-F004B ***	*	. 230 accm	polished seating surfaces and wedge; then repacked the valve
. 12. Core Spray Test Line Inboard Isolation . 2E11-F007A ****		. 30 accm	. cleaned and . repacked the valve
. 13. RHR Shutdown Cooling Suction Isolation . Valve 2E11-F008 **	. 3600 accm	. 50 accm	repolished seats, wedge, and cleaned the pressure seal area; then repacked the valve
. 14. RHR Return to Recirculation Isolation . Valve 2Eil-F015B **	. 1950 accm	: o	repolished seats, wedge, and cleaned the pressure seal area

Would not pressurize

Redundant isolation valve/ barrier in penetration was operable.

\*\*\* Both valves in penetration failed test.
\*\*\*\* ASME test only.

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US NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104 EXPIRES 8/31/85

FACILITY NAME (1)	DOC	KET	NUA	MBER	(2)						LE	R NUM	85R (	11				P	AGE	3)	
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TEXT (If more space is required, use additional NRC Form 386A's) (17)

# TABLE NUMBER 1

(sheet 3 of 6)

# PRIMARY CONTAINMENT PERIODIC TYPE B & TYPE C LEAKAGE TESTS FAILURE SUMMARY

. VALVE DESCRIPTION	:	AS FOUND LEAKAGE	:	AS LEFT LEAKAGE	. REPAIR SUMMARY
. 15. Containment Spray Inboard Isolation . Valve 2E11-F016A **		1000 accm		220 accm	replaced disc, polished stem, and repacked the valve
. 16. RHR Test Line Isolation Valve . 2E11-F028B **		1200 accm	:	800 accm	. replaced packing and 0 rings
. 17. RHR Testable Check Valve 2E11-F050A ***	*.	*		0	. replaced the entire . valve
. 18. RHR Testable Check Valve 2E11-F050B ***	*	*	:	0	. replaced the entire . valve
. 19. Core Spray Inboard Isolation Valve 2E21-F005A ***  (PENETRATION X-16A)	:	1630 accm		20 accm	cleaned wedge and seat, then repacked valve
. 20. Core Spray Outboard Isolation Valve 2E21-F006A *** (PENETRATION X-16A)	:	*		1200 accm	. realigned hinge . pin, repacked valve
. 21. HPCI Turbine Exhaust Inboard Isolation . Valve 2E41-F021 ****	:	1750 accm	•	40 accm	. polished seat and . disc, repacked valve
. 22. HFCI Turbine Exhaust Outboard Isolation . Valve 2E41-F049 ****		1750 accm		40 accm	. cleaned valve, . machined seat, . repaired disc, . repacked valve

Would not pressurize

Redundant isolation valve/ barrier in penetration was operable.

Both valves in penetration failed test.

\*\*\*\* ASME test only.

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104 EXPIRES 8/31/86

FACILITY NAME (1)	DOCKET NUMBER (2)			PAGE (3)
		YEAR	SEQUENTIAL REVISION NUMBER	
EDWIN I. HATCH, UNIT 366	0 15 10 10 10 13 16 16	8 4	_004_011	1   2 OF 1  4

TEXT (If more space is required, use additional NRC Form 366A's) (17)

#### TABLE NUMBER 1

(sheet 4 of 6)

# PRIMARY CONTAINMENT PERIODIC TYPE B & TYPE C LEAKAGE TESTS FAILURE SUMMARY

. VALVE DESCRIPTION .	. AS . FOUND . LEAKAGE	AS LEFT LEAKAGE	REPAIR SUMMARY
. 23. HPCI Pump Suction Inboard Isolation . Valve 2E41-F051 ****	. 1600 accm	0	installed new liner,. flapper, shaft, tappered pins; replaced 0 rings, and repacked valve
. 24. RCIC Turbine Exhaust Inboard Isolation valve 2E51-F001 ****	: •	450 accm	replaced disc seat, replaced valve seat, . then repacked valve .
. 25. RCIC Steam Supply Inboard Isolation . Valve 2E51-F007 **		30 accm	wedge was not . hitting the seat . correctly Realigned and . repacked the valve .
. 26. DRW Pump Discharge Outboard Isolation . Valve 2G11-F004 **		less than 20 accm	. lapped wedge, . cleaned valve, . repacked valve
. 27. CRW Pump Discharge Inboard Isolation valve 2G11-F019 **	. 750 accm	. 40 accm	. cleaned wedge and . repacked the valve .
. 28. Chemical Pump Discharge Inboard . Isolation Valve 2G11-F852 **	•	. 25 accm	cleaned the seat

Would not pressurize Redundant isolation valve/ barrier in penetration was operable.

\*\*\* Both valves in penetration failed test.

\*\*\*\* ASME test only.

US NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APPROVED OMB NO 3150-0104 EXPIRES 8/31/85 DOCKET NUMBER (2) FACILITY NAME (1) LER NUMBER (6) PAGE (3) SEQUENTIAL NUMBER REVISION YEAR EDWIN I. HATCH, UNIT 366 0 |5 |0 |0 |0 | 3 |6 | 6 | 8 | 4 -0 | 1 | 1 | 3 | OF 1 | 4 01014

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#### TABLE NUMBER 1

(sheet 5 of 6)

# PRIMARY CONTAINMENT PERIODIC TYPE B & TYPE C LEAKAGE TESTS FAILURE SUMMARY

:	VALVE DESCRIPTION	:	AS FOUND LEAKAGE		AS LEFT LEAKAGE	. REPAIR SUMMARY	:
. 29.	Torus Purification Suction Inboard Isolation Valve 2G51-F002 ****	:	300 accm	:	25 accm	. Cleaned and . repacked valve	:
. 30.	H202 Sample Outboard Isolation Valve 2P33-F010 **	:	300 acca	::::	20 accm	. Cleaned valve, . replaced gaskets, . and packing; then . set up valve . via a bench test	
. 31.	Chilled Water Supply Isolation Valve 2P64-F045 **	:	2200 accm	:	120 accm	. Machined seat and . globe; repacked . valve	
. 32.	Drywell Pneumatic Return Inboard Isolation valve 2P70-F091 **	:	*	:	190 accm	. cleaned and . repacked the valve	
. 33.	Drywell Differential Pressure System Discharge Inboard Isolation Valve 2T48-F209 **	:	2200 accm		80 accm	. Replaced wedge and . packing	
. 34.	Vent Purge Supply Inboard Isolation Valve 2T48-F307 *** (PENETRATION X-25)	:	125 accs		0	Replaced 0 rings and glands, polished shaft, then repacked the valve	
. 35.	Vent Purge Supply Outboard Isolation Valve 2T48-F103 *** (PENETRATION X-25)	:	3800 accm		700 accm	. Cleaned and . repacked the valve	

Would not pressurize

Redundant isolation valve/ barrier in penetration was operable. Both valves in penetration failed test.

\*\*\*\* ASME test only.

NRC Form 366A

# LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO 3150-0104 EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)			LE	R NUMBER IS	)			PAGE (3	0	-
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TEXT (If more space is required, use additional NRC Form 386A's) (17)

# TABLE NUMBER 1

(sheet 6 of 6)

# PRIMARY CONTAINMENT PERIODIC TYP B & TYPE C LEAKAGE TESTS FAILURE SUMMARY

. VALVE DESCRIPTION	. AS . FOUND . LEAKAGE	. AS . LEFT . LEAKAGE	REPAIR SUMMARY
. 36. Vacuum Relief Inboard Isolation Valve 2T48-F310 **	360 accm	. 80 accm	Replaced "0" rings; cleaned and repacked the valve
. 37. Vacuum Relief Inboard Isolation Valve 2T48-F311 **	. 2200 accm	, 0	Replaced "O" rings; cleaned and repacked the valve
. 38. Vent Purge Outlet Inboard Isolation Valve 2T48-F318 **	. 70 accm	. 0	Replaced "O" rings; cleaned and repacked the valve
. 39. Vent Purge Return Inboard Isolation Valve 2T48-F319 ***  (PENETRATION X-26)	: *	. 0	Replaced "O" rings; cleaned and repacked the valve
. 40. Vent Purge Return Outboard Isolation Valve 2T48-F320 ***  (PENETRATION X-26)	. 690 accm	. 0	Replaced "O" rings; cleaned and repacked the valve
. 41. LOCA H <sub>2</sub> Recombiner Inboard Isolation . Valve 2T49-F002B **	. 350 accm	. 160 accm	. Cleaned, repolished . and repacked valve

Would not pressurize

<sup>\*\*</sup> Redundant isolation valve/ barrier in penetration was operable.

<sup>\*\*\*</sup> Both valves in penetration failed test.

<sup>\*\*\*\*</sup> ASME test only.

Georgia Power

Edwin i. Hatch Nuclear Plant

September 28, 1984 GM-84-862

PLANT E. I. HATCH Licensee Event Report Docket No. 50-366

United States Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Attached is Licensee Event Report No. 50-366/1984-04, Rev. 1. This report is required by 10 CFR 50.73(a)(2)(v)(C) and 10 CFR 50.73(a)(2)(ii).

General Manager

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HCN/TLE/djs

xc: R. J. Kelly

R. E. Conway J. T. Beckham, Jr.

P. D. Rice

K. M. Gillespie

S. B. Tipps

R. D. Baker

Control Room

Document Control