



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 101 TO FACILITY OPERATING LICENSE NO. DPR-57
AND AMENDMENT NO. 38 TO FACILITY OPERATING LICENSE NO. NPF-5

GEORGIA POWER COMPANY
OGLETHORPE POWER CORPORATION
MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA
CITY OF DALTON, GEORGIA

EDWIN I. HATCH NUCLEAR PLANT, UNITS NOS. 1 AND 2

DOCKETS NOS. 50-321 AND 50-366

1. INTRODUCTION

In November 1980, the staff issued NUREG-0737, "Clarification of TMI Action Plan Requirements" which included all TMI Action Plan items approved by the Commission for implementation at nuclear power reactors. NUREG-0737 identifies those items for which Technical Specifications are required. A number of items which require Technical Specifications were scheduled for implementation by December 31, 1981. The staff provided guidance on the scope of Technical Specifications for all of these items in Generic Letter 83-02. Generic Letter 83-02 was issued to all Boiling Water Reactor (BWR) licensees on January 10, 1983. In Generic Letter 83-02 the staff requested licensees to:

- a. review their facility's Technical Specifications to determine if they were consistent with the guidance provided in the generic letter, and
- b. submit an application for a license amendment where deviations or absence of Technical Specifications were found.

By letters dated April 22, 1983 and September 9, 1983, Georgia Power Company (the licensee) responded to Generic Letter 83-02 by submitting Technical Specification change requests for Edwin I. Hatch, Units 1 and 2. By letter dated May 2, 1983, the licensee withdrew its requests for some of the changes it had requested in its April 22, 1983 letter. This evaluation covers the balance of the change requests, which includes the following TMI Action Plan Items:

- a. Limit Overtime (I.A.1.3)
- b. Reporting of Safety and Relief Valve Failures Challenges (II.K.3.3)
- c. RCIC Restart and RCIC Suction (II.K.3.13 and II.K.3.22)
- d. HPCI and RCIC Modification (II.K.3.15)
- e. Common Reference Level (II.K.3:27)

2. DISCUSSION AND EVALUATION

A. Limit Overtime (I.A.1.3)

On June 15, 1982, we transmitted to licensees of operating plants a revised version of the Commission's Policy Statement on nuclear power plant staff working hours (Generic Letter 82-12). This policy statement was also referenced in Generic Letter 83-02.

The licensee has proposed adequate Technical Specifications to establish administrative procedures to limit working hours of the staff who perform safety-related functions and establish guidelines on the use of overtime. We have reviewed the licensee's proposed Technical Specifications and find that they adequately satisfy the intent of the Commission's Policy Statement and the guidelines provided in Generic Letter 83-02. We find these Technical Specifications to be acceptable.

B. Reporting of Safety/Relief Valve Failures and Challenges (II.K.3.3)

In Generic Letter 83-02, the staff requested licensees to formalize the reporting requirements for safety/relief valve failures and challenges. The licensee has proposed Technical Specifications which will require the licensee to report the failures promptly with written follow-up, and the challenges in an annual report. This is consistent with our guidance provided in Generic Letter 83-02. Therefore we find it acceptable.

C. RCIC Restart and RCIC Suction (II.K.3.13 and II.K.3.22)

TMI Action Plan Items II.K.3.13 and II.K.3.22 recommend modifications to the Reactor Core Isolation Cooling System (RCIC) such that:

1. The system will restart on subsequent low water level after it has been terminated by a high water level in the reactor vessel, and
2. RCIC system suction will automatically switchover from the condensate storage tank to the suppression pool when the condensate storage tank level is low.

In Generic Letter 83-02, the staff provided the guidance on necessary changes in the Technical Specifications for implementation of the modifications. The proposed changes in Technical Specifications for RCIC are in response to Generic Letter 83-02. We have reviewed the proposed changes in the Technical Specifications and determined that the changes are consistent with the guidance provided in Generic Letter 83-02. We find the changes acceptable.

D. Isolation of HPCI and RCIC Modifications (II.K.3.15)

TMI Action Plan Item II.K.3.15 recommends that the pipe-break-detection circuitry should be modified so that pressure spikes resulting from high-pressure coolant injection (HPCI) and RCIC system initiation will not cause inadvertent system isolation. The licensee has completed the modification recommended by this item.

The staff provided guidance on the necessary changes in the Technical Specifications by Generic Letter 83-02. The licensee has proposed changes in the Technical Specifications for Hatch Unit 2. The licensee indicated that present Technical Specifications for Hatch Unit 1 do not include isolation system instrumentation response times. However, the surveillance requirements on time delay relay are assured by logic system functional tests which are required by present HPCI and RCIC specifications.

We have reviewed the current Technical Specifications for HPCI and RCIC systems for Hatch Unit 1 and proposed changes in the Technical Specifications for Hatch Unit 2. We have determined that specifications for Unit 1 adequately cover the surveillance requirements on time delay relay included in HPCI and RCIC systems. We have also determined that the proposed changes in Unit 2 are consistent with our guidance in Generic Letter 83-02. We find the proposed changes to be acceptable.

E. Common Reference Level (II.K.3.27)

The guidance provided in Generic Letter 83-02 recommended that the figure defining reactor vessel water levels should be changed to reflect the common reference level established by this Action Plan Item. A sample figure was provided in the guidance.

In response to Generic Letter 83-02, the licensee submitted a proposed change in Figure 2.1-1 for Hatch Unit 1. This figure for Hatch Unit 2 was revised by a previous amendment.

We have reviewed the revised figure which defines reactor vessel water levels. We find that it reflects the common reference level established by the Action Plan Item II.K.3.27. We find the proposed change to be acceptable.

3. ENVIRONMENTAL CONSIDERATIONS

These amendments involve a change in the installation or use of a facility component located within the restricted area as defined in 10 CFR Part 20. We have determined that the amendments involve no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no significant increase in individual or cumulative occupational radiation exposure. The Commission has previously issued a proposed finding that these amendments involve no significant hazards consideration and there has been no public comment on such finding. Accordingly, these amendments meet the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of these amendments.

4. CONCLUSION

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations, and the issuance of these amendments will not be inimical to the common defense and security or to the health and safety of the public.

Dated: July 11, 1984

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