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LWP-95-091

October 5, 1995

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of September 1995.

Respectfully,

ComEd
Quad-Cities Nuclear Power Station

L. W. Pearce
Station Manager

LWP/dak

Enclosure

cc: H. Miller, Regional Administrator
C. Miller, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

September 1995

COMMONWEALTH EDISON COMPANY

AND

MID-AMERICAN ENERGY COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

TABLE OF CONTENTS

- I. Introduction
- II. Summary of Operating Experience
 - A. Unit One
 - B. Unit Two
- III. Plant or Procedure Changes, Tests, Experiments, and Safety Related Maintenance
 - A. Amendments to Facility License or Technical Specifications
 - B. Facility or Procedure Changes Requiring NRC Approval
 - C. Tests and Experiments Requiring NRC Approval
- IV. Licensee Event Reports
- V. Data Tabulations
 - A. Operating Data Report
 - B. Average Daily Unit Power Level
 - C. Unit Shutdowns and Power Reductions
- VI. Unique Reporting Requirements
 - A. Main Steam Relief Valve Operations
 - B. Control Rod Drive Scram Timing Data
- VII. Refueling Information
- VIII. Glossary

I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Mid-American Energy Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Kristal Moore and Debra Kelley, telephone number 309-654-2241, extensions 3070 and 2240.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One spent the month of September 1995 at full power and has been on line for 260 days. A load drop was taken on September 14, 1995 to perform Control Rod Pattern Adjustment. After the Control Rod Pattern Adjustment was completed, the Prime Computer failed affecting Core Monitoring Code. Therefore, the scheduled load drop revolved into a forced load drop.

B. Unit Two

Quad Cities Unit Two started the month of September 1995 shutdown in a Forced Outage due to a auto scram that occurred during EHC Testing in the month of August. The reactor went critical on September 1, 1995 at 2141 hours and the generator was synched to the grid on September 2, 1995 at 1511 hours. A few load drops were performed, however the average daily level remained at 80% or greater.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

Technical Specification Amendment No. 157 was issued on June 23, 1995 to Facility Operating License DPR-29 and Amendment No. 153 to Facility Operating License DPR-30 for Quad Cities Nuclear Power Station. The amendments would revise the Technical Specification Section 3/4.10 (Refueling Operations). These changes are part of the Technical Specification Upgrade Program (TSUP).

Technical Specification Amendment No. 158 was issued on July 27, 1995 to Facility Operating License DPR-29 and Amendment No. 154 to Facility Operating License DPR-30 for Quad Cities Nuclear Power Station. The amendments would revise the Technical Specification Section 3.3 (Reactivity Controls). These changes are part of the Technical Specification Upgrade Program (TSUP).

Technical Specification Amendment No. 159 was issued on September 11, 1995 to Facility Operating License DPR-29 and Amendment No. 155 to Facility Operating License DPR-30 for Quad Cities Nuclear Power Station. The amendments revise the Quad Cities Nuclear Power Station, Units 1 and 2, operating licenses to reflect the transfer of the Iowa-Illinois Gas and Electric Company's (IIGEC) 25 percent undivided ownership to Mid-American Energy Company.

Technical Specification Amendment No. 160 was issued on September 18, 1995 to Facility Operating License DPR-29 and Amendment No. 156 to Facility Operating License DPR-30 for Quad Cities Nuclear Power Station. The amendments would revise the Technical Specification Section 3/4.9 (Electrical Power Systems). These changes are part of the Technical Specification Upgrade Program (TSUP).

Technical Specification Amendment No. 161 was issued on September 20, 1995 to Facility Operating License DPR-29 and Amendment No. 157 to Facility Operating License DPR-30 for Quad Cities Nuclear Power Station. The amendments would revise the Technical Specification Section 3/4.1 (Reactor Protection Systems). These changes are part of the Technical Specification Upgrade Program (TSUP).

Technical Specification Amendment No. 162 was issued on September 21, 1995 to Facility Operating License DPR-29 and Amendment No. 158 to Facility Operating License DPR-30 for Quad Cities Nuclear Power Station. The amendments would revise the Technical Specification Section 3/4.6 (Primary System Boundary). These changes are part of the Technical Specification Upgrade Program (TSUP).

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS,
AND SAFETY RELATED MAINTENANCE

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

UNIT 1

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of occurrence</u>
95-006	8-18-95	1A Recirc Pump speed increased Rx thermal. This LER was downgraded.

UNIT 2

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of occurrence</u>
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There were no Licensee Event Reports for Unit 2 for this reporting period.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Operating Data Report
- B. Average Daily Unit Power Level
- C. Unit Shutdowns and Power Reductions

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-254
 UNIT One
 DATE October 5, 1995
 COMPLETED BY Kristal Moore
 TELEPHONE (309) 654-2241

OPERATING STATUS

0000 090195
 1. REPORTING PERIOD: 2400 093095 GROSS HOURS IN REPORTING PERIOD: 720

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769
 DESIGN ELECTRICAL RATING (MWe-NET): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	720.00	6319.80	158752.30
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	3421.90
7. HOURS GENERATOR ON LINE	720.00	6264.00	154087.90
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	909.20
9. GROSS THERMAL ENERGY GENERATED (MWH)	1755983.00	15178836.10	334334959.50
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	557809.00	4840694.00	108304548.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	533277.00	4637876.00	102226042.00
12. REACTOR SERVICE FACTOR	100.00	96.47	77.17
13. REACTOR AVAILABILITY FACTOR	100.00	96.47	78.83
14. UNIT SERVICE FACTOR	100.00	95.62	74.90
15. UNIT AVAILABILITY FACTOR	100.00	95.62	75.34
16. UNIT CAPACITY FACTOR (Using MDC)	96.32	92.06	64.62
17. UNIT CAPACITY FACTOR (Using Design MWe)	93.87	89.73	62.98
18. UNIT FORCED OUTAGE RATE	0.00	4.38	7.36

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP:

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT Two

DATE October 5, 1995

COMPLETED BY Kristal Moore

TELEPHONE (309) 654-2241

OPERATING STATUS

0000 090195

1. REPORTING PERIOD: 2400 093095 GROSS HOURS IN REPORTING PERIOD: 720

2. CURRENTLY AUTHORIZED POWER LEVEL (Mw): 2511 MAX > DEPEND > CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-NET): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	698.30	2659.80	152435.65
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	2985.80
7. HOURS GENERATOR ON LINE	680.80	2566.70	148498.35
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	702.90
9. GROSS THERMAL ENERGY GENERATED (MWH)	1423117.50	5197103.92	321137808.02
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	442821.00	1612020.00	103125921.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	422180.00	1528435.00	97684344.00
12. REACTOR SERVICE FACTOR	96.99	40.60	74.66
13. REACTOR AVAILABILITY FACTOR	96.99	40.60	76.13
14. UNIT SERVICE FACTOR	94.56	39.18	72.74
15. UNIT AVAILABILITY FACTOR	94.56	39.18	73.08
16. UNIT CAPACITY FACTOR (Using MDC)	76.25	30.34	62.22
17. UNIT CAPACITY FACTOR (Using Design MWe)	74.32	29.57	60.64
18. UNIT FORCED OUTAGE RATE	5.42	11.21	9.95

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP:

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE October 5, 1995
COMPLETED BY Kristal Moore
TELEPHONE (309) 654-2241

MONTH September 1995

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u>739</u>
2.	<u>739</u>
3.	<u>747</u>
4.	<u>750</u>
5.	<u>752</u>
6.	<u>754</u>
7.	<u>754</u>
8.	<u>756</u>
9.	<u>755</u>
10.	<u>751</u>
11.	<u>754</u>
12.	<u>760</u>
13.	<u>754</u>
14.	<u>557</u>
15.	<u>712</u>
16.	<u>749</u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u>742</u>
18.	<u>740</u>
19.	<u>756</u>
20.	<u>723</u>
21.	<u>767</u>
22.	<u>720</u>
23.	<u>688</u>
24.	<u>753</u>
25.	<u>737</u>
26.	<u>768</u>
27.	<u>768</u>
28.	<u>766</u>
29.	<u>770</u>
30.	<u>723</u>
31.	<u> </u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE October 5, 1995
COMPLETED BY Kristal Moore
TELEPHONE (309) 654-2241

MONTH September 1995

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1.	<u> - 9 </u>
2.	<u> 38 </u>
3.	<u> 301 </u>
4.	<u> 373 </u>
5.	<u> 333 </u>
6.	<u> 423 </u>
7.	<u> 514 </u>
8.	<u> 593 </u>
9.	<u> 661 </u>
10.	<u> 652 </u>
11.	<u> 667 </u>
12.	<u> 668 </u>
13.	<u> 667 </u>
14.	<u> 660 </u>
15.	<u> 657 </u>
16.	<u> 657 </u>

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17.	<u> 653 </u>
18.	<u> 619 </u>
19.	<u> 656 </u>
20.	<u> 646 </u>
21.	<u> 664 </u>
22.	<u> 652 </u>
23.	<u> 714 </u>
24.	<u> 714 </u>
25.	<u> 720 </u>
26.	<u> 769 </u>
27.	<u> 754 </u>
28.	<u> 765 </u>
29.	<u> 719 </u>
30.	<u> 679 </u>
31.	<u> </u>

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME One

COMPLETED BY Kristal Moore

DATE October 5, 1995 REPORT MONTH September 1995

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
95-08	9/14/95	S	0	H	5	-----	---	----	Control Rod Pattern Adjustment
95-09	9/14/95	F	0	A	5	-----	---	----	Prime Computer Failed Affecting Core Monitoring Code

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Two

COMPLETED BY Kristal Moore

DATE October 5, 1995 REPORT MONTH September 1995

TELEPHONE 309-654-2241

NO.	DATE	TYPE FOR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
95-08	9-01-95	F	39.2	B	3	-----	---	----	Continued Forced Outage.

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

Unit: One

Date: September 10, 1995

Valve Actuated: No. & Type of Actuation:

1-203-3A	1 Manual
1-203-3B	1 Manual
1-203-3C	1 Manual
1-203-3D	1 Manual
1-203-3E	1 Manual

Plant Conditions: Reactor Pressure 1002 psig

Description of Events: Surveillance Testing

B. Control Rod Drive Scram Timing Data for Units One and Two

There was no Control Rod Drive scram timing data for Units One and Two for the reporting period.

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QUAD CITIES REFUELING
INFORMATION REQUEST

1. Unit: Q1 Reload: 13 Cycle: 14
2. Scheduled date for next refueling shutdown: 2/5/96
3. Scheduled date for restart following refueling: 5/15/96
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

NO
5. Scheduled date(s) for submitting proposed licensing action and supporting information:

7-4-95
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

232 GE10 Fuel Bundles will be loaded during Q1R14.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 1717
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3657
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2006

QUAD CITIES REFUELING
INFORMATION REQUEST

1. Unit: Q2 Reload: 13 Cycle: 14
2. Scheduled date for next refueling shutdown: 1-6-97
3. Scheduled date for restart following refueling: 3-30-97
4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

YES
5. Scheduled date(s) for submitting proposed licensing action and supporting information:

November, 1996
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

Approx. 200 Siemens 9X9IX Power Corporation Fuel Bundles will be loaded during Q2R14.
7. The number of fuel assemblies.
 - a. Number of assemblies in core: 724
 - b. Number of assemblies in spent fuel pool: 3377
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:
 - a. Licensed storage capacity for spent fuel: 3897
 - b. Planned increase in licensed storage: 0
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	- Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	- American National Standards Institute
APRM	- Average Power Range Monitor
ATWS	- Anticipated Transient Without Scram
BWR	- Boiling Water Reactor
CRD	- Control Rod Drive
EHC	- Electro-Hydraulic Control System
EOF	- Emergency Operations Facility
GSEP	- Generating Stations Emergency Plan
HEPA	- High-Efficiency Particulate Filter
HPCI	- High Pressure Coolant Injection System
HRSS	- High Radiation Sampling System
IPCLRT	- Integrated Primary Containment Leak Rate Test
IRM	- Intermediate Range Monitor
ISI	- Inservice Inspection
LER	- Licensee Event Report
LLRT	- Local Leak Rate Test
LPCI	- Low Pressure Coolant Injection Mode of RHRs
LPRM	- Local Power Range Monitor
MAPLHGR	- Maximum Average Planar Linear Heat Generation Rate
MCPR	- Minimum Critical Power Ratio
MFLCPR	- Maximum Fraction Limiting Critical Power Ratio
MPC	- Maximum Permissible Concentration
MSIV	- Main Steam Isolation Valve
NIOSH	- National Institute for Occupational Safety and Health
PCI	- Primary Containment Isolation
PCIOMR	- Preconditioning Interim Operating Management Recommendations
RBCCW	- Reactor Building Closed Cooling Water System
RBM	- Rod Block Monitor
RCIC	- Reactor Core Isolation Cooling System
RHRS	- Residual Heat Removal System
RPS	- Reactor Protection System
RWM	- Rod Worth Minimizer
SBGTS	- Standby Gas Treatment System
SBLC	- Standby Liquid Control
SDC	- Shutdown Cooling Mode of RHRs
SDV	- Scram Discharge Volume
SRM	- Source Range Monitor
TBCCW	- Turbine Building Closed Cooling Water System
TIP	- Traversing Incore Probe
TSC	- Technical Support Center