

U. S. NUCLEAR REGULATORY COMMISSION

REGION V

Report No. 50-397/84-24

Docket No. 50-397

License No. NPF-21

Licensee: Washington Public Power Supply System

P. O. Box 968

Richland, Washington 99352

Facility Name: WNP-2

Inspection at: WNP-2 Site, Benton County, Washington

Inspection conducted: August 20-24, 1984

Inspectors: *Dennis J. Willett* 9-19-84
D. J. Willett, Reactor Inspector Date Signed

L. R. Kanow 9/19/84
L. R. Kanow, Reactor Specialist Date Signed

Approved by: *R. T. Dodds* 9/19/84
R. T. Dodds, Chief Date Signed
Reactor Projects Section 1

Summary:

Inspection on August 20-24, 1984 (Report No. 50-397/84-24)

Areas Inspected: Routine, unannounced safety inspection of follow-up of previous inspection findings, general employee training, non-licensed training, qualification and control of vendor service contract personnel, and follow-up of TMI (NUREG-0737) items. The inspection involved a total of 64 onsite hours by two NRC inspectors.

Results: Of the five areas inspected, no violations or deviations were identified.

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DETAILS

1. Personnel Contacted

- +J. W. Shannon, Director, Power Generation
- +J. D. Martin, WNP-2 Plant Manager
- +C. M. Powers, Assistant Plant Manager
- +D. H. Walker, Plant Quality Assurance Manager
- +J. F. Peters, Plant Administrative Manager
- +J. T. Little, Plant Training Coordinator
- +R. L. Corcoran, Plant Operations Manager
- +G. D. Rhinehart, HP, Chem and GET Supervisor
- +W. L. Fitch, Washington State EFSEC Administrator
- +R. B. Quay, General and Technical Support Training Manager
- +R. Mertins, Compliance Engineer
- +G. C. Sorensen, Regulatory Programs Manager
- +R. S. Barbee, Acting Technical Services Manager
- +G. D. Bouchey, Director Support Services
- R. R. Stickney, Technical Training Manager
- R. E. Ray, I&C
- G. Bradstead, Engineer
- K. C. Bleiler, Fire/Safety Training
- J. A. Landon, Plant Maintenance Manager

The inspectors also held discussions with other licensee and contract personnel during this inspection. This included licensed and non-licensed operators, plant staff engineers, technicians, administrative assistants and quality assurance personnel.

+Denotes those personnel present during the exit interview on August 24, 1984.

2. General Employee Training

The inspector reviewed the General Employee Training (G.E.T.) program to determine the degree of compliance with the following requirements and commitments:

- FSAR - Appendix C.3.0
- Technical Specifications - Section 6, Part 6.4 "Training"
- Operational Quality Assurance Program Description, Appendix II, Section II.1
- Regulatory Guide 1.8, Revision 1-B, May 1977 "Personnel Selection and Training"
- ANSI/ANS 3.1 (1978), "Selection and Training of Nuclear Power Plant Personnel"
- FSAR 13.2.1.1.7 "General Employee Training"

- 10 CFR 50 Appendix B, Criteria II "Quality Assurance Program"
- Plant Procedures Manual, Section 1.8, Procedure Number 1.8.2 "General Employee Training"

The inspectors attended the general employee training short course (for personnel with previous radiological training) and discussed the content with supervisors and personnel responsible for program development and implementation. Training in the areas of: General plant and facility lay-out, plant security, emergency and evacuation response, and radiological health and safety (including prenatal precautions), is provided to satisfy the pre-badging requirement for unescorted access to the WNP-2 site. The inspectors expressed the concern that quality assurance indoctrination and general fire protection/safety were not currently an unescorted access prerequisite.

The licensee stated that quality assurance training is scheduled for plant personnel at some interval after their initial access. The licensee informed the inspectors that they intended to include, as part of the regular G.E.T. program and refresher, quality assurance indoctrination and general fire protection/safety. This commitment will be implemented by October 1, 1984 (84-24-01).

No violations or deviations were identified.

3. Non-Licensed Training

The inspectors reviewed the licensee's program for non-licensed training. The inspector's review included discussions with supervisors and personnel responsible for program management and implementation.

The plant training program (which is the responsibility of the plant manager) is managed by the Plant Training Coordinator. The coordinator conveys the training needs of the plant to the training department, coordinates plant personnel into existing programs, and reviews and comments on the adequacy of training department courses. In addition to the coordinator's primary duties he also reviews plant modifications to ascertain the need to initiate new or additional training which may be warranted by these modifications.

Recently the plant administration has moved from a reactive response mode to an anticipatory one in which plant specific (manufacturer, model, etc) as well as craft skills (soldering, vibration measurements, digital electronics, etc) training can be forecast and scheduled.

The Technical Training Department develops and administers the licensed and non-licensed training program, staffs and manages the courses and simulator, and maintains all training records. The Training Department is comprised of four active sections: General and Technical Support Training, Training Development and Evaluation Programs, Simulator Programs and Nuclear Licensed Training.

The general and technical support training section consists of units responsible for: General Employee Training, Fire Protection/Brigade and Safety, Health Physics and Chemistry, Maintenance Training (I&C, Electrical, Mechanical), and Engineering Fundamentals (math, physics, thermo).

No violations or deviations were identified.

4. TMI Activities (NUREG 0737)

Item II.K.3.22 (Closed): "Automatic switchover of reactor core isolation cooling (RCIC) suction". This item requires that clear and cogent procedures exist for manual switchover of the RCIC suction on low condensate storage tank level to the suppression pool. These procedures are to be in effect until modifications can be implemented to make this switchover an automatic feature. The automatic switchover feature has been incorporated into the WNP-2 RCIC design and is functional.

Item II.3.24 (Open): "Confirm adequacy of space cooling for high-pressure core spray and RCIC systems." This item requires that the pump room temperatures remain within allowable limits during long-term operation. The acceptability should be verified during a complete loss of AC power including loss of off-site AC power to support systems and coolers for a period of at least 2 hours. While the adequacy of space cooling was verified during the loss of off-site power testing the licensee intends to confirm the adequacy once the plant has reached a sustained 100% power level, producing a representative decay heat source for testing as-close-as-practicable. This item will be closed when the above testing is completed.

Item II K.3.27 (Closed): "Common reference level for vessel instrumentation". This item requires all vessel level instruments be referenced to the same point. The guide intimates that the bottom of the vessel or top of active fuel are reasonable reference points. The licensee has referenced all level instruments from the mid-plane of the steam dryers (527.5"). This is appropriate for BWR's because historically BWR technology think's in terms of a negative water level as being a less conservative condition and logically accommodates the spans that the industry is familiar with.

Item II.K.3.28 (Closed): "Qualification of Automatic Depressurization System (ADS) accumulators." This item requires the licensee to verify that the ADS valve accumulators are provided with sufficient capacity to cycle the valves five times (at design pressures), and are designed to function in a hostile environment for a 100 day period following an accident.

During the preoperational testing of the ADS system, it was determined that while the accumulators capacity was sufficient to cycle the valves five times, the recharge time of the accumulators exceeded the NSSS vendor recommendations. This issue is a inspection report follow-up Open Item No. 50-397/83-60-02.

The licensee determined that the excessive recharge times was caused by to strong of a spring on the check valves between the accumulators and supply lines. These springs have been replaced and satisfactory tested.

Initially the containment was not going to be inerted (with nitrogen) so to maintain the containment atmosphere the ADS accumulators can also be charged with nitrogen (since the ADS valve operator would relieve to containment). The accumulators are supplied by two 30 day banks of nitrogen bottles located in the reactor building. Two remote nitrogen connections (outside the reactor building) ensure post accident access for resupply.

This assures that the ADS system can satisfy its long term operability requirement of 148 valve operations over a six month period following an accident. SER supplement 2 concluded that the supply for the ADS accumulators was adequate for long term operation. Follow-up item 83-60-02 is here in closed.

The licensee has qualified the accumulators, valves, and associated equipment and instrumentation in the ADS system to insure that they are capable of functioning during and following exposure to a hostile environment.

No violations or deviations were identified.

5. Qualification and Control of Vendor Service Contract Personnel

The inspectors reviewed the licensees program, practices and procedures for evaluating and certifying vendor services contract personnel and the control of these personnel while on-site and working on equipment or documentation.

The health physics organization identifies pre-purchase criteria for contract services. They then select personnel based on their qualifications, experience and expertise. They test these candidates and send the successful ones through general employee training. After these personnel are badged, they receive special training in administrative procedures and policies before being assigned to a section supervisor.

The Plant Quality Assurance Organization can and has pre-qualified contract organizations. QA performs a evaluation of the organization based on 10 CFR 50 Appendix B Criteria and qualifies the contractor to provide pre-defined services.

The licensee has committed (by October 1, 1984) to expand and elaborate practices, in Plant Procedure Manual PPM 1.8.4 ("Certification of Plant and Support Contractor Personnel"), requiring department managers to certify contract personnel for specific independent unsupervised activities based upon a review of the personnel qualifications, interviews, and personnel knowledge of the individuals. Additional direction in procurement and maintenance procedures will identify that contract personnel will work to plant procedures and controls unless specific unique contract services have been pre-qualified by the QA Department (84-24-02).

Plant philosophy is to perform as much of the work as possible with Supply System personnel in conjunction with contract services to attain desired expertise and activity control.

No violations or deviations were identified.

6. Exit Interview

The inspectors met with representatives (denoted in paragraph 1) at the conclusion of the inspections on August 24, 1984. The scope and findings of these inspections were discussed during the exit interview and are summarized in paragraph 1 through 5 of this report.