

ENCLOSURE 1

NOTICE OF VIOLATION

Florida Power Corporation
Crystal River 3

Docket No. 50-302
License No. DPR-72

The following violations were identified during an inspection conducted on February 28 - March 27 1984. The Severity Levels were assigned in accordance with the NRC Enforcement Policy (10 CFR Part 2, Appendix C).

1. Technical Specification 4.7.7.1.c.4 requires verification of the control room emergency ventilation system flow rate during system operation in the emergency recirculation mode.

Contrary to the above, as of March 15, 1984, the control room emergency ventilation system flow rate had not been verified during operation in the emergency recirculation mode.

This is a Severity Level IV violation (Supplement I).

2. Technical Specification 6.8.1.c requires adherence to procedures involving surveillance and test activities. Surveillance procedure SP-317 provides the instructions for performing a Reactor Coolant System (RCS) leakage check. Procedure steps 6.4.1 through 6.4.3 require data to be recorded on Enclosures 1 and 2 at the end of the data collection interval consisting of at least eight hours of steady state operation. The procedure then requires that the data from Enclosure 1 and Enclosure 2 provide input for the RCS leakage calculation performed on data sheet Enclosure 3.

Contrary to the above, on March 6, 1984, data from Enclosure 1, completed at approximately 1:00 p.m. and representing only six hours of steady state operation, was used to compute the RCS leakage with data from Enclosure 2 that was completed at 4:00 p.m. The Enclosure 2 data represented eight hours of steady state operation.

This is a Severity Level IV violation (Supplement I).

3. 10 CFR Part 50, Appendix B, Criterion 12 requires instrumentation that affects quality to be calibrated at specific periods to maintain accuracy.

Section 1.7.1.12 of the Florida Power Corporation (FPC) Quality Program implements the requirements of 10 CFR Part 50, Appendix B, Criterion 12 and specifies that procedures to control calibration of instrumentation used in the measurement and monitoring of safety-related systems have been established and implemented.

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Surveillance Procedure SP-317 provides the instructions for performing an RCS leakage check and specifies the instrumentation that may be utilized to perform the check. Two of the instruments specified for use are computer points R-731 (which measures average RCS temperature, Tave) and P-714 (which measures reactor power).

Contrary to the above, as of March 22, 1984, computer points R-731 and P-714, that were being used to determined RCS leakage, were not calibrated.

This is a Severity Level IV violation (Supplement I).

Pursuant to 10 CFR 2.201, you are required to submit to this office within 30 days of the date of this Notice, a written statement or explanation in reply, including: (1) admission or denial of the alleged violations; (2) the reasons for the violations if admitted; (3) the corrective steps which have been taken and the results achieved; (4) corrective steps which will be taken to avoid further violations; and (5) the date when full compliance will be achieved.

Security or safeguards information should be submitted as an enclosure to facilitate withholding it from public disclosure as required by 10 CFR 2.790(d) or 10 CFR 73.21.

Date: APR 19 1984