UCENSEE EVENT REPORT (LER)												U.S. NUCLEAR REGULATORY COMMISSION APTROVED OMB NO. 3150-0104 EXPIRES: 8/31/85															
ACILITY	-)			-													DOCK	T NU	MBE	R (2)				P.	AGE	31
Dresden Nuclear Power Stat								ati	ion. Unit 3						0 1	0 15 10 10			101214 0		0	10	FO	12			
TITLE (4)		3-16	01-	63 1	7.21	lve	In	0.00	rah	10									_	-	-		-		-		
EVE	INT DATE	(5)	T	0.5	LEP	NU	AREA	in	:Lau	T		PORT	DATE	(7)			-	EACH	TIER	INV	NVE	0 /11					
MONTH	DAY	YEAR	YE	YEAR SEQUENTIAL				REVER	×	ONTH	TOA	V VEAR		FACILITY NAMES						DOCKET NUMBER(S)							
				NUMBER				NUMBE	BER							N/A			0	15	1010		0	1			
0 8	3 0	8 4	8	4-	-0	1	1 3	-	00	0	Þ	2	6	8 4		N/A				0	15	10	0	0,	1	1	
OPE	RATING		THE	S REPO	-		MITT	ED PU	REUAN	TTO	THE R	EQUIR	EME		CFR - 10	Sheck and	or more	of the	follow	ng) (11)		-				-
NAME				20.406(a)(1)(i) 20.406(a)(1)(ii) 20.406(a)(1)(iii) 20.406(a)(1)(iii) 20.406(a)(1)(iv) 20.406(a)(1)(v)						20.405(c) 50.36(c)(1) 50.36(c)(2) 50.73(c)(2)(1) 50.73(c)(2)(1) 50.73(c)(2)(1) 50.73(c)(2)(1) 50.73(c)(2)(1) 50.73(c)(2)(1)				60,73(a)(2)(*) 50,73(a)(2)(*) 50,73(a)(2)(*ii) 50,73(a)(2)(*ii)(A) 50,73(a)(2)(*iii)(B) 50,73(a)(2)(x) ER (12)				73,71(6) 73,71(6) 07HER / Sam Wow and in 366.4) TELEPHONE NUME				city in Abstract Text, NRC Form					
		Edwa	rd	J. 1	Kot	tri	ch			(X	-52	(3)						A 00	AEA 0	1 5	9	14	12	-	21	912	210
		_			_	COM	PLETE	ONE	LINE F	OR E	ACH C	OMPON	ENT	FAILURE	DESCRIBE	D IN TH	S REPO	RT (13)									
CAUSE	SYSTEM	COMP	ONEN	т	MANUFAC- TURER			REPORTABLE TO NPROS		E				CAUSE	SYSTEM	COMP	COMPONENT	MANUFAC. TURER		-	EPORTABLE			•			
A	1	1	1		1	1	1		N							1	11			1							
		1	1		1	1	1										1.1		1	1							
						SUP	PLEM	ENTA	L REPOI	RT E	XPECTI	ED (14)	1										MO	NTH	DAY	Y	EAR
YES III yes, complete EXPECTED SUBMISSION DATE!									X NO						SUBMISSIO			10N				1		1			

At 1900 hours, the 1601-63 valve, a butter ly valve between the drywell and stand-by gas system, was taken Out-of-Service to repair an air leak. Upon a recheck of the applicable Tech Specs pertaining to the primary containment purge system it was realized that section 3.7.A.6.c requires that the reactor must be taken out of power operation if the primary containment purge system is not operable. At 2055 hours an unusual event was declared and the NRC was notified as the reactor began decreasing load. The outage was cleared, the valve was cycled successfully and the GSEP was terminated at 2235 hours.

IE22

1/1

Safety significance was minimal.

8410120056 840926 PDR ADDCK 05000249 PDR

NRC Form 368 (9-83)

4

NRC Form 366A

...

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES 8/31/85

ACILITY NAME (1)	DOCKET NUMBER (2)	1	LER NUMBER (6)	PAGE (3)			
		NABY	SEQUENTIAL	REVISION		T	
Dresden Nuclear Power Station, Uni	t 3 0 5 0 0 0 2 4 9	814 -	- 01113	- 0 10	012	OF	0 2
TEXT (If more space is required, use additional NRC Form 3864's) (17)							

On August 29, 1984 an operator discovered an air leak in valve 3-1601-63, an air operated butterfly valve between the drywell and the stand-by gas treatment system. A work request was written to repair the air leak. As a part of the Out-of-Service required for the repair an outage was taken on valve 3-1601-63 at 1900 hours on August 30, 1984.

It was realized shortly thereafter that the Tech Spec, section 3.7.A.6.c, requires that the reactor be taken out of power operation when the containment purge system is inoperable. An unusual event was declared and the NRC was immediately notified at 2100 hours as the reactor began a shutdown.

The outage on the valve was cleared at 2200 hours and the valve was successfully tested at 2215 hours. The GSEP was terminated at 2235 hours and the reactor resumed normal operations.

The safety significance of this event was minimal because the integrity of the primary containment was not breached, normal venting of the drywell and torus was still available through the reactor building ventilation system since Indine and Beta-Gamma readings were below required limits, and the outage error was discovered promptly so that a properly controlled shutdown of the unit could be commenced within the time frame allowed for inoperative containment purge systems.

This is the first reported occurrence of this type at Dresden.

To prevent future recurrence;

- The Operating Engineer involved has been made aware of the necessity to research all applicable Tech Spec limitations before taking equipment Out-of-Service.
- 2) This LER will be included in the Licensed Operator required readings.

.



Commonwealth Edison Dresden Nuclear Power Station R.R. #1 Morris, Illinois 60450 Telephone 815/942-2920

September 26, 1984

0

Q.

DJS Ltr #84-967

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Licensee Event Report #84-013-0, Docket #050249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(i)(B).

D.J. Scott Station Superintendent Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

J.E22

-