

Commonwealth Edison Company
LaSalle Generating Station
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October 13, 1995

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Enclosed for your information is the monthly performance report covering
LaSalle County Nuclear Power Station for September, 1995.

A handwritten signature in dark ink, appearing to read "D. J. Ray". The signature is fluid and cursive, with the first letters of the first and last names being capitalized and prominent.

D. J. Ray
Station Manager
LaSalle County Station

DJR/mkl

Enclosure

cc: H. J. Miller, Regional Administrator - Region III
NRC Senior Resident Inspector - LaSalle
IL Department of Nuclear Safety - LaSalle
IL Department of Nuclear Safety - Springfield, IL
NRR Project Manager - Washington, D.C.
GE Representative - LaSalle
Regulatory Assurance Supervisor - LaSalle
Licensing Operations Director - Downers Grove
Nuclear Fuel Services Manager - General Office
Off-Site Safety Review Senior Participant - Downers Grove
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LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

September 1995

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION (UNIT 1)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815) 357-6761, extension 2056.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 1)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1090 Mwe.
11	0200	Reduced power level to 900 Mwe for performance of a rod set and inspection of the Motor Driven Reactor Feed Pump mixing flow valve.
	1000	Increased power level to 1128 Mwe.
16	1015	Reduced power level to 1075 Mwe to swap the Heater Drain and Condensate/Condensate Booster Pumps.
	1400	Increased power level to 1110 Mwe.
24	0408	Manual Peactor scram due to loss of the '1B' Turbine Driven Reactor Feed Pump during surveillance performance.
27	2029	Reactor critical.
28	1017	Generator on-line at 60 Mwe.
29	1738	Generator at 350 Mwe, upshifted Reactor Recirculation pumps.
29	2400	Power level at 1000 Mwe.
30	2400	Reactor critical, Generator on-line at 1000 Mwe.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION

On September 27, 1995, Amendment 106 was issued to license NPF-11 (Unit 1). This amendment allowed the extension of surveillance test intervals to accommodate the start date change of the L1R07 outage from September 30, 1995 to January 27, 1996.

C. SUBMITTED LICENSEE EVENT REPORTS (Unit 1)

<u>LER No.</u>	<u>Occurrence Date</u>	<u>Description</u>
95-012	07/11/95	Two Fire Protection system valves were identified as not being verified to be in the correct position within the time frame as required by the Technical Specifications.
95-014	08/17/95	An automatic Reactor scram occurred due to the '1D' Reactor Protection System trip which caused a group 1 isolation.
95-015	08/24/95	Fire Protection isolation valve 1FP228 was not included in the monthly fire protection system valve surveillance and inturn was not inspected as required by the Technical Specifications.

D. DATA TABULATIONS (Unit 1)

1. Operating Data Report (See Table 1)
2. Average Daily Unit Power Level (See Table 2)
3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 1)

1. Safety Relief Valve Operations
(None)
2. Major Changes to Radioactive Waste Treatment Systems
(None)
3. Static O-Ring Failures
(See Attachment A, Note: The attached Static O-Ring Failure was inadvertently omitted from the July 1995 Monthly Performance Report)
4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LASALLE ONE
DATE October 11, 1995
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: September 1995 GROSS HOURS IN REPORTING PERIOD 720
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323 MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-M 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

4. REASONS FOR RESTRICTION (IF ANY):

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	631.7	6,093.1	72,733.0
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,641.2
7. GENERATOR ON-LINE TIME (HOURS)	617.9	6,020.0	71,131.1
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1.0
9. THERMAL ENERGY GENERATED (MWh _t)	1,925,935	19,333,424	212,020,249
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	642,011	6,480,125	70,871,016
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	617,973	6,270,096	68,039,240
12. REACTOR SERVICE FACTOR (%)	87.7	93.0	70.6
13. REACTOR AVAILABILITY FACTOR (%)	87.7	93.0	72.2
14. UNIT SERVICE FACTOR (%)	85.8	91.9	69.1
15. UNIT AVAILABILITY FACTOR (%)	85.8	91.9	69.1
16. UNIT CAPACITY FACTOR (USING MDC) (%)	82.8	92.4	63.8
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	79.6	88.8	61.3
18. UNIT FORCED OUTAGE FACTOR (%)	14.2	5.8	8.1

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): 01/27/96

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

PROJECT NO. 050-373
UNIT LASALLE ONE
DATE October 11, 1995
COMPLETED BY M.J. GIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: September 1995

DAY	POWER	DAY	POWER
1	1,050	17	1,069
2	1,046	18	1,072
3	1,047	19	1,070
4	1,039	20	1,065
5	1,044	21	1,064
6	1,037	22	1,057
7	1,035	23	1,054
8	1,038	24	167
9	1,035	25	-13
10	1,028	26	-12
11	1,040	27	-12
12	1,075	28	75
13	1,071	29	350
14	1,069	30	954
15	1,071	31	
16	1,065		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 1)

<u>YEARLY SEQUENTIAL NUMBER</u>	<u>DATE (YYMMDD)</u>	<u>TYPE F: FORCED S: SCHEDULED</u>	<u>DURATION (HOURS)</u>	<u>REASON</u>	<u>METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER</u>	<u>CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)</u>
5	950924	F	102.1	A	2	Manual Reactor Scram due to loss of the 'iB' Turbine Driven Reactor Feed pump during surveillance testing.

SUMMARY OF OPERATION:

The unit remained on-line at high power throughout most of the month. The unit experienced a forced outage on 09/24/95 and was returned to service on 09/28/95. Minor power reductions were required due to rod pattern adjustments, maintenance and surveillance testing.

ATTACHMENT A

SOR dp SWITCH FAILURE DATA SHEET

Equipment Piece Number: 1PDS-E12N010AA

Model Number: 103AS-B202-NX-JJTTX6

Serial Number: 92-3-7256

Application: '1A' Residual Heat Removal flow pressure differential switch

Date and Time of Discovery: 07/17/95

1544 hours

Reactor Mode: 1 (Run)

Power Level: 90%

Calibration Tolerance: 23.6 - 24.4 " W.C.

Nominal Setpoint: 23.0 " W.C.

Action Limits: < 17.7 or > 28.3 " W.C.

Reject Limits: < 15.4 or > 30.6 " W.C.

Technical Specification

Limits: 5.9 " W.C.

As Found Setpoint: N/A " W.C.

Date and Time of Return to Service: 07/18/95

1245 hours

Model Number of Replacement Switch: 103AS-B202-NX-JJTTX6

Serial Number of Replacement Switch: 92-5-7835

PIF Number: 373-201-95-01420PIF

Cause: Switch failed diaphragm integrity test.

Corrective Action: The switch was taken out of service and replaced.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

September 1995

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 3. Static O-Ring Failures
 - 4. Off-Site Dose Calculation Manual Changes

I. INTRODUCTION (UNIT 2)

The LaSalle County Nuclear Power Station is a two-Unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois river for make-up and blowdown. The architect-engineer was Sargent and Lundy and the contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on October 19, 1984.

This report was compiled by Michael J. Cialkowski, telephone number (815) 357-6761, extension 2056.

II. MONTHLY REPORT

A. SUMMARY OF OPERATING EXPERIENCE (Unit 2)

<u>Day</u>	<u>Time</u>	<u>Event</u>
1	0000	Reactor critical, Generator on-line at 1030 Mwe. Power level held due to Main Condenser back pressure.
4	0500	Reduced power level to 900 Mwe to take the Circulating Water Pump off-line due to traveling screen repairs.
15	1500	Commenced ramp down in power for maintenace outage for repairs of the '2A' Reactor Recirculation Flow Control Valve.
16	0750	Manual Reactor scram.
23	1655	Reactor Critical.
24	0523	Generator on-line at 60 Mwe.
26	0434	Generator at 390 Mwe, upshifted Reactor Recirculation Pumps.
	0800	Increased power level to 770 Mwe.
	1030	Increased power level to 930 Mwe.
	1800	Reduced power level to 880 Mwe for performance of a rod set.
	2000	Increased power level to 1000 Mwe.
29	0100	Reduced power level to 740 Mwe for performance of a rod set.
	0700	Increased power level to 1130 Mwe.
30	2400	Reactor critical, Generator on-line at 1130 Mwe.

B. AMENDMENTS TO THE FACILITY OR TECHNICAL SPECIFICATION

On September 27, 1995, Amendment 92 was issued to license NPF-18 (Unit 2). This amendment allowed the extension of surveillance test intervals to accommodate the start date change of the L1R07 outage from September 30, 1995 to January 27, 1996.

C. SUBMITTED LICENSEE EVENT REPORTS (Unit 2)
(None)

D. DATA TABULATIONS (Unit 2)

1. Operating Data Report (See Table 1)
2. Average Daily Unit Power Level (See Table 2)
3. Unit Shutdowns and Significant Power Reductions (See Table 3)

E. UNIQUE REPORTING REQUIREMENTS (UNIT 2)

1. Safety Relief Valve Operations
(None)
2. Major Changes to Radioactive Waste Treatment Systems
(None)
3. Static O-Ring Failures
(None)
4. Changes to the Off-Site Dose Calculation Manual
(None)

TABLE 1
D.1 OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE October 11, 1995
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

OPERATING STATUS

1. REPORTING PERIOD: September 1995
GROSS HOURS IN REPORTING PERIOD: 720
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3,323
MAX DEPEND CAPACITY (MWe-Net): 1,036
DESIGN ELECTRICAL RATING (MWe-Net): 1,078

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net):

4. REASONS FOR RESTRICTION (IF ANY):

	REPORTING PERIOD DATA		
	THIS MONTH	YEAR-TO-DATE	CUMULATIVE
5. REACTOR CRITICAL TIME (HOURS)	542.9	3,872.6	69,080.6
6. REACTOR RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	1,716.9
7. GENERATOR ON-LINE TIME (HOURS)	530.5	3,647.0	67,632.5
8. UNIT RESERVE SHUTDOWN TIME (HOURS)	0.0	0.0	0.0
9. THERMAL ENERGY GENERATED (MWh _t)	1,393,378	10,939,041	204,690,969
10. ELECTRICAL ENERGY GENERATED (MWe-Gross)	466,474	3,715,124	68,403,693
11. ELECTRICAL ENERGY GENERATED (MWe-Net)	446,327	3,554,336	65,739,098
12. REACTOR SERVICE FACTOR (%)	75.4	59.1	72.0
13. REACTOR AVAILABILITY FACTOR (%)	75.4	59.1	73.7
14. UNIT SERVICE FACTOR (%)	73.7	55.7	70.5
15. UNIT AVAILABILITY FACTOR (%)	73.7	55.7	70.5
16. UNIT CAPACITY FACTOR (USING MDC) (%)	59.8	52.4	66.1
17. UNIT CAPACITY FACTOR (USING DESIGN MWe) (%)	57.5	50.3	63.5
18. UNIT FORCED OUTAGE FACTOR (%)	26.3	4.9	10.3

19. SHUTDOWNS SCHEDULED OVER THE NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): N/A

20. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

TABLE 2
D.2 AVERAGE DAILY UNIT POWER LEVEL (MWe-Net)

DOCKET NO. 050-374
UNIT LASALLE TWO
DATE October 11, 1995
COMPLETED BY M.J. CIALKOWSKI
TELEPHONE (815)-357-6761

REPORT PERIOD: September 1995

DAY	POWER	DAY	POWER
1	1,002	17	-12
2	1,001	18	-12
3	1,003	19	-12
4	893	20	-12
5	876	21	-12
6	880	22	-12
7	884	23	-12
8	890	24	-17
9	890	25	141
10	887	26	728
11	888	27	1,041
12	887	28	999
13	887	29	1,041
14	883	30	1,092
15	834	31	
16	72		

TABLE 3

D.3 UNIT SHUTDOWNS AND POWER REDUCTIONS > 20%
(UNIT 2)

<u>YEARLY SEQUENTIAL NUMBER</u>	<u>DATE (YYMMDD)</u>	<u>TYPE F: FORCED S: SCHEDULED</u>	<u>DURATION (HOURS)</u>	<u>REASON</u>	<u>METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER</u>	<u>CORRECTIVE ACTIONS/COMMENTS (LER # if applicable)</u>
3	950916	F	189.5	A	2	Manual Reactor scram for maintenance of the '2A' Reactor Recirculation flow control valve.

SUMMARY OF OPERATION:

The unit remained on-line at high power throughout most of the month. The unit experienced a forced outage on 09/16/95 and was returned to service on 09/24/95. Several minor power reductions were required during the month due to rod pattern adjustments.